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> PRAIRIE ISLAND NUCLEAR GENERATING PLANT Docket Nos. 50-282 License Nos. DPR-42 50-306 DPR-60

Potential Generic Problems With Refurbishment of Westinghouse DB-50 Reactor Trip Breakers

The purpose of this letter is to inform the NRC of a potentially generic problem experienced with a Westinghouse DB-50 Reactor Trip Breaker that was recently refurbished by Westinghouse.

Northern States Power has been sending Prairie Island DB-50 reactor trip breakers to the Westinghouse Replacement Component Services Facility for refurbishment over the last three years. Six of the ten Prairie Island DB-50 Reactor Trip Breakers have been refurbished to date. During the receipt inspection of one of these refurbished breakers in February of this year, nine of the breaker's Pole Unit Retaining Bolts were found to be improperly torqued. The bolt torque appeared to be well cutside the specifications as outlined by the Westinghouse technical manual for these breakers.

A Nonconforming Item Report was issued following discovery of the problem and the bolts were torqued to the value specified in the Westinghouse technical manual. The investigation that followed included an audit of the Westinghouse Engineering Control Instruction for the DB-50 breakers. That instruction was found to be deficient in that it did not require bolt torque values to be recorded. The investigation also raised a concern that the Westinghouse torque procedure did not adequately reference the torque specifications included in the technical manual for the DB-50 breakers.

Westinghouse acknowledged the findings of the Northern States Power investigation and committed to revising their Engineering Control Instruction procedure to include recorded torque values.

Norther States Power has requested that Westinghouse issue a Technical Bulletin to other plants using DB-50 reactor trip breakers drawing attention to the torquing requirements described in the technical manual for the DB-50 breakers.

The Northern States Power evaluation of this problem concluded that the improperly torqued bolts on the subject reactor trip breaker did not constitute a significant safety hazard at Prairie Island and thus was not reportable under 10 CFR Part 21. This letter is being provided for the information of the NRC Staff due to the potential generic implications of the deficiencies in the Westinghouse torquing procedures.

Please contact us if you have any questions or require any further information on this issue.

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Manager

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c: Regional Administrator - Region III, NRC Senior Resident Inspector, NRC NRR Project Manager, NRC G Charnoff