



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
2443 WARRENVILLE ROAD, SUITE 210
LISLE, ILLINOIS 60532-4352

July 22, 2021

Mr. David Rhoades
Senior VP, Exelon Generation Company, LLC
President and CNO, Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: BRAIDWOOD STATION – INTEGRATED INSPECTION REPORT
05000456/2021002 AND 05000457/2021002

Dear Mr. Rhoades:

On June 30, 2021, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Braidwood Station. On July 13, 2021, the NRC inspectors discussed the results of this inspection with Mr. John Keenan, Site Vice President, and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

/RA/

Hironori Peterson, Chief
Branch 3
Division of Reactor Projects

Docket Nos. 05000456 and 05000457
License Nos. NPF-72 and NPF-77

Enclosure:
As stated

cc w/ encl: Distribution via LISTSERV®

Letter to David Rhoades from Hironori Peterson dated July 22, 2021.

SUBJECT: BRAIDWOOD STATION – INTEGRATED INSPECTION REPORT
05000456/2021002 AND 05000457/2021002

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**U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report**

Docket Numbers: 05000456 and 05000457

License Numbers: NPF-72 and NPF-77

Report Numbers: 05000456/2021002 and 05000457/2021002

Enterprise Identifier: I-2021-002-0063

Licensee: Exelon Generation Company, LLC

Facility: Braidwood Station

Location: Braceville, IL

Inspection Dates: April 01, 2021 to June 30, 2021

Inspectors: D. Betancourt-Roldan, Senior Resident Inspector (Byron)
R. Bowen, Illinois Emergency Management Agency
J. Bozga, Senior Reactor Inspector
G. Edwards, Health Physicist
M. Holmberg, Senior Reactor Inspector
D. Kimble, Senior Resident Inspector (Braidwood)
P. Smagacz, Resident Inspector

Approved By: Hironori Peterson, Chief
Branch 3
Division of Reactor Projects

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting an integrated inspection at Braidwood Station, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

None.

PLANT STATUS

Unit 1 began the inspection period operating at approximately 89 percent power with end-of-cycle power coast down operations in progress. On April 5, 2021, the unit was shut down for its planned 22nd refueling outage (A1R22). Following completion of refueling and other maintenance activities, the reactor was restarted and taken critical on April 23, 2021, and the main electrical generator was synchronized to the power grid on April 24, 2021. The unit reached full power operation on April 27, 2021. On May 22, 2021, power was reduced to approximately 85 percent to facilitate maintenance on the condensate system. Unit 1 returned to full power later that same day. Just after midnight on June 21, 2021, Unit 1 experienced an automatic trip from full power due to a lightning strike in the station's main switchyard during a severe thunderstorm. After post-trip checks verified that the condition of the unit was acceptable for restart, the reactor was taken critical in the evening on June 21, 2021, and the main electrical generator was synchronized to the power grid on the morning of June 22, 2021. The unit reached full power operation just before midnight on June 22, 2021, and remained operating at or near full power for the remainder of the inspection period.

Unit 2 began the inspection period operating at full power. On May 7, 2021, the unit reduced power to approximately 68 percent to facilitate replacement of a failed relief valve on the 23C Low Pressure (LP) Feedwater (FW) Heater. The unit returned to full power operation the following day, and with the exception of minor reductions in power to support scheduled testing activities and load changes requested by the transmission system dispatcher, it remained operating at or near full power for the remainder of the inspection period.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

Starting on March 20, 2020, in response to the National Emergency declared by the President of the United States on the public health risks of the coronavirus (COVID-19), resident and regional inspectors were directed to begin telework and to remotely access licensee information using available technology. During this time, the resident inspectors performed periodic site visits each week, increasing the amount of time on site as local COVID-19 conditions permitted. As part of their onsite activities, resident inspectors conducted plant status activities as described in IMC 2515, Appendix D; observed risk significant activities; and completed on site portions of IPs. In addition, resident and regional baseline inspections were evaluated to determine if all or a portion of the objectives and requirements stated in the IP could be performed remotely. If the inspections could be performed remotely, they were conducted per the applicable IP. In some cases, portions of an IP were completed remotely and on site. The inspections documented below met the objectives and requirements for completion of the IP.

REACTOR SAFETY

71111.01 - Adverse Weather Protection

Seasonal Extreme Weather Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated summer readiness of offsite and alternate alternating current (AC) power systems during the weeks ending June 26 through June 30, 2021.

71111.04 - Equipment Alignment

Partial Walkdown Sample (IP Section 03.01) (4 Samples)

The inspectors evaluated system configurations during partial physical alignment verifications of the following systems/trains:

- (1) The spent fuel pool (SFP) cooling system while the Unit 1 reactor core was offloaded to the SFP for the A1R22 refueling outage, during the week ending April 17, 2021.
- (2) The Unit 1 component cooling system (CC) during maintenance on 1CC9743A, during the week ending May 8, 2021.
- (3) The 2A Emergency Diesel Generator (EDG) while the unit's station auxiliary transformer was undergoing planned maintenance, during the week ending May 15, 2021.
- (4) The 2A Safety Injection (SI) Train while the 2B SI Train was out-of-service for maintenance, during the week ending June 19, 2021.

Complete Walkdown Sample (IP Section 03.02) (1 Sample)

The inspectors evaluated system configurations during a complete physical alignment verification of the following system/train:

- (1) The Unit 1 residual heat removal (RH) system, during the weeks ending April 10 through April 24, 2021. This activity completed the partial inspection sample that was documented in NRC Inspection Report 05000456/2021001; 05000457/2021001 (ADAMS Accession No. [ML21106A208](#)).

71111.05 - Fire Protection

Fire Area Walkdown and Inspection Sample (IP Section 03.01) (7 Samples)

The inspectors evaluated the implementation of the fire protection program by conducting physical inspections and reviews to verify program compliance, equipment functionality, material condition, and operational readiness of the following fire areas:

- (1) The Unit 1 Containment, 377' Elevation, Inside Missile Barrier area, during the week ending April 17, 2021.
- (2) The Unit 1 Containment, 401' Elevation, Outside Missile Barrier area, during the week ending April 17, 2021.
- (3) The Unit 1 Containment, 426' Elevation, during the week ending April 17, 2021.

