

Duquesne Light Company

Beaver Valley Power Station
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JOHN D. SIEBER
Vice President - Nuclear Group

(412) 393-6255

July 31, 1990

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

Reference: Beaver Valley Power Station, Unit No. 1 and No. 2
BV-1 Docket No. 50-334, License No. DPR-66
BV-2 Docket No. 50-412, License No. NPF-73
Response to Bulletin 90-01

Gentlemen:

The following information is provided to satisfy the reporting requirements of NRC Bulletin 90-01.

Reporting requirement 1.a

Confirm that Items 1, 2, 3, 4, and 5 of Requested Actions for Operating Reactors have been completed.

Response

- Item 1 (identification of transmitters in Safety Related or ATWS Applications) is completed.
- As specified in Item 2, DLC has identified transmitters from the suspect manufacturing lots which are in reactor protection or ESF systems. A replacement plan is in place, but replacement is not complete.
- DLC has reviewed the performance history of identified transmitters in safety-related or ATWS applications (Item 3). Those exhibiting uncertain symptoms of oil loss have been placed on more frequent calibration schedules, are subject to a periodic calibration trend evaluation, and/or are periodically compared to redundant channel information. Transmitters with symptoms of sluggish response and sustained drift are planned for replacement.

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- Item 4 describes a proposed enhanced surveillance program. DLC has no plans for detecting sluggish response via actual planned or unplanned transients or tests since installed ability to gather such information is very limited. Detection of sluggishness during calibration activities is being performed and allows for a more controlled assessment of the transmitter's response. Detection of changes in process noise does not appear to be a reliable symptom of oil loss and is not being pursued. Other aspects of the described program have been implemented.
- Item 5 (Justification for Continued Operation) is completed.

Reporting Requirement 1.6

Identify the indicated manufacturer; the model number; the system the transmitter was utilized in; the approximate amount of time at pressure; the corrective actions taken; and the disposition (e.g., returned to vendor for analysis) of Rosemount Model 1153 Series B, Model 1153 Series D, and Model 1154 transmitters that are believed to have exhibited symptoms indicative of loss of fill-oil or have been confirmed to have experienced a loss of fill-oil. This should include Model 1153 Series B, Model 1153 Series D and Model 1154 transmitters manufactured after July 11, 1989.

Response

DLC has not identified Rosemount transmitters exhibiting clear symptoms of oil loss in any safety related or ATWS applications.

Reporting Requirement 1.C

Identify the system in which the Model 1153 Series B, 1153 Series D, and Model 1154 transmitters from the manufacturing lots that have been identified by Rosemount as having a high failure fraction due to loss of fill-oil are utilized and provides a schedule for replacement of these transmitters which are in use in the reactor protection or engineered safety features actuation systems.

Response

Attachment 1 provides a table of transmitters identified by Rosemount which are installed in reactor protection or ESF systems.

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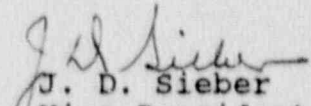
Reporting Requirement 2

Model 1153 Series B, Model 1153 Series D and Model 1154 transmitters that, subsequent to providing the response required by Item 1 above, exhibit symptoms of loss of fill-oil or are confirmed to have experienced a loss of fill-oil should be reviewed for reportability under existing NRC regulations. If determined not to be reportable, addressees are requested to document and maintain, in accordance with existing plant procedures, information consistent with that requested in Item 1b) above for each transmitter identified.

Response

Transmitters identified as having symptoms of oil loss in the future will be evaluated for reportability as required. Documentation, which is consistent with reporting requirement 1b, will be maintained for a reasonable time period (several years) for instances which do not require reporting to the NRC.

Very truly yours,


J. D. Sieber
Vice President
Nuclear Group

cc: Mr. J. Beall, Sr. Resident Inspector
Mr. T. T. Martin, NRC Region I Administrator
Mr. A. W. DeAgazio, Project Manager
Mr. R. Saunders (VEPCO)

ATTACHMENT 1

REACTOR PROTECTION OR ESF TRANSMITTERS FROM ROSEMOUNT SUSPECT LIST

<u>Mark Number</u>	<u>Plant</u>	<u>Model</u>	<u>System</u>	<u>Serial Number</u>	<u>Schedule for Replacement</u>
FT-RC-436	BVPS-1	1154HF5RB	Reactor Coolant	412771	8th Refueling Outage
2CCP-FT-107B	BVPS-2	1153DD5PA	RCP Thermal Barrier Flow	406044	2nd Refueling Outage
2QSS-LT-100A	BVPS-2	1153DB5PA	RWST Level	405684	September, 1991
2QSS-LT-100B	BVPS-2	1153DB5PA	RWST Level	405685	September, 1991

COMMONWEALTH OF PENNSYLVANIA))
COUNTY OF BEAVER) SS:

On this 30th day of July, 1990,
before me, Tracey A. Baczek, a Notary Public in and for said
Commonwealth and County, personally appeared J. D. Sieber, who being
duly sworn, deposed, and said that (1) he is Vice President - Nuclear
of Duquesne Light, (2) he is duly authorized to execute and file the
foregoing Submittal on behalf of said Company, and (3) the statements
set forth in the Submittal are true and correct to the best of his
knowledge, information and belief.

Tracey A. Baczek

Notarial Seal
Tracey A. Baczek, Notary Public
Shippingport Boro, Beaver County
My Commission Expires Aug. 16, 1993

Member, Pennsylvania Association of Notaries