

Attachment I

Updated Pages for Proposed Change to the Technical
Specifications Regarding Extension of Instrumentation and Miscellaneous
Surveillance Test Intervals to Accommodate 24-Month Cycles (JPTS-95-001G)

Request for Amendment Submitted Under JPN-96-003

Pages

183

185

226

239

JAFNPP

3.7 (cont'd)

2. From and after the date that one circuit of the Standby Gas Treatment System is made or found to be inoperable for any reason, the following would apply:
 - a. If in Start-up/Hot Standby, Run or Hot Shutdown mode, reactor operation or irradiated fuel handling is permissible only during the succeeding 7 days unless such circuit is sooner made operable, provided that during such 7 days all active components of the other Standby Gas Treatment Circuit shall be operable.
 - b. If in Refuel or Cold Shutdown mode, reactor operation or irradiated fuel handling is permissible only during the succeeding 31 days unless such circuit is sooner made operable, provided that during such 31 days all active components of the other Standby Gas Treatment Circuit shall be operable.
3. If Specifications 3.7.B.1 and 3.7.B.2 are not met, the reactor shall be placed in the cold condition and irradiated fuel handling operations and operations that could reduce the shutdown margin shall be prohibited.

4.7 (cont'd)

- e. Once per 24 months, manual operability of the bypass valve for filter cooling shall be demonstrated.
- f. Standby Gas Treatment System Instrumentation Calibration:

differential pressure switches	Once per 24 Months
--------------------------------	--------------------
2. When one circuit of the Standby Gas Treatment System becomes inoperable, the operable circuit shall be verified to be operable immediately and daily thereafter.

3. Intentionally Blank

JAFNPP

3.7 (cont'd)

4.7 (cont'd)

- c. Secondary containment capability to maintain a 1/4 in. of water vacuum under calm wind conditions with a filter train flow rate of not more than 6,000 cfm, shall be demonstrated once per 24 months prior to refueling.

D. Primary Containment Isolation Valves

- 1. Whenever primary containment integrity is required per 3.7.A.2, containment isolation valves and all instrument line excess flow check valves shall be operable, except as specified in 3.7.D.2. The containment vent and purge valves shall be limited to opening angles less than or equal to that specified below:

<u>Valve Number</u>	<u>Maximum Opening Angle</u>
27AOV-111	40°
27AOV-112	40°
27AOV-113	40°
27AOV-114	50°
27AOV-115	50°
27AOV-116	50°
27AOV-117	50°
27AOV-118	50°

D. Primary Containment Isolation Valves

- 1. The primary containment isolation valves surveillance shall be performed as follows:
 - a. Once per 24 months, the operable isolation valves that are power operated and automatically initiated shall be tested for simulated automatic initiation and for closure time.
 - b. Once per 24 months, the instrument line excess flow check valves shall be tested for proper operation.
 - c. At least once per quarter:
 - (1.) All normally open power-operated isolation valves (except for the main stream line and Reactor Building Closed Loop Cooling Water System (RBCLCWS) power-operated isolation valves) shall be fully closed and reopened.

4.9 BASES (cont'd)

D. Not Used

E. Battery System

Measurements and electrical tests are conducted at specified intervals to provide indication of cell condition and to determine the discharge capability of the batteries. Performance and service tests are conducted in accordance with the recommendations of IEEE 450-1995.

The battery service (duty cycle) test demonstrates the capacity of the battery to meet the system design requirements. When a service test is used on a regular basis, it will reflect maintenance practices. The FitzPatrick design duty cycle loads are determined by a LOCA concurrent with a loss of normal and reserve power.

The performance (discharge) test is a test of the constant current capacity of a battery and can be conducted with the battery in an as-found condition after being subjected to an equalizing charge. If performance testing is to be used to reflect baselined battery trending capacity, then special conditions (including equalizing) are required to establish the battery in an as known condition prior to the test. If performance testing is to be used to reflect maintenance practices as well as trending, the equalizing charge can be omitted.

The modified performance test is a composite test which envelopes both the service test and performance test requirements. The modified performance test discharge current envelopes the peak duty cycle loads of the service test

followed by a constant discharge current (temperature corrected) for the performance test.

The purpose of the modified performance test is to demonstrate the battery has sufficient capacity to meet the system design requirements and to provide trendable performance data to compare the available capacity in the battery to previous capacity test results. The modified performance test may be performed in lieu of the battery service test.

The station batteries are required for plant operation, and performing the station battery service test and performance (or modified performance) test requires the reactor to be shut down.