- (4) The Unit 1 Auxiliary Building, 330' Elevation, 1A/2A Essential Service Water (SX) Pump Room while performing maintenance/hot work on 1A SX Pump, during the week ending April 17, 2021.
- (5) The Unit 1 Containment, 401' Elevation, Inside Missile Barrier area while performing maintenance/hot work on reactor coolant pumps, during the week ending April 17, 2021.
- (6) The Unit 1 Auxiliary Building, 383' Elevation, 1B Auxiliary Feedwater (AF) Pump Room, during the week ending May 8, 2021.
- (7) The Unit 2 Switchgear area, 451' Elevation, Miscellaneous Electrical Equipment Room, during the week ending May 15, 2021.

71111.08P - Inservice Inspection Activities (PWR)

PWR Inservice Inspection Activities Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors verified the reactor coolant system boundary and risk significant piping system boundaries were appropriately monitored for degradation and that repairs and replacements were appropriately fabricated, examined and accepted by reviewing the following activities from April 7, 2021 to April 16, 2021:

03.01.a - Nondestructive Examination and Welding Activities.

- Ultrasonic (UT) examination of reactor coolant pipe welds 1RC-35-02, 1RC-16-02, 1RC-35-04
- UT examination of pressurizer nozzle to shell welds 1PZR-01-N2, 1PZR-01-N3
- UT examination of reactor vessel hot leg outlet nozzle to safe-end welds 1RV-01-022, 1RV-01-025 and 1RV01-029
- UT examination of steam generator A girth weld 1SG-05-SGC-02
- UT examination of main steam piping welds 1MS-04-34, 35, 36, 37 and 38
- UT of feedwater pipe to elbow welds 1FW-04-34, 35, 37 and 38
- Visual examination for leakage (VT-2) of 1RC-20-B1 and 1RC-27-B1 bolted connections
- Welds 1 and 4, 1SX252 valve replacement, (Work Order (WO) 04817111-01)
- Welds 11-1 and 6-1, 1RC8042B valve replacement, (WO 05046086-01)

03.01.b - Pressurized-Water Reactor Vessel Upper Head Penetration Examination Activities.

- Bare Metal Visual (VE) examination of the reactor vessel upper head penetrations nozzles 2, 5, 13 and 59

03.01.c – Pressurized-Water Reactor Boric Acid Corrosion Control Activities.

- Boric Acid Leakage Evaluation on Component 1SI023A (adapter and instrument cap)
- Boric Acid Leakage Evaluation on Component 1PS9356A (reactor coolant loop sample isolation valve assembly)
- Boric Acid Leakage Evaluation on Component 1RC8042B (reactor coolant loop 1B equalization line main isolation valve)
- Corrective Actions to Leakage at Valve 1RC8042B (Issue Report (IR) 04346974, AR 04391367)
- Corrective Actions to Leakage Downstream of 1SI023A (IR 04345873)

71111.11Q - Licensed Operator Regualification Program and Licensed Operator Performance

Licensed Operator Performance in the Actual Plant/Main Control Room (IP Section 03.01) (1 Sample)

The inspectors observed and evaluated the following licensed operator activities in the control room:

- (1) Various activities involving on-watch operations crews. These activities included, but were not limited to:
 - Shutdown of Unit 1 for the A1R22 refuel outage, during the week ending April 10, 2021.
 - Unit 1 reactor coolant system draining in support of A1R22 refuel outage reactor vessel head lift, during the week ending April 10, 2021.
 - Unit 1 reactor coolant pump startups for A1R22 refuel outage plant heat up, during the week ending April 24, 2021.

Licensed Operator Regualification Training/Examinations (IP Section 03.02) (1 Sample)

The inspectors observed and evaluated the following licensed operator training activity:

- (1) A complex casualty graded scenario involving a crew of licensed operators was observed in facility's simulator on May 25, 2021.

71111.12 - Maintenance Effectiveness

Quality Control (IP Section 03.02) (1 Sample)

The inspectors evaluated the effectiveness of maintenance, as well as certain quality control activities, to ensure the following structure, system, and component (SSC) remains capable of performing its intended function:

- (1) Maintenance effectiveness and performance history review of the 1/2RC8042 A-D 1.5-inch reactor coolant system equalization line manual isolation valves, during the weeks ending April 17 through May 29, 2021.

71111.13 - Maintenance Risk Assessments and Emergent Work Control

Risk Assessment and Management Sample (IP Section 03.01) (3 Samples)

The inspectors evaluated the accuracy and completeness of risk assessments for the following planned and emergent work activities to ensure configuration changes and appropriate work controls were addressed:

- (1) Evaluation of the work activities and risk associated with the emergent replacement of the 1RC8042D, 1.5-inch manual isolation valve as set forth in WO 4969069, during the week ending April 17, 2021.
- (2) Evaluation of the work activities and risk associated with the planned replacement of the 1A SX Pump as set forth in WO 4934317, during the week ending April 17, 2021.
- (3) Evaluation of the work activities and risk associated with emergent troubleshooting and repairs to the Unit 2 Control Bank A Rod Drive Power Supply Cabinet with the unit operating at full power as set forth in WO 5149593, during the week ending May 7, 2021.

71111.15 - Operability Determinations and Functionality Assessments

Operability Determination or Functionality Assessment (IP Section 03.01) (5 Samples 1 Partial)

The inspectors evaluated the licensee's justifications and actions associated with the following operability determinations and functionality assessments:

- (1) Evaluation of the operability of snubber 1RC01BD-B on the 1B Steam Generator following identification of a hydraulic oil leak as documented in IR 4417398, during the weeks ending April 24 through May 29, 2021.
- (2) (Partial)
Evaluation of the operability of the source range nuclear instruments with their high flux trips bypassed during reactor startup as documented in IR 4418688, during the weeks ending May 1 through June 26, 2021.
- (3) Evaluation of the operability of Unit 2 Control Bank 'A' rods following receipt of a rod drive urgent failure alarm during semiannual testing as documented in IR 4421424, during the week ending May 7, 2021.
- (4) Evaluation of the operability of the 1A Overpower Delta Temperature protection circuit following receipt of a spurious alarm as documented in IR 4421778, during the weeks ending May 7 through May 29, 2021.
- (5) Evaluation of the operability of the 1B Overtemperature Delta Temperature and Delta Temperature protection circuits following receipt of spurious indications as documented in IR 4423535, during the weeks ending May 15 through May 29, 2021.
- (6) Evaluation of the operability of the 2A Centrifugal Charging Pump due to recurring low differential pressure issues as documented in IR 4423535, during the weeks ending May 22 through June 5, 2021.

