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Southern Nuclear Operating Company

the southern electric system

Dave Morey
Vice President
Farley Project

July 11, 1995

Docket Nos. 50-348
50-364

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Joseph M. Farley Nuclear Plant
Monthly Operating Data Report

Gentlemen:

Attached are the June 1995 Monthly Operating Reports for Joseph M. Farley Nuclear Plant Units 1 and 2, as required by Section 6.9.1.10 of the Technical Specifications.

If you have any questions, please advise.

Respectfully submitted,

Dave Morey

RWC:jgp(mor)

Attachments

cc: Mr. S. D. Ebnetter
Mr. B. L. Siegel
Mr. T. M. Ross

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Joseph M. Farley Nuclear Plant
Unit 1
Narrative Summary of Operations
June 1995

At 2139 on June 11, 1995, with the Unit 1 reactor in mode 1, operating at 100 percent power, the reactor tripped due to a turbine trip in response to a spurious main steam line isolation valve (MSIV) closure. The unit was synchronized to the grid at 2315 on June 17, 1995.

The following safety related maintenance was performed during the month:

1. Following the June 11, 1995, reactor trip, the unit was cooled down to mode 5 for an inspection of MSIV 3370C. No damage to MSIV 3370C was evident.

OPERATING DATA REPORT

DOCKET NO.	50-348
DATE	July 7, 1995
COMPLETED BY	S. M. Allison
TELEPHONE	(334) 899-5156 ext. 3442

OPERATING STATUS

- | | |
|---|----------------------------------|
| 1. Unit Name: | Joseph M. Farley - Unit 1 |
| 2. Reporting Period: | June 1995 |
| 3. Licensed Thermal Power (MWt): | 2,652 |
| 4. Nameplate Rating (Gross MWe): | 860 |
| 5. Design Electrical Rating (Net MWe): | 829 |
| 6. Maximum Dependable Capacity (Gross MWe): | 855.7 |
| 7. Maximum Dependable Capacity (Net MWe): | 812 |
| 8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: | N/A |
| 9. Power Level To Which Restricted, If Any (Net MWe): | N/A |
| 10. Reasons For Restrictions, If Any: | N/A |

Notes

- 1) Cumulative data since 12-01-77, date of commercial operation.

	This Month	Yr. to Date	Cumulative
11. Hours in Reporting Period	720.0	4,343.0	154,103.0
12. Number Of Hours Reactor Was Critical	590.4	4,123.1	123,379.9
13. Reactor Reserve Shutdown Hours	0.0	0.0	3,650.0
14. Hours Generator On-line	574.4	4,087.3	121,541.2
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWh)	1,432,289.5	10,718,918.2	312,854,635.2
17. Gross Electrical Energy Generated (MWh)	461,977.0	3,513,320.0	100,927,025.0
18. Net Electrical Energy Generated (MWh)	432,843.0	3,329,628.0	95,329,617.0
19. Unit Service Factor	73.8	94.1	78.9
20. Unit Availability Factor	73.8	94.1	78.9
21. Unit Capacity Factor (Using MDC Net)	74.0	94.4	75.9
22. Unit Capacity Factor (Using DER Net)	72.5	92.5	74.6
23. Unit Forced Outage Rate	20.2	5.9	6.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Refueling/Maintenance Outage, September 16, 1995. Approximately 33 days.

25. If Shut Down at End Of Report Period, Estimated Date of Startup:	N/A	
26. Units In Test Status (Prior To Commercial Operation):	Forecast	Achieved
Initial Criticality	08/06/77	08/09/77
Initial Electricity	08/20/77	08/18/77
Commercial Operation	12/01/77	12/01/77

DOCKET NO.	50-348
UNIT	1
DATE	July 7, 1995
COMPLETED BY	S. M. Allison
TELEPHONE	(334) 899-5156 ext. 3442

MONTH June

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	818	17	0
2	815	18	289
3	816	19	335
4	817	20	561
5	818	21	818
6	817	22	819
7	814	23	817
8	813	24	814
9	813	25	815
10	814	26	814
11	733	27	816
12	0	28	816
13	0	29	817
14	0	30	816
15	0	31	NA
16	0		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting period. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-348
 UNIT NAME J. M. Farley - Unit 1
 DATE July 7, 1995
 COMPLETED BY S. M. Allison
 TELEPHONE (334) 899-5156, ext. 3442

