

APPENDIX

U. S. NUCLEAR REGULATORY COMMISSION
REGION IV

NRC Inspection Report: 50-458/85-70

License: NPF-40

Docket: 50-458

Licensee: Gulf States Utilities (GSU)
P.O. Box 2951
Beaumont, Texas 77704

Facility Name: River Bend Station (RBS)

Inspection At: River Bend Station, St. Francisville, Louisiana

Inspection Conducted: October 2-3, 1985

Inspector:

H. Chaney
H. Chaney, Radiation Specialist
Facilities Radiological Protection Section

10/21/85
Date

Approved:

B. Murray
B. Murray, Chief, Facilities Radiological
Protection Section

10/21/85
Date

Inspection Summary

Inspection Conducted October 2-3, 1985 (Report 50-458/85-70)

Areas Inspected: Special, announced inspection of the licensee's actions to resolve Operating License (NPF-40) precriticality conditions involving; (1) plant air flow characteristics, (2) testing of category 1 and 2 air cleaning systems, (3) testing and calibration of certain chemistry sample panels, and (4) actions to improve the integrity of the Post Accident Sampling System (PASS) sample lines. The inspection involved 11 inspector-hours onsite by one NRC inspector.

Results: Within the areas inspected, no violations or deviations were identified.

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DETAILS

1. Persons Contacted

GSU

- *J. Deddens, Vice-President, River Bend Nuclear Group
- *D. Gipson, Assistant Plant Manager
- *P. Freehill, Superintendent Startup and Test
- *J. Hamilton, Projects Supervisor
- *T. Crouse, Manager, Quality Assurance (QA)
- *P. Tomlinson, Director, Operations QA
- *G. Kimmell, Supervisor, Operations QA
- *J. Spivey, Operations QA Engineer
- *B. Hey, Licensing Engineer
- *S. Desai, Chemical Engineer
- *T. Anthony, Senior Mechanical Engineer
- *E. Cargill, Supervisor, Radiological Programs
- C. Nash, Supervisor, Chemistry
- R. Coppo, Senior Projects Engineer
- R. King, Licensing Engineer
- E. Grant, Supervisor, Nuclear Licensing
- D. Zenel, Mechanical Maintenance Supervisor
- D. Speeg, Foreman, Instruments and Controls

Others

- *D. Chamberlain, NRC Senior Resident Inspector
- *B. Dunn, Startup Engineer, Consultant
- J. Morgan, Startup Engineer, Consultant

*Denoted those present during the exit interview.

The NRC inspector also contacted other licensee and contractor employees during the inspection.

2. Licensee Actions on NPF-40, Attachment 1 License Conditions

Item c: "Verify that the plant's air flow characteristics are from areas of potentially low radioactive concentrations to areas of potentially higher radioactive concentrations."

This item has been reviewed in previous NRC inspection reports (50-458/84-06, 85-05, 85-35, and 85-53). The NRC inspector determined that the licensee had nearly completed air flow balancing in all radiological and nonradiological areas, except for the main control room envelope, in accordance with preoperational test (PT) procedure PT-400-2. The test package associated with procedure PT-400-2 is expected to be submitted to the RBS Joint Test Group on or about October 7, 1985, for final review.

The NRC inspector noted that procedure PT-400-2 established test criteria that agree with the commitments contained in the Final Safety Analysis Report (FSAR) chapters 6.5, 9.4, 12.3. Final close out of this licensing condition will be accomplished by the NRC resident inspector after review of the data contained in PT-400-2.

Item d: "Place in service as tested and calibrated, including installed process instrumentation, the following panels: turbine plant sampling, condensate demineralization, and radwaste sampling."

This item has been discussed previously in NRC Inspection Reports 50-458/85-47 and 85-85. The NRC inspector determined that the licensee had performed preventive maintenance on nearly all the required instrumentation on the turbine sampling panels and on all the required instrumentation on the condensate (panel 72) and the radwaste (panel 130) sample panels in accordance with plant maintenance procedures. The licensee had developed contingency procedures for the manual sampling/analysis of influents and effluents serviced by these remaining instruments on the turbine panel. Maintenance Work Requests for these systems have been assigned top priority for corrective action. This item is considered closed for criticality; however, the NRC resident inspector will track to completion the remaining turbine panel instrument qualifications/calibrations.

Item f: "Completion of HVAC preoperational testing and loading of activated charcoal as committed to in GSU's letter RGB-21603 of July 22, 1985. This includes in-place testing of HEPA and charcoal filters and laboratory testing of charcoal for Iodine removal efficiency."

This item has been previously discussed in NRC Inspection Reports 50-458/84-14, 85-46, and 85-53. The NRC inspector determined that the licensee had loaded and successfully tested all category 1 and 2 filtration/absorber units per the criteria of Special Situation Test (SST) procedures 1-SST-17 and 1-SST-18. The licensee's test results agree with the commitments contained in the FSAR, chapters 6.4, 9.4, 12.3, and 14.2.12.1.70. Laboratory testing and in-place tests were found to satisfy the surveillance requirements of Technical Specification 4.7.2 for the main control room HVAC, 4.6.5.4 for the standby gas treatment system, and 4.6.5.6 for the fuel building ventilation system. This item is considered closed as a condition for criticality.

3. Licensee Actions on Previously Identified Inspection Findings

(Open) Open Item (458/8422-05): Post Accident Sampling System - This item has been previously discussed in NRC Inspection Reports 50-458/84-06, 84-22, 85-17, 85-35, 85-47, and 85-53. The NRC inspector discussed the NRC's concerns regarding the large number of mechanical joints associated

with the PASS sample lines. The licensee submitted a schedule for the replacement or seal welding/brazing of all uncontained mechanical joints on the PASS sample lines adjacent to and in the PASS area on the 114 foot level of the auxiliary building. The licensee plans to complete all modifications to the sample lines before reaching 100 effective full-power days of operation. This maintenance activity will be pursued during routine power ascension testing through the five percent power plateau. Since the mechanical joints would not adversely impact on radiological conditions until the reactor core accumulates a significant inventory of radionuclides. The licensee's corrective actions are considered acceptable. The licensee indicated that the remaining NRC concerns on PASS, as referenced in the above reports, would be completed prior to exceeding five percent power. This item will remain open.

(Closed) Open Item (458/8414-03): Liquid Radioactive Waste System - This item was previously discussed in NRC Inspection Reports 50-458/84-14, 84-33, 85-46, and 85-64. The NRC inspector noted that the licensee intends to have two filter/demineralizer trains on line for processing radioactive liquids prior to exceeding five percent power. Currently, the licensee only has one processing train available, but does have contingency plans, contracts and procedures, for the use of a backup vendor provided liquid processing service, if the need arises. Currently, preoperational tests have been completed on train "A" (1-SST-44) and are nearing completion (scheduled completion on or about October 7, 1985) for train "B" (1-SST-45). The licensee committed at the exit interview to have both "A" and "B" trains of the liquid radioactive waste processing system operational prior to exceeding five percent power. This item is considered closed and will be tracked by the NRC resident inspector under license NPF-40 conditions for exceeding five percent power.

4. Exit Interview

The NRC inspector met with the licensee's representatives and the NRC resident inspector identified in paragraph 1 of this report at the conclusion of the inspection on October 3, 1985. The NRC inspector summarized the scope and results of the inspection and discussed the closing of NPF-40 licensing conditions and licensee commitments to the NRC regarding the PASS and liquid radioactive liquid waste processing filter/demineralizer trains.