71111.18 - Plant Modifications

Temporary Modifications and/or Permanent Modifications (IP Section 03.01 and/or 03.02) (1 Sample)

The inspectors reviewed and evaluated the following permanent change to the facility that was enacted on the Unit 1 and Unit 2 SX system. The inspectors' reviews included an in-depth examination of the licensee's use of 10 CFR 50.69, *Risk-Informed Categorization and Treatment of Structures, Systems and Components for Nuclear Power Reactors*, as it was applied to the SX system:

- (1) Engineering Change (EC) 632945: 0SX115C Valve and Piping Carbon Fiber Wrap, during the weeks ending April 1 through June 30, 2021. This activity completed the partial inspection sample that was documented in NRC Inspection Report 05000456/2021001; 05000457/2021001 (ADAMS Accession No. [ML21106A208](#)).

71111.19 - Post-Maintenance Testing

Post-Maintenance Test Sample (IP Section 03.01) (8 Samples)

The inspectors evaluated the following post-maintenance test activities to verify system operability and functionality:

- (1) Comprehensive functional and operational testing of the 1A SX Pump following replacement of the pump rotating element as set forth in WO 4934317, during the week ending April 17, 2021.
- (2) Comprehensive functional testing of the Unit 1 SI and residual heat removal systems following outage maintenance as set forth in WOs 4980282 and 4983054, during the week ending April 17, 2021.
- (3) Functional and operational testing of the 1B Rod Drive Motor-Generator set following multiple actions to balance the motor-generator set as set forth in WO 1953140, during the week ending April 24, 2021.
- (4) AF Train 1B testing following outage system maintenance as set forth in WO 4982132, during the weeks ending April 24 through May 1, 2021.
- (5) Functional and operational testing of Unit 2 Control Bank 'A' control rods following emergent troubleshooting and repairs as set forth in WO 5149593, during the week ending May 7, 2021.
- (6) Low power startup physics testing following reloading of the Unit 1 reactor core for Operating Cycle No. 23 as set forth in WOs 4987491 and 4977795, during the weeks ending April 24 through May 8, 2021.
- (7) System leakage testing at normal operating primary pressure and temperature following return of the reactor coolant system to operation after reactor refueling as set forth in WOs 4985624 and 4985631, during the weeks ending April 24 through May 8, 2021.
- (8) Functional and operational testing of the 1A EDG following maintenance as set forth in WO 5160533, during the week ending June 26, 2021.

71111.20 - Refueling and Other Outage Activities

Refueling/Other Outage Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors reviewed licensee activities and evaluated licensee performance associated with Unit 1 Refuel Outage A1R22, during the weeks ending April 10 through May 1, 2021.

71111.22 - Surveillance Testing

The inspectors evaluated the following surveillance tests:

Surveillance Tests (other) (IP Section 03.01) (4 Samples)

- (1) Unit 1 Refuel Outage A1R22 emergency core cooling system full flow testing for charging and safety injection as set forth in WO 4983054, during the weeks ending April 10 through May 15, 2021.
- (2) Sequencer testing for the 1A EDG for the A1R22 refuel outage as set forth in WO 4803148, during the weeks ending April 10 through May 15, 2021.
- (3) Full load rejection testing for the 1A EDG for the A1R22 refuel outage as set forth in WO 4836102, during the weeks ending April 24 through May 29, 2021.
- (4) Loss of essential 4 KV bus response testing for the 1A EDG for the A1R22 refuel outage as set forth in WO 5067469, during the weeks ending April 24 through May 29, 2021.

Inservice Testing (IP Section 03.01) (1 Sample)

- (1) Observation and assessment of Unit 1 main steam safety valve setpoint testing as set forth in WO 5067469, during the weeks ending April 3 through May 29, 2021.

Containment Isolation Valve Testing (IP Section 03.01) (1 Sample)

- (1) 1BwOSR 3.6.1.1-9: Primary Containment Type C Local Leak Rate Testing for the Unit 1 chemical and volume control system, during the weeks ending April 10 through April 17, 2021.

RADIATION SAFETY

71124.02 - Occupational ALARA Planning and Controls

Implementation of ALARA and Radiological Work Controls (IP Section 03.03) (3 Samples)

The inspectors reviewed as low as reasonably achievable practices and radiological work controls.

- (1) The inspectors evaluated items associated with Radiation Work Permit BW-01-21-00613 and associated ALARA documentation for Reactor Head Disassembly and Reassembly during the spring 2021 outage.
- (2) The inspectors evaluated items associated with Radiation Work Permit BW-01-21-00623 and associated ALARA documentation for Reactor Cavity Decon during the spring 2021 outage.

- (3) The inspectors evaluated items associated with Radiation Work Permit BW-01-21-00612 and associated ALARA documentation for Core Barrel Inservice Inspection (ISI) and all Associated Work Valve Repairs/Inspections during the spring 2021 outage.

Radiation Worker Performance (IP Section 03.04) (1 Sample)

The inspectors evaluated radiation worker and radiation protection technician performance during:

- (1) Fuel move activities.

71124.03 - In-Plant Airborne Radioactivity Control and Mitigation

Permanent Ventilation Systems (IP Section 03.01) (1 Sample)

The inspectors evaluated the configuration of the following permanently installed ventilation systems:

- (1) Unit 1/2 Control Room Emergency Ventilation System.

Temporary Ventilation Systems (IP Section 03.02) (1 Sample)

The inspectors evaluated the configuration of the following temporary ventilation systems:

- (1) Portable high-efficiency particulate air (HEPA) ventilation system used on Reactor Vessel Head Stanchion.

Use of Respiratory Protection Devices (IP Section 03.03) (1 Sample)

- (1) The inspectors evaluated the licensee's use of respiratory protection devices.

Self-Contained Breathing Apparatus for Emergency Use (IP Section 03.04) (1 Sample)

- (1) The inspectors evaluated the licensee's use and maintenance of self-contained breathing apparatuses.

71124.04 - Occupational Dose Assessment

Source Term Characterization (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated licensee performance as it pertains to radioactive source term characterization.

