December 9, 1998

Mr. Michael B. Roche
Vice Fresident and Director
GPU Nuclear, Inc.
Oyster Creek Nuclear Generating Station
P.O. Box 388
Forked River, New Jersey 08731

SUBJECT: Mid-Year Inspection Resource Planning Meeting - OYSTER CREEK NUCLEAR GENERATING STATION

Dear Mr. Roche:

On November 10, 1998, the NRC staff held an inspection resource planning meeting (IRPM). The IRPM provided a coordinated mechanism for Region I to adjust inspection schedules, as needed, prior to the conclusion of the Plant Performance Review cycle in May 1999.

Enclosure 1 contains a historical listing of plant issues, referred to as the Plant Issues Matrix (PIM), that were considered during this IRPM process to arrive at an integrated view of licensee performance trends. The PIM includes only items from inspection reports or other docketed correspondence between the NRC and GPU Nuclear, Inc.. The IRPM may also have considered some predecisional and draft material that does not appear in the attached PIM, including observations from events and inspections that had occurred since the last NRC inspection report was issued, but had not yet received full review and consideration. This material will be placed in the Public Document Room as part of the normal issuance of NRC inspection reports and other correspondence.

This letter advises you of our planned inspection effort resulting from the Oyster Creek Nuclear Generating Station IRPM review. It is provided to minimize the resource impact on your staff and to allow for scheduling conflicts and personnel availability to be resolved in advance of inspector arrival onsite. Enclosure 2 details our inspection plan for the next 6 months. Resident inspections are not listed due to their ongoing and continuous nature.

We will inform you of any changes to the inspection plan. If you have any questions, please contact me at (610) 337-5234.

Sincerely,

Original Signed By:

Peter W. Eselgroth, Chief
Reactor Projects Branch 7
Division of Reactor Projects

Docket Nos. 50-219

Enclosures: 1) Plant Issues Matrix
2) Inspection Plan

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OYSTER CREEK PLANT ISSUES MATRIX

Date Rec #	Type	Source	ID	SFA	Codes	Item Description
8/26/98 613	Positive	IR 98-05	N	1-OPS	1A 3A 5A	Control room operators responded appropriately to a failure of a containment isolation valve to fully stroke while isolating the reactor water cleanup system on two occasions. The chemistry department provided good support to evaluate chemistry parameter action levels and to provide recommendations concerning continued operation without the reactor water cleanup system.
8/26/98 612	Positive	IR 98-05	N	1-OPS	1A 3A	Operators demonstrated good technical specification (TS) awareness, properly documented applicable limiting conditions for operation, and met all TS requirements during periods of diesel inoperability.
8/26/98 611	Positive	IR 98-05	N	1-OPS	5A	The Independent Onsite Safety Review Group conducted a thorough Technical Specification review, identified several value-added improvements, and promptly initiated corrective actions.
8/26/98 610	Positive	IR 98-05	N	1-OPS	5A	The General Office Review Board demonstrated a good safety focus and promptly engaged the station's correction action process to address their safety concerns.
8/26/98 609	Positive	IR 98-05	N	1-OPS	3A	Operations properly planned and effectively implemented a transition to a new electronic control room narrative log format.
8/26/98 608	Positive	IR 98-05	N	1-OPS	1A 3A	Operations conducted safe and high quality fuel handling activities in preparation for the refueling outage. In addition, management provided good oversight.
7/17/98 592	Positive	IR 98-03	N	1-OPS	1B	Control room operators responded promptly and appropriately following a reactor feedwater pump trip while operating at full reactor power.
7/17/98 591	Positive	IR 98-03	N	1-OPS	5A	General Office Review Board members constructively evaluated station activities and asked probing questions focused on plant safety.
7/17/98 590	Negative	IR 98-03	N	1-OPS	3B 5C	Senior reactor operators continued to demonstrate a knowledge weakness relative to technical specification requirements for offsite power sources. Operations management initiated action to address this deficiency.
7/17/98 589	Negative	IR 98-03	N	1-OPS	3A 5A	Senior reactor operators demonstrated good critical self-assessment when they identified that they had inadvertently entered a 30-hour shutdown limiting condition for operation while conducting a containment spray surveillance with the redundant train's emergency diesel out of service. Senior reactor operators and reactor operators did not demonstrate good technical specification awareness as they missed opportunities to prevent the occurrence prior to commencing the surveillance. Operations initiated prompt corrective actions to address this deficiency.

