ENCLOSURE 3

PLANT HATCH - UNITS 1 AND 2
NRC DOCKETS 50-321, 50-366
OPERATING LICENSES DPR-57, NPF-5
REQUEST TO REVISE TECHNICAL SPECIFICATIONS
IMPLEMENTATION OF THE NEW 10 CFR 20 REQUIREMENTS

PAGE CHANGE INSTRUCTIONS

ENCLOSURE 3

PLANT HATCH - UNITS 1 AND 2
NRC DOCKETS 50-321, 50-366
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REQUEST TO REVISE TECHNICAL SPECIFICATIONS
IMPLEMENTATION OF THE NEW 10 CFR 20 REQUIREMENTS

PAGE CHANGE INSTRUCTIONS

The proposed changes to the Plant Hatch Unit 1 and Unit 2 Technical Specifications will be incorporated as follows:

Unit 1

Remove Page	Insert Page
1.0-9	1.0-9
3.2-18	3.2-18
3.15-8	3.15-8
3.15-22	3.15-22*
6-15	6-15*
6-20	6-20*
6-23	6-23*
6-23a	6-23a*

Unit 2

Remove Page	Insert Page
1-8	1-8
3/4 3-44	3/4 3-44
3/4 11-8	3/4 11-8
B 3/4 11-2	B 3/4 11-2*
6-14	6-14*
6-18	6-18*
6-22	6-22*
6-22a	6-22a*

* Supersedes proposed changed Technical Specification page submitted by Georgia Power Company letter dated September 21, 1992.

Unit 1

Ty. 4 Pages

ZZ. MEMBER OF THE PUBLIC

MEMBER OF THE PUBLIC means an individual in a controlled area or UNRESTRICTED AREA. However, an individual is not a MEMBER OF THE PUBLIC during any period in which the individual receives an occupational dose.

AAA. SITE BOUNDARY

The SITE BOUNDARY shall be that line beyond which the land is not owned, based, or otherwise controlled by Georgia Power Company, as shown in figure 3.15-1.

BBB. UNRESTRICTED AREA

UNRESTRICTED AREA means an area, access to which is neither limited nor controlled by the licensee.

CCC. PURGE - PURGING

PURGE or PURGING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration, or other operating condition in such a manner that replacement air or gas is required to purify the confinement.

DDD. VENTING

VENTING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration, or other operating condition in such a manner that replacement air or gas is not provided or required during VENTING. The term "vent" used in system names, does not imply a VENTING process.

Table 3.2-8

RADIATION MONITORING SYSTEMS WHICH LIMIT RADIOACTIVITY RELEASE

Ref. No. (a)	Instrument	Trip Condition Nomenclature	Required Operable Channels per Trip System (b)	Trip Setting	Action to be taken if there are not two operable or tripped trip systems	Remarks
1.	Off-gas Post Treatment Radiation Monitors	Upscale/ Downscale	1	At a value not to exceed the equivalent of the stack release limit indicated in Specification 6.18(7)	(c) (d)	2 upscales, or 1 downscale and 1 upscale, or 2 downscales will isolate the SJAE off-gas
2.	Refueling Floor Exhaust Vent Radiation Monitors	Upscale	2	$\leq$20 mr/hr	Cease refueling operations, if in progress. Isolate the secondary containment and start the standby gas treatment system.	2 upscale will isolate the secondary containment and initiate the standby gas treatment system
3.	Reactor Bldg. Exhaust Vent Radiation Monitors	Upscale	2	$\leq$20 mr/hr	Isolate the secondary containment, start standby gas treatment system, close primary containment and vent valves.	2 upscale will isolate the secondary containment and initiate the standby gas treatment system.
4.	Control Room Intake Radiation Monitors	Downscale Hi	1	$\geq$0.015 mr/hr $\leq$1.0 mr/hr	Refer to Specifications 3.12.C. and 3.12.D.	1 upscale or 2 downscales will actuate the MCRECS in the control room pressurization mode.

RADIOACTIVE EFFLUENTS

LIQUID HOLDUP TANKS

LIMITING CONDITION FOR OPERATION

3.15.1.4[a] The contents within any outside temporary tank shall be limited to ≤10 curies, excluding tritium and dissolved or entrained noble gases.

APPLICABILITY

At all times. This specification does not apply to disposable liners used for shipment of radioactive waste.

ACTION

a. With the contents within any outside temporary tank exceeding the above limit, immediately suspend all additions of radioactive material to the tank and within 48 hours reduce the tank contents within the limit and provide notification to the Commission pursuant to Specification 6.9.1.8.

SURVEILLANCE REQUIREMENTS

4.15.1.4 The quantity of radioactive material contained in any outside temporary tank shall be determined to be within the above limit by analyzing a sample of each batch of radioactive material prior to its addition to the tank.

a. An outside temporary tank is not surrounded by liners, dikes, or walls that are capable of holding the tank contents and not having tank overflows and drains connected to the liquid radwaste treatment system.

3/4.15.1.3 LIQUID WASTE TREATMENT

This specification transferred to the ODCM per NRC Generic Letter 89-01.

3/4.15.1.4 LIQUID HOLDUP TANKS

Restricting the quantity of radioactive material contained in the specified tanks provides assurance that in the event of an uncontrolled release of the tanks' contents, the resulting concentrations would be less than the limits of 10 CFR Part 20.1302(b)(2)(i) at the nearest surface water supply in an UNRESTRICTED AREA.

3/4.15.2 GASEOUS EFFLUENTS

3/4.15.2.1 DOSE RATE

This specification transferred to the ODCM per NRC Generic Letter 89-01.

