

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

5N 157B Lookout Place

**MAY 30 1990**

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

Gentlemen:

In the Matter of )  
Tennessee Valley Authority )

Docket Nos. 50-259  
50-260  
50-296

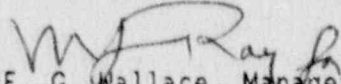
BROWNS FERRY NUCLEAR PLANT (BFN) - STATUS OF IMPLEMENTATION OF GENERIC SAFETY ISSUES (GENERIC LETTER 90-04)

This letter provides TVA's response to Generic Letter 90-04, Request for Information on the Status of Licensee Implementation of Generic Safety Issues (GSIs) Resolved with Imposition of Requirements or Corrective Actions, for BFN. Enclosure 1 provides the status of GSIs for BFN Unit 1. Enclosure 2 provides the status of GSIs for BFN Unit 2. Enclosure 3 provides the status of GSIs for BFN Unit 3.

This letter does not include new commitments. Forecast implementation dates provided for GSIs reflect previous correspondence and anticipated restart dates. If you have any questions, please telephone Patrick P. Carrier, Manager of Site Licensing, at (205) 729-3570.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

  
E. G. Wallace, Manager  
Nuclear Licensing and  
Regulatory Affairs

Enclosures  
cc: See page 2

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U.S. Nuclear Regulatory Commission

MAY 30 1990

cc (Enclosures):

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for Projects  
TVA Projects Division  
U.S. Nuclear Regulatory Commission  
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Rockville, Maryland 20852

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Region II  
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Atlanta, Georgia 30323

ENCLOSURE 1  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 1

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
40 (B065)	Safety Concerns Associated with Pipe Breaks in the BWR Scram System	All BWRs	Complete 1/3/86	Completion based upon issuance of Generic Letter (GL) 86-01, dated January 3, 1986. Implementation of visual verification assumption contained in the GL was never requested by the Staff and is not currently required by the ASME code.
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	Complete 6/27/84	Completion of the June 24, 1983 Order is certified by TVA letter, dated June 27, 1984.
43 (B107)	Reliability of Air Systems	All Plants	Implementing 1/95	TVA's response to GL 88-14 was submitted by letter dated February 23, 1989.
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	Implementing 1/93	TVA's response to GL 89-13 was submitted by letter dated March 16, 1990.
67.3.3 (A017)	Improved Accident Monitoring	All Plants	Implementing 1/95	Implementation with the exception of qualified neutron monitoring instrumentation. Refer to BWR Owners' Group February 21, 1990 letter (BWROG-9025).
75 (B076)	Generic Letter 83-28, Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	Complete 6/17/85	Completion based upon NRC Safety Evaluation, dated June 17, 1985, which stated the program and procedures were acceptable.



ENCLOSURE 1 (Continued)  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 1

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
75 (B085)	Generic Letter 83-28, Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	Implementing 1/95	Completion of item requires installation of upgraded process computer. Reference NRC's June 12, 1985 Safety Evaluation.
75 (B077)	Generic Letter 83-28, Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	Complete 5/1/89	Completion based upon NRC Safety Evaluations, dated September 2, 1986 (Item 2.1, Part 1) and May 1, 1989 (Item 2.1, Part 2). TVA's responses were found to be acceptable.
75 (B086)	Generic Letter 83-28, Item 2.2.1 - Equipment Classification for Safety Related Components	All Plants	Complete 6/1/89	Completion based upon NRC Safety Evaluation, dated June 1, 1989, which states TVA's response was acceptable.
75 (L003)	Generic Letter 83-28, Item 2.2.2 - Vendor Interface for Safety Related Components	All Plants	Under Evaluation	The NRC issued a revised position on this item in GL 90-03, dated March 20, 1990. TVA's response is required by September 25, 1990.
75 (B078)	Generic Letter 83-28, Items 3.1.1 and 3.1.2 - Post- Maintenance Testing (Reactor Trip System Components)	All Plants	Complete 8/15/88	TVA's commitments are reflected in the NRC's March 23, 1988 Safety Evaluation. These commitments were completed July 29, 1988 (Item 3.1.1) and August 15, 1988 (Item 3.1.2).

