KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER
VICE PRESIDENT - NUCLEAR

February 13, 1985

Mr. R.P. Denise, Director
Wolf Creek Task Force
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

KMLNRC 85-045
Re: Docket No. STN 50-482
Ref: Final Report KMLNRC 85-027, dated 1/13/85 from GLKoester, KG&E, to RPDenise, NRC
Subj: Supplemental Final 10CFR50.55(e) Report - Concrete Coatings Inside Containment (53564-K159)

Dear Mr. Denise:

This letter provides additional information, submitted pursuant to 10CFR50.55(e), concerning concrete coatings inside Containment at Wolf Creek Generating Station (WCGS). This matter was originally reported on December 13, 1984, and a final report was provided in the Reference.

As stated in the Reference, Kansas Gas and Electric's (KG&E) investigation into the adequacy of Containment concrete coatings was initiated as a result of an allegation made by an exiting Quality Control Coatings Inspector. Upon exiting he stated that he had inspected coatings work recently performed inside Containment but had not completed some of the corresponding inspection reports. His allegation was that, since he was leaving, the inspection reports may be, or may have been prepared by someone other than himself. This allegation was investigated and proved to be unfounded. However, since the inspector had not documented his inspections, Nonconformance Report (NCR) 1SN 21203C was initiated to document this condition and assure appropriate resolution. In addition, as part of the overall investigation into this matter, a review of concrete coating inspection records was conducted to identify any other areas for which documentation was not available. The areas with unretrievable documentation have been documented on NCR's and included in the overall evaluation to determine "worst case" coatings locations as described in the Reference.

The Reference also stated that the results of the DBA test sequences indicate that the "worst case" coating samples meet the intent of ANSI N101.2 (1972), paragraph 1.4.2.2 in that the coatings will perform satisfactorily under the WCGS design basis accident. After testing, coating samples were evaluated for flaking, delamination, peeling and blistering. It was observed that none of the coating samples exhibited signs of delamination, peeling, chalking, or flaking. Six of the twenty-six core

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samples exhibited blister sizes which exceeded the ANSI N101.2 criteria for blister size. These six samples were determined to be acceptable when evaluated with respect to the type of coating system, substrate, and the condition of the blisters after the completion of the DBA test sequence. In conclusion, it has been determined through testing and an engineering evaluation of test results that the coated concrete surfaces inside containment will perform satisfactorily under WCGS Design Basis Accident conditions in accordance with the commitments described in the SNUPPS FSAR Sections 6.1.1.1.2 and 6.1.2.

Coatings removed from the elevation 2000'-0" floor inside Containment are presently being reapplied. If not completed prior to entering Mode Six this work will be suspended prior to entering Mode Six and then resumed and completed while in Mode Five. KG&E has reviewed the other activities scheduled during the same periods and has determined that no significant schedular, safety or technical impact will result from these recoating activities.

If you have any questions concerning this matter, please contact me or Mr. Otto Maynard of my staff.

Yours very truly,

Glenn L Koester

Glenn L. Koester
Vice President - Nuclear

GLK:sjm

cc: PO'Connor (2)
HBundy
WGuldemond