

U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT

REGION III

Report No. 50-254/78-19; 50-265/78-20

Docket No. 50-254; 50-265 License No. DPR-29; DPR-30

Licensee: Commonwealth Edison Company
P. O. Box 767
Chicago, IL 60690

Facility Name: Quad-Cities Nuclear Power Station, Units 1 and 2

Inspection At: Quad-Cities Site, Cordova, IL

Inspection Conducted: July 13-14, 19-20, 26-27, August 2-3 and
10-11, 1978

I. N. Jackiw
Inspector: I. N. Jackiw

R. C. Knop
Approved By: R. C. Knop, Chief
Reactor Projects Section 1

6/15/78

9/18/78

Inspection Summary

Inspection on July 13-14, 19-20, 26-27 and August 2-3, 10-11, 1978
(Reports No. 50-254/78-19 and No. 50-265/78-20)

Areas Inspected: Review of plant operations and out-of-service activities; review of safety limits, limiting safety system settings and limiting conditions for operations; review of procurement activities; review of rad-waste barrelling activities; 10 CFR Part 21 review; and licensee event report followup. The inspection involved 74 inspector-hours onsite by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

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DETAILS

1. Persons Contacted

- *N. Kalivianakis, Station Superintendent
- K. Graesser, Administrative Assistant
- J. Gudac, Assistant Superintendent
- *L. Gerner, Technical Staff Supervisor
- R. Bax, Operating Engineer, Unit 2
- G. Conschack, Operating Engineer, Unit 1
- T. Tamlyn, Lead Operating Engineer
- *D. Thayer, Quality Assurance Engineer, Maintenance
- R. Demig, Assistant Storekeeper

The inspector also interviewed several other licensee employees including shift engineers and foremen, reactor operators, rad-waste operating personnel, technical staff personnel, storeroom personnel and quality control personnel.

* Denotes those present at the exit interview.

2. Licensee Action on Previous Inspection Findings

(Closed) Unresolved Item (254/77-26): Inoperable Control Rod Accumulators. The inspector confirmed that the licensee has been in contact with NRR and has agreed to comply with the intent of Technical Specification 3.3.D and 3.3.G which states that no two control rods can be inoperable in any nine-rod array configuration.

3. Procurement

Procurement activities were reviewed to ascertain whether the purchase of components, materials and supplies used for safety related functions, is in conformance with the licensee's approved quality assurance program and implementing procedures. The inspector verified that procurement specifications used in the purchase of material included proper approval; quality control inspection requirements; quality record requirements; and evidence of material conformance to procurement requirements. The following components were inspected: Standby Gas Treatment System isolation valve; 250V Battery Charger parts; and three valve manifold for station surveillance.

The inspector also confirmed that spare parts used in safety related functions were supplied by an approved vendor; were

inspected upon delivery; and are being handled in accordance with measures established for control and separation of conforming and nonconforming materials, parts and components.

No items of noncompliance or deviations were identified.

4. Solid Radwaste

Process wastes, such as filter sludges and spent resins are collected in hoppers, dewatered, loaded into 55-gallon drums and mixed with a solidifying material. After loading, the drums are moved to a capping station where drum lids are attached and secured. The capping area is separated from the operator area by a concrete wall penetrated by an opening through which the band and lid holder is moved into the capping area.

The inspector observed the solid radwaste barreling operation and noted that the drum filling and capping operations are performed with remote equipment and conveyors.

No items of noncompliance or deviations were identified.

5. Limiting Safety System Settings and Limiting Conditions for Operations

This area was reviewed to ascertain whether reactor operations are in conformance with Technical Specification requirements for safety limits, limiting safety system settings and limiting conditions for operations.

The review included observation of process instrumentation monitoring current operations; observation of panels and annunciator boards for equipment "out of service;" review of monitoring instrumentation charts; and review of other records such as maintenance, scram and deviation records.

No items of noncompliance or deviations were identified.

6. Out of Service Task Force Conclusions

Equipment outage control events that occurred at Dresden have been reviewed and evaluated by station management. The following actions have been taken relative to the Investigating Committee's recommendations:

- a. Emphasis for rigorous compliance with the out-of-service procedures.

