UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

Docket No. 50-293

APR 14 1989

Mr. Ralph G. Bird
Senior Vice President-Nuclear
Boston Edison Company
Pilgrim Nuclear Power Station
RFD#1, Rocky Hill Road
Plymouth, Massachusetts 02360

Dear Mr. Bird:

The purpose of this letter is to request your prompt review and reporting of the status of implementation of TMI Action Plan Items at your facility. As an enclosure we are providing a printout of all TMI items annotated where our tracking system indicates that implementation is not complete. For the purpose of this request, implementation should be considered complete when, to the best of your knowledge, all actions necessary to satisfactorily meet the requirements of NUREG-0737 and NUREG-0737 Supplement 1 have been completed.

Where you have not fully completed an item as described above, we request that you mark-up the enclosure to reflect your projected implementation date. Add a short note identifying remaining work (e.g., hardware, procedures, training, technical specifications). More explicit instructions are provided as part of the enclosure. We request that this information reach this office by April 18, 1989.

The information we are requesting will be utilized to validate our existing data base so that we can accurately respond to congressional inquiries concerning the status of implementation of TMI Action Plan Items. In view of the importance of the information we request your personal attention and certification that the information is correct to the best of your knowledge.

This request is covered by Office of Management and Budget Clearance Number 3150-0011, which expires December 31, 1989. The estimated average burden hours is 40 person hours per owner response, including searching data sources, gathering and analyzing the data, and preparing the required letter. Send comments regarding this burden estimate or any other aspect of this collection of information, including suggestions for reducing this burden, to the Records and Reports Management Branch, Division of Information Support Services, Office of Information Resources Management, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555; and to the Paperwork Reduction Project (3150-0011), Office of Management and Budget, Washington, D.C. 20503.

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Your prompt attention to this matter within the requested time frame is appreciated.

Sincerely,

Original signed by:

Thomas E. Murley, Director
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

DISTRIBUTION: Docket File, NRC&Local PDRs, PDI-3 r/f, MRushbrook, DMcDonald, SVarga, BBoger, OGC, EJordan, BGrimes, ACRS(10), RWessman, JPartlow, JSniezek, TMurley

[PILGRIM TMI LETTER]

OFC	:PDI-3	:PDI-3	:DIR/PDI-3	:DONRR	:	:	:
NAME	:MRushbrook	:DMcDonald:cb	:RWessman	:TMurley	:	:	: :
DATE	:4/14/89	:4/14/89	:4/14/89	:4/17/89	:	:	:

OFFICIAL RECORD COPY

Mr. Ralph G. Bird
Boston Edison

Pilgrim Nuclear Power Station

cc:

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PILGRIM

ENCLOSURE

GUIDELINES FOR UPDATING TMI STATUS

1. The enclosure provides a complete listing of all TMI requirements from NUREG-0737 and NUREG-0737 Supplement 1. Regulatory Guide 1.97 requirements, although addressed in Supplement 1 to NUREG-0737, are excluded since they did not originate from the TMI Action Plan. This listing consists of a matrix of items applicable to both operating reactors and operating license applicants (NTOLS) as of November, 1980, as shown in NUREG-0737 Enclosure 1 and 2, respectively. All items which are not listed identically in Enclosure 1 and 2 are marked with asterisks to indicate differences between operating reactor and NTOL requirements. Whenever these differences occur, enter data in accordance with the appropriate title, regardless of the TMI Action Plan item number listed. Your NRC Project Manager has indicated by circle which issues are shown as unimplemented in the NRC tracking system. Please review the entire listing for each licensed reactor unit. Where an item was not applicable for your facility, mark N/A in the implementation status column.

2. Where an item was applicable to your facility but no changes were necessary, mark NC in the implementation status column.

3. Where an item was applicable to your facility and changes are complete mark C in the implementation status column. If the date is readily available (month and year) we request you provide it, however, C is sufficient.

4. Where an item was applicable to your facility and is not fully implemented, provide your projected implementation date (month and year) and a short note identifying the outstanding item (e.g. hardware, procedures, training, technical specifications).