ANNUAL REPORTS (Continued)

6.9.1.5 Reports required on an annual basis shall include:

a. A tabulation on an annual basis of the number of station, utility and other personnel, including contractors, receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions,[2] e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

b. Documentation of all challenges to safety/relief valves.

c. The results of specific activity analysis in which the primary coolant exceeded the limits of Specification 3.6.F.1. The following information shall be included: (1) Reactor power history starting 48 hours prior to the first sample in which the limit was exceeded; (2) Results of the last isotopic analysis for radioiodine performed prior to exceeding the limit, results of analysis while limit was exceeded and results of one analysis after the radioiodine activity was reduced to less than limit. Each result should include date and time of sampling and the radioiodine concentrations; (3) Cleanup system flow history starting 48 hours prior to the first sample in which the limit was exceeded; (4) Graph of the I-131 concentration and one other radioiodine isotope concentration in microcuries per gram as a function of time for the duration of the specific activity above the steady-state level; and (5) The time duration when the specific activity of the primary coolant exceeded the radioiodine limit.

d. Any other unit unique reports required on an annual basis.

ANNUAL RADIOLOGICAL ENVIRONMENTAL SURVEILLANCE REPORT[a]

6.9.1.6 The Annual Radiological Environmental Surveillance Report covering the radiological environmental surveillance activities related to the plant during the previous calendar year shall be submitted before May 1 of each year. The report shall include summaries, interpretations, and analyses of trends of the results of the Radiological Environmental Monitoring Program for the reporting period. The material provided shall be consistent with the objectives outlined in the ODCM and Sections IV.B.2, IV.B.3, and IV.C of Appendix I to 10 CFR Part 50.

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a. A single submittal may be made for a multiple-unit station. The submittal should combine those sections common to all units at the station.

[2]This tabulation supplements the requirements of 20.2206 of 10 CFR Part 20.

RECORD RETENTION (Continued)

c. Records of radiation exposure for all individuals entering radiation control areas.

d. Records of gaseous and liquid radioactive material released to the environs.

e. Records of transient or operational cycles for those unit components identified in Table 5.0.G-1.

f. Records of reactor tests and experiments.

g. Records of training and qualification for current members of the unit staff.

h. Records of in-service inspections performed pursuant to these Technical Specifications.

i. Records of Quality Assurance activities required by the QA Manual.

j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.

k. Records of meetings of the PRB and the SRB.

l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.15.

m. Records of analyses required by the Radiological Environmental Monitoring Program.

n. Records of the service lives of all safety-related hydraulic and mechanical snubbers including the date at which the service life commences and associated installation and maintenance records.

o. Records of reviews performed for changes made to the OFFSITE DOSE CALCULATION MANUAL and the PROCESS CONTROL PROGRAM.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1. In lieu of the "control device" or "alarm signal" required by paragraph 20.1601(a) of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit*. Any individual or group of individuals permitted

*Health Physics personnel, or personnel escorted by Health Physics personnel in accordance with approved emergency procedures, shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they comply with approved radiation protection procedures for entry into high radiation areas.

6.16 POST-ACCIDENT SAMPLING AND ANALYSIS

A program shall be established, implemented, and maintained to ensure the capability to obtain and analyze samples of reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere under accident conditions.

The program shall include the following:

(1) Training of personnel,

(2) Procedures for sampling and analysis, and

(3) Provisions for maintenance of sampling and analysis equipment.

6.17 OFFSITE DOSE CALCULATION MANUAL

6.17.1 Licensee-initiated changes to the ODCM shall:

a. Be documented and records of reviews performed shall be retained as required by Technical Specification 6.10.2.n. This documentation shall contain:

1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s), and

2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.

b. Become effective after review and acceptance by the PRB and the approval of the General Manager-Nuclear Plant.

c. Be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of, or concurrent with, the Semiannual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.

6.18 RADIOACTIVE EFFLUENTS CONTROL PROGRAM

A program shall be established, implemented, and maintained conforming with 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to MEMBERS OF THE PUBLIC from radioactive effluents as low as reasonably achievable. The program (1) shall be contained in the ODCM, (2) shall be implemented by operating procedures, and (3) shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

1) Limitations on the OPERABILITY of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the ODCM,

2) Limitations at all times on the concentrations of radioactive material released in liquid effluents to UNRESTRICTED AREAS conforming to 10 times the concentrations stated in 10 CFR Part 20, Appendix B (to paragraphs 20.1001 - 20.2401), Table 2, Column 2,

3) Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR 20.1302 and with the methodology and parameters in the ODCM,

RADIOACTIVE EFFLUENTS CONTROL PROGRAM (Continued)

4) Limitations on the annual and quarterly doses or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released from each unit to UNRESTRICTED AREAS conforming to Appendix I to 10 CFR Part 50,

5) Determination of cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the ODCM at least every 31 days,

6) Limitations on the OPERABILITY and use of the liquid and gaseous effluent treatment systems to ensure that the appropriate portions of these systems are used to reduce releases of radioactivity when the projected doses in a 31-day period would exceed 2 percent of the guidelines for the annual DOSE or dose commitment conforming to Appendix I to 10 CFR Part 50,

7) Limitations at any time on the concentrations of radioactive material released in gaseous effluents to areas beyond the SITE BOUNDARY conforming to 10 times the concentrations stated in 10 CFR Part 20, Appendix B (to paragraphs 20.1001 - 20.2401), Table 2, Column 1, which corresponds to a dose rate of 500 mrem/year total effective dose equivalent,

8) Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from each unit to areas beyond the SITE BOUNDARY conforming to Appendix I to 10 CFR Part 50,

9) Limitations on the annual and quarterly doses to a MEMBER OF THE PUBLIC from Iodine-131, Iodine-133, tritium, and all radionuclides in particulate form with half-lives greater than 8 days in gaseous effluents released from each unit to areas beyond the SITE BOUNDARY conforming to Appendix I to 10 CFR Part 50, and

10) Limitations on the annual dose or dose commitment to any MEMBER OF THE PUBLIC due to releases of radioactivity and to radiation from uranium fuel cycle sources conforming to 40 CFR Part 190.