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Date Rec #	Type	Source	ID	SFA	Codes	Item Description
7/17/98 588	Positive	IR 98-03	N	1-OPS	5A 5C	An operations training instructor demonstrated a good questioning attitude in identifying a potential operability concern involving the control rod drive (CRD) pump breakers. Operations and maintenance responded promptly to ensure that the CRD pump breakers would perform as designed.
7/17/98 587	Positive	IR 98-03	N	1-OPS	5A	Operations demonstrated good awareness to minimize unmonitored releases by identifying and documenting a degraded condition in the EF 1-6 Reactor Building Ventilation fan housing. Minor discrepancies were appropriately addressed.
6/3/98 575	Positive	IR 98-02	N	1-OPS	5A 5B	The management team effectively implemented the new corrective action process (CAP) and affected a smooth transition to the CAP system. Plant personnel initiated CAPs at a low threshold and operations promptly reviewed the CAPs to assess plant impact. The multi-disciplined Management Review Team engaged in active discussions concerning operability, reportability, and safety significance and initiated appropriate corrective actions.
6/3/98 574	Positive	IR 98-02	N	1-OPS	5A	Nuclear Safety Assessment conducted safety-focused and performance-based quality assurance assessments. Assessments provided timely, critical, value-added insight of station performance to plant management. Nuclear Safety Assessment appropriately used the corrective action process to address identified deficiencies.
6/3/98 573	Negative	IR 98-02	N	1-OPS	3B 3A	Senior reactor operators did not demonstrate a high level of knowledge concerning fire protection program requirements governing the redundant fire pump. Operators did not exercise a questioning attitude and did not ensure adequate fire protection system availability prior to removing a diesel-driven fire pump from service.
6/3/98 572	Negative	IR 98-02	N	1-OPS	1A 3A 5C	Operators did not use a questioning attitude to identify several operating procedure deficiencies. Once identified, operators effectively used the corrective action process to address the deficiencies.
6/3/98 571	Negative	IR 98-02	N	1-OPS	1A	Senior reactor operators did not demonstrate a good awareness concerning technical specification limiting conditions for operation while shutdown.
6/3/98 570	Positive	IR 98-02	N	1-OPS	1B 5A	Operators responded appropriately to a reactor building closed cooling water (RBCCW) high temperature condition that resulted in a water hammer in the RBCCW system. Operations management demonstrated a good safety focus and corrective action commitment in identifying several process weaknesses that contributed to the occurrence.
6/3/98 569	Positive	IR 98-02	N	1-OPS	1B	Operators responded promptly and appropriately to an off-site induced electrical transient. Senior reactor operators demonstrated good command and control and effectively controlled recovery actions.