F. LPCI MOV Independent Power Supply

Measurement and electrical tests are conducted at specified intervals to provide indication of cell condition, to determine the discharge capability of the battery. Performance and service tests are conducted in accordance with the recommendations of IEEE 450-1995.

G. Reactor Protection Power Supplies

Functional tests of the electrical protection assemblies are conducted at specified intervals utilizing a built-in test device and once per 24 months by performing an instrument calibration which verifies operation within the limits of Section 4.9.G.

JAFNPP

3.11 (cont'd)

B. DELETED

C. Battery Room Ventilation

Battery room ventilation shall be operable on a continuous basis whenever specification 3.9.E is required to be satisfied.

1. From and after the date that one of the battery room ventilation systems is made or found to be inoperable, its associated battery shall be considered to be inoperable for purposes of specification 3.9.E.

4.11 (cont'd)

B. DELETED

C. Battery Room Ventilation

Battery room ventilation equipment shall be demonstrated operable once/week.

1. When it is determined that one battery room ventilation system is inoperable, the remaining ventilation system shall be verified operable and daily thereafter.
2. Temperature transmitters and differential pressure switches shall be calibrated once per 24 months.

Attachment II

Updated Pages for Proposed Change to the Technical
Specifications Regarding Response Time Testing Requirements (JPTS-96-006)

Request for Amendment Submitted Under JPN-96-024

Pages

30g

38

49

JAFNPP

3.1 LIMITING CONDITIONS FOR OPERATION

3.1 REACTOR PROTECTION SYSTEM

Applicability:

Applies to the instrumentation and associated devices which initiate the reactor scram.

Objective:

To assure the operability of the Reactor Protection System.

Specification:

- A. The setpoints and minimum number of instrument channels per trip system that must be operable for each position of the reactor mode switch, shall be as shown in Table 3.1-1.

4.1 SURVEILLANCE REQUIREMENTS

4.1 REACTOR PROTECTION SYSTEM

Applicability:

Applies to the surveillance of the instrumentation and associated devices which initiate reactor scram.

Objective:

To specify the type of frequency of surveillance to be applied to the protection instrumentation.

Specification:

- A. Instrumentation systems shall be functionally tested and calibrated as indicated in Tables 4.1-1 and 4.1-2 respectively.

The response time of the reactor protection system trip functions listed below shall be demonstrated to be within its limit once per 24 months. Neutron detectors are exempt from response time testing. Each test shall include at least one channel in each trip system. All channels in both trip systems shall be tested within two test intervals.

1. Reactor High Pressure (02-3PT-55A, B, C, D) *
2. Drywell High Pressure (05PT-12A, B, C, D)
3. Reactor Water Level-Low (L3) (02-3LT-101A, B, C, D) *
4. Main Steam Line Isolation Valve Closure
(29PNS-80A2, B2, C2, D2)
(29PNS-86A2, B2, C2, D2)
5. Turbine Stop Valve Closure (94PNS-101, 102, 103, 104)
6. Turbine Control Valve Fast Closure (94PS-200A, B, C, D)
7. APRM Fixed High Neutron Flux
8. APRM Flow Referenced Neutron Flux

* Sensor is eliminated from response time testing for the RPS actuation logic circuits. Response time testing and conformance to the test acceptance criteria for the remaining channel components includes trip unit and relay logic.

4.1 BASES (cont'd)

The individual sensor response time may be measured by simulating a step change of the particular parameter. This method provides a conservative value for the sensor response time, and confirms that the instrument has retained its specified electromechanical characteristics. When sensor response time is measured independently, it is necessary to also measure the remaining portion of the response time in the logic train up to the time at which the scram pilot valve solenoids de-energize. The channel response time must include all component delays in the response chain to the ATTS output relay plus the design allowance for RPS logic system response time. A response time for the RPS logic relays in excess of the design allowance is acceptable provided the overall response time does not exceed the response time limits specified in the UFSAR. The basis for excluding the neutron detectors from response time testing is provided by NRC Regulatory Guide 1.118, Revision 2, section C.5.

The sensors for the Reactor High Pressure and Reactor Water Level - Low (L3) trip functions are exempted from response time testing based on analyses provided in NEDO-32291-A, "System Analyses for the Elimination of Selected Response Time Testing".

Two instrument channels in Table 4.1-1 have not been included in Table 4.1-2. These are: mode switch in shutdown and manual scram. All of the devices or sensors associated with these scram functions are simple on-off switches and, hence, calibration during operation is not applicable.

- B. The MFLPD is checked once per day to determine if the APRM scram requires adjustment. Only a small number of control rods are moved daily and thus the MFLPD is not expected to change significantly, and thus a daily check of the MFLPD is adequate.