External Dosimetry (IP Section 03.02) (1 Sample)

- (1) The inspectors evaluated licensee performance as it pertains to external dosimetry that is used to assign occupational dose.

Internal Dosimetry (IP Section 03.03) (2 Samples)

The inspectors evaluated the internal dosimetry program implementation.

- (1) Intake Investigation Form; Plant ID #7080; 04/22/2020 @1300 hours.
- (2) Intake Investigation Form; Plant ID #5132; 04/22/2020 @1200 hours.

Special Dosimetric Situations (IP Section 03.04) (2 Samples)

The inspectors evaluated the following special dosimetric situation:

- (1) Dose assessment for a declared pregnant worker in 2019.
- (2) Dose assessment for a declared pregnant worker in 2020.

OTHER ACTIVITIES – BASELINE

71151 - Performance Indicator Verification

The inspectors reviewed and verified selected portions of the licensee's performance indicator submittals listed below:

MS05: Safety System Functional Failures (SSFFs) Sample (IP Section 03.04) (2 Samples)

- (1) Unit 1 (April 1, 2020 - March 31, 2021)
- (2) Unit 2 (April 1, 2020 - March 31, 2021)

MS06: Emergency AC Power Systems (IP Section 03.05) (2 Samples)

- (1) Unit 1 (April 1, 2020 - March 31, 2021)
- (2) Unit 2 (April 1, 2020 - March 31, 2021)

MS07: High Pressure Injection Systems (IP Section 03.06) (2 Samples)

- (1) Unit 1 (April 1, 2020 - March 31, 2021)
- (2) Unit 2 (April 1, 2020 - March 31, 2021)

71152 - Problem Identification and Resolution

Semiannual Trend Review (IP Section 02.02) (1 Sample)

- (1) The inspectors conducted a semiannual review of the licensee's corrective action program (CAP) for potential adverse trends that might be indicative of a more significant safety concern during the weeks ending June 5 through June 30, 2021. A specific trend related to the Unit 1 Main Steam Isolation Valve (MSIV) Bypass Valves' operation is documented in the Results Section of this report.

71153 – Follow-up of Events and Notices of Enforcement Discretion

Event Follow-up (IP Section 03.01) (3 Samples)

The inspectors assessed licensee performance related to the following events:

- (1) Review and tracking of licensee actions during the COVID-19 National Emergency declaration as set forth by the President of the United States, during the weeks ending May 15 through June 12, 2021.
- (2) Review and assessment of the licensee and plant response to the Unit 1 automatic reactor trip on June 21, 2021, due to a main electrical generator lockout/load rejection as documented in NRC Event Notification (EN) 55320, during the weeks ending June 26 through June 30, 2021.
- (3) Review and assessment of the loss of emergency response facility capability due to degradation of the Technical Support Center (TSC) ventilation system on June 24, 2021, as documented in NRC EN 55325, during the weeks ending June 26 through June 30, 2021.

INSPECTION RESULTS

Observation: Trend in Unit 1 Main Steam Isolation Valve Bypass Valve Operation	71152
<p>The inspectors performed a review of plant issues, particularly those entered into the licensee's CAP over the past several months. During their review, the inspectors noted a trend on Unit 1 involving the operation of MSIV Bypass Valves (1MS101A-D). Upon further inspection, the trend was noted to extend back a few years in time and seemed to be particularly evident most recently on the 1MS101B and 1MS101C valves. Specific CAP entries reviewed by the inspectors included, but were not limited to:</p> <ul style="list-style-type: none">• IR 4201555; <i>1MS101B Failed To Stroke.</i>• IR 3968314; <i>1MS101C "A" Solenoid Replacement - Extent of Condition.</i>• IR 4256475; <i>1MS101B Would Not Open when Demanded from its Control Switch.</i>• IR 4130120; <i>OSP-A – 1MS101C Will Not Go Full Open.</i>• IR 4427859; <i>1MS101C Failed to Open when Demanded from Control Switch.</i> <p>While the inspectors noted that these issues were exclusively limited to the open direction stroke of the valves, which is not part of the components' safety function, the recurring nature of issues in the open direction without a thorough understanding of the trend might suggest the presence of an underlying component reliability issue.</p> <p>The inspectors did not identify any findings or violations of NRC requirements in the course of their review.</p>	

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On July 13, 2021, the inspectors presented the integrated inspection results to Mr. J. Keenan, Site Vice President, and other members of the licensee staff.
- On April 16, 2021, the inspectors presented the Radiation Protection inspection results to Mr. J. Keenan, Site Vice President, and other members of the licensee staff.
- On April 16, 2021, the inspectors presented the Inservice inspection results to Mr. J. Keenan, Site Vice President, and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.04	Corrective Action Documents	4243133	2DG01KB-Y1 East End Bell Oil Leak 12 DPM When Diesel Running	05/24/2019
		4306628	1RH8703B has Evidence of Packing Leak	12/29/2019
		4367735	1A RH Pump has Boric Acid Buildup on Pump Seal/Bowl	09/06/2020
		4367738	1B RH Pump has Boric Acid Buildup on Pump Seal/Bowl	09/06/2020
		4374335	1RH610 Unsat Grease Inspection	10/05/2020
		4374466	Boric Acid Residue in 1RH01PA Pump Bowl	10/05/2020
		4378052	1B RH Pump Flow Requires IR for Tracking	10/20/2020
		4422278	2A DG Turbocharger Spindown not be Performed as Scheduled	05/07/2021
	4422966	2DG01KA-C, 2A DG Jacket Water Pump is Leaking	05/11/2021	
	Procedures	BwOP CC-E1	Electrical Lineup — Unit 1 Operating	8
		BwOP CC-M1	Operating Mechanical Lineup Unit 1	18
		BwOP DG-E3	Electrical Lineup — Unit 2A Diesel Generator	8
		BwOP DG-M3	Operating Mechanical Lineup Unit 2A Diesel Generator	15
		BwOP FC-M1	Operating Mechanical Lineup Unit 1	10
		BwOP RH-6	Placing the RH System in Shutdown Cooling	60
		BwOP RH-8	Filling the Reactor Cavity for Refueling	24
		BwOP RH-E1	Electrical Lineup — Unit 1 Operating	10
		BwOP RH-M1	Operating Mechanical Lineup Unit 1 1A RH Train	14
		BwOP RH-M2	Operating Mechanical Lineup Unit 1 1B Train	10
BwOP SI-E2		Electrical Lineup — Unit 2 Operating	10	
BwOP SI-M2	Operating Mechanical Lineup Unit 2	24		
71111.05	Fire Plans	Pre-Fire Plan 1	Fire Zone 1.1-1; Containment 377'-0" Elevation Unit 1 Containment Missile Shield Area	1
		Pre-Fire Plan 6	Fire Zone 1.2-1; Containment 401'-0" Elevation Unit 1 Annular Area	1
		Pre-Fire Plan 9	Fire Zone 1.3-1; Containment 426'-0" Elevation Unit 1 Containment Upper Area	2
		Pre-Fire Plan 96	Fire Zone 11.1A-0; Auxiliary Building 330' Elevation, Unit 1 Auxiliary Building Basement (1A/2A SX)	2

