

REGION I PLANT STATUS REPORT

FACILITY: Salem Nuclear Generating Station Units 1 and 2

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	Last Update: March 22, 1994
Update	Approval: Approval:
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CHANGES SINCE THE LAST UPDATE ARE DEMARCATED IN THE BORDER

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ATTACHMENT A

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I. BACKGROUND

1. LICENSEE PARAMETERS

Utility: Company Location:

County:

Public Service Electric & Gas Company (PSE&G) Hancocks Bridge, NJ (18 miles Southeast of Wilmington, DE) Salem

Docket No: CP Issued: Operating License Issued: Initial Criticality: Elec. Ener. 1st Gener: Commercial Operation: Reactor Type: Containment Type: Power Level: Architect/Engineer: NSSS Vendor: Constructor: Turbine Supplier:

Condenser Cooling Method: Condenser Cooling Water:

2. NRC ORGANIZATION

NRC Regional Administrator: (Region I, King of Prussia, PA)

Division of Reactor Projects: (Region I) 50-272 September 25, 1968

UNIT 1

April 6, 1977 December 11, 1976 December 19, 1976 June 30, 1977 PWR 4-Loop Large dry 3411 MWt PSE&G/UE&C Westinghouse PSE&G/UE&C Westinghouse

Once-through Delaware River 50-311 September 25, 1968 May 19, 1981

UNIT 2

August 2, 1980 May 29, 1981 October 13, 1981 Same Same Same Same Same Same Westinghouse (GE Generator) Same Same

Thomas T. Martin (Tel: 610-337-5000)

Richard Cooper, Jr., Division Director (Tel: 8-610-337-5229) Wayne Lanning, Deputy Director (Tel: 8-610-337-5126) Edward C. Wenzinger, Branch Chief (Tel: 8-610-337-5225) John R. White, Section Chief (Tel: 8-610-337-5114)

NRC ORGANIZATION Continued:

Senior	Resident Inspector:	Charles S. Marschall (Tel: 8-609-935-3850)
	Resident Inspector:	Stephen T. Barr (Tel: 8-609-935-3850)
	Resident Inspector:	Joseph G. Schoppy, Jr. (Tel: 8-609-935-3850)
	Resident Inspector:	Todd H. Fish (Tel: 8-609-935-3850)
	Project Engineer:	Robert J. Summers (Tel: 8-610-337-5189)
	Project Manager:	James C. Stone, NRR (Tel: 8-301-504-1419)

3. LICENSEE ORGANIZATION

Management Personnel:

E James Ferland	-Chairman and Chief Executive Officer
Lawrence R. Codey	-President and Chief Operating Officer
Pohert I Dougherty	-Senior Vice President, Electric
Steven E Miltenherger	-Vice President and Chief Nuclear Officer
Steven L. Millenberger	Vice President, Nuclear Engineering
Joseph Hagan	-Vice President Operations and General Manager Salem Operations
Richard N. Swanson	-General Manager, Quality Assurance and Nuclear Safety Review
Lynn K. Miller	-General Manager, Nuclear Operations Support
Francis X. Thomson	-Licensing Manager
Lee Catalfomo	-Operations Manager
Nichael P. Morroni	-Manager, Maintenance-Controls
Arthur Orticelle	-Manager, Maintenance-Mechanical
John W Morrison	-Technical Manager
Terry L. Cellmer	-Radiation Protection/Chemistry Manager
Richard T. Griffith, Sr.	-Station OA Manager
G Charles Munzenmaier	-Manager, Salem Station Planning
Peter Moeller	-Manager, Site Protection
Gree Mecchi	-Manager, Nuclear Training
Christopher Connor	-General Manager, Nuclear Support and Services

Workshifts

5 operations shifts, 2 working 12 hour shifts/day, 1 relief crew, 1 crew in training, 1 crew off.

Shift Complement:	TS minimum	Actual
	3 SRO 4 RO 1 STA	4 SRO 5 RO 1 STA (dual role SRO)
Non-licensed Operators	5	7 or 8

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Maintenance Electrician/I&C	1	2
Chemistry/Rad, Prot.	1	2
Fire Brigade	5	6 (site fire brigade shared with Hope
		Creek)

4. OPERATOR LICENSING

a. Licensed Reactor Operators (Licenses Cover Both Units):

Total number of active SROs:	29
Total number of active ROs:	26
Total number of certified instructors:	13

 In June 1993, NRC performed TI 117, "Licensed Operator Requalification Program Evaluation"; results were satisfactory.

One simulator (modeled after Unit 2) located at the training facility in Salem, NJ, and used for Unit 1 and Unit 2 operator training and NRC administered licensing exams. PSE&G completed a major modeling upgrade package in the summer of 1993.

b. Other Licensed Operator Training / Performance / Staffing Concerns:

Shift Supervisors began working 12 hour shifts during refuel outages conducted in the spring and summer of 1992, formally implementing that schedule in November 1992. The remainder of the shift complement maintained 8 hour shifts until April 1992, when, upca a union vote, they also adopted the 12 hour shifts for a 1 year trial basis. The reactor operators and equipment operators will be voting again in April 1993 as whether to permanently stay on 12 hour shifts.

II. PLANT PERFORMANCE DATA

1. CURRENT OPERATING STATUS (for period 10/1/93 to 3/1/94)

PSE&G shut down Unit 1 on October 1, 1993, to commence a 72 day refueling and maintenance outage. Prior to the shutdown, the unit had been on Refuel line since July 15, 1993, and operating at or near full power. Plant management extended the outage completion date (originally scheduled for December 17) because of emergency diesel generator (EDG) operability concerns. On December 2, 1993, a cracked cylinder liner in a Unit 2 EDG raised generic operability concerns for Unit 1 No. 1B EDG because of the similar liners installed in No. 1B. Operators restarted the unit on January 24; it automatically tripped from 100% power, on January 27 in response to a low water level condition in No. 14 steam generator. Operators restarted the unit on January 31, and operated the unit at power until it automatically tripped, from 100% power, in response to a loss of control power to the main turbine control system. PSE&G restarted the unit February 13, synchronized to the grid February 20, and has operated the unit at or near power through the end of the month.

PSE&G operated Unit 2 at or near full power throughout the fall, until December 3, 1993, when operators shut down the unit due to failure of a cylinder liner in the 2C EDG. After completion of repairs to the EDG, operators restarted the unit on January 3, 1994, and operated at full power until January 19, when the reactor engineering staff discovered that PSE&G had apparently operated Unit 2 in excess of 3411 megawatts (thermal). Since then, and through February, operators have maintained Unit 2 at 95% power.

2. RECENT SIGNIFICANT OPERATING EVENTS AND IDENTIFIED SAFETY CONCERNS

a. Significant Events (of last 12 months)

- Unit 1 automatically tripped on February 10, 1994, from 99% power, in response to a loss of 15 VDC power to the main turbine control system. The plant stabilized at normal operating pressure and temperature. PSE&G determined that the 15 VDC power supplies had tripped when their protective relays sensed an over-voltage condition. (See IR 50-272/94-01)
- Unit 1 automatically tripped on January 27, 1994, from 10% power, in response to a low water level condition in No. 14 steam generator. The cause of the trip was a level error controller in the control circuit for No. 14 steam generator feedwater regulating valve, which caused generator water level control to malfunction in the auto position. This malfunction generated the low water level condition and subsequent reactor trip. (See IR 50-272/94-01)

19/93 01 outing

> 1/44 RT Low S6 Low Constraile Maltune 2450 RT

Loss Contract Pue TO ETK

4/40 RT GRMSS ASTIN N

- Operators shut down Unit 2 on December 3, 1993, from 100% power, due to failure of a cylinder liner in 2C emergency diesel generator (EDG). PSE&G conservatively determined they had a basis for concern about the particular liner's reliability and consequently declared Unit 1 EDG 1B inoperable as well, since the 1B diesel had similar liners installed. (See IR 50-311/93-27)
- On November 2, 1993, operators declared an Unusual Event (UE) in response to a fire in a 230 volt lighting transformer in the Unit 2 turbine building. The fire brigade responded to the scene and extinguished the fire. The station was in the UE for approximately one hour. A loose electrical connection caused the fire. No personnel were injured and no safety-related equipment was affected. (See Ik 50-311/93-23)
- On October 13, 1993, operators declared an UE in response to a fire in the Unit 1 No. 12 service water piping penetration bay. The shift supervisor notified PSE&G Fire Department, which responded to the scene and extinguished the fire. The station was in the UE for about 50 minutes. The fire was caused by sparks from a grinding activity, which ignited insulation from service water piping. Three contractor employees were treated for smoke inhalation; no equipment sustained damage. (See IR 50-272/93-21)
- On August 24, 1993, operators initiated a Technical Specification-required shutdown of Unit 1 in response to a degraded voltage on a cell in the 1C 125 volt battery. The need to shut down was relieved when the NRC exercised enforcement discretion in response to the licensee's request and associated justification. (See IR 50-272/93-20)
- On July 11, 1993, while the repairs to a faulty Unit 1 feedwater isolation protection relay were being performed, the main feedwater regulating valve for the No. 14 steam generator inadvertently went closed at 8:38 p.m., resulting in the water level in that steam generator dropping to a level sufficient to cause an automatic reactor trip. The licensee determined that the technician who was repairing the SSPS relay lifted an improper lead and caused the isolation of the No. 14 steam generator. The licensee additionally determined the root cause of the technician's error was inadequate detail and direction in the SSPS troubleshooting plan. Subsequent to the cause determination of the trip, PSE&G repaired the SSPS and commenced a reactor startup on July 15, 1993. The unit was returned to service on July 16, 1993. (See IR 50-272/93-19)
- On July 10, 1993, toxic gas release (ammonia) in the Unit 1 turbine building caused by a loop seal failure on the ammonia hydroxide storage tank due to overpressure. This apparently resulted from excessive ambient temperature conditions. The licensee will change the concentration of the ammonia hydroxide in the tank to increase the boiling point of the solution to prevent recurrence. (See IR 50-272/93-19)

- On June 8, 1993, Unit 1 automatically tripped following massive intrusion of sea-grass into the circulating water system suction. Four of five operating circulating water pumps tripped, causing a loss of main condenser vacuum, turbine trip, and subsequent reactor trip. (See IR 50-272/93-19)
- On May 28, 1993, Unit 2 was manually tripped by the operators per abnormal operating procedures when control bank "C", group 1 control rods (four rods total) fell into the core during reactor start up operations. At the time the operators were diluting the RCS to criticality for post-refueling startup. A card failure was attributed to a degraded solder trace in the rod control system, which led to the event. (See IR 50-311/93-81)
- On March 16, 1993, Unit 2 automatically tripped from 100% power due to a low-low level condition on the No. 24 steam generator. A failed pressure control switch in the condensate polishing system led to a low suction pressure condition for the No. 22 steam generator feed pump and subsequent feed pump trip, which caused the steam generator low level reactor trip. (See IR 50-311/93-08)
- b. Performance Indicator Data

Units 1 and Unit 2:

- Performance indicators generally show good performance. Capacity factor numbers were low for 1993 due to back-to-back outages of Unit 1 and Unit 2 and shutdowns for potentially generic safety issues such as rod control and diesel generator cylinder liners. No other significant trends are evident in the statistical analysis.
- c. <u>Recently Identified Technical Safety and Managerial Challenges</u> (of last 12 months)
- The NRC Resident Office continues to monitor and evaluate the licensee's efforts to improve plant material condition, repair and replace service water piping, upgrade the RMS system, complete actions relative to Appendix R requirements, issues associated with fire watches and security guards, personnel error reduction efforts, and procedure quality and compliance improvement efforts.
- Reviews were conducted and are planned for erosion/corrosion program.
- Service Water (SW) Lecks: Numerous SW through wall leaks continue to occur due to erosion and microbiologic induced corrosion attack of carbon steel piping. The licensee has a seven year pipe replacement project that will replace 95% (about 19,000 linear feet are safety related) of the safety related SW piping with 6% moly stainless steel. This project will continue through 1995 (two more refueling outages per unit). Currently, approximately 90% of

the safety related portion of the project has been completed, including the majority of the SW piping in containment. Based on NRC inspection, SW pipe replacement project is progressing satisfactorily as scheduled.

- Radiation Monitoring System (RMS) Problems: RMS problems have resulted in numerous ESF actuations and reportable events. Short term corrective actions were completed on both Unit 2 and Unit 1 during the 1992 refueling outages. These changes include electronic upgrades and a new uninterruptible power supply. Longer term actions (1993-4) include a complete system upgrade. Based on NRC inspection, the upgraded RMS operation to date has been satisfactory.
- Failure of Overhead Annunciators: On December 13, 1992, a Unit 2 operator discovered that the overhead annunciators had not been updating alarms for about 1 1/2 hours. This was the result of a member of the operating shift entering a keystroke combination into a remote control workstation that, when input through the wrong system port, prevented the system from updating alarms. An AIT was dispatched to the site and concluded: (1) the root cause was a failure to follow procedure for proper operation of the overhead annunciator system; (2) the design of the OHA system permitted the operator to inadvertently emulate the password-protected software without warning.
- Rod Control System: On May 27, 1993 Unit 2 operators experienced several problems with the rod control system. The most significant event was that during an attempt to insert Shutdown Bank "A", one control rod ac ually withdrew 15 steps of travel. An AIT was dispatched to the site and concluded: (1) the root cause was an introduction of static charges into the solid state electronic components which caused system damage; (2) damage was also caused by voltage spikes originating from "back EMr" in the system's electro-mechanical step counters (the suppression diode installed to mitigate this previously-known phenomenon was disabled due to a failed pin connector on the affected circuit card).

At 5:12 p.m. on July 18, 1993, Salem Unit 2 Control Bank D (8 control rods) began stepping inward at a rate of 72 steps per minute, but only moved a few steps before being detected by operators. At the time, Unit 2 was at 100% power with the control rods in automatic. The operator, finding no apparent cause for the rod insertion, positioned the rods in manual control, which stopped the rod movement. The operators performed all actions per their abnormal rod movement procedure (AB-ROD-0003) and were still unable to positively identify the cause. The licensee installed monitoring instrumentation on the inputs to the automatic rod control signal summator and at 11:40 p.m. on July 18, returned rod control to automatic.

At 11:24 a.m. on July 21, 1993, the licensee again experienced the same phenomenon on Unit 2. As in the previous occurrence, the operator quickly evaluated the situation and appropriately placed the rods in manual control. In both cases the rods only moved inward a few steps (2 and 4 steps respectively). Current traces on the signal summator input revealed no change from the nuclear instrument (NI) or turbine impulse pressure, but some spiking from the average temperature (Tave) and reference temperature (T ref) input. Together these four signals are the input signals to the automatic rod control system. On July 21, the licensee placed additional monitoring instrumentation on the output of the signal summator, output of the "rod in output" signal comparator, and individually on all four Tave channels.

On July 22, 1993, during I&C troubleshooting, the licensee was able to identify a fault in the signal summator, which erroneously produced a high rod inward demand output for a relatively small temperature error input.

- Switchyard Modifications: During the recent outage on Unit 1, PSE&G implemented an extensive design change package involving modifications to the Salem switchyard. These modifications increased voltage recovery on vital and group buses during bus transfers, provided load growth capacity, removed the Salem circulating water system pump motor feeds from the Hope Creek switchyard, improved voltages in both Salem plants, provided margin for short circuit capability, and improved plant reliability. Major components added included two 500/13.8 kv transformers, four 13.8/4.16 kv transformers, four 13.8 kv breakers, and 4.16 kv switchgear for the circulating water system bus.
- Unit 2 Sustained Operation of Greater Than 100% Power: Suspected root cause is erosion of the feedwater flow nozzles resulting in incorrect online calorimetric data. Upon discovery, licensee immediately reduced power for both units, and began adjusting instrument setpoints to insure conservative operation. Licensee is pursuing determination of the exact power level and the effects on the UFSAR Chapter XV analyses. They expect resolution by mid-April 1994.
- Emergency Diesel Generator Cylinder Liner: This caused Salem 2 to shut down as a result of a cracked liner, and delayed Salem 1 to delay startup from the refueling outage. The licensee could not find a clear root cause. The suspected root cause was dimensional tolerance problems with liners distributed by Canadian Allied Diesels. PSE&G determined that only two liners have ever failed, including the Salem liner, in a population of tens of thousands of liners in use world wide (including locomotives and ships).

3. ESCALATED ENFORCEMENT ACTIVITIES

 The NRC issued a Level III Violation on March 8, 1994, documented in NRC Inspection Report 50-272 and 311/93-23; 50-354/93-25. The violation was based on multiple examples of PSE&G's failure to follow procedures and their failure to properly control safety-related activities.

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4. IPE INSIGHTS

 The Salem IPE was submitted to the NRC in July 1993, and is still under NRC review.

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III. ANALYSIS/ASSESSMENT

1. PREVIOUS SALP RATINGS AND OVERVIEW

a. Previous SALP Ratings

Functional Area	December 28, 1991	June 19, 1993
Operations	2	2
Maintenance/ Surveillance	2	2
Radcon Emergency Prepar Security	2, Imp edness 1 1	1 1, Declinin 1
SA/QV	2	2
Engineering & TS	2	2

Current assessment period: June 20, 1993 to December 10, 1994.

b. SALP Overview (derived from the summary paragraph of each SALP section):

OPERATIONS

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On July 29, 1993, the SALP board met to discuss PSE&G's performance at Salem during the period from December 29, 1991 to June 19, 1993. The board concluded that the licensee had operated the Salem units safely and that operator response to operational events was excellent. The overall performance in the Operations area was good. However, weaknesses were noted in the decisions to restart Unit 2 following the rod control system problems, in the failure to follow procedures resulting in the loss of Unit 2 annunciators, and in the inadequate oversight of the fire protection program.

MAINTENANCE/SURVEILLANCE

The board concluded that the Salem maintenance and surveillance programs contributed to the safe operation of the two units during the assessment period. In general, a declining number of personnel errors in both maintenance and surveillance indicated improving performance. However, the number of transients induced by component failures and the significant problems with the rod control system raise questions regarding the overall effectiveness of the maintenance and engineering support functions.