OYSTER CREEK PLANT ISSUES MATRIX

Date Rec #	Type	Source	I/D	SFA	Codes	Item Description
4/2/98 553	Positive	IR 98-80	N	1-OPS	5A 5B 3B	Deviation Reports (DVR) reflected appropriate operability determinations, and management attention necessary to identify causes and put into place effective corrective actions. The Corrective Action training process was acceptable and staff members were generally knowledgeable of the corrective action process. The trending of DVRs was adequate and provided meaningful information to management. Although not all management expectations were met concerning the program issues, industry operating experience was adequately used at the plant.
4/2/98 552	Positive	IR 98-80	N	1-OPS	5C 3A 3C	The assessments and evaluations of the Independent Safety Oversight Review Group, the General Office Review Group, and Nuclear Safety Assessment staff were effective in providing independent oversight of safety significant activities. The experience of the Independent Safety Oversight members, both industry-wide and site-specific, was a significant contributor to the value of their products. Management appeared to be full supportive of the oversight groups and were responsive to their recommendations.
4/2/98 551	Positive	IR 98-80	N	1-OPS	5C 3A 3C	The corrective action and safety review audits were of high quality. Audit findings received appropriate division management attention. The May 1997 safety review process assessment was self-critical and probing. The NSA audit and assessment reports were effective in providing GPU Nuclear management independent identification of significant findings and appropriate recommendations for process improvements.
8/26/98 616	Positive	IR 98-05	N	2-MAINT	3A 5A 5C	Maintenance properly planned and effectively controlled a diesel generator battery replacement. Maintenance demonstrated a good safety focus in conducting increased monitoring following battery replacement and identified an apparent degraded condition on one of the battery cells. Maintenance took prompt and appropriate corrective action to address the condition.
8/26/98 615	Positive	IR 98-05	N	2-MAINT	3A 5B	Maintenance performed thorough and well documented troubleshooting following two trips of the 'B' reactor recirculation pump.
8/26/98 614	Positive	IR 98-05	N	2-MAINT	3A	Planning performed a thorough risk analysis and appropriately adjusted the work week schedule to support an emergent emergency service water (ESW) pump replacement. Maintenance activities were well-controlled and properly documented.
7/17/98 596	Positive	IR 98-03	N	2-MAINT	3A 5C	Maintenance management demonstrated a commitment to quality through their recognition and corrective measures to address an adverse condition involving job order closeout timeliness.
7/17/98 595	Negative	IR 98-03	N	2-MAINT	3A 5C	Management did not demonstrate a good safety focus in that they failed to resolve the post maintenance test requirement for a safety-related battery charger in a timely manner.

OYSTER CREEK PLANT ISSUES MATRIX

Date Rec #	Type	Source	I D	SFA	Codes	Item Description
7/17/98 594	Positive	IR 98-03	N	2-MAINT	3A 5B 5C	Maintenance, engineering, and operations effectively coordinated to methodically troubleshoot, evaluate and affect temporary repair to the 'A' feedwater pump 4Kv cable. Permanent corrective actions were taken for the recirculation pump undervoltage relay cards. Management provided good oversight of these activities.
7/17/98 593	Positive	IR 98-03	N	2-MAINT	3B 4C	The ISI program was properly implemented and controlled. The ISI personnel were properly qualified and knowledgeable of ISI and ASME Code requirements. The documentation supporting the program and ISI examination was satisfactory and readily available. Observations, indications and the resolutions had been clearly documented, and the results were reviewed and accepted by NDE Level III examiners.
6/3/98 578	Positive	IR 98-02	N	2-MAINT	3A	Maintenance demonstrated effective pre-job planning, proper work control, and good radiological worker practices in the conduct of electromatic relief valve pilot valve assembly replacements.
6/3/98 577	Negative	IR 98-02	N	2-MAINT	2B 5C	Maintenance did not effectively control preventive maintenance (PM) planning and execution which contributed to the PM backlog growth. Management initiated corrective actions to review and address PM program weaknesses.
6/3/98 576	Negative	IR 98-02	N	2-MAINT	2A 5B	Maintenance allowed a redundant temperature indicator in the RBCCW system to remain inoperable for a prolonged duration which adversely affected plant operations.
4/2/98 554	Positive	IR 98-80	N	2-MAINT	4A 2B 3A	The Containment Spray System (SS) and Emergency Service Water System (ESWS) were maintained operable and capable of performing their safety functions including during a loss of offsite power and a single active failure. Surveillance testing of system components were conducted in accordance with existing procedures. The CSS and ESWS instruments were properly calibrated and maintained.
8/26/98 621	NCV Positive	IR 98-05 NCV 98-05-02	N	3-ENG	4B 5A 5B 5C	Engineering demonstrated a good questioning attitude in identifying and documenting a diesel generator switchgear seismic nonconformance. Engineering led the efforts to disposition operability issues, analyze and resolve the nonconformance, address reportability, and implement appropriate corrective actions. (Enforcement Discretion exercised per VII.B.1 Of the Enforcement Policy.)
8/26/98 620	Positive	IR 98-05	N	3-ENG	4B 5B	Engineering demonstrated good involvement in evaluating a problem with a reactor water cleanup isolation valve. Engineering root cause evaluation continued at the end of the report period. (URI 98-05-01)
8/26/98 619	Positive	IR 98-05	N	3-ENG	4B 5B	Engineering led a well-controlled and effectively coordinated troubleshooting effort to evaluate a main condenser degraded vacuum condition.