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		Pre-Fire Plan No. 137	Fire Zone 11.4A-1; Auxiliary Building 383' Elevation, Unit 2 Auxiliary Feedwater Pump Diesel Room	3
		Pre-Fire Plan No. 48	Fire Zone 5.4-2; Switchgear Area 451' Elevation, Division 22, MEER and Battery Room	1
	Procedures	BwAP 1110-1	Fire Protection Program System Requirements	44
		BwAP 1110-3	Plant Barrier Impairment Program	39
		BwOP PBI-1	Plant Barrier Impairment Program Pre-Evaluated Barrier Matrix	4
		CC-AA-201	Plant Barrier Control Program	13
		ER-AA-600-1069	High Risk Fire Area Identification	4
		ER-BR-600-1069	Site List of High Risk Fire Areas — Braidwood Unit 1 and Unit 2	0
		OP-AA-201-004	Fire Prevention for Hot Work	17
		OP-AA-201-008	Pre-Fire Plan Manual	4
		OP-AA-201-009	Control of Transient Combustible Material	26
		OP-AA-201-012-1001	Operations On-Line Fire Risk Management	4
	Pre-Fire Plan No. 2	Fire Zone 1.1-1; Containment 401'-0" Elevation Unit 1 Containment Missile Shield Area	1	
71111.08P	Corrective Action Documents	4129240	OSP-X 1RC01R NDE Results for 1RV-03-002 Reactor Head Weld	04/20/2018
		4272501	Notification of Code Nonconformance for Replacement SGs	08/16/2019
		4345873	BA Leak Downstream 1SI023A	05/26/2020
		4346974	Leak Found during Robot Inspection of U1 CNTMNT - IMB 1RC8042B	05/31/2020
		4391367	1RC8042B Exhibiting a Different Leak Path	12/20/2020
		480489	Boric Acid Accumulation Bottom of Pressurizer (Investigate Source)	04/19/2006
		480489	Boric Acid Accumulation Bottom of Pressurizer (Investigate Source)	04/19/2006
	Miscellaneous		ASME Weld Data Record, Weld 1 and Weld 4 - I SXA9A	04/11/2021
			ASME Weld Data Record, Weld 6-1 and Weld 11-1 - 1RC8042B	04/12/2021
			B. Connolly- VT Level II, Certification	01/11/2019

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			R. Kramer – VT Level III, Certification	01/29/2020		
			J. Jennings – UT Level II, Certification	12/03/2020		
			B. Knoff - UT Level II, Certification	01/29/2021		
			J. Simpson - UT Level II, Certification	01/06/2021		
			WPS 1-1-GTSM-PWHT	5		
			PQR A-001	10/19/1998		
			PQR A-002	03/09/1999		
			PQR 1-50C	01/03/1984		
			Rod Ticket - Field Welds 1,2,3,4 (WO 04817111-01)	04/11/2021		
			WPS 8-8-GTSM	6		
			PQR 1-51A	12/28/1983		
			PQR 4-51A	09/12/1986		
			Attachment 2S Evaluation of BACC Leakage (AR 04345873), 1SI023A D/S Adapter and Instrument Cap- SI PMPS INJECTION HDR TO LOOP 3 COLD LEG INST ISOL TO 1FT-S1051	06/22/2020		
			Attachment 2S Evaluation of BACC Leakage (AR 04345861) 1PS9356A-RC Loop Sample Inside Isolation VLV Assembly, PS, A Train	06/18/2020		
			Attachment 2 Evaluation of BACC Leakage 1RC8042B (AR 04346974 & AR 04391367) - RC Loop 1B Equalization Line Main Isolation VLV	07/09/2020		
			CB1404	Welder Certification Record	10/26/2020	
			D9350	Welder Certification Record	09/29/2020	
			EPRI PDQS No. 952	WDI-STD-1165, Revision 3 Addenda 0	01/21/2021	
			NDE Reports	04981252-01	VT-2 Visual Examination - 1 RC-27 and 1 RC-20	04/14/2021
				2021-MT-11	Magnetic Particle Examination- Welds 1,2,3, and 4	09/11/2021
			2021-PT-010	Liquid Penetrant Examination- Field Weld# 6-1, Line 1RC22AB 1-1/2"	04/14/2021	
			2021-PT-011	Liquid Penetrant Examination- Field Weld# 11-1, Line 1RC22AB 1-1/2"	04/14/2021	
			A1R22-UT-001	Ultrasonic Examination- 1RC-16-02	04/10/2021	