6.19 RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

A program shall be established, implemented, and maintained to monitor the radiation and radionuclides in the environs of the plant. The program shall provide (1) representative measurements of radioactivity in the highest potential exposure pathways, and (2) verification of the accuracy of the effluent monitoring program and modeling of environmental exposure pathways. The program shall (1) be contained in the ODCM, (2) conform to the guidance of Appendix I to 10 CFR Part 50, and (3) include the following:

1) Monitoring, sampling, analysis, and reporting of radiation and radionuclides in the environment in accordance with the methodology and parameters in the ODCM,

2) A Land Use Census to ensure that changes in the use of areas at and beyond the SITE BOUNDARY are identified and that modifications to the monitoring program are made if required by the results of this census, and

3) Participation in an Interlaboratory Comparison Program to ensure that independent checks on the precision and accuracy of the measurements of radioactive materials in environmental sample matrices are performed as part of the quality assurance program for environmental monitoring.

Unit 2

Typed Pages

MEMBER OF THE PUBLIC

MEMBER OF THE PUBLIC means an individual in a controlled area or UNRESTRICTED AREA. However, an individual is not a MEMBER OF THE PUBLIC during any period in which the individual receives an occupational dose.

SITE BOUNDARY

The SITE BOUNDARY shall be that line beyond which the land is not owned, leased, or otherwise controlled by Georgia Power Company, as shown in figure 3.11-1.

UNRESTRICTED AREA

UNRESTRICTED AREA means an area, access to which is neither limited nor controlled by the licensee.

DOSE EQUIVALENT IODINE

The DOSE EQUIVALENT I-131 shall be that concentration of I-131 (microcurie/gram), which alone would produce the same thyroid dose as the quantity and isotopic mixture of I-131, I-132, I-133, I-134, and I-135 actually present. The thyroid dose conversion factors used for this calculation shall be those listed in table III of TID-14844 or those in NRC Regulatory Guide 1.109, Revision 1, October 1977.

PURGE - PURGING

PURGE or PURGING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration, or other operating condition in such a manner that replacement air or gas is required to purify the confinement.

TABLE 3.3.6.1-1

RADIATION MONITORING INSTRUMENTATION

INSTRUMENTATION	MINIMUM CHANNELS OPERABLE	APPLICABLE OPERATIONAL CONDITIONS	ALARM/TRIP SETPOINT	MEASUREMENT RANGE	ACTION
1. Off-Gas Post-Treatment Monitors (2D11-K615 A, B)	2	1, 2	(a)	10^{1} to 10^{6} cps	50
2. Control Room Intake Monitors (1Z41-R615 A, B)	2	1, 2, 3, 4, 5	1 mr/hr	0.01 to 100 mr/hr	51

(a) Value not to exceed the equivalent of the stack release limit indicated in Specification 6.1E(7).

RADIOACTIVE EFFLUENTS

LIQUID HOLDUP TANKS

LIMITING CONDITION FOR OPERATION

3.11.1.4[a] The contents within any outside temporary tank shall be limited to ≤ 10 curies, excluding tritium and dissolved or entrained noble gases.

APPLICABILITY

At all times. This specification does not apply to disposable liners used for shipment of radioactive waste.

ACTION

a. With the contents within any outside temporary tank exceeding the above limit, immediately suspend all additions of radioactive material to the tank and within 48 hours reduce the tank contents within the limit and provide notification to the Commission pursuant to Specification 6.9.1.8.

SURVEILLANCE REQUIREMENTS

4.11.1.4 The quantity of radioactive material contained in any outside temporary tank shall be determined to be within the above limit by analyzing a sample of each batch of radioactive material prior to its addition to the tank.

a. An outside temporary tank is not surrounded by liners, dikes, or walls that are capable of holding the tank contents and not having tank overflows and drains connected to the liquid radwaste treatment system.

3/4.11.1.3 LIQUID WASTE TREATMENT

This specification transferred to the ODCM per NRC Generic Letter 89-01.

3/4.11.1.4 LIQUID HOLDUP TANKS

Restricting the quantity of radioactive material contained in the specified tanks provides assurance that in the event of an uncontrolled release of the tanks' contents, the resulting concentrations would be less than the limits of 10 CFR Part 20.1302(b)(2)(i), at the nearest surface water supply in an UNRESTRICTED AREA.

3/4.11.2 GASEOUS EFFLUENTS

3/4.11.2.1 DOSE RATE

This specification transferred to the ODCM per NRC Generic Letter 89-01.

ANNUAL REPORTS (Continued)

6.9.1.5 Reports required on an annual basis shall include:

a. A tabulation on an annual basis of the number of station, utility and other personnel, including contractors, receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions,[2] e.g., reactor operations and surveillance inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

b. Documentation of all challenges to safety/relief valves.

c. The results of specific activity analysis in which the primary coolant exceeded the limits of Specification 3.4.5. The following information shall be included: (1) Reactor power history starting 48 hours prior to the first sample in which the limit was exceeded; (2) Results of the last isotopic analysis for radioiodine performed prior to exceeding the limit, results of analysis while limit was exceeded and results of one analysis after the radioiodine was reduced to less than the limit. Each result should include date and time of sampling and the radioiodine concentrations; (3) Cleanup system flow history starting 48 hours prior to the first sample in which the limit was exceeded; (4) Graph of the I-131 concentration and one other radioiodine isotope concentration in microcuries per gram as a function of time for the duration of the specific activity above the steady-state level; and (5) The time duration when the specific activity of the primary coolant exceeded the radioiodine limit.

d. Any other unit unique reports required on an annual basis.

ANNUAL RADIOLOGICAL ENVIRONMENTAL SURVEILLANCE REPORT[a]

6.9.1.6 The Annual Radiological Environmental Surveillance Report covering the radiological environmental surveillance activities related to the plant during the previous calendar year shall be submitted before May 1 of each year. The report shall include summaries, interpretations, and analyses of trends of the results of the Radiological Environmental Monitoring Program for the reporting period. The material provided shall be consistent with the objectives outlined in the ODCM and Sections IV.B.2, IV.B.3, and IV.C of Appendix I to 10 CFR Part 50.

