

Commonwealth Edison Company
Braidwood Generating Station
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January 31, 1997

Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Attn.: Document Control Desk

Subject: Braidwood Station Unit 1
NRC Docket Number: 50-456

Steam Generator Tube Removal and Insitu Testing

- References:
1. ComEd letter to Office of Nuclear Reactor Regulatory, Braidwood Station Unit 1, NRC Docket Number: 50-456, Operating Interval Between Eddy Current Inspections for Circumferential Indications in the Braidwood Unit 1 Steam Generators, dated May 24, 1996
 2. D. Lynch letter to D. Farrar transmitting Extension of the Operating Interval Between Eddy Current Inspections for Circumferential Indications in the Braidwood Unit 1 Steam Generator Tubes, dated May 22, 1996

In Reference 1, ComEd committed to perform a mid-cycle Steam Generator eddy current inspection at Braidwood Unit 1 prior to October 15, 1996. The staff approved plant operation up to October 15, 1996 subject to the acceptance of four conditions. These conditions were to:

1. Plug or sleeve all steam generator tubes with circumferential indications found in the Braidwood Unit 1 Fall 1996 outage;
2. Use an eddy current inspection methodology during the Braidwood Unit 1 Fall 1996 outage which is equivalent to, or better than that used in the Byron Unit 1 Spring 1996 refueling outage;
3. Remove and destructively analyze tubes with circumferential indications in the roll transition zone during the Braidwood Unit 1 Spring 1997 refueling outage; and
4. Conduct in-situ pressure tests of selected steam generator tubes in the Braidwood Unit 1 Fall 1996 outage.

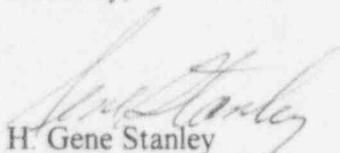
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ComEd performed a mid-cycle eddy current inspection of the Braidwood Unit 1 steam generator's hot-leg top-of-tubesheet roll transition regions during an outage that started October 12, 1996. All circumferential indications found were plugged or sleeved. The eddy current inspection methodology used was equivalent to or better than that used in the Byron Unit 1 Spring 1996 refueling outage. In-situ pressure tests were performed on twenty-three steam generator tubes to verify adequate structural integrity of the more severe circumferential indications detected. This satisfies conditions 1, 2, and 4.

During the Fall 1996 mid-cycle outage, four tubes with circumferential indications in the roll transition region were removed from the Braidwood Unit 1 steam generators for destructive analysis to determine the morphology of the circumferential indications. Therefore, condition 3 was satisfied during the Fall 1996 mid-cycle outage. ComEd does not intend to remove additional tubes with circumferential indications in the roll transition region during the Braidwood Unit 1 Spring 1997 refueling outage.

Please direct any questions regarding these commitments to Douglas S. Huston, Braidwood Licensing Supervisor, (815) 458-2801, extension 2511.

Sincerely,



H. Gene Stanley
Site Vice President
Braidwood Station

cc: D. Lynch, Senior Project Manager - NRR
R. Assa, Braidwood Project Manager - NRR
C. Phillips, Senior Resident Inspector - Braidwood
A. B. Beach, Regional Administrator - RIII
Office of Nuclear Safety - IDNS