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ISSUE NUMBER	MULTI-PLANT ACTION NO.	ISSUE TITLE	LICENSEE IMPLEMENTATION STATUS
I A 1.1.1		SHIFT TECHNICAL ADVISOR - ON DUTY	
*I A 1.1.2		SHIFT TECHNICAL ADVISOR - TECH SPECS	
*I A 1.1.3	F001	SHIFT TECHNICAL ADVISOR - TRAINED PER LL CAT B	
*I A 1.1.4		SHIFT TECHNICAL ADVISOR - DESCRIBE LONG TERM PROGRAM	
I A 1.2		SHIFT SUPERVISOR RESPONSIBILITIES	
I A 1.3.1	F002	SHIFT MANNING - LIMIT OVERTIMES	
I A 1.3.2	F002	SHIFT MANNING - MIN SHIFT CREW	
I A 2.1.1		IMMEDIATE UPGRADING OF RO & SRO TRAINING AND QUAL. - SRO EXPER	
I A 2.1.2		IMMEDIATE UPGRADING OF RO & SRO TRAINING AND QUAL. - SRO'S BE RO'S 1YR	
I A 2.1.3		IMMEDIATE UPGRADING OF RO & SRO TRAINING AND QUAL. - 3 MO. TRAINING	
I A 2.1.4	F003	IMMEDIATE UPGRADING OF RO & SRO TRAINING AND QUAL. - MODIFY TRAINING	
I A 2.1.5		IMMEDIATE UPGRADING OF RO & SRO TRAINING AND QUAL. - FACILITY CERTIF	
I A 2.3		ADMINISTRATION OF TRAINING PROGRAMS	
I A 3.1.1		REVISE SCOPE & CRITERIA FOR LICENSING EXAMS - INCREASE SCOPE	
I A 3.1.2		REVISE SCOPE & CRITERIA FOR LICENSING EXAMS - INCREASE PASSING GRADE	
I A 3.1.3.A		REVISE SCOPE & CRIT. FOR LIC. EXAMS - SIMULATOR PLANTS WITH SIMULATORS	
I A 3.1.3.B		REVISE SCOPE & CRIT. FOR LIC. EXAMS - SIMULATOR - OTHER PLANTS	
*I B 1.2		EVALUATION OF ORGANIZATION & MANAGEMENT	
I C 1.1		SHORT-TERM ACCIDENT & PROCEDURES REVIEW - SB LOCA	
I C 1.2.A	F004	SHORT-TERM ACCID. & PROCEDURES REV. - INADEQ. CORE COOL. REANAL. GUIDELINES	
I C 1.2.B	F004	SHORT-TERM ACCID. & PROCEDURES REV. - INADEQ. CORE COOL. REVISE PROCEDURES	
I C 1.3.A	F005	SHORT-TERM ACCID. & PROCEDURES REV - TRANSIENTS & ACCDTS. REANAL GUIDELINES (PROC. GEN. PKG.)	
I C 1.3.B	F005	SHORT-TERM ACCID. & PROCEDURES REV. - TRANSIENTS & ACCDRS. REVISE PROCEDURES (UPGRADED EOP'S)	
I C 2		SHIFT & RELIEF TURNOVER PROCEDURES	
I C 3		SHIFT-SUPERVISOR RESPONSIBILITY	
I C 4		CONTROL-ROOM ACCESS	
I C 5	F006	FEEDBACK OF OPERATING EXPERIENCE	
I C 6	F007	VERIFY CORRECT PERFORMANCE OF OPERATING ACTIVITIES	
*I C 7.1		NSSS VENDOR REV. OF PROC - LOW POWER TEST PROGRAM	
*I C 7.2		NSSS VENDOR REV. OF PROC - POWER ASCENSION & EMER. PROCS	
*I C 8		PILOT MON OF SELECTED EMERGENCY PROC FOR NIOLS	
I D.1	(See F008 & F071)	CONTROL-ROOM DESIGN REVIEWS (ENTER DATA FOR MPA F008 & MPA F-071)	
I D 2.1	F009	PLANT-SAFETY PARAMETER DISPLAY CONSOLE - DESCRIPTION	