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a. A single submittal may be made for a multiple-unit station. The submittal should combine those sections common to all units at the station.

[2]This tabulation supplements the requirements of 20.2206 of 10 CFR Part 20.

RECORD RETENTION (Continued)

c. Records of radiation exposure for all individuals entering radiation control areas.

d. Records of gaseous and liquid radioactive material released to the environs.

e. Records of transient or operational cycles for those unit components identified in Table 5.7.1-1.

f. Records of reactor tests and experiments.

g. Records of training and qualification for current members of the unit staff.

h. Records of in-service inspections performed pursuant to these Technical Specifications.

i. Records of Quality Assurance activities required by the QA Manual.

j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.

k. Records of meetings of the PRB and the SRB.

l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.15.

m. Records of analyses required by the Radiological Environmental Monitoring Program.

n. Records of the service lives of all safety-related hydraulic and mechanical snubbers, including the date at which the service life commences and associated installation and maintenance records.

o. Records of reviews performed for changes made to the OFFSITE DOSE CALCULATION MANUAL and the PROCESS CONTROL PROGRAM.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.1601(a) of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit*. Any individual or group of individuals permitted

*Health Physics personnel, or personnel escorted by Health Physics personnel in accordance with approved emergency procedures, shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they comply with approved radiation protection procedures for entry into high radiation areas.

6.16 POST-ACCIDENT SAMPLING AND ANALYSIS

A program shall be established, implemented, and maintained to ensure the capability to obtain and analyze samples of reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere under accident conditions.

The program shall include the following:

(1) Training of personnel,

(2) Procedures for sampling and analysis, and

(3) Provisions for maintenance of sampling and analysis equipment.

6.17 OFFSITE DOSE CALCULATION MANUAL

6.17.1 Licensee-initiated changes to the ODCM shall:

a. Be documented and records of reviews performed shall be retained as required by Technical Specification 6.10.2.o. This documentation shall contain:

1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and

2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.

b. Become effective after review and acceptance by the PRB and the approval of the General Manager-Nuclear Plant

c. Be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of, or concurrent with, the Semiannual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.

6.18 RADIOACTIVE EFFLUENTS CONTROL PROGRAM

A program shall be established, implemented, and maintained conforming with 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to MEMBERS OF THE PUBLIC from radioactive effluents as low as reasonably achievable. The program (1) shall be contained in the ODCM, (2) shall be implemented by operating procedures, and (3) shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

1) Limitations on the OPERABILITY of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the ODCM,

6.18 RADIOACTIVE EFFLUENTS CONTROLS PROGRAM (Continued)

2) Limitations at all times on the concentrations of radioactive material released in liquid effluents to UNRESTRICTED AREAS conforming to 10 times the concentrations stated in 10 CFR Part 20, Appendix B (to paragraphs 20.1001 - 20.2401), Table 2, Column 2,

3) Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR 20.1302 and with the methodology and parameters in the ODCM,

4) Limitations on the annual and quarterly doses or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released from each unit to UNRESTRICTED AREAS conforming to Appendix I to 10 CFR Part 50,

5) Determination of cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the ODCM at least every 31 days,

6) Limitations on the OPERABILITY and use of the liquid and gaseous effluent treatment systems to ensure that the appropriate portions of these systems are used to reduce releases of radioactivity when the projected doses in a 31-day period would exceed 2 percent of the guidelines for the annual dose or dose commitment conforming to Appendix I to 10 CFR Part 50,

7) Limitations at all times on the concentrations of radioactive material released in gaseous effluents to areas beyond the SITE BOUNDARY conforming to 10 times the concentrations stated in 10 CFR Part 20, Appendix B (to paragraphs 20.1001 - 20.2401), Table 2, Column 1, which corresponds to a dose rate of 500 mrem/year total effective dose equivalent,

8) Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from each unit to areas beyond the SITE BOUNDARY conforming to Appendix I to 10 CFR Part 50,

9) Limitations on the annual and quarterly doses to a MEMBER OF THE PUBLIC from Iodine-131, Iodine-133, tritium, and all radionuclides in particulate form with half-lives greater than 8 days in gaseous effluents released from each unit to areas beyond the SITE BOUNDARY conforming to Appendix I to 10 CFR Part 50, and

10) Limitations on the annual dose or dose commitment to any MEMBER OF THE PUBLIC due to releases of radioactivity and to radiation from uranium fuel cycle sources conforming to 40 CFR Part 190.

6.19 RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM

A program shall be established, implemented, and maintained to monitor the radiation and radionuclides in the environs of the plant. The program shall provide (1) representative measurements of radioactivity in the highest potential exposure pathways, and (2) verification of the accuracy of the effluent monitoring program and modeling of environmental exposure pathways. The program shall (1) be contained in the ODCM, (2) conform to the guidance of Appendix I to 10 CFR Part 50, and (3) include the following:

Unit 1

Marked Pages

ZZ. MEMBER(S) OF THE PUBLIC

MEMBER(S) OF THE PUBLIC ~~shall include all persons who are not occupationally associated with the plant. This category does not include employees of the utility, its contractors, or its vendors. Also excluded from this category are persons who enter the site to service equipment or to make deliveries. This category does include persons who use portions of the site for recreational, occupational, or other purposes not associated with the plant.~~ means an individual in a controlled area or UNRESTRICTED AREA. However, an individual is not a MEMBER OF THE PUBLIC during any period in which the individual receives an occupational dose.

AAA. SITE BOUNDARY

The SITE BOUNDARY shall be that line beyond which the land is not owned, based, or otherwise controlled by Georgia Power Company, as shown in figure 3.15-1.

BBB. UNRESTRICTED AREA

~~An~~ UNRESTRICTED AREA ~~shall be any area at or~~ beyond ~~the SITE BOUNDARY to which access for purposes of protection of individuals from exposure to radiation and radioactive materials is not controlled by the licensee. This includes any area within the SITE BOUNDARY used for residential quarters or for long-term industrial, commercial, institutional, and/or recreational purposes~~ means an area, access to which is neither limited nor controlled by the licensee.

