Docket No. 50-219

Mr. E. E. Fitzpatrick Vice President and Director Oyster Creek Nuclear Generating Station P.C. Box 388 Forked River, New Jersey 08731

Dear Mr. Fitzpatrick:

SUBJECT: INTERGRANULAR STRESS CORROSION CRACKING (IGSCC) - INSPECTION AND

REPAIRS FOR OYSTER CREEK NUCLEAR GENERATING STATION (OCNGS)

(TAC NO. 67183)

The NRC staff has completed its review of GPU Nuclear Corporation's (licensee) submittals dated January 20, 23, and 31, 1989, October 30, 1989, and February 21, 1990 regarding the IGSCC inspection and repairs, additional information and future inspection plans for the Reactor Water Cleanup System (RWCU) during the 13R refueling outage at OCNGS. The licensee reported that a total of 150 welds susceptible to IGSCC in various stainless steel piping systems were inspected during this outage and that six welds were found to contain IGSCC indications. Five flawed welds were repaired with standard weld overlays and one was evaluated by fracture mechanics as acceptable without repair for one fuel cycle. In addition, IHSI was effectively applied to forty-two welds, and six structural weld overlays were inspected for IGSCC and no indications of IGSCC were found. Furthermore, the licensee submitted its 13R IGSCC inspection plan to inspect 10% of the RWCU weld population outboard of the containment isolation valves.

Based on the review of the information provided by the licensee the staff finds the inspection and overlay repairs that were performed meet the guidelines for GL-88-CI acceptable with the exception of the inspection scope of Category G welds (welds not yet being properly inspected) and the proposed RWCU IGSCC inspection plan subsequent to 13R. Because of the timing of GL-88-OI issuance, the reduced inspection scope of Category G welds for 12R refueling outage was accepted. The staff's review, and conclusions are contained in the enclosed Safety Evaluation.

Sincerely.

original signed by

Alexander W. Dromerick, Senior Project Manager Project Directorate I-4 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosure: As stated

cc w/enclosure: See next page

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## UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

October 18, 1990

Do Met 40. 50-219

Mr. E. E. Fitzpatrick Vice President and Director Oyster Creek Nuclear Generating Station P.O. Box 388 Forked River, New Jersey 08731

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SUBJECT: INTERGRANULAR STRESS CORROSION CRACKING (IGSCC) - INSPECTION AND

REPAIRS FOR OYSTER CREEK NUCLEAR GENERATING STATION (OCNGS)

(TAC NO. 67183)

The NRC staff has completed its review of GPU Nuclear Corporation's (licensee) submittals dated January 20, 23, and 31, 1989, October 30, 1989, and February 21, 1990 regarding the IGSCC inspection and repairs, additional information and future inspection plans for the Reactor Water Cleanup System (RWCU) during the 13R refueling outage at OCNGS. The licensee reported that a total of 150 welds susceptible to IGSCC in various stainless steel piping systems were inspected during this outage and that six welds were found to contain IGSCC indications. Five flawed welds were repaired with standard weld overlays and one was evaluated by fracture mechanics as acceptable without repair for one fuel cycle. In addition, IHSI was effectively applied to forty-two welds, and six structural weld overlays were inspected for IGSCC and no indications of IGSCC were found. Furthermore, the licensee submitted its 13R IGSCC inspection plan to inspect 10% of the RMCU weld coulation outboard of the containment isolation valves.

Based on the review of the information provided by the licensee, the staff finds the inspection and overlay repairs that were performed meet the guidelines for GL-88-01 acceptable with the exception of the inspection scope of Category G welds (welds not yet being properly inspected) and the proposed RWCU IGSCC inspection plan subsequent to 13R. Because of the timing of GL-88-01 issuance, the reduced inspection scope of Category G welds for 12R refueling outage was accepted. The staff's review, and conclusions are contained in the enclosed Safety Evaluation.

Sincerel.

Alexander W. Dromerick, Senior Project Manager

Alexander Marmeret

Project Directorate I-4

Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosure: As stated

cc w/enclosure: See next page

Mr. E. E. Fitzpatrick Oyster Creek Nuclear Generating Station

Oyster Creek Nuclear Generating Station

cc:

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