



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

April 4, 2022

Mr. Eric Carr  
President and Chief Nuclear Officer  
PSEG Nuclear LLC - N09  
P.O. Box 236  
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 – ISSUANCE OF AMENDMENT NOS. 343 AND 324 RE: REVISE TECHNICAL SPECIFICATIONS SURVEILLANCE REQUIREMENTS FOR AUXILIARY FEEDWATER (EPID L-2021-LLA-0117)

Dear Mr. Carr:

The U.S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment Nos. 343 and 324 to Renewed Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating Station, Unit Nos. 1 and 2, respectively. These amendments consist of changes to the technical specifications (TSs) in response to your application dated June 17, 2021.

The changes revised the Salem Nuclear Generating Station (Salem), Unit 2, Technical Specification (TS) Table 4.3-2 Functional Unit 8.f, "Auxiliary Feedwater-Trip of Main Feedwater Pumps," Channel Functional Test surveillance frequency and the Mode in which Salem, Unit 1, TS Table 4.3-2 Functional Unit 8.f, "Auxiliary Feedwater-Trip of Main Feedwater Pumps" is required. The changes also removed Salem, Unit 2, Surveillance Requirement 4.7.1.3.4 to verify the service water spool piece is onsite.

A copy of the related safety evaluation is also enclosed. A Notice of Issuance will be included in the Commission's monthly *Federal Register* notice.

Sincerely,

***/RA/***

James S. Kim, Project Manager  
Plant Licensing Branch I  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

Enclosures:

1. Amendment No. 343 to DPR-70
2. Amendment No. 324 to DPR-75
3. Safety Evaluation

cc: Listserv



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

PSEG NUCLEAR LLC

CONSTELLATION ENERGY GENERATION, LLC

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 343  
Renewed License No. DPR-70

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by PSEG Nuclear LLC, acting on behalf of itself and Exelon Generation Company, LLC (the licensees), dated June 17, 2021, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-70 is hereby amended to read as follows:

- (2) Technical Specifications and Environmental Protection Plan

- The Technical Specifications contained in Appendix A, as revised through Amendment No. 343, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications, and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

James G. Danna, Chief  
Plant Licensing Branch I  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to Renewed Facility Operating  
License and Technical Specifications

Date of Issuance: April 4, 2022

ATTACHMENT TO LICENSE AMENDMENT NO. 343  
SALEM NUCLEAR GENERATING STATION, UNIT NO. 1  
RENEWED FACILITY OPERATING LICENSE NO. DPR-70  
DOCKET NO. 50-272

Replace the following page of Renewed Facility Operating License No. DPR-70 with the attached revised page as indicated. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove  
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Insert  
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Replace the following page of the Appendix A, Technical Specifications, with the attached revised page as indicated. The revised page is identified by amendment number and contain marginal lines indicating the areas of change.

Remove  
3/4 3-33

Insert  
3/4 3-33

- (4) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (5) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (6) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

PSEG Nuclear LLC is authorized to operate the facility at a steady state reactor core power level not in excess of 3459 megawatts (one hundred percent of rated core power).

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 343, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications, and the Environmental Protection Plan.

(3) Deleted Per Amendment 22, 11-20-79

(4) Less than Four Loop Operation

PSEG Nuclear LLC shall not operate the reactor at power levels above P-7 (as defined in Table 3.3-1 of Specification 3.3.1.1 of Appendix A to this renewed license) with less than four (4) reactor coolant loops in operation until safety analyses for less than four loop operation have been submitted by the licensees and approval for less than four loop operation at power levels above P-7 has been granted by the Commission by Amendment of this renewed license.

(5) PSEG Nuclear LLC shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety

TABLE 4.3-2 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION  
SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK<sup>(7)</sup></u>	<u>CHANNEL CALIBRATION<sup>(7)</sup></u>	<u>CHANNEL FUNCTIONAL TEST<sup>(7)</sup></u>	<u>MODES IN WHICH SURVEILLANCE REQUIRED</u>
8. AUXILIARY FEEDWATER				
a. Automatic Actuation Logic	N.A.	N.A.	(2)	1,2,3
b. NOT USED				
c. Steam Generator Water Level--Low-Low				1,2,3
d. Undervoltage - RCP				1,2
e. S.I.	See 1 above (All S.I. surveillance requirements)			
f. Trip of Main Feedwater Pumps	N.A.	N.A.		1,2
g. Station Blackout	See 6b and 7 above (SEC and U/V Vital Bus)			



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

PSEG NUCLEAR LLC

CONSTELLATION ENERGY GENERATION, LLC

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 324  
Renewed License No. DPR-75

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment filed by PSEG Nuclear LLC, acting on behalf of itself and Exelon Generation Company, LLC (the licensees), dated June 17, 2021, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.