CCC. PURGE - PURGING

PURGE or PURGING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration, or other operating condition in such a manner that replacement air or gas is required to purify the confinement.

DDD. VENTING

VENTING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration, or other operating condition in such a manner that replacement air or gas is not provided or required during VENTING. The term "vent" used in system names, does not imply a VENTING process.

Table 3.2-8

RADIATION MONITORING SYSTEMS WHICH LIMIT RADIOACTIVITY RELEASE

Ref. No. (a)	Instrument	Trip Condition Nomenclature	Required Operable Channels per Trip System (b)	Trip Setting	Action to be taken if there are not two operable or tripped trip systems	Remarks
1.	Off-gas Post Treatment Radiation Monitors	Upscale/ Downscale	1	At a value not to exceed the equivalent of the stack release limit indicated in ~~Environmental Tech Specs~~	(c) (d)	2 upscales, or 1 downscale and 1 upscale, or 2 downscales will isolate the SJAE off-gas
2.	Refueling Floor Exhaust Vent Radiation Monitors	Upscale	2	~~At a value not to exceed the equivalent of the stack release limit indicated in Environmental Tech Specs~~ ≤20 mr/hr	Cease refueling operations, if in progress. Isolate the secondary containment and start the standby gas treatment system.	2 upscale will isolate the secondary containment and initiate the standby gas treatment system
3.	Reactor Bldg. Exhaust Vent Radiation Monitors	Upscale	2	≤20 mr/hr	Isolate the secondary containment, start standby gas treatment system, close primary containment and vent valves.	2 upscale will isolate the secondary containment and initiate the standby gas treatment system.
4.	Control Room Intake Radiation Monitors	Downscale Hi	1	≥0.015 mr/hr ≤1.0 mr/hr	Refer to Specifications 3.12.C. and 3.12.D.	1 upscale or 2 downscales will actuate the MCRECS in the control room pressurization mode.

Specification 6.18(7)

≤20 mr/hr

RADIOACTIVE EFFLUENTS

LIQUID HOLDUP TANKS

LIMITING CONDITION FOR OPERATION

3.15.1.4[a] The contents within any outside temporary tank shall be limited to ≤10 curies, excluding tritium and dissolved or entrained noble gases.

APPLICABILITY

At all times. This specification does not apply to disposable liners used for shipment of radioactive waste.

ACTION

a. With the contents within any outside temporary tank exceeding the above limit, immediately suspend all additions of radioactive material to the tank and within 48 hours reduce the tank contents within the limit and provide notification to the Commission pursuant to Specification 6.9.1.8.

~~b. The provisions of Specification 6.9.1.13(b) are not applicable.~~

SURVEILLANCE REQUIREMENTS

4.15.1.4 The quantity of radioactive material contained in any outside temporary tank shall be determined to be within the above limit by analyzing a sample of each batch of radioactive material prior to its addition to the tank.

a. An outside temporary tank is not surrounded by liners, dikes, or walls that are capable of holding the tank contents and not having tank overflows and drains connected to the liquid radwaste treatment system.

treatment systems, the liquid effluents from the shared systems are proportioned among the units sharing that system. DELETED

3/4.15.1.3 LIQUID WASTE TREATMENT

This specification transferred to the ODCM per NRC Generic Letter 89-01

The OPERABILITY of the liquid radwaste treatment system ensures that the system will be available for use whenever liquid effluents require treatment prior to release to UNRESTRICTED AREAS. The requirements that the appropriate portions of this system be used when specified provides assurance that the releases of radioactive materials in liquid effluents will be kept "As Low As Reasonably Achievable". This specification implements the requirements of 10 CFR Part 50.36(a), General Design Criterion 60 of Appendix A to 10 CFR Part 50; and design objective Section II.D of Appendix I to 10 CFR Part 50. The specified limits governing the use of appropriate portions of the liquid radwaste treatment system were specified as a suitable fraction of the guide set forth in Section II.A of Appendix I, 10 CFR Part 50, for liquid effluents. DELETED

3/4.15.1.4 LIQUID HOLDUP TANKS

Restricting the quantity of radioactive material contained in the specified tanks provides assurance that in the event of an uncontrolled release of the tanks' contents, the resulting concentrations would be less than the limits of 10 CFR Part ~~20, Appendix A, Table II (column 2)~~ 20.1302(b)(2)(i) at the nearest surface water supply in an UNRESTRICTED AREA.

3/4.15.2 GASEOUS EFFLUENTS

3/4.15.2.1 DOSE RATE

This specification transferred to the ODCM per NRC Generic Letter 89-01

This specification is provided to ensure that at all times the dose rate at the exclusion area boundary from gaseous effluents from all onsite units will be within the annual dose limits of 10 CFR Part 20 for UNRESTRICTED AREAS. The annual dose limits are the doses associated with the concentrations of 10 CFR Part 20, Appendix B, Table II. These limits provide reasonable assurance that radioactive material discharged in gaseous effluents will not result in the exposure of an individual in an UNRESTRICTED AREA, either within or outside the exclusion area boundary, to annual average concentrations exceeding the limits specified in Appendix B, Table II of 10 CFR Part 20 (10 CFR Part 20.106(b)). For individuals who may at times be within the exclusion area boundary, the occupancy of the individual will be sufficiently low to compensate for any increase in the atmospheric diffusion factor above that for the exclusion area DELETED

ANNUAL REPORTS (Continued)

6.9.1.5. Reports required on an annual basis shall include:

a. A tabulation on an annual basis of the number of station, utility and other personnel, including contractors, receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions,[b] e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

b. Documentation of all challenges to safety/relief valves.

c. The results of specific activity analysis in which the primary coolant exceeded the limits of Specification 3.6.F.1. The following information shall be included: (1) Reactor power history starting 48 hours prior to the first sample in which the limit was exceeded; (2) Results of the last isotopic analysis for radioiodine performed prior to exceeding the limit, results of analysis while limit was exceeded and results of one analysis after the radioiodine activity was reduced to less than limit. Each result should include date and time of sampling and the radioiodine concentrations; (3) Cleanup system flow history starting 48 hours prior to the first sample in which the limit was exceeded; (4) Graph of the I-131 concentration and one other radioiodine isotope concentration in microcuries per gram as a function of time for the duration of the specific activity above the steady-state level; and (5) The time duration when the specific activity of the primary coolant exceeded the radioiodine limit.

d. Any other unit unique reports required on an annual basis.