ENCLOSURE 1 (Continued)  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 1

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
75 (B079)	Generic Letter 83-28, Item 3.1.3 - Post-Maintenance Testing - Changes to Test Requirements (Reactor Trip System Components)	All Plants	Complete 10/27/86	Completion based upon NRC Safety Evaluation, dated October 27, 1986, which states TVA's response was acceptable.
75 (B087)	Generic Letter 83-28, Items 3.2.1 and 3.2.2 - Post- Maintenance Testing (All Other Safety Related Components)	All Plants	Complete 3/23/88	Completion based upon NRC Safety Evaluation, dated March 23, 1988, which states TVA's programs are acceptable.
75 (B088)	Generic Letter 83-28, Item 3.2.3 - Post-Maintenance Testing - Changes to Test Requirements (All Other Safety Related Components)	All Plants	Complete 10/27/86	Completion based upon NRC Safety Evaluation, dated October 27, 1986, which states TVA's response was acceptable.
75 (B080)	Generic Letter 83-28, Item 4.1 - Reactor Trip System Reliability (Vendor Related Modifications)	All Plants	Not Applicable	The applicability section for this item in GL 83-28 states: "This action applies to all PWR licensees and OL applicants."
75 (B081)	Generic Letter 83-28, Items 4.2.1 and 4.2.2 - Reactor Trip System Reliability - Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	Not Applicable	BFN is a BWR



ENCLOSURE 1 (Continued)  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 1

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
75 (B082)	Generic Letter 83-28, Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	Not Applicable	BFN is a BWR
75 (B090)	Generic Letter 83-28, Item 4.3 - Reactor Trip System Reliability - Tech. Spec. Changes (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	Not Applicable	BFN is a BWR
75 (B091)	Generic Letter 83-28, Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	Not Applicable	BFN is a BWR
75 (B092)	Generic Letter 83-28, Item 4.5.1 - Reactor Trip System Reliability - Diverse Trip Features (System Functional Testing)	All Plants	Complete 4/14/86	Completion based upon NRC Safety Evaluation, dated April 14, 1986, which states TVA's response was acceptable.
75 (B093)	Generic Letter 83-28, Items 4.5.2 and 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	Under Evaluation 1/95	Item 4.5.2 is considered complete based upon NRC Safety Evaluation, dated September 2, 1986, which states TVA's response was acceptable. Closure of Item 4.5.3 requires review for conformance to BWROG approved analysis.

ENCLOSURE 1 (Continued)  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 1

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	Completed 1/25/88	Completion based upon issuance of GL 88-01, dated January 25, 1988, which superseded GL 84-11. Actions resulting from GL 88-01 are tracked under USI A-42 (MPA B-05).
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	Not Applicable	BFN is a BWR
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	Not Applicable	BFN is a BWR
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft. Calhoan	Not Applicable	BFN is not listed
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	Complete 5/24/82	Completion based upon issuance of Technical Specification Amendment No. 84, dated May 24, 1982, which stated that the requirements of the Staff's November 20, 1981 Generic Letter are incorporated.

ENCLOSURE 1 (Continued)  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 1

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	Complete 5/24/82	Completion based upon issuance of Technical Specification Amendment No. 84, dated May 24, 1982, which stated that the requirements of the Staff's November 20, 1981 Generic Letter are incorporated.
A-16 (D-012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	Not Applicable	BFN is not listed
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	Complete 1981	Modifications are reflected in Technical Specification Amendment No. 75, dated September 3, 1981.
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	Not Applicable	BFN has a Mark I containment.
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	Not Applicable	BFN docketed its OL application on September 25, 1970.



ENCLOSURE 1 (Continued)  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 1

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	Complete 8/8/80	Completion based upon NRC letter to TVA, dated August 8, 1980, which states the valve configurations discussed in NRC's February 23, 1980 letter do not exist at BFN.

