



Commonwealth Edison

Quad Cities Nuclear Power Station
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Carrtova, Illinois 61242-9740
Telephone 309/654-2241

July 31, 1990

Dr. T. E. Murley
Office of Nuclear Reactor Regulations
U.S. Nuclear Regulatory Commission
Washington, DC 20555

ATTN: Document Control Desk

Subject: Quad Cities Nuclear Power Station Units 1 and 2
Response to Generic Letter 90-04; Request for Information on the
Status of Licensee Implementation of Generic Safety Issues
Resolved with Imposition of Requirements or Corrective Actions.
NRC Docket Numbers 50-254 and 50-265

Reference: (a) J. G. Partlow letter to all holders of Operating Licenses
dated April 25, 1990

(b) T. J. Kovach to T. E. Murley letter dated June 28, 1990

Dr. Murley,

Reference (a) requested that the current implementation status of all generic safety issues (GSI) identified in Generic Letter 90-04 be provided to the Staff by June 29, 1990. Reference (b) requested that the deadline for the submittal be extended to July 31, 1990.

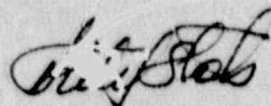
The attached document provides the status for the generic safety issues defined in Generic Letter 90-04. Commonwealth Edison has provided the information in accordance with the guidance established in the Generic Letter. Commonwealth Edison has also provided additional information which describes the basis for the status determination to assist in your Staff's review of the issues.

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If there are any questions or comments regarding this submittal, please direct them to me at 708/515-7283.

Very truly yours,

A handwritten signature in cursive script, appearing to read "Rita Stols".

Rita Stols
Nuclear Licensing Administrator

RS/jmt

cc: A. B. Davis, Regional Administrator
L. N. Olshan, Project Manager
T. Taylor, Senior Resident Inspector

FACILITY NAME: Quad Cities
DOCKET NO.: 254 (Unit 1), 265 (Unit 2)
LICENSEE: Commonwealth Edison

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

| <u>GS1/(MFA No.)</u> | <u>TITLE</u> | <u>APPLICABILITY</u> | <u>STATUS</u> | <u>COMMENTS</u> |
|----------------------|---|----------------------|---------------|---|
| 40 (B065) | Safety Concerns Associated With Pipe Breaks In The BWR Scram System | All BWRs | C | Letters dated 01/19/82, 01/25/82 and 02/23/82 from T. J. Rausch to D. G. Eisenhut provided the plant specific response to MAREG 0803. GL 81-34 evaluations, inspections, and analyses were completed with no further action required. New SDV was installed as required per IEB 80-17: M-4-1-81-021 Completed 08/07/84; additional work completed 03/19/86, 11/20/89. M-4-2-81-021 Completed 05/01/85 |
| 41 (B058) | BWR Scram Discharge Volume (SDV) Systems | All BWRs | C | Initial procedure changes were completed between 07/80 and 07/81. New SDV was installed as follows: M-4-1-81-021 Completed 08/07/84; additional work completed 03/19/86, 11/20/89. M-4-2-81-021 Completed 05/01/85 Modification-related procedure changes were completed between 05/85 and 07/85. R. F. Meishman letter to C. Reed dated 10/15/80 (Inspection report 50-254/80-19, 50-265/80-21) documented the NRC Resident Inspector's review of IEB 80-14. No items of noncompliance were identified. |
| 43 (B107) | Reliability of Air Systems | All Plants | I | CECo will issue letter providing the status of the required actions for GL 88-14 by 08/15/90. |