ANNUAL RADIOLOGICAL ENVIRONMENTAL SURVEILLANCE REPORT[a] Insert 3

6.9.1.6 Routine radiological environmental surveillance reports covering the radiological environmental surveillance activities related to the plant during the previous calendar year shall be submitted prior to May 1 of each year. A single report may fulfill this requirement for both units.

6.9.1.7 The Annual Radiological Environmental Surveillance Report shall include summaries, interpretations, and statistical evaluation of the results of the radiological environmental surveillance activities for the reporting period, including (as appropriate) a comparison with the preoperational studies, operational controls, previous environmental surveillance reports, and an assessment of any observed impacts of the plant operation on the environment. The reports shall also include the

a. A single submittal may be made for a multiple-unit station. The submittal should combine those sections common to all units at the station.

[b]This tabulation supplements the requirements of ~~20.407~~ of 10 CFR Part 20. 20.2206

RECORD RETENTION (Continued)

c. Records of radiation exposure for all individuals entering radiation control areas.

d. Records of gaseous and liquid radioactive material released to the environs.

e. Records of transient or operational cycles for those unit components identified in Table 5.0.6-1.

f. Records of reactor tests and experiments.

g. Records of training and qualification for current members of the unit staff.

h. Records of in-service inspections performed pursuant to these Technical Specifications.

i. Records of Quality Assurance activities required by the QA Manual.

j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.

k. Records of meetings of the PRB and the SRB.

l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.15.

m. Records of analyses required by the Radiological Environmental Monitoring Program.

n. Records of the service lives of all safety-related hydraulic and mechanical snubbers including the date at which the service life commences and associated installation and maintenance records.

o.

Insert 5

6.11. RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12. HIGH RADIATION AREA

20.1601(a)

6.12.1. In lieu of the "control device" or "alarm signal" required by paragraph ~~20.203(c)(2)~~ of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit*. Any individual or group of individuals permitted

*Health Physics personnel, or personnel escorted by Health Physics personnel in accordance with approved emergency procedures, shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they comply with approved radiation protection procedures for entry into high radiation areas.

6.16. POST-ACCIDENT SAMPLING AND ANALYSIS

A program shall be established, implemented, and maintained to ensure the capability to obtain and analyze samples of reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere under accident conditions.

The program shall include the following:

(1) Training of personnel,

(2) Procedures for sampling and analysis, and

(3) Provisions for maintenance of sampling and analysis equipment.

6.17. OFFSITE DOSE CALCULATION MANUAL

6.17.1. Licensee-initiated changes to the ODCM shall: Insert 6

a. Be submitted to the Commission in the semi-annual effluent release report for the period in which the change(s) was made effective. This submittal shall contain:

1. Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information. Information submitted should consist of a package of those ODCM pages to be changed, with each page numbered and provided with an approval and date box, together with appropriate analyses or evaluations justifying the change(s);

2. A determination that the change will not reduce the accuracy or reliability of dose calculations or setpoint determinations; and

3. Documentation that the change has been reviewed and found acceptable by the PRB.

b. Become effective upon review and acceptance by the PRB.

Insert 7

Insert 6

a. Be documented and records of reviews performed shall be retained as required by Technical Specification 6.10.2.o. This documentation shall contain:

   1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and

   2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR ~~20.106~~, 20.1302 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.

b. Become effective after review and acceptance by the PRB and the approval of the General Manager-Nuclear Plant.

c. Be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Semiannual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g. month/year) the change was implemented.

Insert 7

6.18 RADIOACTIVE EFFLUENTS CONTROLS PROGRAM

A program shall be established, implemented, and maintained conforming with 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to MEMBERS OF THE PUBLIC from radioactive effluents as low as reasonably achievable. The program (1) shall be contained in the ODCM, (2) shall be implemented by operating procedures, and (3) shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

1) Limitations on the operability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the ODCM,

2) Limitations at all times on the concentrations of radioactive material released in liquid effluents to UNRESTRICTED AREAS conforming to ~~10 CFR Part 20, Appendix B, Table II, Column 2~~, 10 times the concentrations stated in 10 CFR Part 20, Appendix B (to paragraphs 20.1001-20.2401), Table 2, Column 2,

3) Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR ~~20.106~~ 20.1302 and with the methodology and parameters in the ODCM,

4) Limitations on the annual and quarterly doses or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released from each unit to UNRESTRICTED AREAS conforming to Appendix I to 10 CFR Part 50,

5) Determination of cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the ODCM at least every 31 days,

6) Limitations on the operability and use of the liquid and gaseous effluent treatment systems to ensure that the appropriate portions of these systems are used to reduce releases of radioactivity when the projected doses in a 31-day period would exceed 2 percent of the guidelines for the annual dose or dose commitment conforming to Appendix I to 10 CFR Part 50,

7) Limitations at all times on the ~~dose rate resulting from~~ concentrations of radioactive material released in gaseous effluents to areas beyond the SITE BOUNDARY conforming to ~~the doses associated with 10 CFR Part 20, Appendix B, Table II, Column 1,~~ 10 times the concentrations stated in 10 CFR Part 20, Appendix B (to paragraphs 20.1001 - 20.2401), Table 2, Column 1, which corresponds to a dose rate of 500 mrem/year total effective dose equivalent,

Unit 2

Marked Pages

MEMBER(S) OF THE PUBLIC

MEMBER(S) OF THE PUBLIC ~~shall include all persons who are not occupationally associated with the plant. This category does not include employees of the utility, its contractors, or its vendors. Also excluded from this category are persons who enter the site to service equipment or to make deliveries. This category does include persons who use portions of the site for recreational, occupational, or other purposes not associated with the plant.~~ means an individual in a controlled area or UNRESTRICTED AREA. However, an individual is not a MEMBER OF THE PUBLIC during any period in which the individual receives an occupational dose.