RADIOLOGICAL CONTROLS

PSE&G continued to implement effective radiological controls and ALARA programs during this period. The SALP board noted improvements in this functional area including strong management support and oversight. Quality Assurance audits in this area were of very good quality.

EMERGENCY PREPAREDNESS

The SALP board determined that PSE&G maintained a generally strong and effective emergency preparedness (EP) program. However, the board was concerned with an apparent decline in the ability of the licensee to make correct initial Protective Action Recommendations during training, drills and annual exercises. This concern resulted in the board's assessment of a declining trend for this area. The board also concluded that PSE&G continued to maintain an effective and performance-oriented security program during this period. Overall, license^o performance in both EP and security remained excellent.

ENGINEERING AND TECHNICAL SUPPORT

Engineering and technical support organizations provided good support for refueling and maintenance outages, and strong performance in addressing day-to-day problems. The SALP board noted that training programs for engineering personnel were excellent but that weaknesses were observed in the licensee's non-conformance, erosion/corrosion, and fire protection programs. Although the root cause training program was viewed as a strength, the board noted that the threshold for initiating actual root cause investigation was not clear or consistent.

PSE&G management continued to provide generally effective management support. Significant Event Response Team (SERT) reviews of major events have been effective. However, the board noted that in several instances, PSE&G failed to initiate adequate root cause evaluation or assessment of abnormal conditions. NRC interaction with PSE&G management was needed in a number of cases in order for full evaluation and corrective action to be taken in a timely manner. Once initiated, comprehensive assessment, root cause analysis and effective corrective actions were implemented. Outage planning and training programs in all areas were considered strengths.

2. LICENSEE RESPONSE TO PREVIOUS SALP FUNCTIONAL AREA WEAKNESSES / RECENT LICENSEE PERFORMANCE TRENDS (in the last year)

OPERATIONS

PSE&G continues to safely operate the units. Operator plant knowledge and response to events remains strong, however, operator response has been less than thorough regarding indications of stuck-open RHR check valves, indications of a possible leaking RHR pressure isolation valve, and a case of indeterminate hotwell level.

Recent management changes included the naming of a new Operations Manager in September 1993, and two new Operations Engineers in January 1994. The licensee intends to pursue full unitization of the Salem operating crew shifts.

MAINTENANCE AND SURVEILLANCE

Although maintenance and surveillance activities remain generally good, as exhibited by strong Maintenance Department performance in response to the December 1993 EDG cracked cylinder liner issue, the recent Unit 1 refueling outage was marked by multiple examples of poor work control practices and multiple examples of failure to follow procedures.

In order to improve overall performance and response to emergent issues, PSE&G has reorganized the Maintenance Department. Recent changes include replacing the single Maintenance Manager role with three new positions: 1) Mechanical Maintenance Manager, 2) Controls Maintenance Manager, and 3) Planning Manager. PSE&G is also pursuing unitization in these departments.

ENGINEERING AND TECHNICAL SUPPORT

Both Salem system engineering and PSE&G nuclear engineering have continued to provide good engineering support for plant operations.

An NRC observation related to the Salem rod control issue was that the initial troubleshooting efforts lacked clear leadership and delegation of responsibilities. This resulted in the efforts narrowly focusing on the most recent system malfunction without adequate attention to the repetitive nature of the failures and the need to determine and correct the root cause. The failure of PSE&G to determine the root cause of the failures resulted in numerous aborted startup attempts. The team did observe significant improvements in the control of troubleshooting and root cause determination during the inspection.

PLANT SUPPORT

The NRC noted that PSE&G continued to perform at a noteworthy level in the area of radiological protection through the end of 1993, especially during the recent Unit 1 refueling outage.

The licensee's annual partial-participation emergency preparedness exercise was conducted on June 23, 1993. On-site response to the simulated emergency was very good. An exercise strength was Emergency Response Manager command and control. No exercise weaknesses were identified. Significant areas for potential improvement were maintenance team tracking from the Operational Support Center and public address system operability in the Technical Support Center.

The PSE&G security program continues to be effectively directed towards public health and safety. A strike by the security force was narrowly averted when a new labor agreement was reached in November 1993.

SAFETY ASSESSMENT/QUALITY VERIFICATION

In July 1993, the licensee formed a Comprehensive Performance Assessment team (CPAT) which conducted a special assessment of safety issues and recent plant events using an integrated MORT investigatory analysis. The CPAT developed comprehensive root causes for these events, and the licensee has formed task teams charged with developing corrective actions. PSE&G has held periodic meetings with the NRC to discuss CPAT findings, and the NRC continues to monitor licensee progress in this area.

In February 1994, PSE&G Vice President of Nuclear Operation (VP-NO) assumed the collateral role of General Manager of Salem Operations. The licensee also initiated other management changes under the VP-NO and intends to pursue unitization of the Salem units. PSE&G has implemented these changes in order to achieve sustained improvement in the area of Salem performance.

3. LICENSEE PERFORMANCE STRENGTHS AND WEAKNESSES *

Salem performance continues to be inconsistent.

 Capacity factor has been low due to refueling outages at both units and forced outages due to rod control problems, and diesel liner concerns.

Strengths:

 The licensee continues to increase resources for a material condition improvement program. The NRC has observed noticeable improvement in the material condition of the plant, indicating that the licensee has been earnest in the implementation of improvements.

- The Procedure Upgrade Project (PUP) was closed out in September 1993. A large
 majority of procedures were reviewed and upgraded, and procedure maintenance has
 been made the responsibility of the Technical Department.
- Material condition
- Procedure quality
- Radiation protection program implementation
- When problems or conditions are self-identified and self-detected, event response and root cause determination are through and comprehensive, particularly when the matter is the subject of NRC attention. In other cases, the licensee's performance is considered weaker, as identified below.
- PSE&G has responded to identified performance and management weaknesses relative to approch to problem resolution by initiating the following actions:
- Replacing the Salem General Manager with the Vice President, Nuclear Operations until the licensee's program changes are in place;
- Verifying the effectiveness of numerous supervisors and managers and changing the incumbent when deemed appropriate
- · Pursuing unitization of the maintenance, operations, and planning organizations,
- Implementing the existing performance assessment tools to improve accountability from the highest levels of management down to rank and file workers,
- Forming dedicated teams to implement the corrective actions developed in response to the CPAT findings.

Weaknesses:

Salem performance has been weak in:

- Control of maintenance
- Recognition of the need to due root cause determination,
- · Corrective action effectiveness due to inadequate root cause assessment
- Inadequate approach to problem resolution (i.e., general tendency to fix problems or conditions without assessment or understanting of causal factors. Examples include, but are not limited to the licensee's initial response to cracked diesel liner issues, failure to identify elevated reactor power in 1992, and failure to recognize generic implication of rod control problems

4. NRC TEAM INSPECTIONS WITHIN THE LAST YEAR

Area/Date

EDSFI Assessment August 16 -September 3, 1993

Appendix R Inspection

May 17-21, 1993

Augmented Inspection Team (AIT) June 5 - July 2, 1993 Findings

Licensee-contracted EDSFI has been completed. The NRC assessment of the licensee EDSFI identified a number of minor concerns; but, concluded overall that the licensee's assessment was good.

An AIT was formed to review and evaluate the circumstances surrounding a problem with the Unit 2 rod control system. The components within the control circuitry that led to rod withdrawal when operators were demanding rod insertion.

Identified concerns with Kaowool and 3-M fire wrap material. Also weaknesses in safe shutdown outside the control room and lighting. Re-evaluation to occur during July 1993.

5. PLANNED TEAM INSPECTIONS

SWSOPI

Date and scope to be determined.

DET/OSTI/IPAT?? (Does this team exist yet?)

IV. INSPECTION PROGRAM STATUS

1. STATUS OF INSPECTIONS

The inspection program status is reflected in attached MIPS report #2. The data is current as of the date of the MIP. The MIP indicates that inspection program is ontrack with the planned resource allotment; no significant shift in inspection activities is warranted.

2. PROPOSED CHANGES TO MIP

- Unit 1
- A. DRSS -
- B. DRS -
- C. DRP
- Unit 2
- A. DRSS -
- B. DRS -
- C. DRP -

3. SIGNIFICANT ALLEGATIONS AND INVESTIGATIONS

 There are eight open significant allegations at Salem. (two are common with Hope Creek)

Three allegations are related to harassment and intimidation of licensee personnel, up to and including allegations of promotion denial due to "whistleblowing." One of the allegations asserts that the Offsite Safety Review Group is not performing its function in accordance with technical specifications. OI is actively reviewing these cases.

A fourth allegation asserted that the main security access center at the Salem/Hope Creek site was not manned as required by the NRC approved security plan. DRSS is scheduled to conduct a routine security inspection in March 1994 and will review this matter.

The fifth allegation concerns an operator wrongdoing issue. During and subsequent to the Overhead Annunciator (OHA) AIT in early 1993, neither of the two operators in the control room at the time of the incident admitted to any manipulation of the OHA system, even though clearly operator involvement was a contributor to the event. DRP is reviewing the licensee's investigation and followup into this matter and will determine this issue's resolution on the basis of that review.

The sixth allegation involves a technical question that suggests that HVAC ductwork integrity may not be assured under dynamic loading of new fast-acting curtain fire dampers. DRP is reviewing test procedures and results while DRS is scheduled to review the matter during the next routine fire protection inspection.

The seventh allegation regards evidence that the Rod Control problems experienced by the plant (and followed up by the AIT) occurred during startup testing at the Zion nuclear station, even though Westinghouse representatives denied that the problem had ever occurred before. OI has opened an investigation into this case and is currently reviewing the matter.

The final allegation concerns 6 technical issues raised regarding the environmental qualification of equipment. Upon agreement of the alleger, this matter will be referred to the licensee for resolution. Otherwise, DRS will followup it up.

4. OPEN ITEM STATUS

BACKLOG/No. GREATER THAN 2 YRS

(Unit 1 and 2 - Common) 57/6

NOTE: The large number of open items is due to the issuance of an Appendix R/Fire Protection Team Inspection Report in October 1993 and an EDSFI Team Inspection Report in November 1993.

5. OUTSTANDING LICENSING ISSUES

- GL 89-10 (MOV) technical differences between NRC/PSE&G. (Hope Creek also)
- · EDG amendment meeting held May 11, 1992 to resolve issues.
- TS amendment to resolve AFW/containment spray issue (see Section II.2.a).
- Increase in surveillance test intervals and AOT for reactor trip and ESFAS.
- Install new digital feedwater control system.
- Evaluation of Control Room Design Deficiencies that were not corrected.

 Bulletin 88-08 (Thermal Stess in Piping Systems Connected to the RCS) - licensee is revising their response.

6. LOCAL/STATE/EXTERNAL ISSUES

a. NJ DEPE/BNE

- Now providing input/comments on all PSE&G licensing change requests.
- Letter regarding Salem RMS (see Section II.2.a).
- Provided comments on recent SALP report.
- High interest in resident inspection accompaniment.
- Continuing interest in Salem cooling tower issue: When Salem's renewable variance for the use of the Delaware River as a heat sink came up for renewal in 1984, New Jersey environmentalists appealed to the state to not renew the variance. In 1990, NJ DEPE issued a "draft order" requiring PSE&G to build two cooling towers to support the Salem units' operation. PSE&G responded to the state's order with a 56-volume comment, and the issue is currently under review by NJ DEPE. Recent NJ DEPE decision not to require cooling towers.
- State inspector accompanied AITs that reviewed Salem 2 loss of OHA system and RCS.
- Recent letter (6/29/93) concerning digital feedwater modifications to be performed the next two refueling outages.
- b. Other (Media Interest)
- Minimal interest in SALP Management Meeting.
- Large interest in AIT (Unit 2 TG failure) exit meeting.
- Smaller interest in two AITs (Unit 2 Loss of Alarms and rod control problems) exit meeting.



NRC VISIT

MAY 25, 1994





SALEM GENERATING STATION NRC VISIT AGENDA

Introduction

Strategy for Improvement

Comprehensive Performance Assessment

Communications

Unitization

Improved Oversight

Measures of Success

COMPREHENSIVE PERFORMANCE ASSESSMENT TEAM (CPAT)

Charter Highlights

- Full-time multi-disciplinary, dedicated team
- Report directly to Vice President and Chief Nuclear Officer
- Assess a defined set of 27 occurrences
- Look for previously undiscovered, underestimated, or overlooked root causes, failed barriers, and contributing/causal factors
- Look for "threads" common to multiple occurrences
- Identify responsibility for correcting the root causes, restoring the failed barriers, or eliminating the causal factors
- Act as change agents

COMPREHENSIVE PERFORMANCE ASSESSMENT TEAM

Members

Dana Cooley, Manager - Quality Performance Tom DiGuiseppi, Emergency Preparedness Manager E.J. Galbraith, Chemistry Engineer - Salem John Wilson, Nuclear Engineering Consultant - E&PB Charles Manero, System Engineer - Salem Technical Greg Mecchi, Principal Nuclear Trainer - Operations Roberta Kankus, Senior Strategic Planning Specialist (PECO Energy Co.) Craig Assimos, Nuclear Technical - Controls Special -

Salem

Ron Sutton, Career Pathing Administrator - Human Resources

Steven Spiese, Certified NRRPT Radiation Protection Technologist - Hope Creek

Bruce Little, Former NRC Senior Resident Inspector/DOE Certified Accident Investigator

Judy Almond, Senior Secretary - Site Services

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COMPREHENSIVE PERFORMANCE ASSESSMENT TEAM

- Senior Project Oversight Group
 - Monthly report from Assessment Team
 - Purpose

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- Satisfy group that review was thorough and appropriate
- Ensure both short and long term buy-in from Senior Management
- Provide impetus for timely action
- Share experience with nuclear plant change management
- Counsel Senior Managers and Assessment, Team
- ▲ Foster external credibility

COMPREHENSIVE PERFORMANCE ASSESSMENT TEAM

Senior Project Oversight Group Membership

S.E. Miltenberger, Vice President and Chief Nuclear Officer

J.J. Hagan, Vice President - Nuclear Operations

S LaBruna, Vice President - Nuclear Engineering

M.V. Butz, General Manager - Nuclear Human Resources & Administration

R.N. Swanson - General Manager - QA/Nuclear Safety Review

S.P. Cohen, Director - Nuclear Finance

R.A. Burricelli, General Manager - Information Systems and External Affairs

G. Rainey, Vice President - PECO Energy Company

J. Cross, Senior Vice President - Portland General Electric Company

J.S. Carroll, Professor-Sloan School of Management -MIT

M. Peifer, Institute of Nuclear Power Operations

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COMPREHENSIVE PERFORMANCE ASSESSMENT TEAM PROBLEM STATEMENT CATEGORIES

MANAGEMENT PHILOSOPHY, SKILLS AND PRACTICES

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- M-1 Supervisory practices that properly support professionals who make decisions and perform work.
- M-2 Management risk assessment and prioritization.
- M-3 Management actions and establishment of accountability.
- M-4 Content and delivery of management training to effectively support individuals and groups.
- M-5 Self-assessment processes

PEOPLE PERFORMING THE WORK

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- W-1 Access to timely and accurate technical information versus reliance on interpersonal contacts.
- W-2 Effective use of work planning and schedules.
- W-3 Process work-arounds versus ownership and continuing improvement.
- W-4 Timely and accurate part information and availability with appropriate levels of end-user intervention.
- W-5 Content and delivery of technical training to effectively support individuals and groups.

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W-6 Standards and methods of contractor performance.

PROBLEM SOLVING AND FOLLOW UP

- S-1 Root cause determination.
- S-2 Corrective action follow through.
- S-3 Performance trending for systems and equipment important to reliability and operational control. Action upon results.
- S-4 Operating Experience Feedback (OEF) delivery and tracking that meets the job needs of recipients for information.

Nuclear Department Tactical Plan - CPAT M-1/M-3

Key Focus Items Supported: Supervisory Practices that Support Professionals Who Make Decisions and Perform Work (CPAT M-1)

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Sponsor: Vice President - Nuclear Opertaions

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Activity	Sponsor	Support	Start	Stop
 Develop and implement a supervisory monitorin, program STATUS: Program was developed during 1st quarter 1993, and 10 documented in SL-40. Improving implementation is an ongoing process. SL-40 Program is currently being reviewed to increase effectiveness. 	GM-SO GM-NHR&AS	Sta. Mgrs	1/93	12/95
 Significantly improve two-way communications <u>STATUS:</u> 1. Letter issued by VP-NO in January stating expectations that supervisor spend 16 hours/week in the field. 2. Observation training given by VP-NO & GM-SO to all Salem employees at department engineer level and above during 1994. Being rolled down through organization. 		Mgr-Nuc Comm.		
 Provide 360 degree feedback to Salem supervision <u>STATUS:</u> Ongoing. As of 5/12, 107 1st line supervisors and above have begun the process with 72 having received feedback. 				

	Activity	Sponsor	Support	Start	Stop
Reso signil Expe of sig	olve long-standing equipment deficiencies and reduce number of ficant events, to eliminate chronic drain on resources and morale. ected results are a reduction in the number of events and the elimination gnificant events.				
STA is on two 1994 1st c	TUS: Resolving long standing equipment deficiencies from ngoing. The Salem OEF meeting has determined that Salem events were "notable" during the 1st quarter of 4. This compares to ten "notable" events during the quarter of 1993.				
Esta and	ablish effective vehicles for responding to station workers' issues, concerns I productivity recommendations.				
STA NOI	TUS: This item is being met on an ongoing basis by T teams. NOIT's have been in place since 9/93.				
impi	rove personnel accountability and ownership relative to:				
• F • C • S	procedure compliance compliance to work standards self verification schedule adherence				
STA acro issu thro	ATUS: 1) Salem transitioned to the Star Self-Checking onym during 1/94. 2) Updated work standards handbook led during 3/94. 3) Above being continually reinforced ough supervisory monitoring program.				