2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 324, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

James G. Danna, Chief  
Plant Licensing Branch I  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to Renewed Facility Operating  
License and Technical Specifications

Date of Issuance: April 4, 2022

ATTACHMENT TO LICENSE AMENDMENT NO. 324  
SALEM NUCLEAR GENERATING STATION, UNIT NO. 2  
RENEWED FACILITY OPERATING LICENSE NO. DPR-75  
DOCKET NO. 50-311

Replace the following page of Renewed Facility Operating License No. DPR-75 with the attached revised page as indicated. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove  
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Insert  
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Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages as indicated. The revised pages are identified by amendment number and contains marginal lines indicating the areas of change.

Remove  
3/4 3-36  
3/4 3-37  
3/4 7-7

Insert  
3/4 3-36  
3/4 3-37  
3/4 7-7

- (3) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
  - (4) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use at any time any byproduct, source or special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration and as fission detectors in amounts as required;
  - (5) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
  - (6) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level  
  
PSEG Nuclear LLC is authorized to operate the facility at steady state reactor core power levels not in excess of 3459 megawatts (thermal).
  - (2) Technical Specifications and Environmental Protection Plan  
  
The Technical Specifications contained in Appendix A, as revised through Amendment No. 324, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

TABLE 4.3-2 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION  
SURVEILLANCE REQUIREMENTS

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK<sup>(7)</sup></u>	<u>CHANNEL CALIBRATION<sup>(7)</sup></u>	<u>CHANNEL FUNCTIONAL TEST<sup>(7)</sup></u>	<u>MODES IN WHICH SURVEILLANCE REQUIRED</u>
8. AUXILIARY FEEDWATER				
a. Automatic Actuation Logic	N.A.	N.A.	(2)	1,2,3
b. NOT USED				
c. Steam Generator Water Level--Low-Low				1,2,3
d. Undervoltage - RCP				1,2
e. S.I.	See 1 above (All S.I. surveillance requirements)			
f. Trip of Main Feedwater Pumps	N.A.	N.A.		1,2
g. Station Blackout	See 6 and 7 above (SEC and U/V Vital Bus)			
9. SEMIAUTOMATIC TRANSFER TO RECIRCULATION				
a. RWST Low Level				1,2,3
b. Automatic Initiation Logic	N.A.	N.A.	(2)	1,2,3,4

TABLE 4.3-2 (Continued)

TABLE NOTATION

- \* Outputs are up to, but not including, the Output Relays.
- \*\* The provisions of Specification of 4.0.4 are not applicable.
- (1) Each logic channel shall be tested in accordance with the Surveillance Frequency Control Program. The CHANNEL FUNCTIONAL TEST of each logic channel shall verify that its associated diesel generator automatic load sequence timer is OPERABLE with the interval between each load block within 1 second of its design interval.
- (2) Each train or logic channel shall be tested in accordance with the Surveillance Frequency Control Program.
- (3) The CHANNEL FUNCTIONAL TEST shall include exercising the transmitter by applying either a vacuum or pressure to the appropriate side of the transmitter.
- (4) NOT USED
- (5) NOT USED
- (6) Inputs from undervoltage, Vital Bus, shall be tested in accordance with the Surveillance Frequency Control Program. Inputs from Solid State Protection System, shall be tested in accordance with the Surveillance Frequency Control Program.
- (7) Frequencies are specified in the Surveillance Frequency Control Program unless otherwise noted in the table.
  
- (a) Except when all MSIVs are closed.
- (b) Except when all main feedwater lines are isolated by (1) a closed and de-activated feedwater isolation valve, or (2) closed and de-activated feedwater regulating valve (FRV) and FRV bypass valves, or (3) a closed manual valve.