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ISSUE NUMBER	MULTI-PLANT ACTION NO.	ISSUE TITLE	LICENSEE IMPLEMENTATION STATUS
II.D.2.2	F009	*PLANT-SAFETY PARAMETER DISPLAY CONSOLE - INSTALLED	
II.D.2.3	F009	PLANT-SAFETY PARAMETER DISPLAY CONSOLE - FULLY IMPLEMENTED	
*I.G.1.1		TRAINING DURING LOW-POWER TESTING - PROPOSE TESTS	
*I.G.1.2		TRAINING DURING LOW-POWER TESTING - SUBMIT ANAL. & PROCS	
*I.G.1.3		TRAINING DURING LOW-POWER TESTING - TRAINING & RESULTS	
II.B.1.1		REACTOR-COOLANT SYSTEM VENTS - DESIGN VENTS	
II.B.1.2	F010	REACTOR-COOLANT SYSTEM VENTS - INSTALL VENTS (EL CAT B)	
II.B.1.3	F010	REACTOR-COOLANT SYSTEM VENTS - PROCEDURES	
II.B.2.1		PLANT SHIELDING - REVIEW DESIGNS	
*II.B.2.2		PLANT SHIELDING - CORRECTIVE ACTIONS TO ASSURE ACCESS	
*II.B.2.3	F011	PLANT SHIELDING - PLANT MODIFICATIONS (EL CAT B)	
*II.B.2.4		PLANT SHIELDING - EQUIPMENT QUALIFICATION- NOT TRACKED AS A TMI ACTION ITEM	
*II.B.3.1		POSTACCIDENT SAMPLING - INTERIM SYSTEM	
*II.B.3.2		POSTACCIDENT SAMPLING - CORRECTIVE ACTIONS	
*II.B.3.3		POSTACCIDENT SAMPLING - PROCEDURES	
*II.B.3.4	F012	POSTACCIDENT SAMPLING - PLANT MODIFICATIONS (EL CAT B)	
II.B.4.1	F013	TRAINING FOR MITIGATING CORE DAMAGE - DEVELOP TRAINING PROGRAM	
*II.B.4.2.A	F013	TRAINING FOR MITIGATING CORE DAMAGE - INITIAL	
*II.B.4.2.B	Q13	TRAINING FOR MITIGATING CORE DAMAGE - COMPLETE	
II.D.1.1		RELIEF & SAFETY VALVE TEST REQUIREMENTS - SUBMIT PROGRAM	
*II.D.1.2.A		RELIEF & SAFETY VALVE TEST REQUIREMENTS - COMPLETE TESTING	
*II.D.1.2.B	F014	RELIEF & SAFETY VALVE TEST REQUIREMENTS - PLANT SPECIFIC REPORT	
II.D.1.3		RELIEF & SAFETY VAVLE TEST REQUIREMENTS - BLOCK-VALVE TESTING	
*II.D.3.1		VALVE POSITION INDICATION - INSTALL DIRECT INDICATIONS OF VALVE POS	
*II.D.3.2		VALVE POSTION INDICATION - TECH SPECS	
*II.E.1.1.1 (SEE NOTE 1)	F015	AFS EVALUATION-ANALYSIS	
*II.E.1.1.2 (SEE NOTE 2)	F015	AFS EVALUATION-SHORT TERM MODS	
II.E.1.1.3 (SEE NOTE 3)	F015	AFS -LONG TERM MODS	

NOTE 1 - THE ITEM LISTED IS FROM NUREG-0737, ENCLOSURE 2 AND IS APPLICABLE TO NTOLS'S ONLY
NOTE 2 - THE ITEM LISTED IS FOR ALL PLANTS (OPERATING REACTORS AND NTOL'S)
NOTE 3 - THE ITEM LISTED IS FOR ALL PLANTS (OPERATING REACTORS AND NTOL'S)