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		A1R22-UT-002	Ultrasonic Examination- 1RC-35-02	04/10/2021
		A1R22-UT-003	Ultrasonic Examination- 1RC-35-04	04/10/2021
		A1R22-UT-005	Ultrasonic Examination- 1PZR-01-N3	04/13/2021
		A1R22-UT-006	Ultrasonic Examination- 1PZR-01-N2	04/13/2021
		A1R22-UT-023	1RV-03	04/16/2021
		A1R22-UT-035	Ultrasonic Examination- 1MS-04-37	04/15/2021
		A1R22-UT-036	Ultrasonic Examination- 1MS-04-38	04/15/2021
		A1R22-UT-037	1FW-04-37	04/16/2021
		A1R22-UT-038	Ultrasonic Examination- 1MS-04-34	04/15/2021
		A1R22-UT-039	Ultrasonic Examination- 1MS-04-35	04/15/2021
		A1R22-UT-040	Ultrasonic Examination- 1MS-04-36	04/15/2021
		A1R22-UT-041	Ultrasonic Examination- 1FW-04-35	04/15/2021
		A1R22-UT-042	Ultrasonic Examination- 1FW-04-34	04/15/2021
		A1R22-UT-043	1SG-05-SGC-02	04/16/2021
		A1R22-UT-046	1FW-04-38	04/16/2021
		A1R22-VEN-002	1RC01R Top Head BMV	04/15/2021
		ANALYSIS LOG # WN22-01	1RV-01-022 (DM) / 1 RV-01-030 (SS)	04/16/2021
		ANALYSIS LOG # WN338-01	1RV-01-029 (DM) / 1RV-01-037 (SS)	04/16/2021
		ANALYSIS LOG# WN158-01	1RV-01-025 (DM) / 1RV-01-033 (SS)	04/16/2021
		Procedures	ER-AA-335-002	Liquid Penetrant (PT) Examination
	ER-AA-335-003		Magnetic Particle (MT) Examination	5
	ER-AA-335-015-2013		VT-2 Visual Examination in Accordance with ASME 2013	1
	ER-AA-335-030		Ultrasonic Examination of Ferritic Piping Welds	5
	ER-AA-335-031		Ultrasonic Examination of Austenitic Piping Welds	8
	ER-AA-335-034		Manual Ultrasonic Examination of Reactor Pressure Vessel Welds	3
	ER-AA-335-035		Manual Sizing of Ultrasonic Indications in Reactor Pressure Vessel Welds	3
	ER-AA-335-049	Ultrasonic Examination in Accordance with Section V,	2	

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			Article 4	
		ER-AP-331	Boric Acid Corrosion Control (BACC) Program	10
		ER-AP-331-1001	Boric Acid Corrosion Control (BACC) Inspection Locations, Implementation, and Inspection Guidelines	11
		ER-AP-331-1002	Boric Acid Corrosion Control Program Identification Screening and Evaluation	11
		ER-AP-335-001	Bare Metal Visual Examination for Nickel Alloy Materials	8
		WDI-STD-1165	Automated Ultrasonic Examination of the Remote Inservice Inspection of Reactor Vessel Nozzle to Safe End, Nozzle to Pipe, and Safe End to Pipe Welds Using the Athena Phased Array System	3
71111.11Q	Operability Evaluations	TQ-BR-201-0113	Braidwood Training Department Simulator Examination Security Actions	23
	Procedures	OP-AA-101-111-1001	Operations Standards and Expectations	26
		OP-AA-101-113	Operator Fundamentals	14
		OP-AA-101-113-1006	4.0 Crew Critique Guidelines	11
		OP-AA-103-102	Watch-Standing Practices	20
		OP-AA-103-102-1001	Strategies for Successful Transient Mitigation	2
		OP-AA-103-103	Operation of Plant Equipment	1
		OP-AA-104-101	Communications	5
		OP-AA-111-101	Operating Narrative Logs and Records	17
		OP-AA-300	Reactivity Management	14
		TQ-AA-10	Systematic Approach to Training Process Description	6
		TQ-AA-150	Operator Training Programs	20
		TQ-AA-155	Conduct of Simulator Training and Evaluation	11
		TQ-AA-201	Examination Security and Administration	18
		TQ-AA-306	Simulator Management	10
TQ-BR-201-0113	Braidwood Training Department Simulator Examination Security Actions	23		
71111.12	Corrective Action Documents	1354708	A1R16 (1RC8042B Boric Acid Leakage Refurbish or Replace)	04/16/2012

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		1355845	1RC8042B will not Close	04/08/2012
		4124573	OSP-A Boric Acid Dry on Valve 1RC8042B (Clean)	04/09/2018
	Engineering Changes	394386	Engineering Review of Flowserve Test Report TAL-20820, Revision 2, for a 1.5-Inch ASME Class 1878 Stainless Steel Y-Pattern Globe Valve for Braidwood Station	08/28/2013
	Procedures	ER-AA-310-1001	Maintenance Rule 18-10 — Scoping	0
		ER-AA-310-1002	Maintenance Rule Functions — Safety Significant Classification	3
		ER-AA-320	Maintenance Rule Implementation per NEI 18-10	0
		ER-AA-320-1003	Maintenance Rule 18-10 — Failure Definitions	0
		ER-AA-320-1004	Maintenance Rule 18-10 — Performance Monitoring and Dispositioning Between (a)(1) and (a)(2)	1
	Work Orders	5046086	1RC8042B — Replace Valve	04/10/2021
	71111.13	Corrective Action Documents	4414349	Pre-Freeze NDE UT Readings Below 87.5% Tmin 1SXA9A-6"
4414530			Replacement 1A SX Pump Shaft Scratches/Defects	04/06/2021
4414564			1RC8042D Unable to Close Loop Unable to be Drained	04/06/2021
Drawings		M-60, Sheet 2	Reactor Coolant Loop 2 — Unit 1	BC
Engineering Changes		388765	Freeze Line on 1RC22AB-1 1/2" to Isolate 1FE-0428	04/18/2012
Procedures		BwMP 3100-094	Removal and Installation of Flood Seal Opening Barriers	20
Work Orders		4934317	Replace 1SX01PA in A1R22	04/08/2021
		4969069	1RC8042D Difficult to Operate — Replace Valve	04/12/2021
		5149593	Unexpected 2-10-C6, Rod Control Urgent Failure, SER No. 2165	05/05/2021
71111.15		Corrective Action Documents	2530091	Anomaly in Unit 2 Pressurizer Level / VCT Level Trend
	4417398		14-OSP-X Post-Maintenance Testing: 1RC01BD-B Steam Generator Snubber Found Leaking at 60 Drops per Minute	04/18/2021
	4421424		Unexpected 2-10-C6, Rod Control Urgent Failure, SER# 2165	05/04/2021
	4421660		2-10-C6, Rod Control Urgent Failure, SER# 2165	05/05/2021
	4421778		Unexpected Alarms 1-10-A5 and 1-14-A1 for OPDT	05/05/2021
	4423270		Unexpected Annunciator: Ovation System Trouble	05/13/2021

