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U.S. NUCLEAR REGULATORY COMMISSION
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Gentlemen:

DOCKET 50-266

POINT BEACH NUCLEAR PLANT, UNIT 1
FACILITY OPERATING LICENSE DPR-24
REACTOR CONTAINMENT BUILDING INTEGRATED LEAK RATE TEST

In accordance with Section 15.4.4.I.D of the Point Beach Nuclear Plant Technical Specifications and Section V. B. of 10 CFR 50 Appendix J, the following summary technical report is submitted on the Type A containment integrated leakage rate test conducted during the 1990 Unit 1 refueling outage. Also included is a summary of the results from the type B and C leak tests conducted since the last Type A test. These were performed during the refueling outages of April-May 1987, April-May 1988, and April-May 1989. The type A test was performed by Wisconsin Electric with assistance from Bechtel Power Corporation. We have enclosed a summary report prepared in accordance with the format specified in ANSI/ANS 56.8-1987. The complete documentation, which includes all procedures, equipment calibrations, test data, and results will be retained by Wisconsin Electric and is available for NRC review at the plant.

Summary of Data

The overall as-found mass point leakage rate of this Type A test was 0.067%/day at the 95% upper confidence level. This is 32% of the allowable operational leakage rate. The leakage rate reported included a 0.002%/day penalty for non-standard valve alignments, valve modifications, containment volume changes, and air additions to the containment.

The total as-found leakage of the Type B and C tests conducted during the 1987 refueling outage was greater than 9,498 standard cubic centimeters per minute (sccm), or greater than 4% of 0.6L_a.

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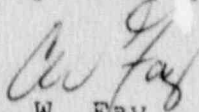
As described further in LER No. 87-003-00, dated May 20, 1987, the actual value could not be quantified because excessive leakage during one Type C test made it impossible to achieve test pressure. However, after repairs, the overall as-left leakage rate was 8,236 sccm, or 3% of $0.6L_a$.

The total as-found leakage of the type B and C tests conducted during the 1988 refueling outage was greater than 60,723 sccm, or greater than 25% of $0.6L_a$. As described further in LER No. 88-004-00, dated May 13,^a 1988, the actual value could not be quantified because excessive leakage during two separate Type C tests made it impossible to achieve test pressure. However, after repairs, the overall as-left leakage rate was 11,358 sccm, or 5% of $0.6L_a$.

The total as-found leakage of the Type B and C tests conducted during the 1989 refueling outage was greater than 20,943 sccm, or greater than 9% of $0.6L_a$. As described further in LER No. 89-002-00, dated May 12,^a 1989, the actual value could not be quantified because excessive leakage during one Type C test made it impossible to achieve test pressure. However, after repairs, the overall as-left leakage rate was 12,504 sccm, or 5% of $0.6L_a$.

If you have any questions concerning this report, please feel free to contact us.

Very truly yours,


C. W. Fay
Vice President
Nuclear Power

Enclosure

Copies to NRC Regional Administrator, Region III
NRC Resident Inspector