OYSTER CREEK PLANT ISSUES MATRIX

Date Rec #	Type	Source	I D	SFA	Codes	Item Description
8/26/98 618	Positive	IR 98-05	N	3-ENG	4B 5B	Engineering provided good support and evaluation of the data, following troubleshooting of the 'B' reactor recirculation pump trips.
8/26/98 617	Positive	IR 98-05	N	3-ENG	3A 5C	The ESW system engineer demonstrated good system ownership and led the efforts to promptly address degraded performance on an ESW pump.
7/17/98 600	NCV Positive	IR 98-03 NCV 98-03-02	L	3-ENG	5A 5B 5C	Engineering properly identified, documented, evaluated, and reported a fire protection duct detector test deficiency. Engineering took prompt and appropriate corrective actions to address this deficiency. (Enforcement Discretion exercised per VII.B.1 Of the Enforcement Policy.)
7/17/98 599	Positive	IR 98-03	N	3-ENG	3A 5A	The Maintenance Rule Coordinator performed a high-quality, value-added, and thorough periodic assessment of maintenance program effectiveness. In addition, the Coordinator demonstrated a good safety focus in identifying negative trends involving power cable failures and material control issues and initiated corrective action via the approved process.
7/17/98 598	Positive	IR 98-03	N	3-ENG	3A 5B	The Operating Experience Coordinator and Core Engineering demonstrated a good safety perspective in promptly evaluating an industry concern involving potential fuel channel bowing to ensure that Oyster Creek took appropriate and timely corrective actions.
7/17/98 597	VIO	IR 98-03 VIO 98-03-01	N	3-ENG	5A 5B	Maintenance and engineering identified the root cause of emergency diesel generator 2's failure to start. In addition, they adequately determined the most probable cause of emergency diesel generator 1's failure to start. During troubleshooting activities, maintenance and system engineers failed to exercise a questioning attitude with regards to an installed jumper on a replacement starter and failed to comply with the receipt deficiency reporting requirements. (Violation of 10 CFR 50, Appendix B, Criterion XV)
7/2/98 586	Positive	IR 98-07	N	3-ENG	5A 5B 5C	GPUN conducted a thorough and self-critical assessment of the Oyster Creek GL 89-10 MOV program. Deficiencies identified by the self-assessment team were addressed adequately.
7/2/98 585	Negative	IR 98-07	N	3-ENG	4C	In evaluating the operability of containment spray valves V-21-5 and V-21-11, GPUN used an inadequately validated method to reduce perceived conservativisms in the Electric Power Research Institute MOV Performance Prediction Methodology. However, the licensee adequately justified current MOV functionality by other means.

