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Quad Cities Generating Station
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ComEd

SVP-98-087

March 10, 1998

U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

SUBJECT: Quad Cities Nuclear Station Units 1 and 2
Monthly Performance Report
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of February, 1998.

If you have any questions concerning this letter, please contact Mr. Charles Peterson, Regulatory Affairs Manager, at (309) 654-2241, extension 3609.

Respectfully,

E. S. Kraft, Jr.
Site Vice President
Quad Cities Station

ESK/dak

Enclosure

cc: A. Beach, Regional Administrator
R. Pulsifer, Project Manager, NRR
C. Miller, Senior Resident Inspector
W. Leech, MidAmerican Energy Company
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SVP Letter File

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QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

FEBRUARY 1998

COMMONWEALTH EDISON COMPANY

AND

MIDAMERICAN ENERGY COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Quad Cities Unit One was off-line the entire month of February 1998 due to a unplanned extension of maintenance Outage Q1P01.

B. Unit Two

Quad Cities Unit Two remained shutdown for the month of February 1998 due to a unplanned extension of maintenance outage Q2P01.

TABLE OF CONTENTS

I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors and Steam Turbine/Generators, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and MidAmerican Energy Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Dick Stockman and Debra Kelley, telephone number 309-654-2241, extensions 3221 and 2240, respectively.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two submitted during the reporting period.

UNIT 1

Licensee Event Report Number	Submission Date	Title of Occurrence
1-98-01	2/4/98	The Unit One Emergency Diesel Generator (EDG) Received an Inadvertent Start Signal When a Relay Was Bumped During Testing, the EDG Failed to Start as Would be Expected on Receipt of this Signal, and the EDG was Inadvertently Started 15 Minutes Later Due to an Error by an Operator Who Was Responding to the Failure to Start.
1-97-26, R1	2/4/98	Technical Specification (TS) Required Instrument Channel Checks and Primary Containment Sump Flowrate Surveillances Were Not Documented Prior to Entering the Applicable Mode Due to Inadequate Procedure Development and Review.
1-98-03	2/5/98	Technical Specification (TS) Limiting Condition for Operation (LCO) was exceeded when both Standby Gas Treatment (SBGT) subsystems were inoperable because both the Unit 1 and the Unit 2 Emergency Diesel Generators (EDG) were inoperable due to inadequate procedure development and review.

UNIT 1 CONTINUED

Licensee Event Report Number	Submission Date	Title of Occurrence
1-98-04	2/12/98	Residual Heat Removal (RHR) Shutdown Cooling Common Suction Header was Made Inoperable Due to a Technical Specification Limiting Condition for Operation (LCO) When Evaluating an Unacceptable Mechanical Shock Arrestor (Snubber) Due to Operations and Engineering Knowledge Deficiencies and an Inadequate Procedure.
1-98-07	2/11/98	The design basis of the Quad Cities Station Reactor Building (RB) superstructure is not in literal conformance with Updated Final Safety Analysis Report (UFSAR) description of Class I loading combinations. This is due to the fact that the FSAR, when originally written, lacked sufficient detail in description of Class I loading combinations for addressing infrequent loading conditions (such as the crane) concurrent with a seismic event.
1-98-08	2/26/98	Residual Heat Removal (RHR) Shutdown Cooling Common Suction Header was Inoperable Due to Inadequate Installation Instructions Resulting in Mechanical Failure of a Mechanical Shock Arrestor (Snubber).
1-98-06	2/25/98	Reactor Building Post Loss of Coolant Accident (LOCA) Temperatures are Higher Than Values Used for the Environmental Qualification of Electrical Equipment Due to Unvalidated Engineering Judgement, the Cause of Which Cannot Be Determined.

UNIT 2

Licensee Event Report Number	Submission Date	Title of Occurrence

The were no U2 LER's for this reporting period.

V. DATA TABULATIONS

The following data tabulations are presented in this report:

A. Average Daily Unit Power Level

B. Operating Data Report

C. Unit Shutdowns and Power Reductions

APPENDIX C

OPERATING DATA REPORT

DOCKET NO. 50-254
UNIT One
DATE March 10, 1998
COMPLETED BY Dick Stockman
TELEPHONE (309) 654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 020198 2400 022898 GROSS HOURS IN REPORTING PERIOD: 672

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX > DEPEND > CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-NET): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	0.00	0.00	172375.40
6. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	3421.90
7. HOURS GENERATOR ON LINE	0.00	0.00	167295.30
8. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	909.20
9. GROSS THERMAL ENERGY GENERATED (MWH)	0.00	0.00	365437242.60
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0.00	0.00	118253369.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)	0.00	0.00	106154969.00
12. REACTOR SERVICE FACTOR	0.00	0.00	75.97
13. REACTOR AVAILABILITY FACTOR	0.00	0.00	77.48
14. UNIT SERVICE FACTOR	0.00	0.00	73.73
15. UNIT AVAILABILITY FACTOR	0.00	0.00	74.13
16. UNIT CAPACITY FACTOR (Using MDC)	0.00	0.00	60.84
17. UNIT CAPACITY FACTOR (Using Design MWe)	0.00	0.00	59.30
18. UNIT FORCED OUTAGE RATE	0.00	0.03	7.19

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD < ESTIMATED DATE OF STARTUP: Q1P01 - 4/17/98

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION): N/A

	FORECAST	ACHIEVED	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			

APPENDIX C

OPERATING DATA REPORT

DOCKET NO. 50-265
UNIT Two
DATE March 10, 1998
COMPLETED BY Dick Stockman
TELEPHONE (309) 654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 020198 2400 022898 GROSS HOURS IN REPORTING PERIOD: 672