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		4423503	Corrective Action to 1TY-0441A3	05/13/2021
		4423535	Unexpected Alarm 1-14-B5 and 1-4-A3	05/14/2021
		4424225	2A CV Pump Significantly Weaker than 2B CV Pump	05/18/2021
		4424510	1C OTDT Rescaling Required	05/19/2021
	Procedures	2BwOSR 3.5.5.1	Unit Two RCS Seal Injection Flow Surveillance	14
		BwAR 1-10-A5	OPDT High Rod Stop C-4	7
		BwAR 1-14-A1	OPDT High Reactor Trip Alert	52
		BwOP RP-9	Disabling and Restoring Source Range Nuclear Instruments	4
		BwVP 312-6	Reactor Coolant System Temperature Instrumentation Delta T and Tave Normalization	3
		ER-AA-600-1012	Risk Management Documentation	14
		OP-AA-106-101-1006	Operational Decision Making Process	22
		OP-AA-108-111	Adverse Condition Monitoring and Contingency Planning	15
	OP-AA-108-115	Operability Determinations (CM-1)	24	
	Work Orders	5150129	Unexpected Alarms 1-10-A5 and 1-14-A for OPDT	05/06/2021
71111.18	Procedures	ER-AA-569	10 CFR 50.69 Program	4
		ER-AA-569-1001	10 CFR 50.69 Active Component Risk Significance Insights	4
		ER-AA-569-1002	10 CFR 50.69 Passive Component Categorization	3
		ER-AA-569-1003	10 CFR 50.69 Risk Informed Categorization for Structures, Systems, and Components	6
		ER-AA-569-1004	10 CFR 50.69 Alternative Treatment Implementation Process	4
		ER-AA-569-1005	Integrated Decision Decision-Making Panel for Risk Informed SSC Categorization Duties and Responsibilities	6
		ER-AA-569-1006	Requirements for Immediate, Interim and Periodic Performance Monitoring Reviews	3
		ER-AA-569-1007	Job Familiarization of 50.69 Project Team and Site Personnel	5
71111.19	Corrective Action Documents	4417074	1A SX Pump Test Results	04/16/2021
		4418193	17-OSP-A 1B Rod Drive Generator Breaker Trip	04/22/2021
	Procedures	1 BwOSR 5.5.8.AF-3B	Group A IST Requirements for Unit One Diesel Driven Auxiliary Feedwater Pump	12
		1BwOSR 3.7.5.7-	Unit One Train B Auxiliary Feedwater Flow Path Operability	6

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		2	Non-Routine Surveillance (Following Cold Shutdown)	
		1BwOSR 3.8.1.2-1	Unit One 1A Diesel Generator Operability Surveillance	49
		1BwOSR 5.5.8.CV-8	Comprehensive Inservice Testing (IST) Requirements for Unit 1 Charging Pumps and Safety Injection System Charging Check Valve Stroke Test	12
		1BwOSR 5.5.8.SX-6A	Comprehensive Inservice Test (IST) Requirements for 1A Essential Service Water Pump (1SX01PA)	16
		BwVS 500-6	Low Power Physics Testing	46
		BwVS TRM 3.1.h.1	Low Power Physics Data Interpretation	23
		ER-AA-330-001	Section XI Pressure Testing	18
	Work Orders	1953140	1RD02E — Rebuild or Replace Rod Drive Motor-Generator Set Motor	04/06/2021
		4793485	1AF01PB — ASME — 1BwOSR 5.5.8.AF-3B	04/21/2021
		4896029	PMT Leak Check 1AF01AB Lubrication Oil Cooler	04/21/2021
		4977795	Low Power Physics Data Interpretation	04/23/2021
		4982132	Train B AF Flowpath Operability Following Cold Shutdown	04/21/2021
		4983054	CV Cold Leg Check Valve Test — 1BwOSR 5.5.8.CV-8	04/07/2021
		4985624	Class 1 System Leakage Test (Ascending Mode 3)	04/22/2021
		4985631	Class 2/3 System Leakage Test (Ascending Mode 3)	04/22/2021
		4987491	Low Power Physics Test Program with Dynamic Rod Worth Measurement	04/23/2021
		4987492	Moderator Temperature Coefficient — Low Power Physics Testing	04/23/2021
		5026352	1B AF Pump Function Run (ASME)	04/21/2021
		5160533	1A Diesel Generator Operability Monthly	06/21/2021
71111.20	Corrective Action Documents	4413057	APS Electrician Received Electrical Arc Flash Burns	03/31/2021
		4414344	01-OSP-A A1R22 GL 88-05 Minor BACC Results for Mode 3 Descending	04/06/2021
	Miscellaneous		A1R22 Shutdown Safety Plan	1
		BR1C22-08.0	Braidwood 1 Cycle 22 Coastdown Reactivity Plan	03/09/2021
	Procedures	1BwGP 100-1	Plant Heat up	38

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		1BwGP 100-2	Plant Start up	47
		1BwGP 100-3	Power Ascension 5% to 100%	85
		1BwGP 100-4	Power Descension	51
		1BwGP 100-5	Plant Shutdown and Cooldown	62
		1BwGP 100-6	Refueling Outage	33
		1BwOS TRM 2.5.b.1	Unit One — Containment Loose Debris Inspection	22
		BwAP 1450-1	Access to Containment	44
		BwGP 100-4	Power Descension	51
		BwVS TRM 3.1.h.1	Core Reload Sequence and Verification	23
		ER-AA-600-1043	Shutdown Risk Management	7
		HU-AA-1211	Pre-Job Briefings	14
		LS-AA-119	Fatigue Management and Work Hour Rules	15
		MA-AA-716-008- 1008	Reactor Services: Refuel Floor FME Plan	17
		OP-AA-108-108	Unit Restart Review	20
		OP-AA-108-110	Evaluation of Special Tests or Evolutions	4
		OP-AA-300-1520	Reactivity Management - Fuel Handling, Storage, and Refueling	7
		OU-AA-103	Shutdown Safety Management Program	21 – 22
		OU-AA-103-1001	Shutdown Safety Plan Independent Review	4
71111.22	Corrective Action Documents	4413124	1MS016C - Pre A1R22 Testing as Found Outside Acceptance Range	03/31/2021
		4413146	Unexpected Annunciator: 1D-14: Fire Alarm	03/31/2021
		4413531	1MS014C - Unsat Testing	04/01/2021
		4413543	Unit 1 Trend Main Steam Safety Valves Lift Exceedance - A1R22	04/01/2021
		4413654	1MS015C - Unsat Testing	04/02/2021
		4413656	1MS015A - Unsat Testing	04/02/2021
	Procedures	1BwOSR 3.6.1.1-9	Primary Containment Type C Local Leakage Rate Test of Chemical and Volume Control System	16
		1BwOSR	1A Diesel Generator Full Load Rejection and Simulated	17