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ISSUE NUMBER	MULTI-PLANT ACTION NO.	ISSUE TITLE	LICENSEE IMPLEMENTATION STATUS
II.E.1.2.1.A		AFS INITIATION & FLOW-CONTROL GRADE	
II.E.1.2.1.B	F016	AFS INITIATION & FLOW - SAFETY GRADE	
II.E.1.2.2.A		AFS INITIATION & FLOW - FLOW INDICATION CONTROL GRADE	
*II.E.1.2.2.B		AFS INITIATION & FLOW - LL CAT A TECH SPECS	
*II.E.1.2.2.C	F017	AFS INITIATION & FLOW - SAFETY GRADE	
*II.E.3.1.1		EMERGENCY POWER FOR PRESSURIZER HEATERS - UPGRADE POWER SUPPLY	
*II.E.3.1.2		EMERGENCY POWER FOR PRESSURIZER HEATERS - TECH SPECS	
II.E.4.1.1		DEDICATED HYDROGEN PENETRATIONS - DESIGN	
*II.E.4.1.2		DEDICATED HYDROGEN PENETRATIONS - REVIEW & REVISE H2 CONTROL PROC	
*II.E.4.1.3	F018	DEDICATED HYDROGEN PENETRATION - INSTALL	
II.E.4.2.1-4		CONTAINMENT ISOLATION DEPENDABILITY - IMP. DIVERSE ISOLATION	
*II.E.4.2.5.A		CONTAINMENT ISOLAT. DEPENDABILITY - CNTMT PRESS. SETPT. SPECIFIY PRESS	
*II.E.4.2.5.B		CONTAINMENT ISOLATION DEPENDABILITY - CNTMT PRESSURE SETPT. MODS	
II.E.4.2.6	F019	CONTAINMENT ISOLATION DEPENDABILITY - CNTMT PURGE VALVES	
II.E.4.2.7	F019	CONTAINMENT ISOLATION DEPENDABILITY - RADIATION SIGNAL ON PURGE VALVES	
*II.E.4.2.8		CONTAINMENT ISOLATION DEPENDABILITY - TECH SPECS	
*II.F.1.1	F020	ACCIDENT - MONITORING - PROCEDURES	
*II.F.1.2.A	F020	ACCIDENT - MONITORING - NOBLE GAS MONITOR	
*II.F.1.2.B	F021	ACCIDENT - MONITORING - IODINE/PARTICULATE SAMPLING	
*II.F.1.2.C	F022	ACCIDENT - MONITORING - CONTAINMENT HIGH-RANGE MONITOR	
*II.F.1.2.D	F023	ACCIDENT - MONITORING - CONTAINMENT PRESSURE	
*II.F.1.2.E	F024	ACCIDENT - MONITORING - CONTAINMENT WATER LEVEL	
*II.F.1.2.F	F025	ACCIDENT - MONITORING - CONTAINMENT HYDROGEN	
*II.F.2.1		INSTRUMENTATION FOR DETECT. OF INADEQUATE CORE COOLING - PROCEDURES	
*II.F.2.2		INSTRUMENTATION FOR DETECT. OF INADEQUATE CORE COOLING - SUBCOOL METER	
*II.F.2.3	F026	INSTRUMENTATION FOR DETECT. OF INADEQUATE CORE COOLING - DESC. OTHER	
*II.F.2.4	F026	INSTRUMENTATION FOR DETECT. OF INADEQUATE CORE CLING INSTLL ADD'L INSTRUMENTATION	
*II.G.1.1		POWER SUPP. FOR PRESSURIZER RELIEF, BLOCK VALVES & LEVEL IND. - UPGRADE	
*II.G.1.2		POWER SUPP. FOR PRESSURIZER RELIEF, BLOCK VALVES & LEVEL IND. - TECH SP	
II.K.1 (Oper. Reactors Only)		IE BULLETINS - 79-05, 79-06, 79-08	