OYSTER CREEK PLANT ISSUES MATRIX

Date Rec #	Type	Source	I/D	SFA	Codes	Item Description
7/2/98 584	Negative	IR 98-07	N	3-ENG	2A 4B	Although action plans were tracked and scheduled, the licensee had not yet adequately verified the design basis capability of approximately two-thirds of the valves in its GL 89-10 program. This included all of the twelve risk-significant high energy line break isolation valves at Oyster Creek. However, current MOV functionality is acceptable, based on independent evaluation of industry data. GPUN performed acceptable technical evaluations of load sensitive behavior, stem friction coefficient, and dynamic unwedging thrust requirements.
6/3/98 580	NCV Positive	IR 98-02 NCV 98-02-02	L	3-ENG	4A 5C	Equipment reliability engineering demonstrated a good design basis awareness in identifying and documenting a degraded condition involving Old Radioactive Waste building ventilation. Management promptly initiated actions to restore operability, address reportability, and implement appropriate corrective measures.
6/3/98 579	NCV Positive	IR 98-02 NCV 98-02-01	L	3-ENG	4A 5C	Structural engineering conducted a thorough review of the shutdown cooling piping design basis and identified that several pipe supports did not meet design basis requirements. Engineering appropriately dispositioned operability, addressed reportability, and initiated appropriate corrective actions.
4/2/98 567	EEI	IR 98-80 EEI 98-80-03	N	3-ENG	4A 4C 5C	Engineering failed to verify that the field installation of the EMRV solenoid voltage was representative of the EQ documentation as required by 10 CFR 50.49, Environmental Qualification. Further, when the NRC identified discrepancies with the EQ program, GPUN was untimely at performing the program required determination of operability. (Apparent violation of 10 CFR 50.49.)
4/2/98 566	VIO	IR 98-80 VIO 98-80-01	N	3-ENG	4A 4B 4C	Engineering did not establish adequate methods to ensure compliance with some Technical Specification requirements associated with the ADS. While the ADS functionality was not affected, an instance that resulted in a violation of Technical Specification was identified. The instance involved the failure, on October 6, 1996, to perform the required High Drywell Pressure Channel Check in accordance with TS 4.1.1, to ensure the ADS was operable during the reactor pressure vessel test as required by Technical Specification 3.4.B.1, ADS. (Violation of TS 3.4.B.1)
4/2/98 565	Negative	IR 98-80	N	3-ENG	4A 4B 5C	While initial conditions and the basis for some assumptions were not always obvious, calculations and safety analysis adequately supported implemented CSS and ESWS modifications. The team identified that some pertinent design information in the UFSAR required updating. The licensee was aware of this and already had efforts ongoing to update the UFSAR. However, with regard to the resolution of seismic deficiencies, there was untimely follow up and reporting to the NRC the status of SQUG outliers.

