

Mr. R. C. Haynes
Administrator
U.S. Nuclear Regulatory
Commission
631 Park Avenue
King of Prussia, PA 19406

Dear Mr. Haynes:

This LER deals with a revision to an LER on uncontrolled release of liquid effluents, per Tech. Spec. 3.8.B.

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | P | A | P | B | S | 3 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | _____

7 8 9 14 15 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40

CONT

01 | R | 0 | 1 | 5 | 0 | - | 0 | 2 | 1 | 7 | 1 | 1 | 1 | 0 | 4 | 8 | 2 | 0 | 1 | 1 | 1 | 1 | 8 | 3 | _____

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | The 3D RHR heat exchanger had leaked slightly radioactive water

03 | into the unit 3 intake structure via the high pressure service water

04 | piping during the period of October 25 through November 2, 1982.

05 | Total release is estimated to be 22.8 millicuries.

06 | _____

07 | _____

08 | _____

09 | _____

SYSTEM CODE (9) C (10) CAUSE CODE (11) E (12) CAUSE SUBCODE (13) B (14) COMPONENT CODE (15) H T E X C H (16) COMP SUBCODE (17) G (18) VALVE SUBCODE (19) Z (20)

17 | LER/RO REPORT NUMBER (21) 82 (22) EVENT YEAR (23) 82 (24) SEQUENTIAL REPORT NO. (25) 022 (26) OCCURRENCE CODE (27) 01 (28) REPORT TYPE (29) X (30) REVISION NO. (31) 1

ACTION TAKEN (32) D (33) FUTURE ACTION (34) D (35) EFFECT ON PLANT (36) Z (37) SHUTDOWN METHOD (38) Z (39) HOURS (40) 0000 (41) ATTACHMENT SUBMITTED (42) N (43) NFRD-4 FORM SUB (44) N (45) PRIME COMP SUPPLIER (46) N (47) COMPONENT MANUFACTURER (48) P I G O

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | The cause of the event was 3D RHR heat exchanger leak. The heat

11 | exchanger was taken out of service and was repaired. The cause of the

12 | leak was an expansion bellows between the inner floating head and

13 | drain. Additional routine radiation survey of HPSW inlet and outlet

14 | to heat exchanger will be initiated.

15 | FACILITY STATUS (28) E (29) % POWER (30) 089 (31) OTHER STATUS (32) NA (33) METHOD OF DISCOVERY (34) C (35) DISCOVERY DESCRIPTION (36) Test Engineer Investigation

16 | ACTIVITY CONTENT (37) L (38) M (39) AMOUNT OF ACTIVITY (40) 22.8 millicuries (41) LOCATION OF RELEASE (42) From 3DHPSW pump to intake structure

17 | PERSONNEL EXPOSURES (43) NUMBER (44) 020 (45) TYPE (46) E (47) DESCRIPTION (48) Max dose rate -1R/HR; 73 maint.-9.1R, 29oper-8R, 4Eng-.2H

18 | PERSONNEL INJURIES (49) NUMBER (50) 000 (51) DESCRIPTION (52) NA

19 | LOSS OF OR DAMAGE TO FACILITY (53) TYPE (54) Z (55) DESCRIPTION (56) NA

20 | PUBLICITY (57) N (58) DESCRIPTION (59) NA

20 | ISSUED (60) N (61) DESCRIPTION (62) NA

8301260462 830111
PDR ADOCK 05000278 PDR
S

M.J. Cooney

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