| GSI/(MPA No.) | TITLE | APPLICABILITY | STATUS | COMMENTS | | | | | | | | | | | | | | | | |
|----------------------|---|---------------|--------|---|-------------------|------------------------------------|----------------------|----------------------------|-------------------|---|-------------------|----------------------------|-----------------|----------------------------|-------------------|----------------------------|---------------|---------------------------|--------------------|---------------------------|
| 51 (L913) | Improving the Reliability of Open-Cycle Service Water Systems | All Plants | I | <p>M. H. Richter letter to NRC dated 01/29/90 provided the schedule for implementation (dates are based on currently scheduled start dates of refueling outage):</p> <table border="0"> <tr> <td>Intake inspection</td> <td>prior to Q1R11, 11/90 (both units)</td> </tr> <tr> <td>Biocide addition mod</td> <td>Q1R12, 03/92; Q2R11, 09/91</td> </tr> <tr> <td>Loop A test/insp.</td> <td>Q1R11, 11/90; Q1R13, 10/93, Q2R11, 09/91; Q2R13, 09/94</td> </tr> <tr> <td>Loop B test/insp.</td> <td>Q1R12, 03/92; Q2R12, 03/93</td> </tr> <tr> <td>E/C inspection,</td> <td>Q1R11, 11/90; Q2R11, 09/91</td> </tr> <tr> <td>Corrosion coupons</td> <td>Q1R12, 03/92; Q2R11, 09/91</td> </tr> <tr> <td>Design Review</td> <td>Q1R11, 11/90 (both units)</td> </tr> <tr> <td>Maintenance Review</td> <td>Q1R11, 11/90 (both units)</td> </tr> </table> <p>Inspection Report 50-254/83-28; 50-265/83-27 documented the Resident Inspector's review of IEB 81-03 inspection and testing programs. No concerns were identified. This issue was reviewed again at Region III's request (I.R. 50-254/84-14; 50-265/84-12). No items of noncompliance or deviations were found.</p> | Intake inspection | prior to Q1R11, 11/90 (both units) | Biocide addition mod | Q1R12, 03/92; Q2R11, 09/91 | Loop A test/insp. | Q1R11, 11/90; Q1R13, 10/93, Q2R11, 09/91; Q2R13, 09/94 | Loop B test/insp. | Q1R12, 03/92; Q2R12, 03/93 | E/C inspection, | Q1R11, 11/90; Q2R11, 09/91 | Corrosion coupons | Q1R12, 03/92; Q2R11, 09/91 | Design Review | Q1R11, 11/90 (both units) | Maintenance Review | Q1R11, 11/90 (both units) |
| Intake inspection | prior to Q1R11, 11/90 (both units) | | | | | | | | | | | | | | | | | | | |
| Biocide addition mod | Q1R12, 03/92; Q2R11, 09/91 | | | | | | | | | | | | | | | | | | | |
| Loop A test/insp. | Q1R11, 11/90; Q1R13, 10/93, Q2R11, 09/91; Q2R13, 09/94 | | | | | | | | | | | | | | | | | | | |
| Loop B test/insp. | Q1R12, 03/92; Q2R12, 03/93 | | | | | | | | | | | | | | | | | | | |
| E/C inspection, | Q1R11, 11/90; Q2R11, 09/91 | | | | | | | | | | | | | | | | | | | |
| Corrosion coupons | Q1R12, 03/92; Q2R11, 09/91 | | | | | | | | | | | | | | | | | | | |
| Design Review | Q1R11, 11/90 (both units) | | | | | | | | | | | | | | | | | | | |
| Maintenance Review | Q1R11, 11/90 (both units) | | | | | | | | | | | | | | | | | | | |

GSI/(MPA No.)

TITLE

APPLICABILITY

STATUS

COMMENTS

67.3.3 (A017)

Improved Accident Monitoring

All Plants

I

GL 82-33 and R.G. 1.97, Revision 2 Items as follows:

SPDS Letter dated 08/25/86 from J.A. Zwolinski to D.L. Farrar

RG 1.97 Letter dated 08/16/88 from T. Ross to H.E. Bliss accepted CECO response to R.G. 1.97 with the exception of wide range neutron monitoring. Five items which have not been completed were identified during the conduct of IR 50-254/88027; 50-265/88028 dated 12/13/88.

Wide Range Neutron Monitoring System

This item is currently being negotiated with NRR by BWROG. Reference letter from T. Kovach to T. Murley dated June 26, 1990.

Range and Taps for Rx Level Instruments

This issue is under review by NRR.

Separation of Safety and Non-Safety Related Equipment

M-4-1(2)-88-101 is currently scheduled to be completed by the end of Q1R11 (11/90) on Unit One and the end of Q2R11 (9/91) on Unit Two.

Torus Temperature Recorder

Minor Design Change 90-041 is currently scheduled for Q1R11 (11/90) and Q2R11 (9/91).

Labeling Identification of R.G. 1.97 Instrumentation

The review of labeling identification is scheduled to be completed by September, 1990.

DCRDR M-4-1-87-051 will be completed during Q1R12 which is currently scheduled for 3/92.

M-4-2-87-051 will be completed during Q2R12 which is currently scheduled for 3/93.

EOP's EOP Team Inspection (50-2254(265)/88200) identified various deficiencies which required revision to EOPs to achieve resolution. The EOP revision was completed in April, 1989. NRR Follow-up Inspection was conducted in November, 1989 (IR 50-254(265)/90006). Although concerns were noted with management involvement in EOP development, no deficiencies with 4/89 procedures were identified.