REPORT MONTH June

NO.	DATE	TYPE (1)	DURATION HOURS	REASON (2)	METHOD OF SHUTTING DOWN REACTOR (3)	LICENSEE EVENT REPORT #	SYSTEM CODE (4)	COMPONENT CODE (5)	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
003	950611	F	145.6	H	3	95-005-0	SB	ISV	<p>At 2139, on 950611, with Unit 1 in mode 1, operating at 100 percent power, the reactor tripped due to a turbine trip in response to a spurious main steam line isolation valve (MSIV) closure. The MSIV closure was attributed to water intrusion into a junction box. Cause and corrective actions are discussed in detail in the referenced LER.</p> <p>The unit was synchronized to the grid at 2315 on 950617.</p>

- | | | | | |
|--------------|---|---------------------|--------------------------------|-------------------------|
| 1: | 2: | 3: | 4: | 5: |
| F: Forced | Reason | Method | Exhibit G- Instructions for | Exhibit I - Same Source |
| S: Scheduled | A - Equipment Failure (Explain) | 1 - Manual | Preparations of Date Entry | |
| | B - Maintenance or Test | 2 - Manual Scram | sheets for Licensee Event | |
| | C - Refueling | 3 - Automatic Scram | Report (LER) File (NUREG-0161) | |
| | D - Regulatory Restriction | 4 - Other (Explain) | | |
| | E - Operator Training & License Examination | | | |
| | F - Administrative | | | |
| | G - Operational Error (Explain) | | | |
| | H - Other (Explain) | | | |

Joseph M. Farley Nuclear Plant
Unit 2
Narrative Summary of Operations
June 1995

There was no major safety related maintenance performed during the month.

At 0058 on June 1, 1995, with the Unit 2 reactor in mode 1, operating at approximately 30 percent power, the reactor was manually tripped following the loss of the operating (2A) steam generator feed pump (SGFP). The loss of the SGFP was caused by a failed electro-hydraulic (EH) fluid supply line and subsequent loss of EH fluid. The failure was in the heat affected zone of the tubing-to-fitting socket weld on the EH supply to the high pressure governor valve to the 2A SGFP. The unit was synchronized to grid at 1912 on June 2, 1995.

Initially, the June 1, 1995, event was attributed to cyclic fatigue (pending the results of metalurgical analysis) caused by a loose support clamp, and higher than normal vibrations, which are associated with feed pump operations at low power. The EH tubing and support clamp were repaired, and the failed tubing was sent off site for detailed failure analysis. Subsequently, at 0948 on June 3, 1995, with the reactor in mode 1, operating at approximately 33 percent power, the reactor was manually tripped following the loss of the operating (2A) SGFP caused by a failed EH fluid supply line on the 2A SGFP and subsequent loss of EH fluid. The unit was synchronized to the grid at 1515 on June 5, 1995.

Examination of the failed tubing revealed that the failures experienced on June 1, 1995 and June 3, 1995 were sudden impact failures caused by servovalve failures, which resulted in mechanical and hydraulic oscillations in the EH system.

At 2000 on June 10, 1995, with the Unit 2 reactor in mode 1, operating at approximately 64 percent power, reactor power was decreased to 15 percent power for steam generator clean-up. At 0300 on June 11, 1995, with the Unit in mode 1, operating at approximately 15 percent power, the 2B SGFP high pressure stop valve closed, which caused the SGFP speed to decrease. Subsequently, the main turbine was removed from the grid and reactor power was decreased to 1.4 percent power. The unit was synchronized to the grid at 1508 on June 12, 1995.

At 2102 on June 21, 1995, with the unit in mode 1, operating at 100 percent power, the unit was ramped to approximately 60 percent power due to 2B SGFP low pressure governor valve oscillations. The 2B SGFP EH servovalves were replaced.

At 1640 on June 25, 1995, with the Unit 2 reactor in mode 1, operating at approximately 63 percent power, the reactor tripped due to a turbine trip caused by 2C steam generator water level reaching the Hi-Hi setpoint of 79 percent. The unit was synchronized to the grid at 0525 on June 29, 1995.

