

## NUCLEAR REGULATORY COMMISSION

50-254

WASHINGTON, D.C. 20556-0001

April 16, 1999

Mr. Oliver D. Kingsley, President Nuclear Generation Group Commonwealth Edison Company Executive Towers West III 1400 Opus Place, Suite 500 Downers Grove, IL 60515

SUBJECT:

FLAW EVALUATION OF RECIRCULATION LINE WELD 02BS-F4 AT QUAD

CITIES NUCLEAR POWER STATION, UNIT 1 (TAC NO. MA4174)

Dear Mr. Kingsley:

During the recent refueling outage for Unit 1 (Q1R15), Commonwealth Edison Company (ComEd, the licensee) performed inservice inspection on the recirculation system piping using ultrasonic examination in accordance with Generic Letter 88-01, "NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping." In a letter dated November 24, 1998, the licensee reported that seven welds were found with flaw indications that exceeded the acceptance criteria in subarticle IWB-3500, "Acceptance Standards," of Section XI of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code), 1989 Edition. Additional information was provided by the licensee in conference calls held on November 19, 24 and 30, 1998. Welds 02AD-F12 and 02AS-S4, exhibiting indications identified in the preceding outage, showed no change in flaw size and the previous flaw evaluation remains applicable and acceptable for continued operation for the next operating cycle. One new flaw indication was detected on weld O2BS-F4. A flaw evaluation was performed on this weld to justify continued operation without repair. Each of the four remaining indications on welds 02AD-F8, O2BS-F7, O2AS-F9, and O2BS-F14 had a weld overlay repair applied in accordance with ASME Code Case N-504, "Alternative Rules for Repair of Class 1, 2, and 3 Austenitic Stainless Steel Piping, Section XI, Division 1."

The licensee performed a flaw evaluation on weld 02BS-F4 based on the indication detected by ultrasonic examination. Weld 02BS-F4 is an induction heating stress improvement treated flux weld fabricated by the shielded metal-arc welding (SMAW) process. The flaw was located on the pump suction side of the recirculation piping. The fracture mechanics evaluation was conducted using the procedures provided in Appendix C and Paragraph IWB-3640 of the ASME Code, Section XI. The crack growth evaluation to determine the projected crack depth was conducted using the procedures outlined in NUREG-0313, "Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping," Revision 2. The results of the licensee's flaw evaluation have shown that Quad Cities Nuclear Power Station, Unit 1, can be safely operated for the next operating cycle because the subject indications meet the criteria of IWB-3640, Section XI of the ASME Code.

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Based on a review of the licensee's submittal, the staff concludes that Quad Cities Nuclear Power Station, Unit 1, can be safely operated for the next fuel cycle with weld O2BS-F4 in its current condition because the structural integrity of the weld will be maintained. However, continued plant operation beyond the next fuel cycle will depend on the satisfactory evaluation of the re-inspection results or implementing acceptable repairs during the next refueling outage.

The staff's safety evaluation is enclosed. This completes our effort for TAC No. MA4174.

Sincerely.

Original signed by

Robert M. Pulsifer, Project Manager, Section 2 Project Directorate III Division of Licensing Project Management Office of Nuclear Reactor Regulation

Docket No. 50-254

Enclosure: Safety Evaluation

cc w/encl: See next page

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O. Kingsley Commonwealth Edison Company

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