SITE BOUNDARY

The SITE BOUNDARY shall be that line beyond which the land is not owned, leased, or otherwise controlled by Georgia Power Company, as shown in figure 3.11-1.

UNRESTRICTED AREA

~~An~~ UNRESTRICTED AREA ~~shall be any area at or beyond the SITE BOUNDARY to which access for purposes of protection of individuals from exposure to radiation and radioactive materials is not controlled by the licensee. This includes any area within the SITE BOUNDARY used for residential quarters or for long-term industrial, commercial, institutional, and/or recreational purposes.~~ means an area, access to which is neither limited nor controlled by the licensee.

DOSE EQUIVALENT IODINE

The DOSE EQUIVALENT I-131 shall be that concentration of I-131 (microcurie/gram), which alone would produce the same thyroid dose as the quantity and isotopic mixture of I-131, I-132, I-133, I-134, and I-135 actually present. The thyroid dose conversion factors used for this calculation shall be those listed in table III of TID-14844 or those in NRC Regulatory Guide 1.109, Revision 1, October 1977.

PURGE - PURGING

PURGE or PURGING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration, or other operating condition in such a manner that replacement air or gas is required to purify the confinement.

TABLE 3.3.6.1-1

RADIATION MONITORING INSTRUMENTATION

INSTRUMENTATION	MINIMUM CHANNELS OPERABLE	APPLICABLE OPERATIONAL CONDITIONS	ALARM/TRIP SETPOINT	MEASUREMENT RANGE	ACTION
1. Off-Gas Post-Treatment Monitors (2D11-K615 A, B)	2	1, 2	(a)	10^{-1} to 10^{4} cps	50
2. Control Room Intake Monitors (1Z41-R6.5 A, B)	2	1, 2, 3, 4, 5	1 mr/hr	0.01 to 100 mr/hr	51

(a) Value not to exceed the equivalent of the stack release limit indicated in ~~the Environmental Technical Specifications~~ Specification 6.18(7).

RADIOACTIVE EFFLUENTS

LIQUID HOLDUP TANKS

LIMITING CONDITION FOR OPERATION

(a) 3.11.1.4[a] The contents within any outside temporary tank shall be limited to ≤10 curies, excluding tritium and dissolved or entrained noble gases.

APPLICABILITY

At all times. This specification does not apply to disposable liners used for shipment of radioactive waste.

ACTION

a. With the contents within any outside temporary tank exceeding the above limit, immediately suspend all additions of radioactive material to the tank and within 48 hours reduce the tank contents within the limit and provide notification to the Commission pursuant to Specification 6.9.1.8.

~~b. The provisions of Specification 6.9.1.13(b) are not applicable.~~

SURVEILLANCE REQUIREMENTS

4.11.1.4 The quantity of radioactive material contained in any outside temporary tank shall be determined to be within the above limit by analyzing a sample of each batch of radioactive material prior to its addition to the tank.

a. An outside temporary tank is not surrounded by liners, dikes, or walls that are capable of holding the tank contents and not having tank overflows and drains connected to the liquid radwaste treatment system.

treatment systems, the liquid effluents from the shared systems are proportioned among the units sharing that system. DELETED

3/4.11.1.3 LIQUID WASTE TREATMENT

This specification transferred to the ODCM per NRC Generic Letter 89-01

The OPERABILITY of the liquid radwaste treatment system ensures that this system will be available for use whenever liquid effluents require treatment prior to release to UNRESTRICTED AREAS. The requirements that the appropriate portions of this system be used when specified provides assurance that the releases of radioactive materials in liquid effluents will be kept ALARA. This specification implements the requirements of 10 CFR Part 50.36(a), General Design Criterion 60 of Appendix A to 10 CFR Part 50; and design objective Section II.D of Appendix I to 10 CFR Part 50. The specified limits governing the use of appropriate portions of the liquid radwaste treatment system were specified as a suitable fraction of the guide set forth in Section II.A of Appendix I, 10 CFR Part 50 for liquid effluents. DELETED

3/4.11.1.4 LIQUID HOLDUP TANKS

Restricting the quantity of radioactive material contained in the specified tanks provides assurance that in the event of an uncontrolled release of the tanks' contents, the resulting concentrations would be less than the limits of 10 CFR Part ~~20, Appendix A, Table II (column 2)~~ 20.1302(b)(2)(i) at the nearest surface water supply in an UNRESTRICTED AREA.

3/4.11.2 GASEOUS EFFLUENTS

3/4.11.2.1 DOSE RATE

This specification transferred to the ODCM per NRC Generic Letter 89-01

This specification is provided to ensure that at all times the dose rate at the exclusion area boundary from gaseous effluents from all onsite units will be within the annual dose limits of 10 CFR Part 20 for UNRESTRICTED AREAS. The annual dose limits are the doses associated with the concentrations of 10 CFR Part 20, Appendix B, Table II. These limits provide reasonable assurance that radioactive material discharged in gaseous effluents will not result in the exposure of an individual in an UNRESTRICTED AREA, either within or outside the exclusion area boundary, to annual average concentrations exceeding the limits specified in Appendix B, Table II of 10 CFR Part 20 (10 CFR Part 20.106(b)). For individuals who may at times be within the exclusion area boundary, the occupancy of the individual will be sufficiently low to compensate for any increase in the atmospheric diffusion factor above that for the exclusion area DELETED

ANNUAL REPORTS (Continued)

