

LER SUPPLEMENTAL INFORMATION

BFRO-50- 259 / 83066 RI Technical Specification Involved 3.10.A.1

Reported Under Technical Specification 6.7.2.b.(2) * Date Due NRC 4/1/84

Event Narrative:

Unit 2 was operating at 73 percent power and both units 1 and 3 were in refueling outages. While performing SI 4.10.A.1 on unit 1, the interlock limit switch No. 1 failed to stop the bridge from traversing over the core with one rod withdrawn, the fuel grapple extended, and the mode switch in refuel. This resulted in a loss of the rod block function with the platform over the core (T.S. 3.10.A.1). The SI was stopped and bridge moved from over the core. The lever arm on limit switch No. 1 (GE Part No. CR 115GW307) was found to be improperly adjusted. The cause is unknown as the interlock switches were checked for proper adjustment, tightness, and operationally tested in previous steps of SI 4.10.A.1. SI 4.10.A.1, Step 18, was revised to verify that limit switches 1 and 2 actuate for both CW and CCW rotation and to verify the limit switch's lever arms are installed and adjusted properly.

* Previous Similar Events:

LER BFRO-50-259/7727
LER BFRO-50-260/8054
LER BFRO-50-296/8053
LER BFRO-50-259/8301

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP

SI 4.10.A.1 Refueling Interlocks Functional Test

NOTE: Omit steps 10 through 17 unless the service platform is over the core and the jib crane is attached. Make appropriate entries in situations 2 and 3 of the data sheet.

NRC/C
NRC Rpt 10.
259/80-40

Check that the mode switch is in "Refuel", that all control rods are fully inserted, (except as noted in Step 1) and that the refueling platform is over the pool.

11. Withdraw one control rod one notch.
12. Attempt to load jib crane hoist with a 313 lb. load (underwater weight.) Observe that hoist cannot be loaded.
13. Fill in situation 2 on data sheet.

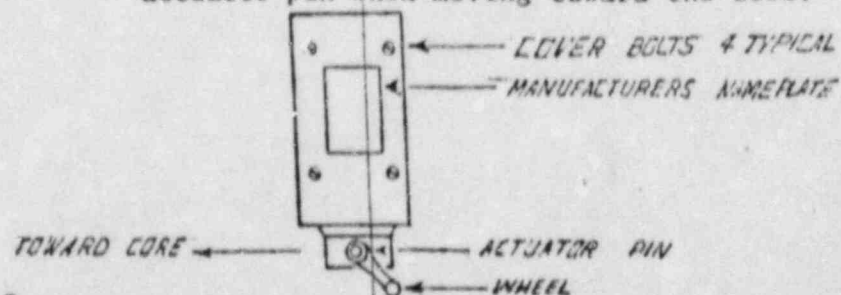
NRC/C
NRC Rpt 14.
259/80-40

14. Insert all control rods (except as noted in Step 1.)
15. Load jib crane hoist with the 313 lb load.
16. Attempt to withdraw one control rod one notch. Observe a rod block. Transfer mode switch to "Startup." Attempt to withdraw one control rod one notch. Observe rod block.
17. Fill in situation 3 on the data sheet and remove the load from the jib crane.
18. Check the following conditions:

NRC/C
NRC Rpt
259/83066

CAUTION: Verify that the refueling platform interlock limit switches actuate for both clock-wise and counter clock-wise operation. (If not, then adjust actuator to do so.)

Verify that the wheel on the actuator arm trails the actuator pin when moving toward the core.



Verify that the actuator arm is adjusted so that the contacts are in (1) the normal position when the cam is not under the limit switch and (2) the non-normal position when the cam is under the switch.

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SI 4.10.A.1 Refueling Interlocks Functional Test (Continued)

NRC/C
NRC Rpt
260/8054

Prior to the performance of any further steps, verify that refuel floor maintenance electricians have checked the refueling platform interlock limit switches for condition and tightness of set screw on the actuating arm of both limit switches.

NRC/C
NRC Rpt
259/80-40

- a. Mode switch in "Startup"
 - b. Refueling platform over pool
 - c. All control rods fully inserted, (except as noted in Step 1) and one control rod selected.
19. Move refueling platform toward core. Observe platform motion is blocked as it approaches core.
 20. Fill in situation 4 on the data sheet.
 21. Attempt to withdraw one rod one notch. Observe rod block.
 22. Fill in situation 5 on the data sheet.
 23. Move refueling platform toward pool to clear refueling interlock.
 24. Withdraw two control rods one notch each.

NOTE: Select control rods in accordance with rod worth minimizer patterns.

*Revision

TENNESSEE VALLEY AUTHORITY

Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, Alabama 35602

March 9, 1984

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Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 1 - DOCKET
NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE OCCURRENCE
REPORT BFRO-50-259/E3C66 R1

Reference: H. J. Green's memorandum to you dated December 8, 1983

The enclosed report provides followup information concerning interlock that prevents bridge travel over the core with one rod withdrawn and fuel grapple extended failed to operate. This report is submitted in accordance with Browns Ferry Unit 1 Technical Specification 6.7.2.b.(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

G. T. Jones

for G. T. Jones
Power Plant Superintendent
Browns Ferry Nuclear Plant

Enclosure

cc (Enclosure)
U. S. Nuclear Regulatory Commission
Document Control Desk
Washington D. C. 20555

NRC Inspector, Browns Ferry Nuclear Plant

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