



Ralph E. Beedle
Executive Vice President
Nuclear Generation

December 28, 1993
JPN-93-084

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

SUBJECT: James A. FitzPatrick Nuclear Power Plant
Docket No. 50-333
Individual Plant Examination

- References:
1. NYPA letter, R. E. Beedle to the NRC dated September 1, 1992 (JPN-92-046), providing NYPA's response to an NRC Request for Additional Information on the Individual Plant examination for the FitzPatrick Nuclear Power Plant.
 2. NYPA letter, R. E. Beedle to the NRC dated September 13, 1991 (JPN-91-048), providing the Individual Plant Examination for the FitzPatrick Nuclear Power Plant.

Dear Sir:

During the NRC's review of the Authority's response to a Request for Additional Information on the FitzPatrick Individual Plant Examination (IPE) (References 1 and 2), the NRC identified a potential concern with the depth of concrete between the bottom of the drywell sump and the containment liner beneath. In the highly unlikely event of a core-melt accident which propagates through the bottom of the reactor vessel, the molten core debris would collect in the drywell sumps located beneath the reactor. Localized molten core-concrete interactions (MCCI) may thus ablate the floor and possibly fail the containment liner. The distance between the bottom of the sump and the containment liner beneath was stated erroneously in Reference 1, Response to Item 25 as being "only a few inches."

A recent review of plant drawings has determined that the actual distance between the bottom of the drywell sump and the containment liner beneath is 4 ft.- 8 in. at the reactor center line, not "a few inches" as described in the IPE RAI response quoted above. The clarification makes the containment failure scenario described above even more unlikely. This containment failure mode was not considered generically by the NRC in NUREG/CR-5423, and the FitzPatrick IPE likewise did not consider this scenario in the quantification of the IPE results. Since this containment failure scenario was not quantified in the IPE, this new information has no effect on the numeric or qualitative results of the IPE.

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If you have any questions, please contact Mr. J. A. Gray, Jr.

Very truly yours,


Ralph E. Beedle

cc: Regional Administrator
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