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Activity	Sponsor	Support	Start	Stop
evelop Descriptive Supervisor Behavior Model (CPAT M1)	VP-NO	GM-NS&S	5	
Describe/reinforce model at spring supervisors dialogue.			2/94	3/94
STATUS: complete				
Follow up at fall supervisors dialogue			9/94	9/94
All managers/supervisors to spend at least 16 hours/week of their time in the field.		All Mgrs/Supvr	1/94	12/95
STATUS: Reported data indicates that slightly greater than				
16 hours/week is being achieved.				
mprove the Performance Appraisal Process (CPAT M3)	VP-NO	All Mgrs	1/94	12/94
Managers to review existing performance appraisals for all employees three levels down in their organizations to insure the appraisals accurately reflect individual performance		All Mgrs	1/94	4/94
STATUS: Process underway throughout the Nuclear Department.				
Managers prepare and deliver new performance appraisals to employees, as required, to ensure a current performance appraisal (within 12 months) exists for all employees that meets standards (accurate reflection of individual performance).		All Mgrs	1/94	4/94
STATUS: Process underway throughout the Nuclear Department.				

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		Nuclear Department Tactical Plan - CPAT M-1/M-3 (cont'd)				
Activity	Sponsor	Support	Start	Stop		
Continue to reinforce performance appraisal expectations at manager dialogues			1/94	12/94		
STATUS: Ongoing	Componention		1/94	4/94		
Review/revise guidance, policies, and rating definitions for performance appraisals	Compensation					
STATUS: One to three months behind schedule, revised schedule to January 1995 for implementation. Based on inabiliity to present information during January & February at Managers Dialogue - due to weather conditions.						
	Continue to reinforce performance appraisal expectations at manager dialogues STATUS: Ongoing Review/revise guidance, policies, and rating definitions for performance appraisals STATUS: One to three months behind schedule, revised schedule to January 1995 for implementation. Based on inabiliity to present information during January & February at Managers Dialogue - due to weather conditions.	Continue to reinforce performance appraisal expectations at manager dialogues STATUS: Ongoing Review/revise guidance, policies, and rating definitions for performance appraisals STATUS: One to three months behind schedule, revised schedule to January 1995 for implementation. Based on inability to present information during January & February at Managers Dialogue - due to weather conditions.	Continue to reinforce performance appraisal expectations at manager dialogues STATUS: Ongoing Review/revise guidance, policies, and rating definitions for performance appraisals STATUS: One to three months behind schedule, revised schedule to January 1995 for implementation. Based on inability to present information during January & February at Managers Dialogue - due to weather conditions.	Continue to reinforce performance appraisal expectations at manager dialogues STATUS: Ongoing Review/revise guidance, policies, and rating definitions for performance appraisals Compensation 1/94 STATUS: One to three months behind schedule, revised schedule to January 1995 for implementation. Based on inability to present information during January & February at Managers Dialogue - due to weather conditions. Compensation 1/94		

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BLD Content Applicability Nuclear Dept





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Work Practices & Standards Performance Line Management Monitoring Salem Station



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Supervisory Face-to-Face Time Salem Station



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Nuclear Department Tactical Plan - CPAT M-2

Key Focus Item Supported: Management Risk Assessment & Prioritization (CPAT M-2)

Sponsor: Vice President - Nuclear Engineering

Activity	Sponsor	Support	Start	Stop
Risk Assessment				
Select M2 team and meet with CPAT representatives to establish clarity of issues and brainstorm actions.	VP-NE	M2 Team	1/94	
STATUS: Complete.				
Establish draft framework and plan to address risk	VP-NE	MAP Team/	11/93	
STATUS: Complete.	(MAP Team)	FINI-IF		
Obtain acceptance of framework and plan from VP-NE/VP-NO and E&PB peer group (approximately 6 separate sessions).	PM-IP PMA	Peer Group	1/94	
STATUS: Complete.			1.2.1	
Revise framework plan based on acceptance dialogue sessions	PM-IP		1/94	
STATUS: Complete.	PMA	184 S. S. S.	1.0	
Obtain acceptance of framework and plan from E&PB and station department managers (THEY Bashers population).	VP-NE VP-NO	PM-IP PMA	3/1/94	
STATUS: Complete.	A Second			
Investigate the development of risk assessment policy to embrace key components	VP-NE	M2 Team	3/1/94	6/15/94
STATUS: Working.	VP-NO			

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Activity	Sponsor	Support	Start	Stop
Design communication plan through 1994	VP-NO	M2 Team/Nuc Comm Mgr	4/1/94	6/15/94
STATUS: Working.				
Implement Communication plan	VP-NE/VP-NO	RC Mgrs PMA/PM-IP	6/1/94	ongoin
Set up Socratic Dialogue. (method to demonstrate framework application by Nuclear Department Leadership) (Video session for roll down)	PM-IP/PMA	VP-NE VP-NO	3/1/94	7/1/94
STATUS: Working	F 18 1 18			
Design mini tool, thought process aid as handout	VP-NE VP-NO	PMA/PM-IP	4/1/94	6/1/94
STATUS: Working	NA SECON	1845243		
Roll out framework and Socratic Dialogue. Roll out per communication plan (use video as aid)	VP-NE VP-NO	PM-IP/PMA	7/1/94	9/1/94
Design measures through 1994 to assess what changes have occurred in prudent risk taking	VP-NE	M2 Team RC Mgrs	7/1/94	7/1/94
Prioritization			125	187
Select M2 team and meet with CPAT representatives to establish clarity of issues and brainstorming actions	VP-NE	M2 Team	1/94	
STATUS: Complete.		Provide Ser		1.55
Survey RC managers for work in department that can be a) stopped, b) given lower priority, c) emphasized less by management	VP-NE	RC Mgrs	2/8/94	
STATUS: Complete.		1.2.2.2.2.2.1		

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Activity Review NDRAP projects to reduce workload on ND	Ops/E&PB Mgrs	NSM Team	10/93	6/15/94
STATUS: Working. Resolve NDRAP inconsistencies and insure use of system STATUS: Working	VP-NE E&PB MAP team (work control)	VP-NE	10/93	6/1/94
Executive decision to cut work, reprioritize, de-emphasize based on survey results	VP-NE VP-NO VP-CNO	NSM Team	6/1/94	7/1/94
NDRAP process	Mgr-NED	TBD	6/1/94	
Perform collegial assessment on Horow pro- Design Communication Plan Implement Communication Plan	VP-NE VP-NE VP-NO	RC Mgrs	6/1/94 7/1/94	7/1/94 ongoing
Design measures to assess what changes have occurred regarding prioritization effectiveness	VP-NE	M2 Team	7/1/94	7/1/94

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Measures

M-2 Management Risk Assessment and Prioritization

MEASURES:

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- Corrective Maintenance Backlog
- Preventive Maintenance Backlog
- Engineering Work Requesis
- NDRAP
- Repetitive Equipment Problems (under development)
- DCP SORC Status Approval
- Total Human Performance Events
- Licensee Event Reports (Personnel Error)
- Work Practices and Standards Monitoring by QA

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Corrective Maintenance Backlog All Priorities

Salem Station



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Preventive Maintenance Overdue

Hope Creek Station



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Nuclear Dept Resource Allocation Process



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Work Practices & Standards Performance QA Monitoring

Salem Station



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Nuclear Department Tactical Plan - CPAT M-4

Key Focus Item Supported: Content and Delivery of Management Training to Effectively Support Individuals and Groups (CPAT M-4)

Sponsor: General Ma orger - Nuclear Human Resources and Administrative Services

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00011001	Support	Start	Stop
GM-NHR&AS	Mgrs	1/94	12/94
	GM-NHR&AS	GM-NHR&AS Mgrs	GM-NHR&AS Mgrs 1794

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Measures

M-4 Create environment to experience and develop new behavior based on new leaderhsip values

MEASURES:

- Business Leadership Development Content Applicability
- Total Human Performance Events
- Licensee Event Reports (Personnel Error)

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Nuclear Department Tactical Plan - CPAT M-5

Key Focus Item Supported: Management Self-Assessment Process (CPAT M-5)

Sponsor: General Manager - Quality Assurance/Nuclear Safety Review

-	Activity	Sponsor	Support	Start	Stop
Α.	Expectations for Self-Assessment and Corrective Action (Promulgate uniform understanding of Self Assessment and Corrective Action Process).				
	1. Presentation to SEM/Director Reports	GM-QA/NSR	S-2 Team	2/94	2/8/94
	STATUS: Completed 2/8/94.	144 25			
	1a. Updated Presentation	GM-QA/NSR	Mgr-QAP&A	5/94	6/94
	STATUS: Ready for presentation on 6/1/94.	1.1.1.1.1.1.1			
	2. Manager's Dialogue Presentation	1.00	HR Plan &	3/94	7/94
	STATUS: Awaiting Schedule Slot.		Coroning		
	3. Supervisorys Dialogue Breakout Groups: Self Assessment & Corrective Action	Mgr-QA P&A		8/94	9/94
	4. Revise Corrective Action Procedure (NAP 58)	Mgr-QAP&A		7/94	10/94
	5. Define Expectations at Ke Manager's staff meetings	Mgr-QAP&A	1.00	9/94	1/95
B.	Corrective Action Data Base Project	Principal Engr	QA Staff	1992	8/93
	1. Test Module in Procurement	Principal Engr			
COMPANY OF TAXABLE	STATUS: Completed 8/93.	QA Programs			
and the second se	2. DR, IR, DEF Inclusion	1946	Mgr-M&S	9/93	5/94
Contract of the local division of the local	STATUS: Implement 6/1/94.				

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Nuclear Department Tactical Plan - CPAT M-5 (cont'd)					
Sponsor	Support	Start	Stop		
Each Dept.	Mgr-M&S	5/94	7/94		
Mgr-QAP&A	Principal Engr QA Programs	12/94	4/95		
GM-QA/NSR	Mgr-QAP&A	5/94	7/94		
GM-QA/NSR	Mgr-QAPA&	9/94	quarterly		
Mgr-QAP&A	QA Staff	9/94	quarterly		
GM-QA/NSR	Mgr-QAP&A				
		1/94	3/94		
		3/94	6/94		
		7/94 10/94	10/94 10/94		
VPs & GMs	GM-QA/NSR		ongoing		
GM-QA/NSR	Mgr-QAP&A Station QA	3/94	12/94		
	Migrs.				
	CPAT M-5 (Sponsor Each Dept. Mgr-QAP&A GM-QA/NSR Mgr-QAP&A GM-QA/NSR VPs & GMs CM-QA/NSR	SponsorSupportEach Dept.Mgr-M&SMgr-QAP&APrincipal Engr QA ProgramsGM-QA/NSRMgr-QAP&AGM-QA/NSRMgr-QAPA&GM-QA/NSRMgr-QAPAAGM-QA/NSRMgr-QAP&AVPs & GMsGM-QA/NSRGM-QA/NSRGM-QA/NSRGM-QA/NSRGM-QA/NSRMgr-QAP&AMgr-QAP&AMgr-QAPAMgr-QAP&AMgr-QAPAMgr-QAP&AMgr-QAPAMgr-QAP&AMgr-QAPAMgr-QAP&AMgr-QA/NSRMgr-QAP&AMgr-QAPAMgr-QAP&AMgr-QAPAMgr-QAP&AMgr-QAPAMgr-QAP&AMgr-QAPAMgr-QAP&AMgr-QAPAMgr-QAP&AMgr-QAPAMgr-QAP&AMgr-QAPAMgr-QAP&AMgr-QAPAMgr-QAP&AMgr-QAPMgr-QAP&AMgr-QAPMgr-QAP&AMgr-QAPMgr-QAP&AMgr-QAPMgr-QAP	SupportStartSponsorSupportStartEach Dept.Mgr-M&S5/94Mgr-QAP&APrincipal Engr QA Programs12/94GM-QA/NSRMgr-QAP&A5/94GM-QA/NSRMgr-QAPA&9/94Mgr-QAP&AQA Staff9/94GM-QA/NSRMgr-QAP&A1/94GM-QA/NSRMgr-QAP&A1/94VPs & GMsGM-QA/NSR7/94VPs & GMsGM-QA/NSRMgr-QAP&AGM-QA/NSRMgr-QAP&A3/94Mgr-QAP&AGM-QA/NSR3/94		

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Measures

M-5 Measurement Self-Assessment Process

MEASURES:

- Total Human Performance Events
- Composites Safety Index Performance
- NRC Violations

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Salem Composite Safety Index Status as of 4/30/94 (Year-End Goal/YTD Actual)



Legend:

* Green - significantly exceeds goal

Green - achieves goal

Yellow - needs improvement Red - significant weakness

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Hope Creek Composite Safety Index (Year-End Goal/YTD Actual) Status as of 4/30/94



Legend:

* Green - significantly exceeds goal Green - achieves goal

Yellow - needs improvement Red - significant weakness

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Nuclear Department Tactical Pla	n · CPAT W	/-1		
Key Focus Item Supported: (CPAT W-1) Sponsor: Vice President - Nuclear Operations	ation versus reli	ance on interper	sonal con	tacts
Activity	Sponsor	Support	Start	Stop
 Re-enforce requirements for control of vendor information received from sources other than the TDRs or EDCC to all personnel. 	NEStd Mgr	CCG Supv.	4/92	6/94
 STATUS: Annual letter on vendor information control process to be issued by 6/1/94. Confirm adequacy of procedural guidance for control of vendor information received from sources other than the TDRs or EDCC. 	NEStd Mgr	CCG Supv. Mgr - NP&MM	1/94	2/94
 STATUS: Completed - (ref. STN-94-0111) Confirm Salem Revitalization Project is correctly processing vendor information received with shipments. 	NEStd Mgr	CCG Supv. Mgr - Sp Proj	2/94	2/94
 STATUS: Completed - (ref. STN-94-0111) Retrain stock handlars to the procedural requirements for processing of vendor information received with shipments. 	Mgr-NP&MM		2/94	2/94
STATUS: Open - awaiting verification				

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Activity	Sponsor	Support	Start	Stop
IMIS Maintenance				10.55
Confirm that no gaps exist in the DCP process to assure that database impacts are recognized & incorporated.	NEStd Mgr	CCG Supv.	2/94	6/94
STATUS: In process as part of BOM collegial self assessment.				
Complete BOM Validation Project (1994 Scope)	NEStd Mgr	CCG Supv.	1/94	12/94
STATUS: BOM collegial self assessment for assessing BOM Control and Validity in process.				
 Assess BOM Control Assess BOM Validity FDR/FDDI Impact Review (HC) Solenoid Valve Verification (Salem) Recommend future needs 				
Communicate MDF Resolution Status to Nuclear Department	NEStd Mgr	CCG Supv	2/94	7/94
STATUS: Communication plan under development. Roll-cut dependent on collegial self assessment results due 6/94.				
Communicate MMIS Control & "Get-Well" Process	NEStd Mgr	CCG Supv	2/94	7/94
STATUS: Communication plan under development. Roll-out dependent on collegial self assessment results due 6/94.				
Develop ASME parts/Component Specification Sheets	NEStd Mgr	NME Mgr NESci Mgr	5/94	8/94
 Screen established, never populated May be able to drive completion from CJP process 				
STATUS: Collegial self assessment initiated. Results will address this issue and are due 8/94.				

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Activity	Sponsor	Support	Start	Stop
Establish standard construction material lists and invertory levels Technical Standard (TS) under consideration	Mgr-NP&MM	NEStd Mgr NME Mgr NEE Mgr NESci Mgr	1/93	TBD
STATUS:		in Local angle		
Computer Hardware and Software Control	1.20			
 Support NC.NA-AP.ZZ-0036(Q) policies regarding the procurement of computer hardware and software 	Mgr-M&S	All ND Mgrs	2/94	ongoing
 STATUS: No activity started - may be deleted once NA-AP- 0036(D) requirements are clarified. See next item. Clarify NC.NA-AP.ZZ-0036(Q) requirements regarding the procurement and development of customized computer software and hardware 	Mgr-M&S	N/A	2/94	6/94
STATUS: Communication plan under development. Engineering Document Control and Distribution				
 Provide "Working Copies" from EDCC (DCPIT Task #30) 	Mgr - NED	CCG Supv	1/93	12/94
STATUS: In process				
 Provide "Working Copies" from DMS (DMS Project Scope) 	Mgr - M&S	DMS Proj. Mgr Disc	1/93	12/94
STATUS: In process.				
 Bank Changes to Drawings other than OWDs (DCPIT Task #32) - DUTT activity 	Mgr - NED	CCG Supv	1/93	7/94
STATUS: In process.				
 Post MCRs against DCP CDs and MDs (DCPIT Task #31) 	Mgr - NED	CCG Supv	1/93	12/94
STATUS: In process.			-	
Access scanning MCRs into DMS	Mgr - M&S	DMS Proj Mgr Disc	2/93	12/94
STATUS: In process.			1	

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Activity	Sponsor	Support	Start	Stop
Communicate CBD Development Plan/Status for Salem - original plan	Mgr - NED	CBD Proj Mgr S&A Supv	1/91	ongoing
STATUS: Communication plan under development.				
Communicate DMS Development Plan/Status - original plan	Mgr - M&S	DMS Proj Mgr Disc	1/93	6/95
STATUS: Communication plan under development.				
Communicate DCPIT Task #30, 31, 32 Implementation Plans	Mgr - NED	CCG Supv	2/93	ongoing
STATUS: Monthly reports generated through Mgr - NED				1005
Complete Salem Setpoint Project	NEE Mgr	Salem I&C Supv	6/91	12/95
STATUS: On schedule.			0.04	4/04
Communicate Current TDR/EDCC Services and responsibilities	Mgr - M&S	DCG Supv CCG Supv	2/94	4/94
STATUS: Communication plan under development.			4/04	12/04
Baseline Reference Documents" identified and input to DMS	Mgr - M&S	CCG Supv	1/94	12/34
STATUS: Assessment of which documents are considered "Baseline Reference Documents" in process.		DCG Supv		
Spare Parts/Components and Construction Component Availability				
 Communicate the inventory "Write-Off" strategy to the working levels of the impacted organizations 	Mgr - NP&MM	Mgr - P&MC Ping Mgr- Salem	2/94	5/94
STATUS: Communication plan under development.		Maint Mgr- Salem Mgr - NED		

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Activity	Sponsor	Support	Start	Stop
alem Maintenance Procedures	CPEILER			
Eliminate procedure revision backlog	Tech Mgr	Tech Staff	1/94	TBD
STATUS: Reducing backlog to meet planned maintenance needs.				
Improve turn-around time for procedure revisions	Tech Mgr	Tech Staff	1/95	TBD
STATUS: The long term goal is less than two month turnaround depending on priority.				
Develop "New/Old" procedure cross-reference for repetitive tasks	Tech Mgr	Tech Staff	1/94	TBD
STATUS: Cross reference index developed.				