## PLANT SYSTEMS

### AUXILIARY FEED STORAGE TANK

#### LIMITING CONDITION FOR OPERATION

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3.7.1.3 The auxiliary feed storage tank (AFST) shall be OPERABLE with a contained volume of at least 200,000 gallons of water.

APPLICABILITY: MODES 1, 2 and 3.

#### ACTION:

With the auxiliary feed storage tank inoperable, within 4 hours either:

- a. Restore the AFST to OPERABLE status or be in HOT SHUTDOWN within the next 12 hours, or
- b. Demonstrate the OPERABILITY of a demineralized water or a fire protection/domestic water storage tank as a backup supply to the auxiliary feedwater pumps and restore the auxiliary feed storage tank to OPERABLE status within 7 days or be in HOT SHUTDOWN within the next 12 hours.

#### SURVEILLANCE REQUIREMENTS

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4.7.1.3.1 The auxiliary feed storage tank shall be demonstrated OPERABLE in accordance with the Surveillance Frequency Control Program by verifying the water level is within its limits when the tank is the supply source for the auxiliary feedwater pumps.

4.7.1.3.2 A demineralized water storage tank shall be demonstrated OPERABLE in accordance with the Surveillance Frequency Control Program by verifying the tank contains greater than or equal to 200,000 gallons of water and by verifying proper alignment of valves for taking suction from this tank when it is the supply source for the auxiliary feedwater pumps.

4.7.1.3.3 A fire protection/domestic water storage tank shall be demonstrated OPERABLE in accordance with the Surveillance Frequency Control Program by verifying the tank contains greater than or equal to 200,000 gallons of water and by verifying proper alignment of valves for taking suction from this tank when it is the supply source for the auxiliary feedwater pumps.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 343 AND 324 TO

RENEWED FACILITY OPERATING LICENSE NOS. DPR-70 AND DPR-75

PSEG NUCLEAR LLC

CONSTELLATION ENERGY GENERATION, LLC

SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

1.0 INTRODUCTION

By application dated June 17, 2021 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML21173A090), PSEG Nuclear LLC (PSEG) (the licensee) submitted a license amendment request (LAR) for the Salem Generating Station (Salem), Units 1 and 2.

The amendment includes three proposed changes to Salem, Units 1 and 2, technical specifications (TSs). The first proposed change modifies Salem, Unit 2, TS Table 4.3-2 Functional Unit 8.f "Auxiliary Feedwater - Trip of Main Feedwater Pumps," Channel Functional Test surveillance frequency from an event driven frequency to a periodic frequency controlled in accordance with the Surveillance Frequency Control Program (SFCP). The second proposed change modifies Salem, Unit 1, TS Table 4.3-2 Functional Unit 8.f "Modes in Which Surveillance Required" of the TS table to include Mode 2. The third proposed change deletes Salem, Unit 2, Surveillance Requirements (SR) 4.7.1.3.4, which verifies the onsite presence of a spool piece to demonstrate the capability of the Service Water (SW) system to provide a backup water supply to the Auxiliary Feedwater (AFW) system.

2.0 REGULATORY EVALUATION

In Title 10 of the *Code of Federal Regulations* (10 CFR) 50.36, "Technical specification," the Commission established its regulatory requirements related to the content of the TSs. Pursuant to 10 CFR 50.36(c)(3), SR are requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the limiting conditions for operation will be met.

3.0 TECHNICAL EVALUATION

3.1 Licensee’s Proposed Changes - Channel Functional Test Surveillance for Unit 2 TS Table 4.3-2 Functional Unit 8.f

Salem Unit 2 TS Table 4.3-2 Functional Unit 8 currently reads:

<u>FUNCTIONAL UNIT</u>	<u>CHANNEL CHECK</u> <sup>(7)</sup>	<u>CHANNEL CALIBRATION</u> <sup>(7)</sup>	<u>CHANNEL FUNCTIONAL TEST</u> <sup>(7)</sup>	<u>MODES IN WHICH SURVEILLANCE REQUIRED</u>
f. Trip of Main Feedwater Pumps	N/A	N/A	S/U(4)	1,2

Table 4.3-2. Table Notations:

(4) If not performed in the previous 92 days.

