

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3180-0104
EXPIRES 8/31/85

FACILITY NAME (1)
Peach Bottom Atomic Power Station - Unit 2

DOCKET NUMBER (2)
0 8 0 0 0 2 7 7 1 OF 0 2

PAGE (3)
1 OF 0 2

TITLE (4)
Reactor Vessel Heatup Rate Exceeded 100 Degrees Fahrenheit per hour.

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 1	3 1	8 4	8 4	0 0 3	0 1	0 5	0 3	8 4			0 8 0 0 0
											0 8 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)

OPERATING MODE (8)	20.402(b)	20.406(a)	80.73(a)(2)(iv)	72.71(b)
POWER LEVEL (10)	20.406(a)(1)(i)(B)	80.36(a)(1)	80.73(a)(2)(iv)	72.71(a)
	20.406(a)(1)(i)(B)	80.36(a)(2)	80.73(a)(2)(iv)	OTHER (Specify in Abstract below and in Text, NRC Form 364A)
	20.406(a)(1)(iii)	X 80.73(a)(2)(ii)	80.73(a)(2)(iv)(A)	
	20.406(a)(1)(iv)	80.73(a)(2)(iv)	80.73(a)(2)(iv)(B)	
	20.406(a)(1)(v)	80.73(a)(2)(ii)	80.73(a)(2)(iv)	

LICENSEE CONTACT FOR THIS LER (12)

NAME: B. L. Clark, Senior Engineer - Special Projects

TELEPHONE NUMBER: 2 1 5 8 4 1 1 - 5 0 1 7

AREA CODE: 2 1 5

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 words, i.e., approximately fifteen single space typewritten lines) (16)

Abstract 2-84-03

Technical Specification 3.6.A.1 states that the average rate of reactor coolant temperature change during normal heatup or cooldown shall not exceed 100 degrees Fahrenheit increase (or decrease) in any one hour period.

During startup on Unit 2, the Reactor Operator noted that the heatup rate was exceeding the Technical Specification limit of 100 degrees Fahrenheit in a 1-hour period per Surveillance Test 9.12 (Reactor Vessel Temperatures). In responding, the operator failed to take adequate corrective action rapidly enough and the reactor coolant temperature increased by 110 degrees Fahrenheit within a one hour period.

General Electric has evaluated the excess heatup rate and concluded that it had a minimal effect on the reactor pressure vessel.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Peach Bottom Atomic Power Station Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 2 7 7 8 4 - 0 0 3 - 0 1 0 2 OF 0 2	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (if more space is required, use additional NRC Form 368A (17))

Description of the Event:

During startup on Unit 2, the Reactor Operator noted that the heatup rate was exceeding the Technical Specification limit of 100 degrees Fahrenheit in a 1-hour period per Surveillance Test 9.12 (Reactor Vessel Temperature). In responding, the operator failed to take adequate corrective action (inserting control rods and opening turbine bypass valves) rapidly enough. The result was that the reactor coolant temperature increased by 110 degrees Fahrenheit within a one hour period. Applicable Technical Specification Reference 3.6.A.1, states that the average rate of reactor coolant temperature change during normal heatup or cooldown shall not exceed 100 degrees Fahrenheit increase (or decrease) in any 1-hour period.

Consequences of the Event:

Heatup rates far greater than 100 degrees Fahrenheit per hour were considered in the original reactor vessel design analysis. This transient has been evaluated by GE and determined to have imposed only a negligible increase in the usage factor for the most limiting reactor pressure vessel component.

Cause of the Event:

The cause of the event was personnel error.

Corrective Actions:

The responsible operator has been disciplined regarding his inadequate performance (for exceeding the heatup rate and failing to take adequate action promptly enough). The station superintendent has issued a memo to all licensed operators to ensure that the intent of the Technical Specification heatup rate requirement is understood and to emphasize the use of diverse instrumentation available in the control room as a cross check on reactor heatup rates.

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET
P.O. BOX 8699
PHILADELPHIA, PA. 19101
(215) 841-4000

May 3, 1984

Docket No. 50-277

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

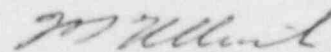
SUBJECT: Licensee Event Report

This LER concerns the Unit 2 reactor vessel heatup rate exceeding its Technical Specification limit of 100 degrees Fahrenheit increase in a 1-hour period, while in the startup mode.

Reference:	Docket No. 50-277
Report Number:	2-84-03
Revision Number:	01
Event Date:	January 31, 1984
Report Date:	May 3, 1984
Facility:	Peach Bottom Atomic Power Station RD #1, Box 208, Delta, PA 17314

This LER is submitted pursuant to the requirements of 10 CFR 50.73(A)(2)(i)(B). The revised portion of this LER is identified by a vertical bar in the margin.

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator
Region I, USNRC

Mr. A. R. Blough, Site Inspector

IE22
/1