OYSTER CREEK PLANT ISSUES MATRIX

Date Rec #	Type	Source	ID	SFA	Codes	Item Description
4/2/98 564	Positive	IR 98-80	N	3-ENG	4A 4B 4C	Apart from EQ and available voltage issues, the ADS was maintained well. System modifications were properly implemented. The discharge piping vacuum breakers were adequately tested to the IST requirements and the acceptance criteria was consistent with the design basis. Instrumentation and control design for the ADS and pressure relief function was consistent with the design basis and licensing documents.
4/2/98 563	Positive	IR 98-80	N	3-ENG	3B 3A 4C	Overall, the quality of the 10 CFR 50.59 training program for certification of RTR and ISR was excellent. The contents of the training were appropriate and supported the OCNGS review process that is being implemented. Safety evaluations were prepared and reviewed by individuals who had received training regarding the preparation and review of SEs in accordance with the facility procedure.
4/2/98 562	EEI	IR 98-80 EEI 98-80-02	N	3-ENG	4A 4B 4C	Engineering failed to establish adequate design control measures to verify or check the adequacy of design voltage required for the ADS EMRV solenoid valves as required by 10 CFR 50, Appendix B, criterion III, Design Control. (Apparent violation of 10CFR 50, Appendix B, Criterion III.)
4/2/98 561	EEI	IR 98-80 EEI 98-80-04	N	3-ENG	4A 4B 4C	Failure to maintain ADS operable as of March 19, 1998, as required by Technical Specification 3.4.B.1, ADS. (Apparent violation of TS 3.4.B.1.)
4/2/98 560	VIO	IR 98-80 VIO 98-80-05	N	3-ENG	3A 4C	Engineering failed to comply with their procedure for Safety Evaluations on several occasions as required by 10 CFR 50, Appendix B, Criterion V, Instructions, Procedures, and Drawings. (This is a Violation of 10 CFR 50, Appendix B, Criterion V.)
4/2/98 559	VIO	IR 98-80 VIO 98-80-07	N	3-ENG	3A 4A 4C	Engineering failed to submit the required changes, test, and experiments reports in 1983, 1986, and 1998 as required by 10 CFR 50.59(b) and 10 CFR 50.71(e). (This is a violation of 10 CFR 50.59(b) and 50.71(e).)
4/2/98 558	Positive	IR 98-80	N	3-ENG	4A 3B 3A	10 CFR 50.59 Safety Evaluations (SEs) were of good quality and performed in accordance with the requirements of 10 CFR 50.59 and the applicable procedures by qualified and certified personnel. However, some SEs exhibited a lack of thoroughness and attention to detail that was expected by the facility procedures.
4/2/98 557	Positive	IR 98-80	N	3-ENG	4C	In the 10 CFR 50.59 program area, procedures were found to be comprehensive and detailed in providing guidance and assigning responsibility for implementing the requirements of 10 CFR 50.59 and updating the UFSAR. The procedures were also up to date in incorporating revised industry and NRC 10 CFR 50.59 guidance and GPUN self assessment findings.

OYSTER CREEK PLANT ISSUES MATRIX

Date Rec #	Type	Source	ID	SFA	Codes	Item Description
4/2/98 556	VIO	IR 98-80 VIO 98-80-05	N	3-ENG	4A 4B 4C	Engineering failed to verify design calculations that supported the seismic adequacy of safety related equipment as required by procedure 10 CFR 50, Appendix B, Criterion V, Instructions, Procedures, and Drawings. (This is a violation of 10CFR 50, Appendix B, Criteron V.)
4/2/98 555	VIO	IR 98-80 VIO 98-80-06	N	3-ENG	4A 4C 5C	Engineering failed to adequately correct a seismic deficiency with a containment spray heat exchanger as of February 26, 1998, as required by 10 CFR 50, Appendix B, Criterion XVI, Corrective Actions. (This is a violation of 10 CFR 50, Appendix B, Criterion XVI.)
8/26/98 626	Negative	IR 98-05	N	4-PS	3A	The annual review of the radiation protection program was adequate. Weakness in the scope and depth of the summary self-assessment for 1997 was noted; however, the sum total of self-assessments and safety assessments met the requirements of 10 CFR 20.1101.
8/26/98 625	Positive	IR 98-05	N	4-PS	3B	The presentation of continuing training to radiation protection technicians was appropriate with regard to scope and depth of presentation for two classes reviewed during the inspection period.
8/26/98 624	Positive	IR 98-05	N	4-PS	3A	Appropriate controls for high, locked high and very high radiation areas were established. Control of radiological work for the shipment of a radwaste liner, receipt of radioactive materials (new fuel), and spent fuel pool work were also effective.
8/26/98 623	Negative	IR 98-05	N	4-PS	4C	The licensee established an adequate program for the sampling, analysis and assessment of personnel exposure to airborne radionuclides. A programmatic weakness was identified regarding the lack of reverification of the basis documents for this area since 1993. (IFI 98-05-03)
8/26/98 622	Positive	IR 98-05	N	4-PS	3A	Radiological controls effectively supported refueling activities. In addition, management provided a good oversight.
7/17/98 607	Positive	IR 98-03	N	4-PS	3A	The licensee met their notification requirements in response to the turbine generator exciter fire on August 1, 1997.
7/17/98 606	Positive	IR 98-03	N	4-PS	3A	The response to the fire in the 'A' motor generator set, on June 27, 1997, was appropriate and the fire burned for less than ten minutes. The fire brigade response to the fire was enhanced by improvements to the fire brigade turnout area.
7/17/98 605	VIO	IR 98-03 VIO 98-03-04	N	4-PS	2A 5C	The licensee failed to maintain adequate communications capability for fire fighting and safe shutdown activities during the turbine generator exciter fire on August 1, 1997. (Violation of Fire Protection License Condition 2.C.3)
7/17/98 604	Negitive	IR 98-03	N	4-PS	3B	Operations displayed a weakness with regard to the manual operation of fire suppression systems. The specific cause of the main generator exciter fire was unrelated to the other fire events during the period.