OPERATING DATA REPORT

DOCKET NO.	50-364
DATE	July 7, 1995
COMPLETED BY	S. M. Allison
TELEPHONE	(334) 899-5156 ext. 3442

OPERATING STATUS

- | | |
|---|----------------------------------|
| 1. Unit Name: | Joseph M. Farley - Unit 2 |
| 2. Reporting Period: | June 1995 |
| 3. Licensed Thermal Power (MWt): | 2,652 |
| 4. Nameplate Rating (Gross MWe): | 860 |
| 5. Design Electrical Rating (Net MWe): | 829 |
| 6. Maximum Dependable Capacity (Gross MWe): | 863.6 |
| 7. Maximum Dependable Capacity (Net MWe): | 822 |
| 8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: | N/A |
| 9. Power Level To Which Restricted, If Any (Net MWe): | N/A |
| 10. Reasons For Restrictions, If Any: | N/A |

Notes

- 1) Cumulative data since 07-30-81, date of commercial operation.

	This Month	Yr. to Date	Cumulative
11. Hours in Reporting Period	720.0	4,343.0	122,016.0
12. Number Of Hours Reactor Was Critical	641.0	2,890.0	104,547.7
13. Reactor Reserve Shutdown Hours	0.0	0.0	138.0
14. Hours Generator On-line	503.5	2,662.2	102,884.8
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	880,007.9	5,474,786.9	261,814,492.6
17. Gross Electrical Energy Generated (MWH)	266,973.0	1,754,944.0	85,809,486.0
18. Net Electrical Energy Generated (MWH)	239,327.0	1,618,738.0	81,341,036.0
19. Unit Service Factor	69.9	61.2	84.3
20. Unit Availability Factor	69.9	61.3	84.3
21. Unit Capacity Factor (Using MDC Net)	40.4	45.3	81.3
22. Unit Capacity Factor (Using DER Net)	40.1	45.0	80.4
23. Unit Forced Outage Rate	30.1	7.5	4.1
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):	N/A		

25. If Shut Down at End Of Report Period, Estimated Date of Startup:	N/A	
26. Units In Test Status (Prior To Commercial Operation):	Forecast	Achieved
Initial Criticality	05/06/81	05/08/81
Initial Electricity	05/24/81	05/25/81
Commercial Operation	08/01/81	07/30/81

DOCKET NO.	50-364
UNIT	2
DATE	July 7, 1995
COMPLETED BY	S. M. Allison
TELEPHONE	(334) 899-5156 ext. 3442

MONTH June

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	815
2	2	18	827
3	53	19	828
4	0	20	828
5	36	21	789
6	209	22	470
7	372	23	462
8	459	24	460
9	444	25	307
10	473	26	0
11	0	27	0
12	0	28	0
13	67	29	12
14	397	30	323
15	727	31	NA
16	306		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting period. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH June

DOCKET NO. 50-364
 UNIT NAME J. M. Farley - Unit 2
 DATE July 7, 1995
 COMPLETED BY S. M. Allison
 TELEPHONE (334) 899-5156, ext. 3442

NO	DATE	TYPE (1)	DURATION HOURS	REASON (2)	METHOD OF SHUTTING DOWN REACTOR (3)	LICENSEE EVENT REPORT #	SYSTEM CODE (4)	COMPONENT CODE (5)	CAUSE AND CORRECTIVE
									ACTION TO PREVENT RECURRENCE
005	950601	F	42.2	A	2	95-005-0	SJ	XXXXXX	At 0058 on 950601, with the unit in mode 1, operating at approximately 30 percent power, the reactor was manually tripped following the loss of the operating (2A) steam generator feed pump (SGFP). The loss of the SGFP was caused by a failed electro-hydraulic (EH) fluid supply line and subsequent loss of EH fluid. Cause and corrective actions are discussed in detail in the referenced LER. The unit was synchronized to the grid at 1912 on 950602.
006	950603	F	53.5	A	2	95-005-0	SJ	XXXXXX	At 0948 on 950603, with the unit in mode 1, operating at approximately 33 percent power, the reactor was manually tripped following the loss of the operating (2A) SGFP caused by a failed EH fluid supply line on the 2A SGFP and subsequent loss of EH fluid. The cause and corrective actions are discussed in detail in the referenced LER. The unit was synchronized to the grid at 1515 on 950605.