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Document Management System Workstations Installed Nuclear Dept



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Document Management System Documents into System Nuclear Dept

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Document Management System Time Available Nuclear Dept

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MMIS Work-in-Progress Load Nuclear Dept



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Nuclear Department Tactical Plan - CPAT W-2

Key Focus Item Supported: Effective use of Work Planning & Scheduling (CPAT W-2)

Sponsor: Vice President - Nuclear Operations

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	Activity	Sponsor	Support	Start	Stop
	plement Recommendations of Corporate Maintenance Performance hancement Program				
•	Salem Phase I	GM-SO	Corp. Perf. Salem Staff	11/91	
	• Implement productivity enhancement recommendations STATUS: Recommendations approximately 2/3 implemented. PC based software to onsite manpower (supvr & worker) availability to work schedule currently being implemented. Stewart communications is on-site faciliation, installation and use.				
•	Share transferable practices from best-in-class plant visits			11/91	
	STATUS: Complete. Included in Phase 1 recommendation. Hope Creek	GM-HCO	Corp. Perf. HC Staff	6/92	12/94
	 Layout and implement assessment plan Continue work on 30 day outage initiative 			ongoing	9/95
COMPANY OF THE OWNER OF THE OWNER	STATUS: Ongoing				

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Activity	Sponsor	Support	Start	Stop
Nuclear Procurement and Material Management	Mgr-NP&MM	M&S S/HC-Maint Planning	7/92	
 Assess and optimize new warehouse productivity 				
STATUS: Complete				
Optimize utilization of WAMMS System				
 Performance indicators Management reporting Parts availability to clients 				
STATUS: Ongoing, performance indicator in place.				
 Implement corporate material management personnel development program 	GM-Mat Mgmt (Corporate)	Mgr-NP&MM MMSO (Corp)	3/94	ongoir
STATUS: Working - bargaining unit (3rd quarter 1994), Material Control (2nd half of 1994), Procurement (1st quarter 1994).				
Implement Work Packages	Sta Planning - Mgr Salem		1/94	ongoir
 Work package standards monitoring Realign customer focus with work departments 				
STATUS: Working				

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Activity	Sponsor	Support	Start	Stop
Activity • Work towards including: • Why is job being done • Tech Spec Action Statement • ALARA • Heat Stress • Plant Conditions • Work Impact • Contingency Plans • Tagging Requirements STATUS: Working plement Salem Unitization Plan STATUS: On scheduie. Imm a process improvement team to "Work Control" and implement activities. STATUS: Full time team assigned, process on schedule.	Sponsor	Support		

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Measures

W-2 Effective use of work planning and schedules

MEASURES:

- Corrective Maintenance Backlog and Aging
- Preventative Maintenance Overdue
- Schedule Achievement (non-outage)
- Outage Window Performance
 - Shutdown Rx Disassembly Drain RCS Midloop Core Reload Mode 5 Mode 5 to 4 Mode 4 to Unit Synchronization

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U1 NON-OUTAGE PREVENTIVE MAINT. TREND STATION DEPT'S ONLY (EXCEPT AS NOTED) EXCLUDES HISTORY, REJECT, RDYRT & RTCPT



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U1 MAINTENANCE W/O BACKLOG NON-OUTAGE CM/PL WORKORDERS STATION DEPARTMENTS ONLY



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U1 NON-OUTAGE CORRECTIVE MAINT. W/O'S SORTED BY DEPARTMENT/GROUPS/PRI A,B,1,2

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U2 MAINTENANCE W/O BACKLOG

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MAJOR WINDOW COMPARISON SHUTDOWN THRU RX HEAD ON STAND

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MAJOR WINDOW COMPARISON REACTOR DISASSEMBLY THRU CORE OFFLOAD

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MAJOR WINDOW COMPARISON DRAIN RCS

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MAJOR WINDOW COMPARISON MIDLOOP OPERATIONS

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MAJOR WINDOW COMPARISON CORE RELOAD

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MAJOR WINDOW COMPARISON COMPLETED CORE RELOAD TO MODE V



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MAJOR WINDOW COMPARISON MODE V TO IV

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MODE IV TO FINAL SYNC



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Nuclear Department Tactical Plan - CPAT W-3

Key Focus Item Supported: Process work-around versus ownership and continuing improvement (CPAT W-3)

Sponsor: Vice President - Nuclear Operations

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Activity	Sponsor	Support	Start	Stop
Combine NAP 1 (Nuclear Department Procedure System) and NAP 32 (Preparation, Review and Approval of Procedures) into one procedure	GM-HCO	W-3 Team	1/94	5/1.4
 STATUS: Completed 3/94 Review NAP 59 (10CFR50.59 Reviews and Safety Evaluations) for simplification, followed by the approximately 40 NAPs which are spons Nuclear Operations 	GM-HCO	W-3 Team	3/94	5/94
 STATUS: NAP 59 issued 5/17/94, others by 12/94 Further simplify NAPs by separating those procedures which directly in plant safety and are more subject to regulatory scrutiny from those whi not, permitting more simple processes for writing, reviewing, and imple administrative procedures 	mpact GM-HCO ich are menting	W-3 Team	3/94	12/94
STATUS: In progress Roll out lessons learned to RC Managers Evaluate transition of station procedures back to responsible departm	ents GM-HCO GM-HCO	W-3 Team W-3 Team	5/94 6/94	12/94 12/94
Identify and Correct Work-arounds				
 Evaluate work-arounds during field time and/or work monitoring 	VP-NO	GM-SO GM-HCO	2/94	ongoing
 STATUS: Working Include lessons learned in SL-40, SD-16, work standards handbook Review work continue process to eliminate potential for work-arounds 	GM-HCO Dir-Pl	W-3 Team Work Control PIT	11/94 2/94	6/95 12/94
Review development of software on LAN's	Mgr-M&S	Staff		

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Measures

W-3 Process work arounds versus ownership and continuing improvement

MEASURES:

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- Work Practices and Standards Monitoring by Line Management
- Work Practices and Standards Monitoring by QA
- Total Human Perrformance Events
- Licensee Event Reports (Personnel Error)

Work Practices & Standards Performance Line Management Monitoring Salem Station



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Nuclear Department Tactical Plan - CPAT W-4

Key Focus Item Supported: Timely and Accurate Part Information and Availability with Appropriate levels of end-user intervention (CPAT W-4)

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Sponsor: Vice President - Nuclear Operations

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Activity	Sponsor	Support	Start	Stop
Formalize Training Program (Codes, Standards, Hardware, Forklift, Safety	Mgr-NP&MM	Mgr-NTC	1/94	ongoing
Procedures				
 Continuing training MMIS, PDIS, VPNO, AP-18, AP-19, AP-09, AP-38 		Mgr-NTC	1/94	ongoing
Conduct training needs survey (W4, Bullet 6)		GM-NHR&AS	4/94	7/94
STATUS: Work started - formed NP&MM Team & J. Samson is doing client survcy. (5/24/94) J. Samson is still developing client network for WAMMS and APPO Training Issues/Concerns.				
Implement Recommendations of Process Improvement Team on Obsolete	Mgr-NP&MM		11/93	6/94
Spare Parts (W4, Bullet 2, 8)	12555	Contraction of the	1.0	
STATUS: Complete implementation - procurement engineering is tracking status of PEs & OSPs. (5/24/94) NP&MM/Per are actively developing tracking process/client feedback and status	1.5			
tool.	100.00184			
Align Nuclear Procurement & Material Management with Corporate	Mgr-NP&MM		1/94	12/95
Organization Structures				
STATUS: New NP&MM organization announced on 5/16/94.		1.440.577.0		
Now implementing.	1.2.5	GM-NHR&AS	1/94	4/94
		Mgr-P&MC	2/94	7/94
Conduct organization				
errorus, Working, expect implementation May 1994.				
STATUS: Working - expect implementation				, e

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Nuclear Department Tactical Plan - CPAT W-4 (cont'd)						
	Activity	Sponsor	Support	Start	Stop	
 Monitor NP&MM effectiven performance indicators Conduct client survey & im 4, 5, 7, 10, 14) 	ess by client perception/service and specific plement client feedback mechanism (W4, Bullets 3,		Mgr-P&MC S/HC/E&PB Planning	3/94	7/94	
STATUS: Per Salem for parts >90 days of survey completed. F evaluating comments	Maint. Planning - report on CM/PLS hold d w/priority 1, 2, A or B. (5/24/94) Client eedback of results to NP&MM 5/1/94 (still s).	Mar-NP&MM	Depts.	1/94	ongoing	
Conduct periodic self asse STATUS: Specific s	trategy working - not yet finalized.	ling. the second				
Implement On-Line Purchas	se Requisitions with APPO (W4, Bullet 14)	Mgr-NP&MM	Mgr-M&S	1/94	12/94	
 Develop Nuclear specific p Implement on-line System P3/B3 Services P2/B2 Inventory P1/B1 Direct Charge Mate 	programming		Dir-NF Corp IS			
STATUS: Current - & working. (5/24/94) Phase II Project Initia Business Partners.	P1, P2, P3 and B3, needs B1, B2, Material Phase I APPO implemented 3/28/94. ation approved/funded by Corporate Scheduling still being developed for 94/95.					
Reduce Current Inventory L	_evel					
 Absorb inflation/supplier p Obtain funding (including) 	rice increases co-owner approval) to support five year reduction plan	Mgr-NP&MM	Dir-Nuc Fin NEStd Mar	1994 6/93	Ongoing 1996	
<u>STATUS:</u> 1994 goa (5/24/94) New year-e =\$110.3M.	I \$107m - current level (3/31/94) \$111.5m. and 1994 goal is =\$102.5M 5/23/94 value					

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Activity	Sponsor	Support	Start	Stop
 Implement corporate material management process improvements 		GM-MM	1/94	12/95
 Common coding (W4, Bullet 2, 11) Inventory consolidation <u>STATUS:</u> Still in process/\$2M Inventory Reduction so far. Supplier performance (W4, Bullet 1) <u>STATUS:</u> Supplier performance - Implementation plan to be issued in June 94. Contact M. Rosenzweig. Material & resource planning (MRP) Resource recovery Resource recovery 	Mgr-NP&MM	Mgr-P&MC GM-MM Mgr-Nuc Purch Mgr-MGP Corp Mgr-Res Recovery	9/94 1/94 9/93 1/94 1/94 9/93	7/95 10/94 6/94 1/95 1/95 12/95
Continue to develop supplier partnerships		Mgr-Nuc Purch	1/94	7/94
Continue expansion c JPC (Joint Procurement Corporation) activities		Mgr-P&MC	1/93	ongoing
STATUS: Master Purchase Agreement - 1. MCCB's, 2. Limitorque Parts, 3. Bearings - PC2 and PC3. Still development with bearings. MCB's/Limitorque parts in place.				
 Develop/communicate corporate inventory reduction strategy (W4, Bullet 9) 	1	Mgr-P&MC Dir-NF	1/94	5/94
STATUS: Common Coding - started 1/94 inventory consolidation on track, supplier performance - corporate PIT - finalizing plan - target 6/94. (5/24/94) Consolidation Project in progress. No action on common coding at nuclear - scheduled Nov. 1994.		GM-MM		

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Workorders on Hold for Inventory Parts

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Nuclear Department Tactical Plan - CPAT W-5

Key Focus Item Supported: Content & Delivery of Technical Training to Effectively Support Individuals and Groups (CPAT W-5)

Sponsor: Vice President - Nuclear Operations

Activity	Sponsor	Support	Start	Stop
Intergrated Total Quality Initiatives into Initial and Continuing Technical Training Programs	Mgr-NTC			
 Assess Mechanical Maintenance program Assess Controls Maintenance program based upon results of above 			5/94 6/94	complete 12/94
STATUS: Process Improvement Team named, first meeting 5/94. Postponed due to HC SERT involvement. Plan start 6/94.				
 Continue to integrate increased awareness to work standards and management expectations in all training programs. (Train-the-Trainer on Quality/Diversity conducted Jan/Feb, 1994) 			1/92	ongoing
STATUS: Working				
 Assess value of on-the-job refresher and/or increased use of "just-in-time" training 		Dept. Mgrs	4/94	12/94
Implement enhanced outage planner/scheduler training	Sta Planning - Mgr Salem	Mgr-NTC GM-NHR&AS	1/94	ongoing
Planning skills and technical knowledge		Mgr-M&S		
 Scheduling skills and project management 		Supervisors		
Computer				
Rotational Assignments				
Job Observation (Planners) Work Control Center (Schedulers)			1.1	
STATUS: Working		1		
STATUS. WORKING		1	1	

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Activity	Sponsor	Sur	rt	Start	Stop
OPTIMIZE REFUELING OUTAGE DURATION	Sta Planning			1/94	ongoing
mprove "Planned" Outage Schedules	Mgr				
Outage critimization with Westinghouse					
STATUS: Working					
Achieve Key Milestone Dates for 2R8					
STATUS: Working					

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Nuclear Department Tactical Plan - CPAT W-6

Key Focus Item Supported: Standards and Methods of Contractor Performance (CPAT W-6)

Sponsor: Vice President - Nuclear Engineering

Activity	Sponsor	Support	Start	Stop
Improve contractor industrial safety performan	ce through:	GM-SO GM-HCO Mgr-NEP Nuc. Med Dir. Contractor Mgmt.	1993	ongoing
 Monitoring previous safety compliance problem 	n areas (safety team) Mgr-Site Pro.			
STATUS: Fully implemented				
 Conducting safety talks with all contractors price emphasizing safety priority and performance e 	pr to the start of outages, Mgr-NEP xpectations	Mgr - Site Pro.	1993	ongoing
STATUS: Fully implemented				
 Including contractors in Safety Dept. "Safety/P Program" 	rofessional Recognition Gift Mgr-Site Pro.	Mgr-NEP	1993	ongoing
STATUS: Fully implemented				
 Conducting special training for all contractor m personnel in the new "Confined Space Permit" 	anagement and supervisory Mgr-Site Pro. program	Mgr-NEP		
STATUS: Fully implemented				

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Activity	Sponsor	Support	Start	Stop
mprove overall contractor performance				
Maintain adequate supervisory - to - craft ratio.	Mgr - NEP	Station Maintenance.	1/14/94	ongoing
STATUS: Complete for HC outage.		Nuclear Support &		
Increase supervisory field presence	Mgr - NEP	Jervices	1/14/94	ongoing
STATUS: Complete for HC outage				
Evaluate the need to manage and train contractors with single group	Mgr - NEP	Station Maintenance	1/14/94	9/1/94
STATUS: Open - discussion & negotiation with Station Maintenance Managers required.				
Complete job observation training for ITEs and PMs	VP-NE	Mgr-NEP	1/14/94	ongoin
STATUS: Complete and implemented in HC outage			1.1.1	6
 Specify training manhours separately in proposals Bring foreman/non-manuals in earlier for DCP familiarization 	Mgr - NEP Mgr - NEP		6/1/94 1/14/94	ongoin ongoin
STATUS: Implemented for HC outage.	1.331.37	김 화가 가지?		1976
 Implement initiatives from Safety Tagging Review Team Report 	Ops Mgr - Salem	Mgr - NEP Ops Mgr - HC	1/14/94	ongoin
STATUS: Recommendation No. 2 Implemented for HC outage and ongoing for future outages				

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Measures W-6 Standards and methods of contractor performance **MEASURES:** OSHA Accident Rate (Contractor) - Work Practices and Standards Monitoring (Contractor) - Number of Contractor Related Incident Reports (under development) Outage Performance Goals (under development) - First Aid Incident Rate - Hold Point Rework - Cost Performance

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Work Practices & Standards Contractor Performance, QA Monitoring Salem Station



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OSHA Accident Rate Contractors



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Nuclear Department Tactical Plan - CPAT S-1

Key Focus Item Supported: Root Cause Determination (CPAT S-1)

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Sponsor: Vice President - Nuclear Operations

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	Activity	Sponsor	Support	Start	Stop
1. 1	Develop and implement process for identifying and documenting root cause(s) for all corrective maintenance work orders, and preventive maintenance work orders which result in some corrective maintenance. A graded approach would insure that root cause determinations are made when appropriate, and at the proper level of detail.	GM-HCO	S-1 Team	1/94	6/94
	 Develop process and describe in appropriate procedures. STATUS: In progress Aligned CM work handling with Maintenance Rule implementing group. Implement on balance of Salem and Hope Creek systems 			1/94 11/94	12/94 5/95
	 Implement on balance analysis (RCA) procedure for use within the Develop generic Root Cause Analysis (RCA) procedure for use within the Nuclear Department. This procedure would provide guidance for conducting appropriate RCA on a variety of problems, from low level of significance and complexity to those highly significant and/or complex problems requiring extensive investigation and 	см-нсо	S-1 Team	3/94	6/94
3.	STATUS: In progress Develop measures of effectiveness. STATUS: In progress	GM-HCO	S-1 Team	3/94	12/94

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Measures

S-1 Root Cause Determination

MEASURES:

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- Repetitive Equipment Problems (under development)