OYSTER CREEK PLANT ISSUES MATRIX

Date Rec #	Type	Source	I/D	SFA	Codes	Item Description
7/17/98 603	VIO	IR 98-03 VIO 98-03-03b	N	4-PS	3A	The licensee failed to provide adequate fire watches when the fixed CO_2 fire protection system was disabled. (Violation of Fire Protection License Condition 2.C.3)
7/17/98 602	VIO	IR 98-03 VIO 98-03-03a	N	4-PS	3A	A fire watch failed to conduct an adequate watch turnover. (Violation of Fire Protection License Condition 2.C.3)
7/17/98 601	Positive	IR 98-03	N	4-PS	3A	The fire brigade response to a fire drill scenario met the drill objectives and the fire brigade leader executed effective command and control of the brigade.
6/3/98 583	VIO	IR 98-02 VIO 98-02-03	N	4-PS	1C	The licensee was conducting its security and safeguards activities in a manner that protected public health and safety. Security personnel maintained an effective security program and management competently administered the program. The licensee met their commitments and NRC requirements, with the exception of the performance of the required background checks of the Fitness-For-Duty (FFD) personnel. Security personnel failed to perform background checks on the personnel administering the FFD program at least once every 3 years, as required by 10 CFR 26. (Violation of 10 CFR 26, Appendix A, Section 2.3(2))
6/3/98 582	Negative	IR 98-02	N	4-PS	5C	The licensee's pursuit of corrective actions, related to identified EP problems, is inconsistent, as evidenced by the aggressive pursuit of corrective actions for some identified problems and slow pursuit of corrective actions for other equally significant problems.
6/3/98 581	MISC	IR 98-02	N	4-PS	2B	Despite some minor discrepancies in the surveillance procedures, the major licensee emergency response facilities were adequately maintained. The existence of discrepancies in two surveillance procedures warrants additional oversight by the emergency preparedness (EP) organization of the surveillance activities performed by other licensee organizations.

ABBREVIATIONS USED IN PIM TABLE

ADS	Automatic Depressurization System
ALARA	As Low As Reasonably Achieveable
AOG	Augumented Off Gas
CSS	Containment Spray System
DVR	Deviation Report
EMRV	Electromatic Relief Valve
EQ	Environmental Qualifications
ESWS	Emergency Servic e Water System
GPU	General Public Utilities
IR	Inspection Report
ISR	Independent Safety Review
IST	In-Service Testing
MDRG	Multi-displinary Review Group
NRW	New Rad Waste
NSA	Nuclear Safety Assessment
OCNGS	Oyster Creek Nuclear Generating Station
RMS	Radiation Monitoring System
RTR	Responsible Technical Reviewer
SEs	Safety Evaluations
TS	Techncial Specification
UFSAR	Updated Final Safety Analysis Report