- 1: F - Forced
 S - Scheduled
- 2: Reason
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & License Examination
 F - Administrative
 G - Operational Error (Explain)
 H - Other (Explain)

- 3: Method
 1 - Manual
 2 - Manual Scram
 3 - Automatic Scram
 4 - Other (Explain)

- 4: Exhibit G - Instructions for
 Preparations of Date Entry
 Sheets for Licensee Event
 Report (LER) File (NUREG-0161)

- 5: Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-364
 UNIT NAME J. M. Farley - Unit 2
 DATE July 7, 1995
 COMPLETED BY S. M. Allison
 TELEPHONE (334) 899-5156, ext. 3442

REPORT MONTH June

NO.	DATE	TYPE (1)	DURATION HOURS	REASON (2)	METHOD OF SHUTTING DOWN REACTOR (3)	LICENSEE EVENT REPORT #	SYSTEM CODE (4)	COMPONENT CODE (5)	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
007	950610	S	0.0	H	4	N/A	AB	SG	At 2000 on 950610, with the unit in mode 1, operating at approximately 64 percent power, the reactor was decreased to 15 percent power for steam generator clean up.
008	950611	F	36.1	A	4	N/A	SJ	SHV	At 0300 on 950611, with the unit in mode 1, operating at approximately 15 percent power, the 2B steam generator feed pump (SGFP) high pressure stop valve closed, which caused the SGFP speed to decrease. Subsequently, the main turbine was removed from the grid and reactor power was decreased to 1.4 percent power. The high pressure stop valve closure was attributed to a failed solenoid valve, which was replaced. The unit was synchronized to the grid at 1508 on 950612.

1: F - Forced
 S - Scheduled

2: Reason
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & License Examination
 F - Administrative
 G - Operational Error (Explain)
 H - Other (Explain)

3: Method
 1 - Manual
 2 - Manual Scram
 3 - Automatic Scram
 4 - Other (Explain)

4: Exhibit G- Instructions for
 Preparations of Date Entry
 Sheets for Licensee Event
 Report (LER) File (NUREG-0161)

5: Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH June

DOCKET NO. 50-364
 UNIT NAME J. M. Farley - Unit 2
 DATE July 7, 1995
 COMPLETED BY S. M. Allison
 TELEPHONE (334) 899-5156, ext. 3442

NO.	DATE	TYPE (1)	DURATION HOURS	REASON (2)	METHOD OF SHUTTING DOWN REACTOR (3)	LICENSEE EVENT REPORT #	SYSTEM CODE (4)	COMPONENT CODE (5)	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
009	950621	F	0.0	A	4	N/A	SJ	SCV	At 2102 on 950621, with the unit in mode 1, operating at 100 percent power, the unit was ramped to approximately 60 percent power due to 2B SGFP low pressure governor valve oscillations. The 2B SGFP EH servovalves were replaced.
010	950625	F	84.7	G	3	95-007-0	AB	SG	At 1640 on 950625, with the Unit in mode 1, operating at approximately 63 percent power, the reactor tripped due to a turbine trip caused by the 2C steam generator water level reaching the Hi-Hi setpoint of 79 percent. This occurred during the evolution of transferring feedwater flow from the (2A) operating SGFP to the 2B SGFP. The cause and corrective actions are discussed in detail in the referenced LER. The unit was synchronized to the grid at 0525 on 950629.

- | | | | | |
|---------------|---|---------------------|--|-------------------------|
| 1: | 2: | 3: | 4: | 5: |
| F - Forced | Reason | Method | Exhibit G- Instructions for Preparations of Date Entry | Exhibit I - Same Source |
| S - Scheduled | A - Equipment Failure (Explain) | 1 - Manual | Sheets for Licensee Event | |
| | B - Maintenance or Test | 2 - Manual Scram | Report (LER) File (NUREG-0161) | |
| | C - Refueling | 3 - Automatic Scram | | |
| | D - Regulatory Restriction | 4 - Other (Explain) | | |
| | E - Operator Training & License Examination | | | |
| | F - Administrative | | | |
| | G - Operational Error (Explain) | | | |
| | H - Other (Explain) | | | |