ENCLOSURE 2  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 2

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
40 (B065)	Safety Concerns Associated with Pipe Breaks in the BWR Scram System	All BWRs	Complete 1/3/86	Completion based upon issuance of Generic Letter (GL) 86-01, dated January 3, 1986. Implementation of visual verification assumption contained in the GL was never requested by the Staff and is not currently required by the ASME code.
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	Complete 12/11/88	Modifications described in TVA letter, dated June 27, 1984.
43 (B107)	Reliability of Air Systems	All Plants	Implementing 9/90	TVA's response to GL 88-14 was submitted by letter dated February 23, 1989.
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	Implementing 1/93	TVA's response to GL 89-13 was submitted by letter dated March 16, 1990.
67.3.3 (A017)	Improved Accident Monitoring	All Plants	Implementing 9/92	Implementation with the exception of qualified neutron monitoring instrumentation. Refer to BWR Owners' Group February 21, 1990 letter (BWROG-9025).
75 (B076)	Generic Letter 83-28, Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	Complete 6/17/85	Completion based upon NRC Safety Evaluation, dated June 17, 1985, which stated the program and procedures were acceptable.

ENCLOSURE 2 (Continued)  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 2

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
75 (B085)	Generic Letter 83-28, Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	Implementing 9/92	Completion of item requires installation of upgraded process computer. Reference NRC's June 12, 1985 Safety Evaluation.
75 (B077)	Generic Letter 83-28, Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	Complete 5/1/89	Completion based upon NRC Safety Evaluations, dated September 2, 1986 (Item 2.1, Part 1) and May 1, 1989 (Item 2.1, Part 2). TVA's responses were found to be acceptable.
75 (B086)	Generic Letter 83-28, Item 2.2.1 - Equipment Classification for Safety Related Components	All Plants	Complete 6/1/89	Completion based upon NRC Safety Evaluation, dated June 1, 1989, which states TVA's response was acceptable.
75 (L003)	Generic Letter 83-28, Item 2.2.2 - Vendor Interface for Safety Related Components	All Plants	Under Evaluation	The NRC issued a revised position on this item in GL 90-03, dated March 20, 1990. TVA's response is required by September 25, 1990.
75 (B078)	Generic Letter 83-28, Items 3.1.1 and 3.1.2 - Post-Maintenance Testing (Reactor Trip System Components)	All Plants	Complete 8/15/88	TVA's commitments are reflected in the NRC's March 23, 1988 Safety Evaluation. These commitments were completed July 29, 1988 (Item 3.1.1) and August 15, 1988 (Item 3.1.2).



ENCLOSURE 2 (Continued)  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 2

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
75 (B079)	Generic Letter 83-28, Item 3.1.3 - Post-Maintenance Testing - Changes to Test Requirements (Reactor Trip System Components)	All Plants	Complete 10/27/86	Completion based upon NRC Safety Evaluation, dated October 27, 1986, which states TVA's response was acceptable.
75 (B087)	Generic Letter 83-28, Items 3.2.1 and 3.2.2 - Post- Maintenance Testing (All Other Safety Related Components)	All Plants	Complete 3/23/88	Completion based upon NRC Safety Evaluation, dated March 23, 1988, which states TVA's programs are acceptable.
75 (B088)	Generic Letter 83-28, Item 3.2.3 - Post-Maintenance Testing - Changes to Test Requirements (All Other Safety Related Components)	All Plants	Complete 10/27/86	Completion based upon NRC Safety Evaluation, dated October 27, 1986, which states TVA's response was acceptable.
75 (B080)	Generic Letter 83-28, Item 4.1 - Reactor Trip System Reliability (Vendor Related Modifications)	All Plants	Not Applicable	The applicability section for this item in GL 83-28 states: "This action applies to all PWR licensees and OL applicants."
75 (B081)	Generic Letter 83-28, Items 4.2.1 and 4.2.2 - Reactor Trip System Reliability - Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	Not Applicable	BFN is a BWR

ENCLOSURE 2 (Continued)  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 2

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
75 (B082)	Generic Letter 83-28, Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	Not Applicable	BFN is a BWR
75 (B090)	Generic Letter 83-28, Item 4.3 - Reactor Trip System Reliability - Tech. Spec. Changes (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	Not Applicable	BFN is a BWR
75 (B091)	Generic Letter 83-28, Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	Not Applicable	BFN is a BWR
75 (B092)	Generic Letter 83-28, Item 4.5.1 - Reactor Trip System Reliability - Diverse Trip Features (System Functional Testing)	All Plants	Complete 4/14/86	Completion based upon NRC Safety Evaluation, dated April 14, 1986, which states TVA's response was acceptable.
75 (B093)	Generic Letter 83-28, Items 4.5.2 and 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	Under Evaluation 9/90	Item 4.5.2 is considered complete based upon NRC Safety Evaluation, dated September 2, 1986, which states TVA's response was acceptable. BFN is reviewing Item 4.5.3 for conformance to BWROG approved analysis.