The licensee proposed to modify the Channel Functional Test frequency of “S/U(4)”, which currently requires the Channel Functional Test to be performed prior to each Unit startup if not performed within the prior 92 days, to frequency “R”, which requires the functional test to be performed at least once every 18 months. The licensee also proposed to delete the Note (4) and relocate the new frequency, "R", to the Salem, Unit 2, SFCP.

U.S. Nuclear Regulatory Commission (NRC) Staff’s Evaluation

The AFW system serves as a backup system for supplying feedwater to the secondary side of the steam generators at times when the Main Feedwater (MFW) system is not available. The AFW system is designed to automatically initiate in response to a trip of the MFW system in order to maintain inventory in the steam generators to maintain secondary cooling to the core. In order to assure the proper operation of these systems, the instrumentation and logic associated with this automatic initiation must undergo a routine Channel Functional Test.

The LAR indicates that the current frequency of the Channel Functional Test is a burden to operations and maintenance personnel during startup and results in distractions from more critical actions required during the startup evolution. The licensee identified that the instrumentation and logic configuration between Salem, Unit 1 and Unit 2, are identical, but the surveillance frequency of the Channel Functional Test are different. The frequency for the corresponding Channel Functional Test for Salem, Unit 1, TS Table 4.3-2, Functional Unit 8.f is contained in the Salem SFCP which identifies the frequency as "R", at least once every 18 months. This frequency is also consistent with the frequency identified in NUREG-1431, “Standard Technical Specifications for Westinghouse plants” Revision 5.

The NRC staff reviewed the licensee’s proposed change, the systems description provided in the LAR, and the Salem, Unit 2, final safety analysis report and confirmed that a Channel Functional Test surveillance frequency of every 18 months, would be consistent with the testing frequency described in NUREG-1431, and with the testing frequency used in Salem, Unit 1. Based on its review, the staff finds that changing the Channel Functional Test surveillance frequency to every 18 months does not reduce the margin of safety nor does it create or



increase the probability of an accident. Therefore, the staff finds the proposed change to modify Salem, Unit 2, TS Table 4.3-2 Functional Unit 8.f "Auxiliary Feedwater - Trip of Main Feedwater Pumps," Channel Functional Test surveillance frequency from every startup if not performed within the past 92 days to once per 18 months is acceptable because the new SR remains consistent with the requirement specified in 10 CFR 50.36(c)(3).

The licensee also proposed to relocate the surveillance frequency from Salem, Unit 2, TS Table 4.3-2 to the licensee-controlled surveillance frequency control program. The LAR indicates that Salem, Units 1 and 2 have implemented (with NRC approval of Amendment Nos. 299 and 282 (ADAMS Accession No. ML110410691)) TSTF Change Traveler TSTF-425, Revision 3 (ADAMS Accession No. ML090850642) "Relocate Surveillance Frequencies to Licensee Control - RITSTF [Risk-Informed Technical Specifications Task Force] Initiative 5b." As part of the implementation of this TSTF, the Salem, Unit 1, "Auxiliary Feedwater - Trip of Main Feedwater Pumps," Channel Functional Test surveillance frequency was relocated to the Salem, Unit 1, SFCP.

TSTF-425 indicates that:

All Surveillance Frequencies are relocated except:

- Frequencies that reference other approved programs for the specific interval (such as the Inservice Testing Program or the Primary Containment Leakage Rate Testing Program);
- Frequencies that are purely event driven (e.g., "Each time the control rod is withdrawn to the 'full out' position");
- Frequencies that are event-driven but have a time component for performing the surveillance on a one-time basis once the event occurs (e.g., "within 24 hours after thermal power reaching  $\geq 95\%$  RTP"); and
- Frequencies that are related to specific conditions (e.g., battery degradation, age, and capacity) or conditions for the performance of a surveillance requirement (e.g., "drywell to suppression chamber differential pressure decrease").

Since the Salem, Unit 2, "Auxiliary Feedwater - Trip of Main Feedwater Pumps," Channel Functional Test surveillance frequency is an event-driven frequency, it could not be relocated to the SFCP as part of Amendment 282. The LAR indicates that once the Channel Functional Test surveillance frequency is changed to a periodic surveillance (once per 18 months), it will fall within the scope of TSTF-425 for relocation to the licensee-controlled SFCP.