ERF's A-model is scheduled to be completed by 03/29/91.

| <u>GSI/MPA No.</u> | <u>TITLE</u> | <u>APPLICABILITY</u> | <u>STATUS</u> | <u>COMMENTS</u> |
|--------------------|--|----------------------|---------------|--|
| 75 (B076) | Item 1.1 - Post-Trip Review (Program Description and Procedure) | All Plants | C | Letter dated 07/30/85 from D.B. Vassallo to D.L. Farrar |
| 75 (B085) | Item 1.2 - Post-Trip Review - Data and Information Capability | All Plants | C | Letter dated 03/13/86 from J.A. Zwolinski to D.L. Farrar |
| 75 (B077) | Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components) | All Plants | C | Letter dated 02/22/89 from T.M. Ross to H.E. Bliss (Part 2) Letter dated 08/19/87 from T.M. Ross to L.D. Butterfield (Part 1) |
| 75 (B086) | Item 2.2.1 - Equipment Classifica- tion for Safety-Related Components | All Plants | C | Letter dated 11/28/89 from B.L. Siegel to T.J. Kovach |
| 75 (L003) | Item 2.2.2 - Vendor Interface for Safety-Related Components | All Plants | I/E | Clarification of NRC Staff position is provided in GL 90-03. CECo response will be submitted by 09/26/90. |
| 75 (B078) | Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components) | All Plants | C | Letter dated 05/31/85 from D.B. Vassallo to D.L. Farrar |
| 75 (B079) | Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components) | All Plants | C | Letter dated 09/25/85 from J. A. Zwolinski to D. L. Farrar |
| 75 (B087) | Items 3.2.1 & 3.2.2 - Post-Mainten- ance Testing (All Other Safety- Related Components) | All Plants | C | Letter dated 05/31/85 from D.B. Vassallo to D.L. Farrar |
| 75 (B088) | Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components) | All Plants | C | Letter dated 09/25/85 from J.A. Zwolinski to D.L. Farrar |
| 75 (B080) | Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications) | All Plants | NA | GL 83-28 states this applicable to all PWRs |

| <u>GSI/IMPA No.</u> | <u>TITLE</u> | <u>APPLICABILITY</u> | <u>STATUS</u> | <u>COMMENTS</u> |
|---------------------|---|----------------------|---------------|---|
| 75 (B081) | Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability - Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers) | All PWRs | NA | |
| 75 (B082) | Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants) | All W and B&W Plants | NA | |
| 75 (B090) | Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants) | All W & B&W Plants | NA | |
| 75 (B091) | Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants) | All B&W Plants | NA | |
| 75 (B092) | Item 4.5.1 - Reactor Trip System Reliability - Diverse Trip Features (System Functional Testing) | All Plants | C | Letter dated 05/31/85 from D.B. Vassallo to D.L. Farrar |
| 75 (B093) | Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing) | All Plants | C | Letter dated 02/03/89 from T.M. Ross to H.E. Bliss (Item 4.5.2) Letter dated 05/17/90 from L.N. Olshan to T.J. Kovach (Item 4.5.3) |

| <u>GSI/(MPA No.)</u> | <u>TITLE</u> | <u>APPLICABILITY</u> | <u>STATUS</u> | <u>COMMENTS</u> |
|----------------------|--|--|---------------|--|
| 86 (B084) | Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping | All BWRs | I | Commitments required by IEB 82-03 (including Revision 1), IEB 83-02, and GL 84-11 are complete. Those commitments which have been superseded by GL 88-01 commitments are also complete. Three GL 88-01 commitments are not complete: Leakage Measurement Requirements - These requirements are being incorporated into the Tech Specs as part of the Tech Spec Improvement Program which should be implemented by August of 1991. Hydrogen Addition - Modifications (M-4-0-86-011) are scheduled for completion by 1992. Change to ISI Program - Inspections are conducted each outage in accordance with GL 88-01. The ISI program will be revised to reflect these inspections by October, 1990. |
| 93 (B098) | Steam Binding of Auxiliary Feedwater Pumps | All PWRs | NA | |
| 99 (L817) | RCS/RHR Suction Line Valve Interlock on PWRs | All PWRs | NA | |
| 124 | Auxiliary Feedwater System Reliability | ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3, Ft. Calhoun | NA | |
| A-13 (B017) | Snubber Operability Assurance - Hydraulic Snubbers | All Plants | C | Safety-related hydraulic snubbers replacement (with mechanical snubbers) was completed in 1985 as required by IEB 79-14. Hydraulic snubbers were removed from Tech Specs by Amendments 115 (Unit 1) and 111 (Unit 2) approved by SER dated 02/22/89. |
| A-13 (B022) | Snubber Operability Assurance - Mechanical Snubbers | All Plants | C | Requirements for snubber operability, inspections, testing, and monitoring are defined in Technical Specification 3.6.I/4.6.I. Procedure (QTS 180-2) for mechanical snubber inspection was created in 05/82. Procedure (QAP 350-6) which reflects current Inservice Testing Program requirements for standard snubber testing methods was created in 06/89. In-place inservice testing of snubbers is NA to Station. |

