

NOTICE OF VIOLATION

Northern States Power Company
Prairie Island Nuclear Generating Plant

Docket Nos. 50-282; 50-306
License Nos. DPR-42; DPR-60

During an NRC inspection conducted on February 3 through 13, 1997 and February 20, 1997, a violation of NRC requirements was identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," NUREG 1600, the violation is listed below:

Technical Specification 4.2.A.1 requires that the "inservice inspection of ASME Code Class 1, Class 2, and Class 3 components be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55(g), except where relief has been granted by the Commission pursuant to 10 CFR 50.55(g)(6)(i)."

ASME Code, Section XI, 1980 Edition Winter 1981 Addenda, IWB-2500(a) requires "Components shall be examined and tested as specified in Table IWB-2500-1," IWC-2500(a) required "Components shall be examined and pressure tested as specified in Table IWC-2500-1" and IWF-2500 required "Components supports subject to examination shall be examined and tested as specified in Table IWF-2500-1."

10 CFR 50.55a(g)(5)(iii) requires "If the licensee has determined that a Code requirement is impractical for its facility, the licensee shall notify the commission and submit, as specified in 50.4, information to support the determinations."

10 CFR 50.55a(g)(5)(iv) requires "Where an examination requirement by the Code or addenda is determined to be impractical by the licensee and is not included in the revised inservice inspection program as permitted by paragraph (g)(4) of this section, the basis for this determination must be demonstrated to the satisfaction of the Commission no later than 12 months after the expiration of the initial 120-month period of operation and each subsequent 120-month period of operation during which the examination is determined to be impractical."

10 CFR 50.55a(g)(6)(i) states "The Commission will evaluate determinations under paragraph (g)(5) of this section that code requirements are impractical. The Commission may grant such relief and may impose such alternative requirements as it determines is authorized"

Contrary to these requirements:

- a. On February 10, 1997, the inspector identified seven welds in each unit (listed in report section M4.1) for which Code volumetric examination requirements had not been met and for which authorized Code relief had not been obtained pursuant to 10 CFR 50.55a(g)(6)(i) and Technical Specification 4.2.A.1 requirements.

- b. On February 12, 1997, the inspector identified 73 examinations affecting both units (identified in Attachment A as a "Limited exam") for which Code examination requirements had not been met and for which authorized Code relief had not been obtained pursuant to 10 CFR 50.55a(g)(6)(i) and Technical Specification 4.2.A.1 requirements.

This is a Severity Level IV violation (Supplement 1).

Pursuant to the provisions of 10 CFR 2.201, Commonwealth Edison is hereby required to submit a written statement of explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington D.C. 20555 with a copy to the Regional Administrator, Region III, and a copy to the NRC Resident Inspector at the facility that is the subject of this Notice, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved. Your response may reference or include previous docketed correspondence, if the correspondence adequately addresses the required response. If an adequate reply is not received within the time specified in this Notice, an order or a Demand for Information may be issued as to why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time. Because your response will be placed in the NRC Public Document Room (PDR), to the extent possible, it should not include any personal privacy, proprietary, or safeguards information so that it can be placed in the PDR without redaction. However, if you find it necessary to include such information, you should clearly indicate the specific information that you desire not to be placed in the PDR, and provide the legal basis to support your request for withholding the information from the public.

Dated at Lisle, Illinois,
this 14th day of March 1997