The NRC staff evaluated the proposed change discussed in the LAR and the staff's issuance of the Amendment Nos. 299 and 282 (ADAMS Accession No. ML110410691) which approved the modifications to the TSs that relocated specific surveillance frequencies to a licensee-controlled program, based on TSTF-425, Revision 3. The staff observed that once Salem, Unit 2, "Auxiliary Feedwater - Trip of Main Feedwater Pumps," Channel Functional Test surveillance frequency has been changed to "once per 18 months," it will fall within the applicability criteria of TSTF-425 for relocation to a licensee-controlled SFCP. The staff also noted that the revised frequency would be consistent with the frequency of Salem, Unit 1, "Auxiliary Feedwater - Trip of Main Feedwater Pumps," Channel Functional Test surveillance, which was relocated to the

SFCP when the licensee implemented TSTF-425. Therefore, the staff finds the licensee's request to relocate the surveillance frequency to the SFCP acceptable, because the SR remains consistent with the requirement specified in 10 CFR 50.36(c)(3).

### 3.2 Licensee's Proposed Changes - Mode of Applicability for Performance of Channel Surveillance Tests for Salem, Unit 1, TS Table 4.3-2 Functional Unit 8.f

Salem, Unit 1, TS Table 4.3-2 Function Unit 8.f currently identifies the required modes as Mode 1 while Unit 2 identifies both Mode 1 and 2. The LAR indicates that the lack of a Mode 2 applicability in Salem, Unit 1, TS Table 4.3-2, Functional Unit 8.f stems from an oversight in a 1992 LAR. The 1992 LAR added Mode 2 in Unit 1 TS Table 4.3-2 Functional Unit 8.f; however, the mode applicability in Table 4.3-2 Functional Unit 8.f was not revised.

#### NRC Staff's Evaluation

The NRC staff reviewed the information provided in the LAR and noted that there is an inconsistency in the modes of applicability between Salem, Unit 1, TS Table 4.3-2 and Salem, Unit 2, TS Table 4.3-2. The licensee's proposed change would correct this inconsistency in the Unit 1 TS and add consistency between both units. The staff finds that the proposed change to add Mode 2 to Salem, Unit 1, TS Table 4.3-2 Function Unit 8.f modes of operability corrects an oversight and increases consistency in the TS; therefore, the staff finds the proposed change acceptable because the SR remains consistent with the requirement specified in 10 CFR 50.36(c)(3).

### 3.3 Licensee's Proposed Changes - Deletion of Unit 2 SR 4.7.1.3.4

The LAR proposes to delete Salem, Unit 2, SR 4.7.1.3.4, which states:

The Service Water System shall be demonstrated capable of providing a water supply to the Auxiliary Feedwater System in accordance with the Surveillance Frequency Control Program by verifying that the required spool-piece is on site.

The LAR indicates that the systems have been modified. The spool piece is no longer necessary to achieve alignment of the SW system to the AFW system; the system alignment is now performed via a permanently installed spectacle flange.

#### NRC Staff's Evaluation

The NRC staff evaluated the LAR and noted that the LAR indicates that Salem, Unit 1, utilizes the same spectacle flange arrangement in order to align the SW system as a backup water supply for the AFW system, and Unit 1 TS does not include a similar SR. The staff noted that the licensee has completed plant modifications that allow the alignment of the SW system to the AFW system without relying on the spool piece. Therefore, the staff finds that Salem, Unit 2, SR 4.7.1.3.4 is no longer required to meet the requirement specified in 10 CFR 50.36(c)(3), and that the licensee's proposed deletion of SR 4.7.1.3.4 is acceptable.

## 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendments on March 11, 2022. The State official had no comments.

## 5.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20 and change SRs. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve NSHC, published in the *Federal Register* on August 10, 2021 (86 FR 43691), and there were no public comments on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

## 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Hernandez

Date: April 4, 2022

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 – ISSUANCE OF AMENDMENT NOS. 343 AND 324 RE: REVISE TECHNICAL SPECIFICATIONS SURVEILLANCE REQUIREMENTS FOR AUXILIARY FEEDWATER (EPID L-2021-LLA-0117) DATED APRIL 4, 2022

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