ENCLOSURE 2 (Continued)  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 2

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	Completed 1/25/88	Completion based upon issuance of GL 88-01, dated January 25, 1988, which superseded GL 84-11. Actions resulting from GL 88-01 are tracked under USI A-42 (MPA B-05).
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	Not Applicable	BFN is a BWR
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	Not Applicable	BFN is a BWR
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft. Calhoun	Not Applicable	BFN is not listed
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	Complete 5/24/82	Completion based upon issuance of Technical Specification Amendment No. 81, dated May 24, 1982, which stated that the requirements of the Staff's November 20, 1981 Generic Letter are incorporated.



ENCLOSURE ? (Continued)  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 2

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	Complete 5/24/82	Completion based upon issuance of Technical Specification Amendment No. 81, dated May 24, 1982, which stated that the requirements of the Staff's November 20, 1981 Generic Letter are incorporated.
A-16 (D-012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	Not Applicable	BFN is not listed
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	Complete 1981	Modifications are reflected in Technical Specification Amendment No. 72, dated September 3, 1981.
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	Not Applicable	BFN has a Mark I containment.
E-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	Not Applicable	BFN docketed its OL application on September 25, 1970.

ENCLOSURE 2 (Continued)  
STATUS OF GENERIC SAFETY ISSUES  
BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 2

<u>GSI/ (MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS</u>	<u>COMMENTS</u>
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	Complete 8/8/80	Completion based upon NRC letter to TVA, dated August 8, 1980, which states the valve configurations discussed in NRC's February 23, 1980 letter do not exist at BFN.

ENCLOSURE 3  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) -- UNIT 3

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
40 (B065)	Safety Concerns Associated with Pipe Breaks in the BWR Scram System	All BWRs	Complete 1/3/86	Completion based upon issuance of Generic Letter (GL) 86-01, dated January 3, 1986. Implementation of visual verification assumption contained in the GL was never requested by the Staff and is not currently required by the ASME code.
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	Implementing 1/93	Modifications described in TVA letter, dated June 27, 1984.
43 (B107)	Reliability of Air Systems	All Plants	Implementing 1/93	TVA's response to GL 88-14 was submitted by letter dated February 23, 1989.
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	Implementing 1/93	TVA's response to GL 89-13 was submitted by letter dated March 16, 1990.
67.3.3 (A017)	Improved Accident Monitoring	All Plants	Implementing 1/93	Implementation with the exception of qualified neutron monitoring instrumentation. Refer to BWR Owners' Group February 21, 1990 letter (BWROG-9025).
75 (B076)	Generic Letter 83-28, Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	Complete 6/17/85	Completion based upon NRC Safety Evaluation, dated June 17, 1985, which stated the program and procedures were acceptable.



ENCLOSURE 3 (Continued)  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 3

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
75 (B085)	Generic Letter 83-28, Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	Implementing 1/93	Completion of item requires installation of upgraded process computer. Reference NRC's June 12, 1985 Safety Evaluation.
75 (B077)	Generic Letter 83-28, Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	Complete 5/1/89	Completion based upon NRC Safety Evaluations, dated September 2, 1986 (Item 2.1, Part 1) and May 1, 1989 (Item 2.1, Part 2). TVA's responses were found to be acceptable.
75 (B086)	Generic Letter 83-28, Item 2.2.1 - Equipment Classification for Safety Related Components	All Plants	Complete 6/1/89	Completion based upon NRC Safety Evaluation, dated June 1, 1989, which states TVA's response was acceptable.
75 (L003)	Generic Letter 83-28, Item 2.2.2 - Vendor Interface for Safety Related Components	All Plants	Under Evaluation	The NRC issued a revised position on this item in GL 90-03, dated March 20, 1990. TVA's response is required by September 25, 1990.
75 (B078)	Generic Letter 83-28, Items 3.1.1 and 3.1.2 - Post- Maintenance Testing (Reactor Trip System Components)	All Plants	Complete 8/15/88	TVA's commitments are reflected in the NRC's March 23, 1988 Safety Evaluation. These commitments were completed July 29, 1988 (Item 3.1.1) and August 15, 1988 (Item 3.1.2).

