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The Northeast Utilities System

December 17, 1997

Docket No. 50-423
B16873

Re: 10 CFR 50.73(a)(2)(i)(B)

Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555


Millstone Nuclear Power Station Unit 3
Licensee Event Report 96-034-02
Submitted Pursuant to 10 CFR 50.73(a)(2)(i)(B)

This letter forwards Supplement 2 to Licensee Event Report 96-034-00, documenting a condition that was initially determined reportable at Millstone Unit No. 3 on September 18, 1996. This LER is submitted pursuant to 10 CFR 50.73(a)(2)(i)(B). NNECO's commitments in response to this event are contained within Attachment 1 to this letter.

Should you have any questions regarding this submittal, please contact Mr. David A. Smith at (860) 437-5840.

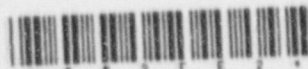
Very truly yours,

NORTHEAST NUCLEAR ENERGY
COMPANY


G. D. Hicks

Unit Director, Millstone Unit No. 3

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U.S. Nuclear Regulatory Commission
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Attachment: 1) NNECO's commitments in response to LER 96-034-02
2) LER 96-034-02

cc: H. J. Miller, Region I Administrator
A. C. Cerne, Senior Resident Inspector, Millstone Unit No. 3
J. W. Andersen, NRC Project Manager, Millstone Unit No. 3
W. D. Travers, Dr., Director, Special Projects
Corrective Action Group Files (CR M3-97-2864, CR M3-96-0812)

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Attachment 1

Millstone Nuclear Power Station, Unit No. 3
NNECO's Commitments
In Response To
(LER 96-034-02)

December 17, 1997

Enclosure
List of Regulatory Commitments

The following table identifies those actions committed to by NNECO in this document. Please notify the Manager - Nuclear Licensing at the Millstone Nuclear Power Station Unit No. 3 of any questions regarding this document or any associated regulatory commitments.

Number	Commitment	Due
115939-01	As a result of the September 18, 1996 event, a Technical Specifications Change was implemented on July 10, 1997 that revised the setpoint value identified in Technical Specifications 3.4.9.3.a.2 and 3.4.9.3.a.3 from the current value of 450 psig to ≥ 426.8 psig and ≤ 453.2 psig.	Complete
B16777-02	The setpoint calculation will be revised to properly incorporate the RHR pump suction relief valve temperature correction for the full temperature range.	Prior to entry into Mode 4
B16777-03	An evaluation of the RHR relief valve setpoint effect on the RHR piping stress analyses will be completed.	Prior to entry into Mode 4

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Attachment 2

Millstone Nuclear Power Station, Unit No. 3
NNECO's Submittal of
(LER 96-034-02)

December 17, 1997