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- Time Between Events

- Total Human Performance Events

Time Between Events Nuclear Dept



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Nuclear Department Tactical Plan - CPAT S-2

Key Focus Item Supported: Corrective Action and Follow-Through (CPAT S-2)

Sponsor: General Manager - Quality Assurance/Nuclear Safety Review

Activity	Sponsor	Support	Start	Stop
Management Expectations for Corrective Action (Promulgate uniform understanding of the Corrective Action Process)	GM-QA/NSR			
 Presentation to CNO Team (raise standard for corrective action thru existing process) 			2/94	6/94
STATUS: Presentation made 2/94. Decision made to combine roll-out of S-2 (Corrective Action and Follow-through) with M-5. New presentation to CNO Team will be ready June 1994.				
Manager's Dialogue presentation (combined with self-assessment)		199 3 13	6/94	6/94
STATUS: Modified schedule to deal with issue of prioritization.				
 Supervisors Dialogue breakout groups GM quarterly meetings Follow-up Communications Activities 		GM's Mgr-Nuc	7/94 TBD TBD	9/94
Corrective Action Data Base Project	GM-QA/NSR			
 Phase I (Procurement Module Pilot) implemented. The consolidation of various independent processes for identification of discrepancies in receiving, warehouse, vendor programs/process, procurement documents, and QA into a single "problem report" system using the Corrective Action Database (CADB) is underway. 			12/92	complete
STATUS: On schedule.		a la carda		1 2 4 3

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Activity	Sponsor	Support	Start	Stop
Phase II currently under development for consolidation of Hope Creek, Salem and E&PB corrective action processes (DEF, IR, DR)			1/94	7/94
STATUS: On schedule.	1		5/0.1	7/94
Training and Implementation	CHICANSP		0.0.1	
ollow-up	GM-GANNON		5/94	7/94
Develop indicators to trend of corrective action Develop indicators to trend corrective action database contents			7/94	ongoing

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Nuclear Department Tactical Plan - CPAT S-3

Key Focus Item Supported: Safe, Uneventful Operations/Performance Trending for Systems and Equipment (CPAT S-3)

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Sponsor: Vice President - Nuclear Operations

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Activity	Sponsor	Support	Start	Stop
Cotimize Preventive Maintenance (PM) through Reliability Centered	Mgr-R&A		1/94	12/94
Complete RCM implementation at Salem			1994	1994
STATUS: Analysis complete, implementation by mid-year.			1995	1995 6/96
 Perform RCM analysis on 8 Hope Creek Systems 				
STATUS: Ongoing				
Eliminate Scrams through Identification of Design Changes		Sta. Mgrs Mgr-NSR		
STATUS: Ongoing		Mgr-NED		
Continue Technical Specification Improvements to Reduce Risk of On-Line Surveillance	Mgr-L&R	Sta. Mgrs	1/94	
STATUS: Ongoing				
Continue Procedure Improvements to Reduce Incidence of Personnel Error Improve Use of Operating Experience (internal/external) • Continue OEF trending program	Sta. Mgrs Mgr-R&A	Sta. Mgrs	1/94	
STATUS: Improvement plan is in progress. NOIT effort has started.				

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Activity	Sponsor	Support	Start	Stop
mprove Maintenance Effectiveness Perform maintenance self-assessments as part of NUMARC commitment by 1994.	Maint. Mgr.	Sta. Mgrs NE/QA/NSR	1/94	
STATUS: In progress. Continue Focus on Work Standard & Compliance STATUS: Ongoing process continually reinforced through	GM-SO			
Performance Trending for Systems and Equipment (S3) 1. Break down performance trending information subactivities as follows:	HC Tech			complet
 Data collection Database input Graph generation Graph and database monitoring & analysis Negative trend actions Identification of trend parameters, goals, and action levels Setup and revisions to database and graphs Configuration of database (software) 				
2. Analyze each subactivity, implement solutions. One subactivity each month STATUS: Same as above - database upgrades first subactivity.	HC Tech	HC/Salem: Technical Maintenance Operations R&A	3/1/94	1/1/95

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Measures

S-3 Performance trending for systems and equipment important to reliability and operational control action upon results

MEASURES:

- Capacity Factor
- Licensee Event Reports (Equipment)
- Repetitive Equipment Problems (under development)
- Unplanned Automatic Scrams per 7000 Hours Critical
- Number of systems which have a prepared list of performance indicators (under development)

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- Number of workorders generated as a result of trending performance indicators (under development)
- Number of parameters trended (by group) (under development) By manual means By electronic means



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Capacity Factor Hope Creek Station



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Licensee Event Report Salem Unit 1



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Licensee Event Report Hope Creek Station



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Unplanned Auto Scrams/7k Hours Salem Unit 1



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Unplanned Auto Scrams/7k Hours Salem Unit 2



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Unplanned Auto Scrams/7k Hours Hope Creek



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Nuclear Department Tactical Plan - CPAT S-4							
Key Focus Item Supported: Operating Experience Feedback (OEF) delivery and tracking that meets job needs of recipients for information (CPAT S-4) Sponsor: Vice President - Nuclear Operations							
Activity	Sponsor	Support	Start	Stop			
Salem Review operating experience on an ongoing basis	GM-SO	Mgr-R&A	1/94	ongoing			
 Hold small group meetings to increase familiarity with OEF program <u>STATUS</u>: Requires data base to be available for island use - in development with other departments. 	Mgr-R&A		5/1/94	5/1/95			
Provide personalized distribution of Daily Nuclear Network STATUS: Complete	Mgr-R&A		5/1/94	7/1/94			
 Provide individual access to historical OEF information STATUS: Working with methods tied to first item. 	Mgr-R&A		5/1/94	5/1/95			
Improve decision making at weekly OEF for those events that need in-depth root cause follow-up	GM-SO		3/1/94	12/31/94			
discuss each IR to determine root cause analysis depth.							

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Activity	Sponsor	Support	Start	Stop
Set expectations for timeliness of completing incident report close-out	GM-SO		3/1/94	9/1/94
STATUS: Working.				
Improve accountability for close-out of external OEF documents	GM-SO	Station Mgrs	3/1/94	9/1/94
STATUS: Working.				
Improve Manager accountability for review of responses to internal and external documents	GM-SO		3/1/94	9/1/94
STATUS: Working.				
Provide more in-depth review of re-opened external documents based on internal event trends	GM-SO	Station Mgrs	3/1/94	12/1/94
STATUS: Working.			10.5	
Increase frequency of management review of causal factor and event trends (present quarterly)	GM-SO	Mgr-R&A	3/1/94	
STATUS: Ongoing.			1000	
Re-assess station NPRDS coordinator function	GM-SO	Mgr-R&A	3/1/94	5/1/94
STATUS: Station coordinator assigned.	(* . L. ?		1.1.1.2	
Increase priority of preparing operating experience reports to share with industry based on internal event experience	GM-SO	Mgr-R&A	3/1/94	9/1/94
STATUS: OEs to be prepared for notable events, 4 OEs shared w/industry 1st quarter.				

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Activity	Sponsor	Support	Start	Stop
tope Creek	(
Review operating experience on an ongoing basis	GM-HCO	Mgr-R&A	1/94	
Implement NOIT OER improvements	GM-HCO	Mgr-R&A	4/94 3/1/94	5/94 5/1/94
Re-assess OEFRDS coordinator function within R&A				
STATUS: Working	1000 Sec. 1			
E&PB				
Review operating experience on an ongoing basis	GM-HCO	Mgr-R&A	1/94	
 Hold small group meetings to increase familiarity with OEF program STATUS: Requires data base to be available for island use. 	Mgr-R&A		5/1/94	5/1/95
Provide personalized distribution of Daily Nuclear Network	Mgr-R&A		5/1/94	7/1/94
STATUS: Complete.				
 Provide individual access to historical OEF information 	Mgr-R&A		5/1/94	5/1/95
STATUS: Working.				
 Set expectations for timeliness of completing incident report close-out 	Mgr-NED		3/1/94	9/1/94
STATUS: R&A to provide list of all IR open items.				
 Improve accountability for close-out of external OEF documents 	Mgr-NED	Station Mgrs	3/1/94	9/1/94
STATUS: Working				
 Improve Manager accountability for review of responses to internal and external documents 	Mgr-NED		3/1/94	9/1/94
STATUS: Working				

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Nuclear Department Tactical Plan - (CPAT S-4	(cont a)		
Activity	Sponsor	Support	Start	Stop
 Provide more in-depth review of re-opened external documents based on internal event trend 	Mgr-NED	Station Mgrs	3/1/94	12/1/94
STATUS: Current trend being developed.				
Training				
 Provide "Train the Trainer" program for all instructors by the end of 1994. 	Mgr-NTC	Mgr-R&A	1/94	12/94
STATUS: Working	Star and			
 Present OEF Training twice a year rather than the current four times per year to allow greater preparation time. 	Mgr-NTC		1/94	12/94
STATUS: Working				
 Perform a "Validity of Commitment" check to determine value added in training on SOER's in the Controls area on a repetitive basis. 	Mgr-NTC		1/94	12/94
STATUS: Working	1.00			
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MEASURE	S:
	- Licensee Event Reports
	- Licensee Event Reports (Personnel Error)
	- Repetitive Equipment Problems (under development)
	- Time Between Events
	- Total Human Performance Events

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Licensee Event Report Salem Unit 2



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Licensee Event Report Hope Creek Station



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COMMUNICATION OF EXPECTATIONS

- 3 VP's meet with all employees on Island (completed 1993)
 - Reinforce standard of quality and barrier model
 - Salem/HC status and progress to date
 - CPAT rolldown (completed 1/94)
- Salem employee meeting with new VP/GM (completed 2/94)
- New vision rolldown (completed 2/94)
- Status of the department year end meeting (completed 2/94)
- S. Miltenberger meeting with Salem employees
- Management time in plant to observe and enforce standards
- Expectation that all personnel will self-identify errors
 - Establish standards which strive for mistake-free performance
 - Create climate in which all co-workers honor good faith efforts
 - Culture supports owning up to mistakes

UNITIZATION OF SALEM

Objectives

- Keynote: Focus Ownership Teamwork
- Improve at all levels
- Position individuals appropriately skills match job

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- Provide time to do right thing (proactively solve problems)
- Minimize challenges (single unit focus)
- Provide opportunity for significant changes
- Less overtime per person
- Improved quality of work life
- Scope
 - Operations
 - Maintenance
 - Station planning and scheduling
 - Outage planning and scheduling

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UNITIZATION OF SALEM

- Implementation
 - Requires additional staff
 - Raname Management/Supervisory team
 - Re-bid Supervisory/Management positions as needed
 - Hiring methods (Targeted Selection)

Behavioral component

Supervisory as well as technical skills

- Hire from outside opportunity to raise qualification/standards
- Schedule
 - Interim division of Maintenance Mechanical and Control groups - completed February 1994
 - Planning and Scheduling Spring 1994
 - Mechanical Maintenance Summer 1994
 - Controls Maintenance 4th Qtr 1994/1st Qtr 1995
 - Operations based on license classes 1st Qtr 1996

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PSE&G SALEM UNITIZATION ORGANIZATION



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IMPROVED MANAGEMENT/SUPERVISORY OVERSIGHT

Organizational changes

- J. Hagan Vice President Nuclear Operations and (acting) Salem Station General Manager
 - Reassigned other duties to VP-NE and VP-CNO
- Station management enhancements
- Maximize effective management time in field
- Improved monitoring/assessment/feedback
 - Enforcement of standards of performance

Supervisor/Manager oversight

Increased time in field

Maintenance - Controls

- Additional management oversight
- Controls troubleshooting
 - Mid-level management personnel provide on-shift reviews of I&C troubleshooting plans

Work standards monitoring

Performance indicators

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IMPROVED SAFETY REVIEW/QUALITY ASSURANCE OVERSIGHT

Implemented four barrier model

- Role of independent and self assessment
- Rolled down through organization
- Findings and corrective actions include barrier assessment

- QA an safety philosophies
 - "Safety is our first priority"
 - Roles of QA and nuclear safety
 - Risk based assessments
- Reorganization of NSR and formation of Nuclear Review Board (NRB)
 - Focus to more global assessment of nuclear safety
- Independent NSR effectiveness review performed by outside organization



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AUGMENTED ON-SHIFT OVERSIGHT

Purpose

 Provide additional independent management oversight of plant operations

Scope

 All plant operations with the potential to impact plant reliability and safety

Responsibilities

 Monitor/Observe plant evolutions to assess compliance with work standards, procedures, and professional conduct

Typical evolutions to be monitored

- Reactor startup and shutdown
- Low power operations
- Special tests
- Selected surveillance
- Selected major system evolutions
- Shift turnovers and Plan of the Day meetings
- Key maintenance evolutions
- Material condition walkdowns
- Control room demeanor and conduct

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AUGMENTED ON-SHIFT OVERSIGHT

- Implementation
 - Initiated May 17, 1994
 - 5 people covering 5 shifts
 - Senior, well-experienced people respected, high credibility
 - Free reign to look at anything
 - Provide daily feedback to Station GM and Managers
 - Provide weekly feedback to Chief Nuclear Officer

SALEM GENERATING STATION NRC VISIT MEASURES OF SUCCESS

MONITORING EFFECTIVENESS OF PERFORMANCE THROUGH PEOPLE

Performance Indicators

- Work practices and standards monitoring by line management and Quality Assurance
- Supervisory face-to-face time
- Human performance indicators
- Leadership feedback results
- Personnel error Licensee Event Report
- Composite safety index performance

SALEM GENERATING STATION NRC VISIT SUMMARY

- PSE&G's commitment to continued improvement has been demonstrated by a comprehensive self-diagnostic assessment
- Physical changes occur more readily than cultural/people changes
- Focus on Salem improvement
- Nuclear Department Priorities for 1994
 - Emphasis on people and performance
 - Safe uneventful operations
 - Successful refueling outages
 - Results oriented, cost effective operations

ENCLOSURE 10

OPEN ENFORCEMENT CONFERENCE SURVEY

3.

Lice	nsee Public Service Electric + Gas
Faci	lity Salem
Date	of Enforcement Conference 7/28/94
Pres	iding NRC Official Tim MARTIN
•	Impact on The NRC's Ability to Conduct an Enforcement Conference and/or Implement The Agency's Enforcement Program
1.	Was there a delay in the enforcement process due to holding Slightly an open enforcement conference? a. Yes b. No
	If yes, what was the cause for the delay?
	 a. Providing sufficient public notice of the conference. b. Licensee requested additional time to prepare for the open enforcement conference. c. Other. Explain. <u>Had to Send Commission Paper to get approval</u> <u>to be Open because of Public interpt</u>
	If yes, how long was the delay? _ couple weeks
2.	Were any members of the public disruptive to the proceedings? a. Yes b. No
•	Impact on Licensee's Participation During the Open Enforcement Conference
3.	Does the staff believe that the licensee's communication with the staff during the open enforcement conference was less candid or more guarded than in past enforcement conferences or in other meetings where the public was not present? In answering this question, consideration should be given to whether the licensee tended to answer staff questions more narrowly or whether the licensee volunteered additional information or whether the staff had to be more persistent in questioning the licensee to gain full information during the open enforcement conference. Consideration should also be given to whether there was any change in practice in the licensee having an attorney present at the conference.
<	a. No difference.
	c. Big difference. Explain.
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a. Yes (b. No)
If yes, explain.
Did the licensee provide a thorough explanation of the root cause(s) of the violation? (a. Yes b. No
Did the licensee admit the violation(s)? (a. Yes) b. No Ex
Does the staff believe that the licensee's presentation during the open enforcement conference was more formal than in past enforcement conferences?
a. No difference.
b. Little difference.
c. Big difference. Explain.
was significantly longer or shorter than other enforcement
conferences? a. No difference. b. Longer. Explain. Slightly longer but due to wome, rathen then fact the conference was open.
conferences? a. No difference. b. Longer. Explain. <u>Slightly longer</u> but due to some, rathen <u>then fact the conference was open</u> c. Shorter. Explain.
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 conferences? a. No difference. b. Longer. Explain. <u>Slightly longer</u> but due to usue, rathen <u>then fact the conference was open</u>. c. Shorter. Explain. Impact on NRC Resources Was there adequate seating capacity for all persons interested in attending the open enforcement conference? a. Yes b. No Was it necessary for the NRC to arrange for a public meeting room outside of the regional office? a. Yes b. No If so, what was the cost? Did the regional office need to purchase any equipment/services as a result of holding the open enforcement conference?

12. Was substantially more staff time spent in preparing for the open enforcement conference? a. Yes (b. No)

If yes, explain. Although more people involvent

In answering questions thirteen through nineteen, the staff should give consideration to such issues as the need for certain staff members to attend the open enforcement conference, the need to provide escorts, the need to make copies of handouts, the need to answer questions from the audience after the conference, the need to respond to the open enforcement conference survey, etc.

13. Was a higher level of management involved in the open conference than the level of management typically involved in closed conferences? a. Yes b. No

It yes, explain. Reysonal Administrator normally does not attend Els, but attended this one

14. Were there substantially increased demands on the public affairs staff? a. Yes (b. No)

If yes, explain. Although alot of media affended

15. Were there substantially increased demands on the legal staff? a. Yes b. No

If yes, explain.

16. Were there substantially increased demands on the security staff? a. Yes b. No

If yes, explain.

17. Were there substantially increased demands on the enforcement staff? a. Yes (b. No)

If yes, explain.

18. Were there substantially increased demands on the technical staff? a. Yes b. No

If yes, explain.