The sensitivity of LPRM detectors decreases with exposure to neutron flux at a slow and approximately constant rate. This is compensated for in the APRM system by calibrating twice a week using heat balance data and by calibrating individual LPRM's every 1000 effective full power hours, using TIP traverse data.

3.2 LIMITING CONDITIONS FOR OPERATION

3.2 INSTRUMENTATION

Applicability:

Applies to the plant instrumentation which either (1) initiates and controls a protective function, or (2) provides information to aid the operator in monitoring and assessing plant status during normal and accident conditions.

Objective:

To assure the operability of the aforementioned instrumentation.

Specifications:

A. Primary Containment Isolation Functions

When primary containment integrity is required, the limiting conditions of operation for the instrumentation that initiates primary containment isolation are given in Table 3.2-1.

4.2 SURVEILLANCE REQUIREMENTS

4.2 INSTRUMENTATION

Applicability:

Applies to the surveillance requirement of the instrumentation which either (1) initiates and controls protective function, or (2) provides information to aid the operator in monitoring and assessing plant status during normal and accident conditions.

Objective:

To specify the type and frequency of surveillance to be applied to the aforementioned instrumentation.

Specifications:

A. Primary Containment Isolation Functions

Instrumentation shall be functionally tested and calibrated as indicated in Table 4.2-1. System logic shall be functionally tested as indicated in Table 4.2-1.

The response time of the main steam isolation valve actuation instrumentation isolation trip functions listed below shall be demonstrated to be within their limits once per 24 months. Each test shall include at least one channel in each trip system. All channels in both trip systems shall be tested within two test intervals.

1. MSIV Closure - Reactor Low Water Level (L1) *
(02-3LT-57A,B and 02-3LT-58A,B)
2. MSIV Closure - Low Steam Line Pressure *
(02PT-134A,B,C,D)
3. MSIV Closure - High Steam Line Flow *
(02DPT-116A-D, 117A-D, 118A-D, 119A-D)

* Sensor is eliminated from response time testing for the MSIV actuation logic circuits. Response time testing and conformance to the test acceptance criteria for the remaining channel components includes trip unit and relay logic.

Attachment III

Updated Pages for Proposed Change to the Technical
Specifications Regarding Implementation of BWROG Option I-D Long-Term
Solution for Thermal Hydraulic Stability (JPTS-96-005)

Request for Amendment Submitted Under JPN-96-009

Pages

ii

JAFNPP

TABLE OF CONTENTS (Cont'd)

			<u>Page</u>
F.	ECCS-Cold Condition	F.	122
G.	Maintenance of Filled Discharge Pipe	G.	122a
H.	Average Planar Linear Heat Generation Rate (APLHGR)	H.	123
I.	Linear Heat Generation Rate (LHGR)	I.	124
J.	Thermal Hydraulic Stability	DELETED	124a
K.	Single-Loop Operation	NONE	124a

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

3.6	Reactor Coolant System	4.6	136
	A. Pressurization and Thermal Limits	A.	136
	B. DELETED		
	C. Coolant Chemistry	C.	139
	D. Coolant Leakage	D.	141
	E. Safety and Safety/Relief Valves	E.	142a
	F. Structural integrity	F.	144
	G. Jet Pumps	G.	144
	H. DELETED		
	I. Shock Suppressors (Snubbers)	I.	145b
3.7	Containment Systems	4.7	165
	A. Primary Containment	A.	165
	B. Standby Gas Treatment System	B.	181
	C. Secondary Containment	C.	184
	D. Primary Containment Isolation Valves	D.	185
3.8	Miscellaneous Radioactive Material Sources	4.8	214
3.9	Auxiliary Electrical Systems	4.9	215
	A. Normal and Reserve AC Power Systems	A.	215
	B. Emergency AC Power System	B.	216
	C. Diesel Fuel	C.	218
	D. Diesel-Generator Operability	D.	220
	E. Station Batteries	E.	221
	F. LPCI MOV Independent Power Supplies	F.	222a
	G. Reactor Protection System Electrical Protection Assemblies	G.	222c
3.10	Core Alterations	4.10	227
	A. Refueling Interlocks	A.	227
	B. Core Monitoring	B.	230
	C. Spent Fuel Storage Pool Water Level	C.	231
	D. Control Rod and Control Rod Drive Maintenance	D.	231
3.11	Additional Safety Related Plant Capabilities	4.11	237
	A. Main Control Room Ventilation	A.	237
	B. DELETED		
	C. Battery Room Ventilation	C.	239