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		3.8.1.10-1	Safety Injection in Conjunction with Undervoltage During Load Testing	
		1BwOSR 3.8.1.11-1	1A Diesel Generator Loss of Emergency Safety Features Bus Voltage with No Safety Injection Signal	22
		1BwOSR 5.5.8.CV-8	Comprehensive Inservice Testing (IST) Requirements for Unit 1 Charging Pumps and Safety Injection System Charging Check Valve Stroke Test	12
		BwMP 3305-003	Main Steam Safety Valve Testing Using Setpoint Verification Device	9
		BwOP DG-1	Diesel Generator Alignment to Standby Condition	31
		BwOP DG-11	Diesel Generator Startup and Operation	53
		BwOP DG-12	Diesel Generator Shutdown	30
		BwOSR 3.8.1.19-1	1A Diesel Generator Emergency Core Cooling System Sequencer Surveillance	24
		ER-AA-321	Administrative Requirements for Inservice Testing	13
	Work Orders	4803148	1A Diesel Generator 24-Hour Load Test and Emergency Core Cooling System Sequencer Surveillance	04/20/2021
		4817382	1A Diesel Generator Loss of Emergency Safety Features Bus Voltage with NO Safety Injection	04/20/2021
		4836102	1A Diesel Generator Safety Injection Signal Override of Test Mode	04/20/2021
		4983054	OP — CV Cold Leg Check Valve Test — 1BwOSR 5.5.8.CV-8	04/07/2021
		5066993	As Found LLRT — 1CV8152 and 1CV8160 — 1BwOSR 3.6.1.1-9	04/09/2021
		5067469	IST – 1MS013s/014s/015s/016s/017s	03/31/2021
71124.02	ALARA Plans	BW-01-21-00612	Core Barrel ISI and all Associated Work Valve Repairs/Inspections	0
		BW-01-21-00613	Reactor Head Disassembly and Reassembly	0
		BW-01-21-00623	Reactor Cavity Decon	0
71124.03	Corrective Action Documents	04254289	0EA01C SCBA Compressor has a Small Leak	06/04/2019
		04263013	Mask Fit not Completed for Licensed Operator	07/10/2019
		04370062	Evaluate Need for SCBA Kits in the Unit 1 and Unit 2 Containment Access Facility	09/17/2020

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	Miscellaneous		MSA SCBA/Firehawk Monthly Inspection	03/2021
	Procedures	RP-AA-825-101-1001	Operation and use of The MSA Firehawk Self-Contained Breathing Apparatus (SCBA)	0
		RP-AA-870-1003	Testing Portable HEPA Filter Units	6
71124.04	Calibration Records	RP-AA-232-1001	FASTSCAN Calibration Acceptance Criteria	02/24/2021
		RP-AA-700-1235	PM-12 Calibration Data Sheet	07/08/2020
	Corrective Action Documents	04268944	NOS ID 2018 Bioassay Program Periodic Measurement not Performed	08/01/2019
	Radiation Surveys	RP-AA-220	Intake Investigation Form; 7080	04/22/2020
		RP-AA-220	Intake Investigation Form; 5132	04/22/2020
	Self-Assessments	PI-AA-126-1001-F-01	Radiation Protection Dosimetry and Occupational Dose Assessment	10/31/2019
71151	Miscellaneous		NRC Performance Indicator Data; Mitigating Systems – Mitigating System Performance Index for High Pressure Injection;	04/01/2020 – 03/31/2021
			NRC Performance Indicator Data; Mitigating Systems — Safety System Functional Failures	04/01/2020 – 03/31/2021
			NRC Performance Indicator Data; Mitigating Systems – Mitigating System Performance Index for Emergency AC Power	04/01/2020 – 03/31/2021
71152	Corrective Action Documents	3968314	1MS101C "A" Solenoid Replacement — Extent of Condition	01/30/2017
		4130120	OSP-A, 1MS101C will not go Open	04/24/2018
		4201555	1MS101B Failed to Stroke	12/09/2018
		4256475	1MS101B would not Open when Demanded from its Control Switch	06/13/2019
		4427859	1MS101C Failed to Open when Demanded from its Control Switch	06/07/2021
	Procedures	NO-AA-10	Quality Assurance Topical Report (QATR)	96 – 97
		PI-AA-120	Issue Identification and Screening Process	11
		PI-AA-125	Corrective Action Program (CAP) Procedure	7
		PI-AA-125-1001	Root Cause Analysis Manual	6
		PI-AA-125-1003	Corrective Action Program Evaluation Manual	6
71153	Corrective Action	4430406	Multiple MCR Alarms Received Due to Adverse Weather	06/20/2021

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	Documents	4430440	Indication Lost for 1MS-147A Following Unit 1 Reactor Trip	06/21/2021
		4430441	Unit 1 Reactor Trip — Load Reject	06/21/2021
		4430442	Unexpected Annunciator: 1-4-D5	06/21/2021
		4430444	1 A/D MSIV Room Fire Alarm Following Reactor Trip	06/21/2021
		4430446	N35 and N36 Appear to be Undercompensated	06/21/2021
		4430586	4.0 Critique Unit Reactor Trip Due to Generator Load Reject	06/21/2021
		4430895	Need WO Generated for Testing Station Site Grounding System	06/22/2021
		4431283	0VV23C has a Sheared Belt	06/24/2021
		4432865	Braidwood Site Ground Testing	07/01/2021
	Procedures	1BwGP 110-4T4	Reactor Trip Post Response Guideline	0
		OP-AA-106-101-1001	Event Response Guidelines	31
		OP-AA-108-108	Unit Restart Review	20
		OP-AA-108-114	Post Transient Review	13
	Work Orders	1685480	0VV23C — Perform Belt Inspection	08/09/2017
		5164828	0VV23C has a Sheared Belt	06/24/2021