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ISSUE NUMBER	MULTI-PLANT ACTION NO.	ISSUE TITLE	LICENSEE IMPLEMENTATION STATUS
*II K.1.5		IE BULLETINS - REVIEW ESF VALVES	
*II K.1.10		IE BULLETINS - OPERABILITY STATUS	
*II K.1.20		IE BULLETINS - PROMPT MANUAL REACTOR TRIP	
*II K.1.21		IE BULLETINS - AUTO SG ANTICIPATORY REACTOR TRIP	
*II K.1.22		IE BULLETINS - AUX. HEAT REM SYSTEM, PROC	
*II K.1.23		IE BULLETINS - RV LEVEL, PROCEDURES	
*II K.2.2		ORDERS ON B&W PLANTS - PROCEDURES TO CONTROL AFW IND OF ICS	
*II K.2.8		ORDERS ON B&W PLANTS - UPGRADE AFW SYSTEM	
II K.2.9	F027	ORDERS ON B&W PLANTS - FEMA ON ICS	
II K.2.10	F028	ORDERS ON B&W PLANTS - SAFETY-GRADE TRIP	
*II K.2.11	F029	ORDERS ON B&W PLANTS - OPERATOR TRAINING	
II K.2.13	F030	ORDERS ON B&W PLANTS - THERMAL MECHANICAL REPORT (CE & W PLANTS ALSO)	
II K.2.14	F031	ORDERS ON B&W PLANTS - LIFT FREQUENCY OF PORV'S & SV'S	
II K.2.15		ORDERS ON B&W PLANTS - EFFECTS OF SLUG FLOW	
II K.2.16	F032	ORDERS ON B&W PLANTS - RCP SEAL DAMAGE	
II K.2.17	F033	ORDERS ON B&W PLANTS - VOIDING IN RCS (CE & W PLANTS ALSO)	
II K.2.19		BENCHMARK ANALYSIS OF SEQUENTIAL AFW FLOW TO ONCE THROUGH STM GENERATOR	
*II K.2.20	F035	ORDERS ON B&W PLANTS - SYSTEM RESPONSE TO SB LOCA	
*II K.3.1.A	F036	B&O TASK FORCE - AUTOMATIC PORV ISOLATION DESIGN	
*II K.3.1.B		FINAL RECOMMENDATIONS, B&O TASK FORCE - AUTO PORV ISO TEST/INSTALL	
II K.3.2	F037	B&O TASK FORCE - REPORT ON PORV FAILURES	
II K.3.3	F038	B&O TASK FORCE - REPORTING SV & RV FAILURES AND CHALLENGES	
II K.3.5.A	F039	B&O TASK FORCE - AUTO TRIP OF RCP'S PROPOSED MODIFICATIONS	
II K.3.5.B	F039	B&O TASK FORCE - AUTO TRIP OF RCP'S MODIFICATIONS	
II K.3.7		B&O TASK FORCE - EVALUATION OF PORV OPENING PROBABILITIES	
II K.3.9	F040	B&O TASK FORCE - PID CONTROLLER MODIFICATION	
II K.3.10	F041	B&O TASK FORCE - PROPOSED ANTICIPATORY TRIP MODIFICATIONS	
II K.3.11		B&O TASK FORCE - JUSTIFY USE OF CERTAIN PORV	
II K.3.12.A		B&O TASK FORCE - ANTICIPATORY TRIP ON TURBINE TRIP PROPOSED MODS	
II K.3.12.B	F042	B&O TASK FORCE - ANTICIPATORY TRIP ON TURBINE TRIP INSTALL MODS	
II K.3.13.A	F043	B&O TASK FORCE - HPCI & RCIC SYSTEM INITIATION LEVELS ANALYSIS	
II K.3.13.B	F043	B&O TASK FORCE - HPCI & RCIC INITIATION LEVELS MODIFICATION	

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ISSUE NUMBER	MULTI-PLANT ACTION NO.	ISSUE TITLE	LICENSEE IMPLEMENTATION STATUS
*II.K.3.14	F044	B&O TASK FORCE - ISO CONDENSER ISOLATION ON HIGH RAD	
II.K.3.15	F045	B&O TASK FORCE - MODIFY HPCI & RCIC BRK DETECTION CIRCUITRY	
II.K.3.16A	F046	B&O TASK FORCE - CHALLENGE & FAILURE OF RELIEF VALVES STUDY	
II.K.3.16.B	F046	B&O TASK FORCE - CHALLENGE & FAILURE OF RELIEF VALVES MODIFICATIONS	
II.K.3.17	F047	B&O TASK FORCE - ECC SYSTEM OUTAGES	
II.K.3.18.A	F048	B&O TASK FORCE - ADS ACTUATION STUDY	
II.K.3.18.B	F048	B&O TASK FORCE - ADS ACTUATION PROPOSED MODIFICATIONS	
II.K.3.18.C	F048	B&O TASK FORCE - ADS ACTUATION MODIFICATIONS	
*II.K.3.19	F049	B&O TASK FORCE - INTERLOCK RECIRCULATORY PUMP MODIFICATIONS	
*II.K.3.20		B&O TASK FORCE - LOSS OF SVC WATER AT BRP	
II.K.3.21.A	F050	B&O TASK FORCE - RESTART OF CSS & LPCI LOGIC DESIGN	
II.K.3.21.B	F050	B&O TASK FORCE - RESTART OF CSS & LPCI LOGIC DESIGN MODIFICATIONS	
II.K.3.22.A	F051	B&O TASK FORCE - RCIC SUCTION VERIFICATION PROCEDURES	
II.K.3.22.B	F051	B&O TASK FORCE - RCIC SUCTION MODIFICATION	
II.K.3.24	F052	B&O TASK FORCE - SPACE COOLING FOR HPCI/RCI LOSS OF AC POWER	
II.K.3.25.A		B&O TASK FORCE - POWER ON PUMP SEALS PROPOSED MODIFICATIONS	
II.K.3.25.B	F053	B&O TASK FORCE - POWER ON PUMP SEALS MODIFICATIONS	
II.K.3.27	F054	B&O TASK FORCE - COMMON REFERENCE LEVEL FOR BWRS	
II.K.3.28	F055	B&O TASK FORCE - QUALIFICATION OF ADS ACCUMULATORS	
*II.K.3.29	F056	B&O TASK FORCE - PERFORMANCE OF ISOLATION CONDENSERS	
II.K.3.30.A		B&O TASK FORCE - SCHEDULE FOR OUTLINE OF SB LOCA MODEL	
II.K.3.30.B	F057	B&O TASK FORCE - SB LOCA MODEL, JUSTIFICATION	
II.K.3.30.C		B&O TASK FORCE - SB LOCA METHODS NEW ANALYSES	
II.K.3.31	F058	B&O TASK FORCE - COMPLIANCE WITH CFR 50.46	
*II.K.3.40		B&O TASK FORCE - RCP SEAL DAMAGE - COVERED BY II.K.2.16 AND II.K.3.25	
*II.K.3.43		B&O TASK FORCE - EFFECTS OF SLUG FLOW - COVERED BY II.K.2.15	
II.K.3.44	F059	B&O TASK FORCE - EVALUATE TRANSIENT WITH SINGLE FAILURE	
II.K.3.45	F060	B&O TASK FORCE - ANALYSES TO SUPPORT	
II.K.3.46	F061	RESPONSE TO LIST OF CONCERNS FROM ACRS CONSULTANT	
*II.K.3.57	F062	IDENTIFY WATER SOURCES PRIOR TO MANUAL ACTIVATION OF ADS	
III.A.1.1		EMERGENCY PREPAREDNESS, SHORT TERM	