| <u>GSI/MPA No.</u> | <u>TITLE</u> | <u>APPLICABILITY</u> | <u>STATUS</u> | <u>COMMENTS</u> |
|--------------------|--|--|---------------|---|
| A-16 (D012) | Steam Effects on BWR Core Spray Distribution | Oyster Creek & NMP-1 | NA | |
| A-35 (B023) | Adequacy of Offsite Power Systems | All Plants | C | Second level undervoltage (degraded voltage) relays were installed under following modifications: M-4-1-80-015, Completed 05/05/84 M-4-2-80-015, Completed 02/10/84 M-4-1-82-029, Completed 05/01/84 M-4-2-82-029, Completed 12/20/83 M-4-1/2-82-029, Completed 05/05/84 Changes were incorporated into Technical Specification Table 3.2-2, Table 4.2-1, 4.9.A.1, and 4.9.E.2. |
| B-10 | Behavior of BWR Mark III Containments | All BWR Mark III Plants | NA | Quad Cities is a Mark I Plant. |
| B-35 | Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems | All Plants with OL Applications After 4/1/80 | NA | Quad Cities received its operating licenses in 1972. |
| B-63 (B045) | Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary | All Plants | NC | W.F. Naughton letter to D.G. Eisenhut dated 03/14/80 transmitted the requested information. Response was provided for GL 87-06 by M.S. Turbak letter to NRC dated 06/11/87. No further actions were required. |

SAFETY INSPECTION

90-01

1. LICENSEE

VA Dept of TRANSPORTATION
MATERIALS Division
1401 EAST BROAD STREET
RICHMOND VA 23219

2. REGIONAL OFFICE

Region II Suite 2900
101 MARIETTA ST. NW
Atlanta GA 30323

3. DOCKET NUMBER(S)

30-06624

4. LICENSE NUMBER(S)

45-13380-01

5. DATE OF INSPECTION

MAY 31, 1990

Licensee:

The inspection was an examination of the activities conducted under your license as they relate to radiation safety and to compliance with the Nuclear Regulatory Commission's (NRC) rules and regulations and the conditions of your license. The inspection consisted of selective examinations of procedures and representative records, interviews, with personnel, and observations by the inspector. The findings as a result of this inspection are as follows:

- 1. Within the scope of this inspection, no violations were observed.
- 2. The inspector also verified the steps you have taken to correct the violations identified during the last inspection. We have no further questions on those actions at this time.
- 3. During this inspection certain of your activities, as checked below, were in violation of NRC requirements.
THIS IS A NOTICE OF VIOLATION which is required to be posted in accordance with 10 CFR 19.11.
 - A. _____ was not properly posted to indicate the presence of a _____ 10 CFR 20.203(b), (c), (d), (e) or 34.42.
 - B. Containers located in _____ were not properly labeled to indicate the presence of radioactive material. 10 CFR 20.203(f)(1), or (f)(2).
 - C. _____ of sealed sources were not performed at the proper frequencies. 10 CFR _____ License Condition Number _____.
 - D. Records of _____ were not properly maintained. 10 CFR _____ or License Condition Number _____.
 - E. Documents were not properly posted or otherwise made available. 10 CFR 19.11.
 - F. Reports or notifications of _____ were not made in accordance with 10 CFR _____ or License Condition Number _____.
 - H. _____
 - I. _____
 - J. _____
 - K. _____

I hereby state that within 30 days the actions described by me to the inspector will be taken to correct the violations identified in the items checked above. This statement of corrective actions is made in accordance with the requirements of 10 CFR 2.201. No further response will be submitted unless required by the NRC.

[Handwritten signature] 5/31