- 3 -

19. Were there substantially increased demands on the administrative staff? (a. Yes) b. No

If yes, explain. TapEL+ provide escorture If not specifically addressed in the responses to questions 20. thirteen through nineteen, estimate the additional demands on the staff in staff-hours. Zero to five staff-hours. 8. b. Five to 10 staff-hours. C. Ten to 20 staff-hours. Over 20 staff-hours. Specify. d. Public Interest/Public Benefit How many members of the media attended the open enforcement 21. conference? 13 How many members of the public attended the open enforcement 22. conference? 7 How many State Government representatives, attended the open 23. enforcement conference? (including 2 montos of Senitor, Biden's Were any interested individuals denied access to the 24. staff enforcement conference due to conference room limitations? a. Yes b. No Did the majority of the audience stay for the duration of 25. the enforcement conference? (a. Yes) 1 Media b. No hit staff Did members of the media or public ask the NRC questions / 26. with come in after the enforcement conference? a. Yes b. No imme diafel By fime Please provide any additional comments based on either positive media or negative impacts of conducting the open enforcement Stoppo conference. no other mombas remained Please enclose the attendance sheet with the completed survey.

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Last Day Briefing w/PSE & G

- Purpose of the AIT:
 - A. Verify the circumstances and evaluate the significance of the following event: The Salem Unit 2 OHA system was lost without knowledge or response by the operating staff for about 1.5 hours.
 - B. The Charter gives the scope of the inspection. This scope included detailed factfinding, identification of generic issues, determination of root causes, and examination of PSE&G operational and managerial performance.
- II. The inspection included document review, over 2 dozen interviews, observation of a simulator demonstration, extensive discussions with OHA system vendor, review of test and troubleshooting, equipment examination with walkthru's, and observation of operating crew performance.
- III. The team believes that the OHA system is now performing its function to provide information to the operating crews. Current system checks and admin controls adequately verify system status.
- IV. The failure of the OHA system was most likely initiated by a person from the control room operating crew making the wrong key strokes on a computer workstation for the OHA system. This error, coupled with a panel switch in the wrong position, placed the OHA system CPU in a mode where it was waiting for additional commands that never came. This prevented the OHA system from displaying alarms in the control room.
- V. The team determined that the root causes for the event were as follows (please note that these are only preliminary conclusions and subject to additional team and NRC management review):
 - A. Procedure STOP-SO.ANN-0001(Q) was not followed. (switch, keystrokes)
 - Operators were not trained to recognize system problems.
 - C. Poor Human Machine Interface (failure not readily detectable, workstation keystrokes & lack feedback & black box sw)
 - D. Design vulnerabilities (response to annunciators overridden by lower priority task.
- VI. Observations
 - A. Failure to notify the NRC within one hour of the event.
 - 3 Operations had concerns about some system problems before the event.

- C. Delay in notifying senior management of the event.
- D. Knowledge LTA of OHA system coverage by technical and operations.
- E. Communication LTA about A45 window between Eng and Ops.
- F. Lack of technical understanding of the detail operation of the OHA system through understanding of the software.
- G. Design vulnerabilities exist in the system that make the OHA system susceptible to errors (data link to the distributed logic cards, workstation)
- H. Operators were not sensitized to the complexity of the system.
- I. Lack of Abnormal Procedure for partial or total loss of annunciator systems. Also, individual ABs don not contain alternate indication for annunciators.
- J. No simulator training on loss of annunciators.
- K. No operator classroom training on the OHA system.
- L. On Unit 1, there is no alternate continuous monitoring of the RWST tank level in the control room. (May affect ECCS system operation during post-LOCA injection phase).
- M. Operations personnel did not observe precaution 3.1 in ANN-0001 concerning unauthorized personnel trying to gain access to password protected features of the OHA workstation.
- N. Vendor manual was LTA. No top level software description. The word "lockup" is used but not defined or discussed.
- O. Routine test of redundant components was not in place (was planned).
- P. Software review of DCP LTA.
- Q. System not designed to aid troubleshooting redundant components.
- R. No system virus check.
- NCO actions to deduce the OHA loss w/o training showed a good questioning attitude.
- T. System engineer responded well to the event (showed ownership).
- U. PM process looked good.

V. DCP was well documented.

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- W. Installation of the OHA system went smoothly and was well-planned.
- X. Use of OHA system to enhance trending a plus.

Your staff provided outstanding support. Formal exit date TBD. Thank you. Merry Christmas.



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General Activity and Licensee Response History

- 1/94: PSE&G concludes comprehensive assessment, concludes there are significant deficiencies in root cause determination, and the performance of offsite and line QA organizations. Subsequently, a complex Strategic Improvement Plan is established that identifies corrective measures and schedule for completion.
- 2/94: After being at power for 3 days, unit 1 tripped.

2/94: Cal Vondra, General Manager-Salem Operations reassigned to non-nuclear posistion in PSE&G. Joe Hagan, Vice President-Nuclear Operations assigned as General Manager until permanent replacement is appointed.

4: Salem reorganization initiated, including , ant unitization, and establishment of new department managers for System Engineering/ Tech Support, Maintenance, and Outage Planning.

2/4/94: Salem management took both units off line to dredge grass and mud in front of the Salem circulating water intake structure.

2/24/94: Management Meeting to discuss CPAT findings and licensee plans and schedules for program improvement.

 6/24/94: Salern Unit 1 rapid shutdown from 75% power due to condensate suction header overpressurization and water hammer.

General Occurence and Event History

1/27/94 - 2/13/94: Salem Unit 1 was subject to 2 reactor trips (1/27/94-trip from 10% due to feed reg valve problems attributable to previously ineffective troubleshooting efforts; 2/10/94-trip from 100% due to coincident loss of both 15 VDC control power supplies to EHC system due to unexpected actuation of overvoltage protection (crowbar), suspected to be caused by maintenance activities. On 2/11/94licenses discovered that the mode switches to both air compressors for 1B EDG were in the off position due to work control problems; and on 2/13/94, while the unit was in Mode 2, an 1&C technician error involving a pressure transducer associated with the atmospheric steam dump system caused the steam dumps to actuate. Consequently, excessive cooldown occurred and power increase from 2% to 5.6%, causing an unplanned mode change.

4/7/94: Salem Unit 1 trip from 25% due to operator error (Operators reduced power to 10% to compensate for grass intrusion, which enabled the 25 low power trip setpoint. To resi to lower than normal Tave, operators withdrew J me control rods which increased power in excess of 25%, which resulted in trip. Trip was complicated when two MSIVs and two FW isolation valve failed to close; and two turbine driven feed pumps failed to trip. SI actuated, PRT rupture disk blew-out, and the licensee declared a UE followed by an ALERT. AIT dispatched. Consequently, escalated enforcement action was taken (4 Severity Level Ills with a \$500,000 CP). General cause involve ineffective corrective action for preexisting equipment deficiencies that provided challenges to operators (MS10), inadequate operator command and control, and ineffective management communication of expectations to the staff, and poor operator performance issues. (AIT)

Precursors to this event (grass affecting plant operation) occurred twice in 1993, and once in 2/ 94. Subsequent problems with grass occurred in 6/94 and 12/94.

 6/14/94: Salem Unit 2 rapidly reduced power from 100% to 70% due to grass intrusion.

Jan.- June 1994

SALEM

General Activity and Licensee Response History

 7/94: NRC Commissioners receive PSE&G presentation on April 7, event. Chairman informs PSE&G that Salem performance (4 AITs in 4 years) was unacceptable.

7/11/94 - 8/25/94: NRC conducted a special Performance Assessment of Salem. Generally the assessment team found that there was no aggressive quality oversight of activities, and no proactive effort existed to correct existing system and equipment deficiencies that had the potential to challenge operators and system performance. Weakness were also found in maintenance programs relative to procedure adherence, postmaintenance testing, and control of work activities. In operations, a significant number of vork-around" issues were identified that

verators had accomodated and accepted as normal. Though engineering activities were generally assessed positively, weaknesses were noted in engineering oversight of vendor-designed modifications. Plant support activities were acceptable.

7/30/94: PSE&G executive management, as part of an overall performance improvement effort (which involved assessing the performance of all personnel assign 1 to support Salem), terminated or otherwise forced the resignation of about 55 personnel that were deemed to be low-level performers in the Salem orgainization. The terminations mainly affected supervisors and technical personnel in non- bargaining positions, and included L. Reiter, General Manager-Quality Assurance and Nuclear Safety Review and other managers in that organization.

34: EDO informs Miltenberger, LaBruna, and Hagan that he will not be able to defend continued Salem operation in the event of another AIT.

10/94 Steve Miltenberger, Vice President and Chief Nuclear Officer was replaced by Leon Eliason. Subsequently, Eliason presides over the reorganization of PSE&G's nuclear division into a subsidian reorganization of Nuclear Division, Nuclear Business unit. Eliason is named President of NBU. 10/13/94 - 2/16/95: Salem Unit 2 refueling outage (2R8). Projected 77 day refueling outage delayed due to leaking pressurizer code safety valves and single failure susceptibility of the Solid State Protection System.

10/29/94-11/4/94: several ineffectively controlled non-safety related maintenance activities, including near miss cutting 4160V cable.

July-Oct. 1994

General Occurence and Event History

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General Activity and Licensee Response History

- 11/94: As a result of investigation of a Hope Creek matter involving insufficient staffing of the control room in 1992, the licensee determined that some similar staffing issues occurred at Salem over an extended period of time. OI investigation is progressing.
- 12/13/94: John Summers named as new General Manager-Salem Operations; Joe Hagan resumes normal duties of Vice President of Nuclear Operations.
- 1/95: PSE&G named Jeffery Benjamin as new General Manager-Quality Assurance and Nuclear Safety Review. A new manager and new supervisors were subsequently appointed to the Salem QA organization.

1/30/95: Leon Eliason announced functional realignment of Nuclear Business Unit and Performance Improvement initiatives. NBU functional areas were established as Operations (Hagan), Support (LaBruna), Assessment (Benjamin), Human Resources and Administration (Johnson), External Affairs, (Burricelli), and Strategic Planning and Financial (Cohen). Reorganization to be announced later.

2/13/95: Leon Eliason announced reorganization of NBU to support previously announced functional realignment.

2/17/95: Leon Eliason informed the EDO that he has commissioned an independent team of senior nuclear industry executives to perform a Organizational Effectiveness Review of Salem in an effort to determine why program improvement has not been realized.

2/27 - 3/10/95: INPO Plant Evaluation and Accreditation Team (28 persons) in progress.

2/95: Enforcement Conferences were held with PSE&G and three former managers (Cal Vondra, former GM-Salem Operations; Vince Polizzi, former Operations Manager-Salem, and L. Reiter, former GM-QA/NSR) relative to harassment and intimidation issues stemming from a 12/3/92 incident involving two SRG engineers. Enforcement pending,

3/95: John Morrison, formerly Manager-Technical Department is reassigned. Licensee is currently considering outside replacement. Nov. 1994 - Present

General Occurence and **Event History**

- 11/18 28/94: the Salem Units experienced four electrical transients including two losses of 4160V station power transformers, one loss of a 13 KV substation, and arcing of a 4KV supply cable to safety related busses.
- 12/11/94: rapid Salem Unit 1 power reduction from 100% power to 51% power in response to grass intrusion.
- 10/94 2/95: Unit 2 outage, planned to be 60 days but extended due to extensive problems involving leak-by on pressurizer safety relief valves.
- 1/95: Salem Unit 2 replaces no 23 RCP seal due to seal leakoff valve failure.

1/95: SALP 93-99 (6/93 - 11/94) OPS 3. M/ S 3, E/TS 2, PS 1

- 2/2/95: While unit 1 at 100% and unit 2 in Mode 2, PSE&G determined that SSPS vulnerability existed due to design deficiency. Requested NOED to effect design change. NOED granted by NRC. On 2/3/95 licensee determined that power supply problem existed relative to expected functioning of SSPS circuits and commenced troubleshooting activities. NRC subsequently rescinded NOED. Both units were required to go to Mode 5.
- 2/94 3/95: While shutdown, the licensee experienced difficulty in relative to MS10 performance on both units. Extensive troubleshooting and root cause analysis initiated. Determined several design and component deficiencies existed on controller cards and modules associated with MS10 operation that were not previously revealed by other licensee efforts to understand and resolve MS10 performance issues that contributed to the 4/7/94 Unit 1 trip.
- 2/95: After start up of the unit, within 3 days Salem Unit 2 was shutdown again to replace No. 21 RCP seal. Seal failure resulted from low leakoff flow apparently caused by a small amount of crud.

Mr. Leon Eliason Chief Nuclear Officer and President Nuclear Business Unit Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey, 08038

SPO

SUBJECT: SYSTEMATIC PERFORMANCE OF LICENSEE PERFORMANCE (SALP) REPORT NO. 50-272;50-311/93-99

Dear Mr. Eliason:

This letter forwards the SALP report/for Salem Generating Stations, Units 1 and 2, for the period between June 20, 1993 and November 5, 1994. The SALP was conducted in accordance with the Nuclear Regulatory Commissions's revised process that was implemented July/19, 1993. This revised process assesses licensee performance in four functional areas: Operations, Maintenance, Engineering, and Plant Support (which includes radiation protection, physical plant protection and security, emergency preparedness, fire protection, chemistry, and housekeeping).

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Operators generally responded appropriately with good command and control to the many plant trips and operational transients that occurred in this period except during the April 7, 1994, grass intrusion event. However, the operators did not effectively assure that plant systems and equipment were always sufficiently maintained to perform as designed. Toc often, the operations organization accommodated long-standing equipment or system problems that from commodated long-standing equipment or system problems that from commodated the operation of the plant in normal and upset conditions. Further, the general lack of a questioning attitude by operators resulted in anomalous indications or conditions being unnoticed or not understood, and consequently, ineffectively resolved. Weaknesses in operability decision-making resulted in some determinations that were not conservative or otherwise lacked a solid technical basis. This functional area was rated as Category 3.

The maintenance organization was weak in the implementation of programs and activities. Consequently, there were frequent ecourrences involving procedure adherence, procedural adequacy, and control and oversight of work. Some improvements, such as better communications with the operating organization, improved prioritization and scheduling of work, and improved material condition have been achieved. Notwithstanding, weaknesses still prevail relative to the effectiveness of corrective actions, troubleshooting and resolution of recurrent equipment problems, and management oversight of work activities. This functional area was rated as Category 3.

The performance of engineering was inconsistent. The quality of design and modification activities was generally good. However, engineering priorities and did rule moments reflect the needs of the plant. Significant problems were

evident in the quality of root cause assessment activities and resolution of repetitive equipment problems. While the quality and technical ability of the Engineering and Plant Betterment organization appears good, the organization did not effectively engage itself in the diagnosis, root cause assessment, and resolution of the chronic plant system and equipment problems that have adversely affected overall plant performance. This functional area was rated as Category 3.

Performance in the plant support area continued to be strong. Well trained and capable management and staff contributed to the effectiveness of radiation protection and ALARA efforts, and the radiological environmental and effluent monitoring programs. The performance of the emergency preparedness organization improved in this period and was effectively demonstrated in drills and actual event responses. The plant security organization performed well, notwithstanding problems with assessment aids and occasional weaknesses relative to supervisory oversight and personnel performance. Performance relative to fire protection program and activities improved during this period. This functional area was rated as Category 1.

In summary, the NRC is concerned with the performance decline in three of the four areas during this period. The NRC is particularly concerned with the frequent challenges to plant systems and to the operators caused by repetitive equipment problems and personnel errors that had the potential to, or actually did, adversely affect plant or personnel safety. We recognize that your organization has, within the last year, initiated several comprehensive actions that have the potential to improve overall plant performance. While we acknowledge some recent incremental performance gains, these efforts have not yet resulted in any noticeable overall performance improvement.

In arriving at this assessment, our staff determined the following apparent contributing factors; (1) The tendency of your operations staff to accept and otherwise, less than optimum; (2) The tendency of your organization to assume guest accommodate system performance that was not in accordance with design, or the most proximate cause of degraded conditions or unexpected system performance, and dismiss or not adequately consider other possible contributors or factors without substantial technical basis or rationale; (3) The reluctance, of maintenance and operations organizations to solicit technical support from the engineering organization for the resolution of plant system or equipment issues; and the engineering organization's reticence to engage in the diagnosis or resolution of plant technical problems without requirement or request; (4) The lack of value attributed to, or expected from, on-site safety review and quality assurance activities, and the consequent ineffectiveness of the function; and (5) Insufficient critical self-assessment initiatives to evaluate the adequacy and performance of personnel, procedures, and hardware.

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We have scheduled a management meeting on January 12, 1995, at the Salem Generating Station Access Processing Facility to formally present this Systematic Assessment of Licensee Performance. The meeting will be open for public observation in accordance with NRC policy. Following that meeting, we request that you provide written comments, including any correction of factual information, within 20 days of the date of the meeting. The enclosed report and your response will be placed in the NRC Public Document Room.

We appreciate your cooperation.

Sincerely,

Thomas T. Martin Regional Administrator

Docket No. 50-272/50-311

Enclosure: Systematic Assessment of Licensee Performance Report No. 50-272/93-99 & 50-311/93-99

cc w/encl: J. J. Hagan, Vice President-Operations/General Manager-Salem Operations S. LaBruna, Vice President - Engineering and Plant Betterment C. Schaefer, External Operations - Nuclear, Delmarva Power & Light Co. F. Thomson, Manager, Licensing and Regulation J. Robb, Director, Joint Owner Affairs A. Tapert, Program Administrator A. Giardino, Acting Manager, Quality Assurance B. Hall, Acting Manager, Nuclear Safety Review R. Fryling, Jr., Esquire M. Wetterhahn, Esquire P. J. Curham, Manager, Joint Generation Department, Atlantic Electric Company Consumer Advocate, Office of Consumer Advocate William Conklin, Public Safety Consultant, Lower Alloways Creek Township Public Service Commission of Maryland The Chairman **Commissioner Rogers** Commissioner de Planque Public Document Room (PDR) Local Fublic Document Room (LPDR) Nuclear Safety Information Center (NSIC) K. Abraham, PAO (24 copies) NRC Resident Inspector State of New Jersey State of Delaware

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bcc w/enc: Region I Docket Room (with concurrences) J. Taylor, EDO J. Milhoan, DEDO SALP Program Manager, NRR/ILPB (2) J. White, DRP S. Barber, DRP K. Gallagher, DRP

bcc w/enc (VIA E-MAIL): L. Olshan, NRR W. Dean, OEDO J. Stolz, PDI-2, NRR M. Shannon, ILPB M. Callahan, OCA W. Russell, NRR R. Zimmerman, NRR J. Lieberman, OE C. Holden, NRR/RPEB A. Thadani, NRR

bcc via E-Mail: Region I Staff (Refer to SALP Drive)

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DATE	12/16/94		12/11/94	12/16/99	
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SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE (SALP)

SALER UNITS 1 AND 2

REPORT NO. 50-272/93-99 & 50-311/93-99

I. BACKGROUND

The SALP Board convened on December 1, 1994, to assess the nuclear safety performance of the Salem Units 1 and 2 for the period June 20, 1993, to November 5, 1994. The board was convened pursuant to U.S. Nuclear Regulatory Commission (NRC) Management Directive (MD) 8.6, "Systematic Assessment of Licensee Performance (SALP)" (see NRC Administrative Letter 93-02). Board members were Richard W. Cooper, II (Board Chairman), Director, Division of Reactor Projects, NRC RI; James T. Wiggins, Director, Division of Reactor Safety, NRC Region I (RI); Charles W. Hehl, Director, Division of Radiation Safety and Safeguards, NRC RI; and John F. Stolz, Director, Project Directorate I-2, NRC Office of Nuclear Reactor Regulation. The board developed this assessment for approval by the Region I Administrator.