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ISSUE NUMBER	MULTI-PLANT ACTION NO.	ISSUE TITLE	LICENSEE IMPLEMENTATION STATUS
III.A.1.2.1		*UPGRADE EMERGENCY SUPPORT FACILITIES - INTERIM TSC OSC & EOF	
III.A.1.2.2 (SEE F063, F064, F065)		UPGRADE EMERGENCY SUPPORT FACILITIES-DESIGN-INCORP. INTO F063/F064/F065	
III.A.1.2.3 (SEE F063, F064, F065)		UPGRADE EMER SUPPORT FACILITIES - MODS INCORPOR. INTO F063, F064 & F065	
III.A.2.1	F067	UPGRADE PREPAREDNESS - UPGRADE EMERGENCY PLANS TO APP. E. 10 CFR 50	
III.A.2.2	F068	UPGRADE PREPAREDNESS - METEOROLOGICAL DATA	
*III.D.1.1.1		PRIMARY COOLANT OUTSIDE CONTAINMENT - LEAK REDUCTION	
*III.D.1.1.2		PRIMARY COOLANT OUTSIDE CONTAINMENT - TECH SPECS	
III.D.3.3.1		INPLANT RAD. MONIT. - PROVIDE MEANS TO DETER. PRESENCE OF RADIOIODINE	
III.D.3.3.2	F069	INPLANT RADIATION MONIT. - MODIFICATIONS TO ACCURATELY MEAS. IODINE	
III.D.3.4.1	F070	CONTROL ROOM HABITABILITY - REVIEW	
*III.D.3.4.2	F070	CONTROL ROOM HABITABILITY - SCHEDULE MODIFICATIONS	
*III.D.3.4.3		CONTROL ROOM HABITABILITY - IMPLEMENT MODIFICATIONS	
MPA-F008	F008	I.D.1.1 DETAILED CONTROL ROOM DESIGN REVIEW PROGRAM PLAN	
MPA-F063	F063	III.A.1.2 TECHNICAL SUPPORT CENTER	
MPA-F064	F064	III.A.1.2 OPERATIONAL SUPPORT CENTER	
MPA-F065	F065	III.A.1.2 EMERGENCY OPERATIONS FACILITY	
MPA-F071	F071	I.D.1.2 DETAILED CONTROL ROOM REVIEW (FOLLOWUP TO F-8)	
MPA-B072	B072	NUREG-0737 TECH SPECS (GENERIC LETTERS 82-16 & 83-02)	
MPA-B083	B083	TECH SPEC COVERED BY GENERIC LETTERS 83-26 & 83-37 FOR NUREG-0737	
67.4.1	G001	REACTOR COOLANT PUMP TRIP (GENERIC LETTER 85-12)	