ENCLOSURE 3 (Continued)  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 3

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
75 (B079)	Generic Letter 83-28, Item 3.1.3 - Post-Maintenance Testing - Changes to Test Requirements (Reactor Trip System Components)	All Plants	Complete 10/27/86	Completion based upon NRC Safety Evaluation, dated October 27, 1986, which states TVA's response was acceptable.
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75 (B088)	Generic Letter 83-28, Item 3.2.3 - Post-Maintenance Testing - Changes to Test Requirements (All Other Safety Related Components)	All Plants	Complete 10/27/86	Completion based upon NRC Safety Evaluation, dated October 27, 1986, which states TVA's response was acceptable.
75 (B080)	Generic Letter 83-28, Item 4.1 - Reactor Trip System Reliability (Vendor Related Modifications)	All Plants	Not Applicable	The applicability section for this item in GL 83-28 states: "This action applies to all PWR licensees and OL applicants."
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ENCLOSURE 3 (Continued)  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 3

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	REMARKS
75 (B082)	Generic Letter 83-28, Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	Not Applicable	BFN is a BWR
75 (B090)	Generic Letter 83-28, Item 4.3 - Reactor Trip System Reliability - Tech. Spec. Changes (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	Not Applicable	BFN is a BWR
75 (B091)	Generic Letter 83-28, Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	Not Applicable	BFN is a BWR
75 (B092)	Generic Letter 83-28, Item 4.5.1 - Reactor Trip System Reliability - Diverse Trip Features (System Functional Testing)	All Plants	Complete 4/14/86	Completion based upon NRC Safety Evaluation, dated April 14, 1986, which states TVA's response was acceptable.
75 (B093)	Generic Letter 83-28, Items 4.5.2 and 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	Under Evaluation 1/93	Item 4.5.2 is considered complete based upon NRC Safety Evaluation, dated September 2, 1986, which states TVA's response was acceptable. Closure of Item 4.5.3 requires review for conformance to BWROG approved analysis.



ENCLOSURE 3 (Continued)  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 3

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	Completed 1/25/88	Completion based upon issuance of GL 88-01, dated January 25, 1988, which superseded GL 84-11. Actions resulting from GL 88-01 are tracked under USI A-42 (MPA B-05).
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	Not Applicable	BFN is a BWR
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	Not Applicable	BFN is a BWR
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft. Calhoun	Not Applicable	BFN is not listed
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	Complete 5/24/82	Completion based upon issuance of Technical Specification Amendment No. 55, dated May 24, 1982, which stated that the requirements of the Staff's November 20, 1981 Generic Letter are incorporated.

ENCLOSURE 3 (Continued)  
 STATUS OF GENERIC SAFETY ISSUES  
 BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 3

GSI/ (MPA No.)	TITLE	APPLICABILITY	STATUS	COMMENTS
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	Complete 5/24/82	Completion based upon issuance of Technical Specification Amendment No. 55, dated May 24, 1982, which stated that the requirements of the Staff's November 20, 1981 Generic Letter are incorporated.
A-16 (D-012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	Not Applicable	BFN is not listed
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	Complete 1982	Modifications are reflected in Technical Specification Amendment No. 52, dated March 29, 1982.
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	Not Applicable	BFN has a Mark I containment.
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	Not Applicable	BFN docketed its OL application on September 25, 1970.

ENCLOSURE 3 (Continued)  
STATUS OF GENERIC SAFETY ISSUES  
BROWNS FERRY NUCLEAR PLANT (BFN) - UNIT 3

<u>GSI/ (MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS</u>	<u>COMMENTS</u>
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	Complete 8/8/80	Completion based upon NRC letter to TVA, dated August 8, 1980, which states the valve configurations discussed in NRC's February 23, 1980 letter do not exist at BFN.