The following performance category ratings and the assessment functional areas are defined and described in NRC MD 8.6.

II. PERFORMANCE ANALYSIS - OPERATIONS

The Operations functional area was rated category 2 in the last SALP period. The licensee's performance was characterized by excellent operator response to trips and other operational transients. Supervision and management oversight of refueling and day-to-day operations was very good. However, the operators' attempt at several startups of Unit 2 without sufficiently determining the cause of repetitive rod control problems and effectively resolving the problem, was identified as a significant management control and oversight weakness.

Throughout the current SALP period, operators were often challenged by plant trips and other operational transients. Operators exhibited generally strong food command and control of the response to these events. For example, on June 10, 1994, operators demonstrated appropriate command and control in response to an automatic trip caused by failure of a main generator potential transformer. Likewise, on August 30, 1994, Unit 2 operators responded well to a condenser water box manway failure and reduced power to 75 percent. However, during the April 7, 1994, grass intrusion event, shift management personnel did not remain free to survey and analyze all operating parameters and, for a short period of time, lost control and perspective of the overall operations in the midst of attempting to stabilize plant conditions.

Operations and plant management made operating decisions that exhibited conservative operation of the plant. For example, in June 1994, plant staff performed a methodical, controlled, safe startup of Unit 1 following the April 7, 1994 trip, after delaying startup in order to repair small leaks in the reactor head vents and a pressurizer safety valve. Additionally, operators exhibited proficiency in making conservative, proper, and timely emergency declarations for six actual events that necessitated consideration of entering an emergency action level.

Notwithstanding the performance noted above, overall operations performance during this assessment period was characterized by significant weaknesses in several areas. Slow or inadequate resolution of equipment problems by other plant departments caused operators to become accustomed to working around or living with problems that created additional challenges to them in operating the plant in normal and upset conditions. For example, the licensee provided inadequate training, guidance, and procedures to the operators to cope with plant transients resulting from grass intrusion events that had occurred frequently at Salem and that had caused numerous safety system challenges, reactor trips, and significant conditions adverse to quality. Operator response to the April 7, 1994, grass intrusion event was also complicated by a safety injection that was caused by a spurious high steam flow signal of short duration that had been observed during three previous reactor and turbine trips but had never been fully investigated and resolved. In addition, during the transient, the atmospheric relief valve control system exhibited a recurring problem in which it had to be shifted to manual, then back to automatic after a short time delay to ensure proper operation. This problem existed for many years with no management action to correct it. Operators also did not aggressively pursue correction of longstanding problems with the rod control system that caused numerous occurrences of rods stepping into the core in half steps without appropriate process demand signals. the les water inventory decrease in

Some events that occurred during this assessment period demonstrated a weak the duestioning attitude by operators. For example, in April 1994 with the plant conditions shut down, Unit 1 operators did not question a reading of 93% on the reactor vessel water level indication system (RVLIS). When brought to their attention by NRC, operators attributed the reading to a calibration problem instead of an actual reactor vessel void. Subsequently, a void was confirmed that was and adequately eliminated by venting. Earlier in the SALP period, a cold leg subseque accumulator's level was recorded in the control room logs as being above the upper technical specification limit without a corresponding technical specification entry. However, this was not identified by either self checking or supervisory review.

Operability decisions made by the Operations staff were often weak due to a poor understanding of the design basis of safety related equipment and systems, as well as, a lack of clear guidance and training on Generic Letter 91-18. The engineering organization was not consistently consulted on many of these more difficult operability determinations. For example, an initial operability evaluation for CPR25 did not involve any consultation with the engineering organization and failed to consider the design basis requirements of the valve. Other noteworthy examples that occurred during the SALP period includ. involved weak operability determinations for degraded performance of the 1A involved weak operability determinations for degraded performance of the 1A and safety injection relief valve leakage. In addition, there were several examples over the assessment period in which operators took a non-conservative approach to entering and exiting Technical Specification limiting conditions

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for operation (LCOs) for the same underlying problem. For example, in May 1994, during a Unit 1 startup, operators made repeated entries into the Technical Specification (TS) LCO for the pressurizer vent path in response to minor leakage through two head vent valves, but inappropriately re-initialized the LCO entry each time. Operators also entered and exited a containment isolation TS LCO for the service air system twice in the same shift to perform maintenance that would have exceeded the original LCO time period.

Operations also exhibited difficulty managing and controlling outage activities. For example, operators created or contributed to a number of tagging errors. These included an operator who removed tags from a bleed steam coil drain tank pump, which allowed steam to escape through an unsecured drain line, and an operator who erroneously opened a boundary valve that allowed water to a downstream valve that was undergoing a maintenance activity. Subsequently, the licensee established corrective measures and similar occurrences have not been observed. Also, during the refueling outage in October 1993, with the spent fuel pools cross connected, operators did not identify that a pre-existing high level condition in the Unit 2 spent fuel pool masked further increases in pool level which, when such an increase occurred, resulted in an overflow of the spent fuel pool water into the fuel handling building ventilation exhaust ductwork.

Inspection activities late in the SALP period revealed that Quality Assurance surveillance of Operations was not performance-based and was ineffective in identifying significant previously existing weaknesses in the Operations department. The lack of self assessment activities within the Operations organization, coupled with ineffective independent oversight by the Quality Assurance organization, resulted in little or no feedback to the operators and their management relative to the existence of significant performance problems in Operations.

In summary, operators generally responded appropriately with good command and control to the many plant trips and operational transients that occurred over the SALP period. Likewise, they demonstrated good proficiency in making emergency declarations for events for which such declarations should have been considered. However, performance over the assessment period demonstrated significant weaknesses in several areas. Operators did not practice ownership of the plant and did not aggressively enlist other plant departments to resolve longstanding equipment problems which frequently challenged them in normal and upset plant conditions. A lack of an appropriately questioning attitude by operators resulted in anomalous indications, or conditions being unnoticed or not understood and not being acted upon. A lack of guidance for and training of operators on operability decisions resulted in some decisions being nonconservative or having weak technical bases. Examples of nonconservative approaches to entering and exiting LCOs occurred over the period. Some difficulties were experienced managing and controlling outage activities. A lack of self assessment within the Operations department coupled with ineffective independent assessment of Operations by the Quality Assurance department contributed to the continuation of performance problems throughout most of the period.

The Operations functional area is rated as Category 3.

III. PERFORMANCE ANALYSIS - MAINTENANCE

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In the previous assessment period, the Salem maintenance and surveillance functional area was rated Category 2. Personnel errors had decreased, but still caused three reactor trips and four engineered safety features actuations. A number of trends indicated continuing improvement. Three refueling outages were performed with strong planning and implementation. Improvements were noted in the preventive maintenance program, procurement, material control, and surveillance procedures quality through the procedures upgrade program.

During the latter part of this assessment period, management improved its safety focus in prioritizing and scheduling maintenance activities. In the plan-of-the-day meetings and other work planning meetings and activities involving both operations and maintenance personnel, the emphasis was on safety rather than production. Interdepartmental communication, especially between maintenance and systems engineers, improved. However, supervisors did not always communicate effectively with workers while they were in the field and during pre-job briefings as evidenced by maintenance errors involving the governor gear box oil change and turbine overspeed trip test device made during preventive maintenance work on the Number 23 auxiliary feedwater (AFW) pump. The AFW pump tripped twice during post-maintenance testing before appropriate supervisory guidance was obtained for returning the pump to service.

Management did not provide sufficient oversight and control of the maintenance programs. Salem had a high recurrent equipment failure rate indicating that corrective action effectiveness remains a problem. There were several examples of the licensee's inability to resolve longstanding equipment and system deficiencies. For example, inadequate root cause analysis and training contributed to the delay in correcting long-term deficiencies in the various radiation monitoring systems; defayed are inadequate root cause analysis also contributed to repetitive failures of the automatic control of the steam generator feedwater regulating valves (BF19) over a two year period. Supervisory control and management oversight was also lacking for the numerous groups and organizations that perform maintenance work on site. Personnel errors, problems with procedural adherence, and excessive reliance on "skills of the craft" contributed to inconsistent implementation of the maintenance program. Most recently, the licensee found that a contractor electrician cut into the wrong 4160 VAC cable. A fatality was avoided only because the affected cable was tagged out of service to support other unrelated work.

In the area of problem identification and resolution, the licensee implemented an effective way of tracking equipment problems using a process called the equipment malfunction identification system (EMIS). However, the feedback process regarding problems that occur during maintenance activities was not effectively implemented by field maintenance personnel. This primarily delayed affected the correction of deficient procedures and work packages. Feedback did not always get into the planning system and in some instances the initiator of the feedback form was not informed as to the resolution of the

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problem. Proubleshooting and implementation of the root cause program was inconsistent, even though the licensee established a good root cause capability. For example, the licensee did a good job of troubleshooting and determining the root cause of intermittent rod stepping and oscillations on the AFW pump. However, in addition to inadequate root cause analysis and failure to resolve longstanding problems cited earlier, the licensee performed inadequate troubleshooting and root cause analysis on the four electrohydraulic control power supply failures before determining the fundamental root cause of the failures. Some of the maintenance performance problems were related to conducting troubleshooting without a procedure such as the example where the ability to capture as-found defects was lost during removal of a failed emergency diesel generator cylinder liner.

The material condition of the plant improved following the licensee's establishment of the Salem Material Condition Revitalization Project. However, there remains evidence of degraded conditions in the service water intake structure and the residual heat removal pump rooms.

In general, surveillance testing activities were effective with respect to meeting the surveillance program objectives. However, the licensee failed to demonstrate the design basis capability of the emergency diesel generators to start on a single air start system while performing maintenance on the remaining air start system. Also during this period, a surveillance procedure deficiency resulted in the inadvertent discharge of a safety injection accumulator into the reactor coolant system while at low pressure.

Although the licensee completed a formal procedures upgrade program (PUP) in 1993, procedure adequacy continues to be a problem. For example, an excellent troubleshooting procedure was developed and implemented in the controls area, but a similar procedure in the mechanical maintenance area was not implemented. There were recurring maintenance problems that needed specific procedure changes which were being delayed because of an excessive procedure change backlog. In several instances, there was a planning failure to specify appropriate post-maintenance testing requirements in work order packages. This was attributed to the inadequacy of the controlling procedures for the planning process and training of planners in post-maintenance testing requirements.

The Salem in-service testing program was adequate. The use of spectrum analysis for vibration and high quality procedures were noteworthy. However, several shortcomings were identified in program oversight by station management. Many program weaknesses were identified by comprehensive and self-critical audits, but were not acted upon. The programs for inservice inspection, erosion/corrosion and steam generator leakage monitoring were adequately implemented.

In summary, weaknesses were evident in the implementation of the maintenance programs and activities, such as procedural adherence and adequacy, the feedback process, specification of post-maintenance testing requirements, and control of work activities by numerous onsite groups. Management improved its safety focus in prioritizing and scheduling maintenance activities. However, management oversight of corrective action program activities was weak as

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evidenced by the high recurrent equipment failure rates. Inconsistencies in troubleshooting activities and root cause analysis contributed to the delay in correcting recurring problems. Material condition of the plant continued to improve, but there were several areas that still need improvement. Although the in-service testing program was adequate, management did not effectively resolve associated self-assessment findings. Programs for inservice inspection, erosion/corrosion and steam generator leakage monitoring were adequately implemented.

The Maintenance functional area is rated Category 3.

IV. PERFORMANCE ANALYSIS - ENGINEERING

In the last SALP, engineering was rated Category 2. Engineering provided good support for refueling and maintenance outages and strong performance was noted in addressing day-to-day activities. The training programs for engineering personnel were excellent. Weaknesses were noted in handling of engineeringrelated nonconformances, in the erosion/corrosion program implementation and in fire protection programs. Also, while the root cause training program was found to be strong, the threshold for initiating root cause analyses was not clear or consistent.

During this period, the quality of engineering activities was inconsistent and varied significantly from activity to activity. Quality depended on the issue involved and the perceived importance of that issue by engineering and plant management and staff. Management expectations for engineering performance were clearly articulated but were implemented inconsistently throughout the organization.

Communication and coordination among the Engineering and Plant Betterment (E&PB) organization, the Technical Department of the plant staff and the balance of the plant staff were not always effective. While there was good communication and coordination of highly-visible problems, day-to-day interactions were ineffective in resolving some repetitive equipment problems that continued to challenge the operation of the facility. While close interactions occurred between the Maintenance organization and Technical Department system engineers, the engineering expertise of the E&PB organization was not always effectively engaged. Engineering did not always proactively seek out and correct system and component deficiencies before they led to increasingly challenging plant events. Further, E&PB did not effectively involve itself in support of plant operations as demonstrated by the fact that, while backlogs of its activities were well controlled, its work priorities were not well-integrated with those of the operating organization. For example, the "Engineering Critical Issues List" did not match the plant's critical issues list and was not prioritized by safety significance. Further, none of the items that were being tracked as operator work-arounds made the engineering list. Notwithstanding, significant positive engineering leadership and good quality engineering work were demonstrated in the recovery from the overhead annunciator and rod control systems problems, in the main

steam line flow monitoring modifications and in the commitment of resources toward the switchyard betterment and radiation monitoring system upgrade programs.

Design engineering procedures were comprehensive and their quality was good. Work instructions associated with modification installation were generally good. Temporary modification activities were well controlled, with installed temporary modifications tracked and periodically assessed by the system

The quality of technical support provided to the Operations and Maintenance organizations was mixed. Engineering support was good in a number of instances, such as those associated with indications of condensate pump pedestal damage, with the identification of thermal fatigue cracks in Unit 1 steam generator feedwater nozzles, and with a leaking flange joint associated with the #22 reactor coolant pump. Further, the engineering evaluation of emergency diesel generator cylinder liner cracks was comprehensive and of high quality. However, several instances were noted where engineering support in response to equipment problems was poor. Examples included the ineffective response to control air compressor problems and the lack of a timely and effective review of the main steam line pressure pulse phenomenon prior to the April 7, 1994 event.

In a number of programmatic areas, performance was good. The motor-operated valve testing program was found to be progressing well toward its planned completion date. The erosion/corrosion program improvements achieved at the end of the last SALP period were maintained in effect. The steam generator inspection program was well controlled and implemented. Engineering support to maintenance troubleshooting activities was, in general, good. The Environmental Qualifications Master List was appropriately maintained. In addition, the engineering assurance program was revised and improved during this period. Configuration baseline documents were found to be of good quality, but a licensee self-assessment noted opportunities to improve their use. In the procurement area, commercial grade dedication packages were complete and the warehouse storage areas were well maintained; however the material issuance process failed to prevent issuance of the incorrect materials to support a modification of Unit 2 power-operated relief valves and to support emergency diesel generator fuel injector stud changeouts. Also, notwithstanding the problems identified in the licensee's reaction to the April 7, 1994, event, the licensee provided for an excellent and comprehensive investigation and monitoring program for grass intrusion into the circulating water/service water intake structure.

Problems with root cause analyses continued from the last SALP period and contributed to weaknesses in resolution of long-standing problems. In several instances, such as in response to indications of ground water leakage near auxiliary feedwater system piping penetrations, to indications of operation at greater than 100% power, and to repeated steam generator feedwater pump control oil power unit problems, root cause analyses performed by the plant maintenance and technical organizations tended to focus narrowly on the symptoms of equipment problems at hand. In reaction to NRC interest or as a result of an event, senior licensee management focused on specific issues and commissioned more in-depth root cause activities, such as Significant Event Review Teams. The outcomes of these focused efforts were markedly better than those done routinely by the line organizations, indicating the licensee had the capability to perform these assessments and suggesting that the performance problem continued to be associated with the threshold established for initiating thorough root cause evaluations.

Engineering personnel, particularly reactor engineering personnel, were found to be very knowledgeable of their discipline, however system engineers were not trained in current NRC operability guidance despite the fact that they are routinely engaged in operability assessments. Personnel performance was generally good, however two noteworthy contractor control problems were noted associated with the auxiliary feedwater system controller and the primary water oxygen reduction modifications where the contractors engaged in installation activities failed to follow established station work process control procedures.

In summary, Engineering performance was inconsistent, with substantial variation in quality. The quality of the discipline design work was good, with significant engineering management focus shown in several modification activities. However, engineering work priorities did not always reflect plant needs. In several significant programmatic areas in which the Engineering organization had an important role, performance was, on balance very good. Significant problems, nonetheless were noted associated with root cause assessments and with equipment problem resolution. The fact that there existed engineering capability, that when focused by station management and brought to bear on important issues, demonstrated the ability to achieve very good performance suggested that a significant aspect of the problem was associated with the effective engagement of available engineering expertise in activities important to safe plant operations, such as in root cause assessment and equipment problem resolution. Gends like

The Engineering functional area is rated as Category 3.

