



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
799 ROOSEVELT ROAD
GLEN ELLYN, ILLINOIS 60137

DCD/DCB
RIDS

OCT 23 1990

Docket No. 50-456

Commonwealth Edison Company
ATTN: Mr. Cordell Reed
Senior Vice President
1400 Opus Place
Downers Grove, IL 60515

Gentlemen:

This refers to the special onsite review conducted by the NRC Augmented Inspection Team composed of Messrs. W. D. Shafer, W. J. Kropp, S. G. DuPont, and J. A. Hopkins of this office, Mr. S. P. Sands, and S. Diab of the Office of Nuclear Reactor Regulation (NRR), E. A. Trager, Jr., Office for Analysis and Evaluation of Operational Data (AEOD) and J. L. Harbour, of Idaho National Engineering Laboratory (INEL) on October 4-6, 1990, of activities associated with the October 4, 1990, loss of reactor coolant system inventory event at the Braidwood Station, Unit 1, Operating License No. NPF-72, and to the discussion of our findings with Messrs. K. L. Graesser, K. L. Kofron, and others of your staff at the conclusion of the inspection.

The enclosed copy of our Augmented Inspection Team report identifies areas examined during the inspection. Within these areas, the inspection consisted of a selective examination of procedures and representative records, observations, and interviews with personnel.

No violations of NRC requirements were identified during the course of this inspection; however, the major purpose of the AIT was to perform onsite fact finding regarding the event. Issues identified by the AIT may be examined for possible regulatory violations in subsequent inspections. We are, however, concerned with the lack of command and control demonstrated by the operating crew during the March 18, 1990, loss of inventory event and again during the October 4, 1990, event. The corrective actions implemented as a result of the first event were apparently not sufficient to prevent a similar occurrence. In this regard, we plan to meet with you to discuss the corrective actions you have or will implement as a result of the concerns identified (Section VII of the enclosed report) during the AIT investigation and the results of your own investigation. We will contact you to schedule this meeting once we have received the results of your investigation and conclusions.

The AIT also concluded that while the October 4, 1990 loss of inventory event was not an intersystem loss of coolant accident (ISLOCA), the act of conducting multiple surveillances on systems with a low/high pressure interface, coupled with the loss of command and control by the Operations Department, is considered a precursor to an ISLOCA. We request that you be prepared to discuss this matter in our forthcoming meeting.

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In accordance with 10 CFR 2.790 of the Commission's regulations, a copy of this letter and the enclosed inspection report will be placed in the NRC Public Document Room.

We will gladly discuss any questions you have concerning this inspection.

Sincerely,


for Edward G. Greenman, Director
Division of Reactor Projects

Enclosure: Augmented Inspection
Team Report No. 50-456/90-020(DRP)

See Attached Distribution

Distribution

cc w/enclosures:
M. Wallace, Vice President,
PWR Operations
T. Kovach, Nuclear
Licensing Manager
A. Checca, Nuclear
Licensing Administrator
K. L. Kofron, Station Manager
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