- b. Procedures dealing with shift change, operating logs and equipment control and tagging have been added to the required reading list for all operating license holders.
- c. Assignments of Technical Staff Engineers have been made to work with station Operating Engineers on specific projects during normal plant operation and for short outages.

No items of noncompliance or deviations were identified.

7. 10 CFR Part 21 Inspection

This area was inspected to ascertain whether the licensee and individuals subject to Part 21 regulations have established and are implementing procedures and controls to assure the reporting of defects and noncompliance. The inspector verified that the following controls have been established for implementation of Part 21 requirements:

- a. Quality Procedures 15-51, 15-52 and 15-53 have been revised to include evaluation of deviations with regard to Part 21 requirements and informing the Commission when receiving information of a defect or reportable failure to comply.
- b. Maintenance of Part 21 records will be in accordance with 10 CFR Part 50 requirements.
- c. The Part 21 document has been posted at the facility.

Discussions with plant personnel revealed that no system exists for the tracking and maintaining of Part 21 reports and documentation generated by the licensee's contractors and subcontractors. The inspector stated that this item will be reviewed during a subsequent IE inspection.

No items of noncompliance or deviations were identified.

8. Review of Plant Operations

The inspector reviewed facility operations to ascertain whether they are in conformance with Technical Specification requirements, 10 CFR requirements and in accordance with administrative procedures. The inspection included reviews of selected operating logs, daily orders, jumper logs, licensee deviation reports and a tour of the facility.

The following records were reviewed:

- a. Units 1 and 2 control room logs for April, May, June and July 1978.
- b. Daily Order Book for April, May, June and July 1978.
- c. Jumper Log for the period April to July 1978.
- d. Deviation Reports D4-1-77-25 to D4-1-77-56 for Unit 1 and D4-2-77-25 to Dr-2-77-45 for Unit 2.

A plant tour of the Units 1 and 2 areas determined that monitoring instrumentation is recording as required, radiation controls are being properly implemented, plant housekeeping conditions are adequate, existence of fluid leaks have been identified and are being properly controlled, pipe hanger/seismic restraint settings and oil levels are normal, valve positions and equipment start switches are as required, equipment caution and out-of-service tag control is proper, responses to control room annunciators are adequate, and control room manning is in conformance with 10 CFR 50.54(k) and facility Technical Specifications.

9. Office Review of Licensee Event Reports

The inspector reviewed licensee's event reports to ascertain whether corrective actions discussed in the reports appear appropriate and whether information reported to the NRC satisfies reporting requirements. The following event reports were reviewed and are considered closed:

Unit 1

RO 78-16/03L dated June 26, 1978, "Vacuum Breaker 1601-33E Would Not Close."

RO 78-18/03L dated July 14, 1978, "Low-Low Reactor Level Switch Failed."

RO 78-20/03L dated July 7, 1978, "Drywell Equipment Sump Discharge Valve Inoperable."

10. Site Review of Licensee Event Reports

Licensee Event Reports were reviewed to verify that details were accurately reported; that the cause was identified; that the event was reviewed and evaluated in accordance with Technical Specifications; and the corrective action was taken as described in the report. The following LER's were reviewed and are considered closed:

No items of noncompliance or deviations were identified

Unit 1

RO 78-18/03L dated July 14, 1978, "Low-Low Reactor Level Switch Malfunctioned." On June 15, 1978, while performing reactor water level functional testing, switch LIS-1-263-72A failed to operate as required. Failure of the switch is attributed to instrument setpoint drift cause by magnet misalignment. The inspector verified that the switch was adjusted and retested satisfactorily.

Unit 2

RO 78-20/03L dated May 8 and 16, 1978, "RCIC Inboard Steam Inlet Valve Inoperable." On April 9, 1978, while performing monthly valve operability tests, valve 1301-16 would not close. Cause of the problem is attributed to torque switch failure. The inspector verified that the valve was made operable and also confirmed that a modification has been initiated to the valve control circuit to preclude valve closure if a torque switch failure occurs.

11. Exit Interview

The inspector met with licensee representatives (denoted in Paragraph 1) on August 11, 1978. The inspector summarized the scope and findings of the inspection.

With regard to the maintenance of Part 21 records, the licensee stated that this matter would be reviewed.