PERFORMANCE ANALYSIS - PLANT SUPPORT ۷.

This functional area is new, representing a significant change from the previous SALPs. The plant support functional area covers all activities related to plant support functions, including radiological controls, emergency preparedness, security, chemistry, fire protection, and housekeeping controls.

In the previous SALP the radiological controls, emergency preparedness and security functional areas were all rated as Category 1; however a declining trend was assigned to the emergency preparedness area. Performance observations in the radiation protection area included: strong management involvement, as shown by excellent as-low-as-reasonably-achievable (ALARA) oversight; effective supervision of on-going work; and challenging occupational exposure goals. The radioactive waste, transportation and contamination control programs demonstrated continued strong performance. The chemistry, effluent and environmental monitoring programs remained highly effective. Performance in the emergency preparedness area was excellent with

a high quriity drill and exercise program, and extensive management involvement. Although the emergency plan was effectively implemented for four events requiring declarations of Unusual Events, weaknesses were identified in classifying and reporting the December 1992 loss of control room annunciator event at Salem 2. Additionally, problems with formulation of event classification and protective action recommendations during exercises were identified. The licensee maintained a very effective security program, with good management support, high quality maintenance support, excellent rapport with other plant groups, and effective audit and self-assessment programs. Although rated in conjunction with the Operations Area during the last SALP, the fire protection program exhibited some programmatic and personnel performance problems.

During the current SALP period, the licensee's radiation protection program performance continued to be a significant strength. Effective external and internal exposure control programs continued to be implemented. Effective application of engineering controls to control contamination resulted in commendably low air activity levels, resulting in low internal exposures. Continued effective ALARA program implementation was evidenced by dose reductions achieved through extensive application of temporary shielding during both unit outages, good radiation safety work coverage and pre-job briefings, and appropriate work area postings. The licensee effectively implemented the revised 10 CFR 20 by integration of the new requirements in applicable radiation protection procedures and in timely training of the work force. High quality training for radiation protection technicians and staff was evident. A very effective radioactive material and contamination control program was implemented. Radiological housekeeping was generally very good. Audits and surveillances of the radiation protection area were performancebased, performed by appropriately qualified individuals, and were effective in identifying performance problems. Corrective actions taken in response to identified problems were effective. The radioactive waste handling, processing, packaging, storage, and transportation programs continued to be very good. The licensee completed construction of a state-of-the-art radwaste storage facility. Radwaste generation reduction efforts were very effective as evidenced by the continuing downward trend in radwaste produced.

Performance in the radiological environmental monitoring and effluent control programs continued to be strong. Effective programs for measuring radioactivity in process and effluent samples were implemented as well as an effective program for the radiation environmental monitoring. Quality assurance audits were thorough and of good technical quality. Responses to audit findings were timely and identified appropriate corrective actions.

Continued excellent emergency preparedness (EP) program performance was noted during drills and exercises. An exercise strength was highlighted regarding Emergency Response Manager command and control. Effective management support was evidenced by active involvement of upper level management in the emergency response organization(ERO) qualification and drills, and rapid replacement of ERO members following recent employee layoffs. Several improvements were implemented during the period, including development of a radiologically-based protective action recommendation flow chart and improved containment boundary emergency action level, which enhanced response capability. The emergency response facilities were well equipped and generally well maintained, however, problems were identified regarding periodic efficiency tests on the high efficiency particulate filters associated with the Emergency Operations Facility and radiation monitors for the Technical Support Center heating and ventilation system being out of service for 18 months.

The licensee continued to implement a very effective security program. Management attention and involvement generally continued at a high level. Maintenance support of security equipment from the maintenance staff was some assessment aids had deteriorated to a point that even aggressive maintenance was not entirely effective in maintaining this equipment. The licensee continued to implement a good performance-oriented training and qualification program. However, personnel performance issues raised questions regarding complacency of security force members and supervisory oversight of routine security program implementation. The licensee initiated actions to address problems in this area.

The fire protection and prevention program was effectively implemented. Corrective actions put in place to address equipment and personnel performance problems highlighted in the previous SALP were effective. There was good fire-fighting equipment maintenance and surveillance. Responses to emergent equipment conditions were appropriate. Combustibles and ignition sources were well controlled. Performance during drills demonstrated the licensee's readiness and fire fighting capabilities. Audits were detailed and of appropriate depth.

In summary, the plant support functions contributed effectively to safe plant performance. Performance in the radiation protection area continued to be a significant licensee strength. Well trained technician and staff coupled with effective management resulted in aggressive ALARA program implementation with significant dose savings realized. Excellent performance in the radiological effluent and environmental monitoring programs was again noted. There was continued excellent performance in the emergency preparedness area. Security program performance continued to be a strength. Fire protection program implementation was substantially improved.

The Plant Support functional area is rated as Category 1.

SALEM EXECUTIVE SUMMARY

Overview

On July 29, 1993, the SALP board met to discuss PSE&G's performance at Salem during the period from December 29, 1991 to June 19, 1993. The board concluded that the licensee had operated the Salem units safely and that operator response to operational events was excellent. The overall performance in the Operations area was good. However, weaknesses were noted in the decisions to restart Unit 2 following the rod control system problems, in the failure to follow procedures resulting in the loss of Unit 2 annunciators, and in the inadequate oversight of the fire protection program.

PSE&G continued to implement effective radiological controls and ALARA programs during this period. The SALP board noted improvements in this functional area including strong management support and oversight. Quality Assurance audits in this area were of very good quality.

The board concluded that the Salem maintenance and surveillance programs contributed to the safe operation of the two units during the assessment period. In general, a declining number of personnel errors in both maintenance and surveillance indicated improving performance. However, the number of transients induced by component failures and the significant problems with the rod control system raise questions regarding the overall effectiveness of the maintenance and engineering support functions.

The SALP board determined that PSE&G maintained a generally strong and effective emergency preparedness (EP) program. However, the board was concerned with an apparent decline in the ability of the licensee to make correct initial Protective Action Recommendations during training, drills and annual exercises. This concern resulted in the board's assessment of a declining trend for this area. The board also concluded that PSE&G continued to maintain an effective and performance-oriented security program during this period. Overall, licensee performance in both EP and security remained excellent.

Engineering and technical support organizations provided good support for refueling and maintenance outages, and strong performance in addressing day-to-day problems. The SALP board noted that training programs for engineering personnel were excellent but that weaknesses were observed in the licensee's non-conformance, erosion/corrosion, and fire protection programs. Although the root cause training program was viewed as a strength, the board noted that the threshold for initiating actual root cause investigation was not clear or consistent.

PSE&G management continued to provide generally effective management support. Significant Event Response Team (SERT) reviews of major events have been effective. However, the board noted that in several instances, PSE&G failed to initiate adequate root cause evaluation or assessment of abnormal conditions. NRC interaction with PSE&G management was needed in a number of cases in order for full evaluation and corrective action to be taken in a timely manner. Once initiated, comprehensive assessment, root cause analysis and effective corrective

actions were implemented. Outage planning and training programs in all areas were considered strengths.

Facility Performance Analysis Summary

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	Functional Area	Rating, Trend Last Period	Rating, Trend This Period
1.	Plant Operations	2	2
2.	Radiological Controls	2, Improving	1
3.	Maintenance/Surveillance	2	2
4.	Emergency Preparedness	1	1, Declining
5.	Security	1	1
6.	Engineering/Tecnnical Support	2	2
7.	Safety Assessment/Quality Verification	2	2

Previous Assessment Period: August 1, 1990 through December 28, 1991 Present Assessment Period: December 29, 1991 through June 19, 1993

SALEM AND HOPE CREEK ISEG INPUT

Evaluation of Salem/Hope Creek ISEG equivalent organization

- I. Plant Tech Specs for Hope Creek and both Salem units require an onsite Safety Review Group (SRG), the ISEG equivalent.
 - A. Salem 1 is a pre-TMI plant. The licensee proposed a Tech Spec change in 1981 to require an SRG. The NRC approved the change in 1984.
 - B. Salem 2 and Hope Creek are post-TMI plants. The original Tech Specs for both units required SRGs.

II.

- A. The SRGs review appropriate documents with the potential for identifying issues. For example, the SRGs review plant Incident Reports (LER precedents), participate in the Operating Experience Feedback meetings, and scan available industry sources of information such as newsletters and electronic bulletin boards.
- B.
- 1. The form of the SRG product is practical and useable by line organizations. SRG issues monthly summary reports of activities. They provide independent reports on specific activities. SRG makes recommendations to line organizations; they are negotiated and tracked in the Action Tracking System.
- 2. When the SRGs make recommendations, they are sound and justified. However, during interviews line management had difficulty recalling specific safety significant recommendations made by the SRGs. Also, line management identified that, at times, SRG may identify findings without making recommendations on how to resolve the problem. Based on review of SRG monthly summaries of findings for the past year, the inspectors concluded that the SRG recommendations were sound and practical, although generally not safety significant.
- C. Line organizations tolerate/accept the SRG function. Some members of SRG are more respected for their individual accomplishments and qualifications. SRG recommendations are negotiated with line departments and subsequently tracked and implemented. Receptiveness to SRG opinions varies with the SRG member expressing the opinion (see respect comments above) and the line manager receiving the opinion. The licensee's organizational structure is designed to provide SRG independence from line organizations. Additionally, licensee Tech Specs and procedures do not prescribe or limit the SRG role to one typical of traditional QA/QC organizations. However, plant managers sometimes compromise SRG independence by often using SRG personnel to perform routine activities normally performed by line organizations. The inspectors determined that the Tech Specs and procedures do not establish a clear mission for the SRGs. As a result, SRG has no detailed guidance how to accomplish their function as defined by Tech Specs and does not develop a systematic approach to providing an assessment of the effectiveness of line organization activities. SRG rarely identifies opportunities for major improvements in plant safety. For example, the

majority of SRG findings identify minor procedure discrepancies and process or equipment deficiencies with little or no effect on nuclear safety.

- D. The inspectors were unaware of any inspection or recognition by outside entities (INPO/NRC/others) of SRG contributions.
- E. Plant organizations occasionally seek out SRG to participate in special activities. The SRG participates in most, perhaps all, Safety Evaluation Review Teams (SERTs). The SRG assists SERT reviews of plant trips and significant plant events.
- F. The plant managers frequently request SRG review of events or activities. Some reviews are related to nuclear safety; many are not.
- The Hope Creek organization performs SA/QV functions well. The Salem organization has been slow to identify issues and significant event precursors. Once the issues had been identified, the Salem organization responded with comprehensive efforts to understand and resolve them. Licensee senior management considered the Offsite Safety Review (OSR) group ineffective and assessed that SRG made some positive contributions to the plant. Licensee senior management initiated a contractor review of the effectiveness of OSR, SRG, and the quality organizations. Management intends to improve the effectiveness of these organizations. At the time of the inspection, the contractor had not completed the review of the SA/QV organizations.

III.

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SALP

On its most recent SALP report, dated 1/3/95 covering the period between 6/20/93 and 11/5/94, Salem was awarded the following SALP Scores:

Operations-3; Maintenance-3; Engineering-2; Plant Support-1;

Overall performance noted to have declined, as both Operations and Maintenance had been rated category 2 the previous SALP. Accommodation and inability to resolve long-standing equipment problems and a general lack of a questioning attitude were major concerns.

Senior Management Meeting

Recently, Salem was a full discussion plant at the June 1994 and January 1995 SMMs. It will be a full discussion plant again at the June 1995 SMM. Though not put on the watch list or sent a trending letter, the senior managers recommended that the EDO, Regional Administrator, and Director, NRR meet with the Board of Directors of PSE&G. This meeting took place on March 21.

Current Issues

The licensee has implemented a plan to change the culture at the facility and achieve meaningful and measurable performance improvements. This will be a challenging task. Over the past year, several management changes have occurred, and there will be a few more before the licensee has the management team it wants in place. Continuing problems with the feedwater system plague the operation of the plant, causing frequent power changes to effect repairs. Other equipment problems occur with high frequency precluding sustained operation at 100% power. Unit 1 has a feedwater heater leak that will limit power to 94% until the next outage.

Since the beginning of 1995, the two units have had to shut down, reduce power, or delay startup due to problems with their solid state protection system, safety valves, reactor coolant pump seals, feed pump governors, main steam atmospheric relief valves, and heater drain pump level controllers.

Recent Resident Inspector inspection reports highlight continued problems in corrective action determination and effectiveness. Inconsistent system engineering involvement and effectiveness has also been a persistent problem.

1) - Hope Ereck relance committeed - contaminated No gas tank - able to track off- ite -- brief - Mike Me Cale .. Toma Baker - entrotrating - received allegations - ancied the Union. 2) what I to taken higher the normal is containment to whit I - dide it follow perceduces - with a] contrinents - onthe pares - presenting of the form of pinto - weaks marry ment contrinents -- offe passon pressantages "

SALEM GENERATING STATION SENIOR MANAGEMENT MEETING HISTORICAL PERSPECTIVE

COMPLETED FOR UNIT 1 AND/OR UNITS

- · CONTROL ROOM MODIFICATIONS AND HUMAN FACTOR UPGRADES
- # UPGRADE OF 14,000 LINEAR FEET OF BERVICE WATER PIPING
- · CLEAN CHEMISTRY LABORATORY
- . MAJOR SWITCHY ARD EXPANSION AND UPGRADE
- . BUS INSTRUMENT INVERTER REPLACEMENT
- . NO. 1 MAIN TRANSPORMER UPGRADES
- & STEAM GENERATOR BAFETY VALVE REPLACEMENT
- . CIRCULATING WATER FIGH TROUGH REPLACEMENTS
- · CONTAINMENT STEAM GENERATOR BLOW DOWN VALVE UPGRADE
- · PRESSURCER INSULATION REPLACEMENT
- · BAFEGUARDE EQUIPMENT CONTROLLER INSTALLATION
- & LUBE OIL STORAGE FACILITY
- . INSTALLATION OF SYSTEM TO ADD CHEMICALS TO AUDILLARY FEED SYSTEM
- . UPGRADE RADWARTE PANEL (I&C)
- · REPLACEMENT OF MINCELLANEOUS CONDEMNATE STRAIMERS
- · ROD CONTROL & VDC POWER SUPPLY REPLACEMENT
- · MID LOOP INETRUMENTATION MODIFICATIONS
- · GENERATOR HYDROGEN DRYER DEW POINT MONITOR PURGE/VENT
- C DEBNEL GENERATOR HVAC IMPROVEMENTS
- . GAD FEED FUND INDEPENDANT CONTROL OIL SYSTEM
- · CONDEMNATE POLISHING BUILDING UPGRADES

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SENIOR MANAGEMENT MEETING HISTORICAL PERSPECTIVE

- . CIRCULATING WATER MECHANICAL UPGRADES
- BORIC ACID CONCENTRATION REDUCTION
- . CIRCULATING WATER AIR REMOVAL SYSTEM
- UPGRADED INTERNALS FOR PRESSURIZEF. CODE SAFETY VALVES AND ELIMINATED LOOP SEALS
- . UPGRADED PORV AND SPRAY VALVE ACTUATORS
- . UPGRADED INTERNALS OF ALL AUX FEED WATER CONTROL VALVES
- . UPGRADED WASTE GAS SYSTEM ANALYZER
- . UPGRADED CONTROL AIR AND NITROGEN VALVES TO CONTAINMENT
- . UPGRADED BORIC ACID AND PRIMARY WATER FLOW INSTRUMENTATION
- . SMALL BORE PIPING REPLACEMENT > 5,000 FEET
- . ELECTRO HYDRAULIC CONTROL PUMP UPGRADES
- . STEAM GENERATOR FEED PUMP CONTROL OIL SYSTEM UPGRADE
- INSTALLED PERMANENT BACK-UP POWER SUPPLIES TO ELIMINATE TEMPORARY POWER FEEDS
- · DUNING OUTAGES
- . UPORADED THE SEC AUTO TEST CIRCUIT
- . UPORADED THE CONTROL ROOM ANNUNCIATOR SYSTEM
- e REPLACED ROD CONTROL STEP COUNTERS
- · UPORADED MAIN STEAM FLOW MEASUREMENT

9414040-116

Corrective Maintenance Backlog Salem Station



2/22/94 18A0-35

Preventive Maintenance Overdue Salem Station (Maint Dept)



1/17/94 20TO-3MS

Preventive Maintenance Ratio Salem Station







Reliability Centered Maintenance Salem Station





Procedures Upgrade Project Salem Station



1/10/94 PUPLIC (Project Completed)

Licensee Event Reports Salem Station



2/7/94 28TO-35

Personnel LER's Salem Station



1/1/94 280-35

PSE+6Co Meeting 4/19/93 STATI LABRUNA JOE HAGAN STEVE MILTONBORGOR FRANK - Dick Suman - former exqueries not QA to onteres wive to 4 barrier concept (induided, aspectran, QA, ortaider - Lerry kyder now heads up TQM team astablishandt Vill de full time termo to re-erginan proveres - TOM - nuclear quality consul; use executive exercises of each QIT; had usto now used PT teams - Redeployment - a right signing effort 12000 - 500 Completeer scheduled & Fat-94 work -50 Will be adding 81 people at Statt, love added maite and risiaterane group, are cutter bak on contractors. Shorty to and 300-400 untrastore in 1/2 Hing page with Millesbarger safety volve for skills winx Expert S/HC perminent stop to atally winder 657 - 85-86 unplotes of DB reconstitution offert shalle - Now wing PSE+6Co person responsible for - LERS V work Mont

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MANAGEMENT MEETING - REGION I APRIL 19, 1993

AGENDA

I. UNGAMEATION CHARTS

II. PSE&G REDEPLOYMENT

III. NUCLEAR DEPARTMENT BUSINESS PLAN

IV. 1993 PERFORMANCE OBJECTIVES

V. SALEM IMPROVEMENT RESULTS
NUCLEAR DEPARTMENT

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