

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) **Monticello** DOCKET NUMBER (2) **05000263** PAGE(S) **1 OF 2**

TITLE (4) **Crack Indications on Primary System Pressure Boundary Piping**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
02	18	84	84	011	010	05	15	84			050000
											050000

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)

OPERATING MODE (9) N	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(a)	<input type="checkbox"/> 80.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
POWER LEVEL (10) 01010	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 80.36(a)(1)	<input type="checkbox"/> 80.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 80.36(a)(2)	<input type="checkbox"/> 80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 308A)
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 80.73(a)(2)(i)	<input type="checkbox"/> 80.73(a)(2)(vii)(A)	
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input checked="" type="checkbox"/> 80.73(a)(2)(ii)	<input type="checkbox"/> 80.73(a)(2)(vii)(B)	
	<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 80.73(a)(2)(iii)	<input type="checkbox"/> 80.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME **Michael Keller, Production Engineer** TELEPHONE NUMBER **612 295-5151**

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS
B	AIDP	SIF	G101810	Y					

SUPPLEMENTAL REPORT EXPECTED (14) YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Crack indications have been found in the following locations:

1. Recirculation System Sensing Line Penetration Piping (previously reported in LER 84-011-00).
2. Residual Heat Removal System (previously reported in LER 84-011-00).
3. Recirculation System Pump Suction and Discharge Valves.
4. Recirculation System Discharge Riser "D" Safe End.

The degraded piping will be replaced with IGSCC resistant material. The valve material with indications will be machined so as to remove all affected material.

8405220295 840515
PDR ADOCK 05000263
S PDR

IE22

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Monticello	DOCKET NUMBER (2) 0 5 0 0 0 2 6 3	LER NUMBER (6)			PAGE (3)	
		YEAR 8 4	SEQUENTIAL NUMBER 0 1 1	REVISION NUMBER 0 1	0 2	OF 0 2

TEXT (if more space is required, use additional NRC Form 388A's) (17)

On February 18, 1984 and February 23, 1984 crack indications were detected on the B Loop Jet Pump (P) Instrument Seal (SEAL) and on both A&B Loops of the Residual Heat Removal (RHR) Low Pressure Coolant Injection (LPCI) lines. On April 13 and 14, 1984, crack indications were detected on the A Loop Recirculation Pump Discharge Valve (V) and the B Loop Recirculation Pump Suction and Discharge Valves. On May 5, 1984 crack indications were detected on the B Loop Recirculation System Discharge Riser D Safe End (PSF). All crack indications have been detected during normal Non-Destructive Examination activities for replacement of Recirculation System Piping (PSP). All cracks described above were detected while in cold shutdown.

All crack indications were located within the heat affected zone of welds to 304 Stainless Steel. B Loop Jet Pump Instrument Seal crack indications were located within the heat affected zone of a weld between two 304 Stainless Steel eccentric reducers. Indications appear to have been on the inside surface oriented parallel to the pipe axis. Crack indications on the A&B Loops of the RHR LPCI lines were located on the reactor side of the stainless steel LPCI testable check valves AO 10-46 A/B in the heat affected zone of welds joining a stainless steel pup piece to the valve and carbon steel piping. Indications on the Recirc Valves were located within the heat affected zone of welds to 304 Stainless Steel pipe. Indications on the Riser D Safe End were located within the heat affected zone of the 304 Stainless Steel Safe End to Vessel Nozzle Weld.

All detected crack indications are attributed to Intergranular Stress Corrosion Cracking (IGSCC). All susceptible 304 Stainless Steel is to be replaced with IGSCC resistant material. Welds to the valves and safe ends will be post weld heat treated using Induction Heating Stress Improvement techniques.

There have been two previous similar occurrences:

- M-RO-82-13 Recirc System IGSCC Problems
- AO-75-03 Recirc Pump Bypass Line IGSCC Problems

With the exception of one Thru-Wall crack on the Discharge Riser D Safe End, all indications have been found to be surface defects. There were no radioactive materials released and no personnel exposures or injuries took place. There was no effect upon public health or safety.

HHH095



Northern States Power Company

414 Nicollet Mall
Minneapolis, Minnesota 55401
Telephone (612) 330-5500

May 15, 1984

U S Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Revised Licensee Event Report Related to Crack
Indications on Primary System Pressure Boundary

A revised Licensee Event Report for this occurrence is attached.

These events were reported via the Emergency Notification System per
10 CFR Part 72 on February 18, February 23, April 13, April 14,
and May 7, 1984.

David Musolf
Manager - Nuclear Support Services

cc: Regional Administrator - III, NRC
NRR Project Manager, NRC
Resident Inspector, NRC
MPCA
Attn: J W Ferman

Attachment

IE22
11