GENERAL DESCRIPTION OF PIM TABLE COLUMNS

Date	The actual date of an event or significant issue for those items that have a clear date of occurrence (mainly **LERs**), the date the source of the information was issued (such as for **EALs**), or the last date of the inspection period (for **IRs**).
Type	The categorization of the item or finding - see the Type / Findings Type Code table, below.
Source	The document that describes the findings: **LER** for Licensee Event Reports, **EAL** for Enforcement Action Letters, or **IR** for NRC Inspection Reports.
ID	Identification of who discovered issue: **N** for NRC; **L** for Licensee; or **S** for Self Identifying (events).
SFA	SALP Functional Area Codes: **OPS** for Operations; **MAINT** for Maintenance; **ENG** for Engineering; and **PS** for Plant Support.
Code	Template Code - see table below.
Item Description	Details of NRC findings on LERs that have safety significance (as stated in IRs), findings described in IR Executive Summaries, and amplifying information contained in EALs.

TYPE / FINDINGS CODES

ED	Enforcement Discretion - No Civil Penalty
Strength	Overall Strong Licensee Performance
Weakness	Overall Weak Licensee Performance
EEI *	Escalated Enforcement Item - Waiting Final NRC Action
VIO	Violation Level I, II, III, or IV
NCV	Non-Cited Violation
DEV	Deviation from Licensee Commitment to NRC
Positive	Individual Good Inspection Finding
Negative	Individual Poor Inspection Finding
LER	Licensee Event Report to the NRC
URI **	Unresolved Item from Inspection Report
Licensing	Licensing Issue from NRR
MISC	Miscellaneous - Emergency Preparedness Finding (EP), Declared Emergency, Nonconformance Issue, etc. The type of all MISC findings are to be put in the Item Description column.

TEMPLATE CODES

1	Operational Performance: **A** - Normal Operations; **B** - Operations During Transients; and **C** - Programs and Processes
2	Material Condition: **A** - Equipment Condition or **B** - Programs and Processes
3	Human Performance: **A** - Work Performance; **B** - Knowledge, Skills, and Abilities / Training; **C** - Work Environment
4	Engineering/Design: **A** - Design; **B** - Engineering Support; **C** - Programs and Processes
5	Problem Identification and Resolution: **A** - Identification; **B** - Analysis; and **C** - Resolution

NOTES:

* EEIs are apparent violations of NRC requirements that are being considered for escalated enforcement action in accordance with the "General Statement of Policy and Procedure for NRC Enforcement Action" (Enforcement Policy), NUREG-1600. However, the NRC has not reached its final enforcement decision on the issues identified by the EEIs and the PIM entries may be modified when the final decisions are made. Before the NRC makes its enforcement decision, the licensee will be provided with an opportunity to either (1) respond to the apparent violation or (2) request a predecisional enforcement conference.

** URIs are unresolved items about which more information is required to determine whether the issue in question is an acceptable item, a deviation, a nonconformance, or a violation. However, the NRC has not reached its final conclusions on the issues, and the PIM entries may be modified when the final conclusions are made.

Enclosure 2

OYSTER CREEK INSPECTION PLAN FOR DECEMBER 1998 THROUGH MAY 1999

Inspection No.	Program Area/Title	Planned Dates	No. Inspectors	Type
IP 40501	Licensee Self-Assessment Related to Area-of-Emphasis Inspections	12/14/1998	3	RI
IP 83728	Maintaining Occupational Exposures ALARA	1/25/1999	1	RI
IP 84750	Radioactive Waste Treatment, and Effluent and Environmental Monitoring - Environmental	2/1/1999	1	Core
IP 62706	Maintenance Rule Inspection Procedure - Follow-up	2/1/1999	1	OA
IP 86750	Solid Radwaste Management and Transportation of Radioactive Material	05/10/99	1	Core

Legend:

- IP - Inspection Procedure Number
- TI - Temporary Instruction Program / Sequence Number
- Core - Minimum NRC Inspection Program (mandatory at all plants)
- OA - Other Inspection Activity
- RI - Additional Inspection Effort Planned by Region I
- SI - Safety Initiative Inspection