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX > DEPEND > CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-NET): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	0.00	0.00	164367.05
6. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	2985.80
7. HOURS GENERATOR ON LINE	0.00	0.00	159969.85
8. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	702.90
9. GROSS THERMAL ENERGY GENERATED (MWH)	0.00	0.00	348356256.32
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0.00	0.00	111762544.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)	0.00	0.00	105947458.00
12. REACTOR SERVICE FACTOR	0.00	0.00	72.94
13. REACTOR AVAILABILITY FACTOR	0.00	0.00	74.27
14. UNIT SERVICE FACTOR	0.00	0.00	70.99
15. UNIT AVAILABILITY FACTOR	0.00	0.00	71.30
16. UNIT CAPACITY FACTOR (Using MDC)	0.00	0.00	61.14
17. UNIT CAPACITY FACTOR (Using Design MWe)	0.00	0.00	59.59
18. UNIT FORCED OUTAGE RATE	0.00	0.00	11.19

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): N/A

20. IF SHUTDOWN AT END OF REPORT PERIOD < ESTIMATED DATE OF STARTUP: Q2P01 - 4/12/98

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254
UNIT One
DATE March 10, 1998
COMPLETED BY Dick Stockman
TELEPHONE (309) 654-2241

MONTH February 1998

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	- 7	17.	- 7
2.	- 7	18.	- 7
3.	- 7	19.	- 7
4.	- 6	20.	- 7
5.	- 7	21.	- 7
6.	- 8	22.	- 8
7.	- 8	23.	- 8
8.	- 8	24.	- 7
9.	- 8	25.	- 7
10.	- 8	26.	- 7
11.	- 8	27.	- 7
12.	- 8	28.	- 7
13.	- 8	29.	
14.	- 7	30.	
15.	- 7	31.	
16.	- 7		

INSTRUCTIONS
On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265
UNIT Two
DATE March 10, 1998
COMPLETED BY Dick Stockman
TELEPHONE (309) 654-2241

MONTH February 1998

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	- 7	17.	- 7
2.	- 7	18.	- 7
3.	- 7	19.	- 7
4.	- 7	20.	- 7
5.	- 7	21.	- 6
6.	- 8	22.	- 8
7.	- 8	23.	- 7
8.	- 8	24.	- 6
9.	- 8	25.	- 7
10.	- 8	26.	- 7
11.	- 8	27.	- 7
12.	- 8	28.	- 7
13.	- 8	29.	
14.	- 8	30.	
15.	- 7	31.	
16.	- 7		

INSTRUCTIONS
On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-254

UNIT NAME One

COMPLETED BY Dick Stockman

DATE March 10, 1998 REPORT MONTH February 1998

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
98-02	980201	S	672.0	B	4	-----	--	---	Continuation of Maintenance Outage Q1P01 (Unplanned Extension).

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-265

UNIT NAME Two

COMPLETED BY Dick Stockman

DATE March 10, 1998 REPORT MONTH February 1998

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
98-01	980201	S	672.0	B	4	--------	----	---	Continuation of Maintenance Outage Q2P01 (Unplanned Extension).

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

There were no Main Steam Relief Valve Operations for the reporting period.

B. Control Rod Drive Scram Timing Data for Units One and Two

There was no Control Rod Drive scram timing data for Units One and Two for the reporting period.

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QTP 0300-S32
Revision 3
April 1997

QUAD CITIES REFUELING INFORMATION REQUEST

1. Unit: Q1 Reload: 14 Cycle: 15

2. Scheduled date for next refueling shutdown: 11/07/98

3. Scheduled date for restart following refueling: 12/17/98

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

   Yes

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   Approved

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   Approx. 216 SPC 9X9IX Fuel Bundles will be loaded.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 1933

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3657

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2002

QUAD CITIES REFUELING
INFORMATION REQUEST

1. Unit: Q2 Reload: 14 Cycle: 15

2. Scheduled date for next refueling shutdown: 1/8/2000

3. Scheduled date for restart following refueling: 2/17/2000

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

   Yes

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   August, 1999

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   N/A

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 2943

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3897

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2002

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM - Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI - American National Standards Institute
APRM - Average Power Range Monitor
ATWS - Anticipated Transient Without Scram
BWR - Boiling Water Reactor
CRD - Control Rod Drive
EHC - Electro-Hydraulic Control System
EOF - Emergency Operations Facility
GSEP - Generating Stations Emergency Plan
HEPA - High-Efficiency Particulate Filter
HPCI - High Pressure Coolant Injection System
HRSS - High Radiation Sampling System
IPCLRT - Integrated Primary Containment Leak Rate Test
IRM - Intermediate Range Monitor
ISI - Inservice Inspection
LER - Licensee Event Report
LLRT - Local Leak Rate Test
LPCI - Low Pressure Coolant Injection Mode of RHRs
LPRM - Local Power Range Monitor
MAPLHGR - Maximum Average Planar Linear Heat Generation Rate
MCPR - Minimum Critical Power Ratio
MFLCPR - Maximum Fraction Limiting Critical Power Ratio
MPC - Maximum Permissible Concentration
MSIV - Main Steam Isolation Valve
NIOSH - National Institute for Occupational Safety and Health
PCI - Primary Containment Isolation
PCIOMR - Preconditioning Interim Operating Management Recommendations
RBCCW - Reactor Building Closed Cooling Water System
RBM - Rod Block Monitor
RCIC - Reactor Core Isolation Cooling System
RHRS - Residual Heat Removal System
RPS - Reactor Protection System
RWM - Rod Worth Minimizer
SBGTS - Standby Gas Treatment System
SBLC - Standby Liquid Control
SDC - Shutdown Cooling Mode of RHRS
SDV - Scram Discharge Volume
SRM - Source Range Monitor
TBCCW - Turbine Building Closed Cooling Water System
TIP - Traversing Incore Probe
TSC - Technical Support Center