6.9.1.5 Reports required on an annual basis shall include:

a. A tabulation on an annual basis of the number of station, utility and other personnel, including contractors, receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions,[a] e.g., reactor operations and surveillance inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

b. Documentation of all challenges to safety/relief valves.

c. The results of specific activity analysis in which the primary coolant exceeded the limits of Specification 3.4.5. The following information shall be included: (1) Reactor power history starting 48 hours prior to the first sample in which the limit was exceeded; (2) Results of the last isotopic analysis for radioiodine performed prior to exceeding the limit, results of analysis while limit was exceeded and results of one analysis after the radioiodine was reduced to less than the limit. Each result should include date and time of sampling and the radioiodine concentrations; (3) Cleanup system flow history starting 48 hours prior to the first sample in which the limit was exceeded; (4) Graph of the I-131 concentration and one other radioiodine isotope concentration in microcuries per gram as a function of time for the duration of the specific activity above the steady-state level; and (5) The time duration when the specific activity of the primary coolant exceeded the radioiodine limit.

d. Any other unit unique reports required on an annual basis.

ANNUAL RADIOLOGICAL ENVIRONMENTAL SURVEILLANCE REPORT[a]

Insert 3

6.9.1.6 Routine radiological environmental surveillance reports covering the radiological environmental surveillance activities related to the plant during the previous calendar year shall be submitted prior to May 1 of each year. A single report may fulfill this requirement for both units.

6.9.1.7 The Annual Radiological Environmental Surveillance Report shall include summaries, interpretations, and statistical evaluation of the

a. A single submittal may be made for a multiple-unit station. The submittal should combine those sections common to all units at the station.

[a] This tabulation supplements the requirements of ~~20.407~~ of 10 CFR Part 20.

20.2206

RECORD RETENTION (Continued)

c. Records of radiation exposure for all individuals entering radiation control areas.

d. Records of gaseous and liquid radioactive material released to the environs.

e. Records of transient or operational cycles for those unit components identified in Table 5.7.1-1.

f. Records of reactor tests and experiments.

g. Records of training and qualification for current members of the unit staff.

h. Records of in-service inspections performed pursuant to these Technical Specifications.

i. Records of Quality Assurance activities required by the QA Manual.

j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.

k. Records of meetings of the PRB and the SRB.

l. Records for Environmental Qualification which are covered under the provisions of paragraph 6.15.

m. Records of analyses required by the Radiological Environmental Monitoring Program.

n. Records of the service lives of all safety-related hydraulic and mechanical snubbers, including the date at which the service life commences and associated installation and maintenance records.

o. ← Insert 5

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph ~~20.203(c)(2)~~ of 10 CFR 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit*. Any individual or group of individuals permitted

20.1601(a)

*Health Physics personnel, or personnel escorted by Health Physics personnel in accordance with approved emergency procedures, shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they comply with approved radiation protection procedures for entry into high radiation areas.

6.16 POST-ACCIDENT SAMPLING AND ANALYSIS

A program shall be established, implemented, and maintained to ensure the capability to obtain and analyze samples of reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere under accident conditions.

The program shall include the following:

(1) Training of personnel,

(2) Procedures for sampling and analysis, and

(3) Provisions for maintenance of sampling and analysis equipment.

6.17 OFFSITE DOSE CALCULATION MANUAL

6.17.1 Licensee-initiated changes to the ODCM shall: Insert 6

a. Be submitted to the Commission in the semi-annual effluent release report for the period in which the change(s) was made effective. This submittal shall contain:

1. Sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information. Information submitted should consist of a package of those ODCM pages to be changed, with each page numbered and provided with an approval and date box, together with appropriate analyses or evaluations justifying the change(s);

2. A determination that the change will not reduce the accuracy or reliability of dose calculations or setpoint determinations; and

3. Documentation that the change has been reviewed and found acceptable by the PRB.

b. Become effective upon review and acceptance by the PRB.

Insert 7

Insert 6

a. Be documented and records of reviews performed shall be retained as required by Technical Specification 6.10.2.o. This documentation shall contain:

   1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and

   2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR ~~20.106~~, 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.

b. Become effective after review and acceptance by the PRB and the approval of the General Manager-Nuclear Plant.

c. Be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Semiannual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g. month/year) the change was implemented.

Insert 7

6.18 RADIOACTIVE EFFLUENTS CONTROLS PROGRAM

A program shall be established, implemented, and maintained conforming with 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to MEMBERS OF THE PUBLIC from radioactive effluents as low as reasonably achievable. The program (1) shall be contained in the ODCM, (2) shall be implemented by operating procedures, and (3) shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

1) Limitations on the operability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the ODCM,

2) Limitations on the concentrations of radioactive material released at all times in liquid effluents to UNRESTRICTED AREAS conforming to ~~10 CFR Part 20, Appendix B, Table II, Column 2,~~ 10 times the concentrations stated in 10 CFR 20, Appendix B (to paragraphs 20.1001-20.2401), Table 2, Column 2,

3) Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR ~~20.106~~ 20.1302 and with the methodology and parameters in the ODCM,

4) Limitations on the annual and quarterly doses or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released from each unit to UNRESTRICTED AREAS conforming to Appendix I to 10 CFR Part 50,

5) Determination of cumulative and projected dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the ODCM at least every 31 days,

6) Limitations on the operability and use of the liquid and gaseous effluent treatment systems to ensure that the appropriate portions of these systems are used to reduce releases of radioactivity when the projected doses in a 31-day period would exceed 2 percent of the guidelines for the annual dose or dose commitment conforming to Appendix I to 10 CFR Part 50,

7) Limitations on the at all times concentrations of ~~dose rate resulting from~~ radioactive material released in gaseous effluents to areas beyond the SITE BOUNDARY conforming to ~~the doses associated with 10 CFR Part 20, Appendix B, Table II, Column 1,~~ 10 times the concentrations stated in 10 CFR Part 20, Appendix B (paragraphs 20.1001-20.2401), Table 2, Column 1, which corresponds to a dose rate of 500 mrem/year total effective dose equivalent,