SCHEDULING NOTE

Title: BRIEFING ON ADVANCED REACTORS AND NEW REACTOR TOPICS (Public)

- **Purpose:** To provide the Commission with an update on (1) NRC's efforts to modernize and risk-inform regulatory processes to effectively and efficiently license advanced reactors, (2) the NRC's efforts to complete the regulatory readiness activities under the Nuclear Energy Innovation and Modernization Act (3) Vogtle project activities, and (4) stakeholder perspectives on advanced reactor industry initiatives and NRC developmental activities, including potential policy and program challenges.
- Scheduled: February 6, 2020 9:00 a.m.
- Duration: Approx. 3 hours
- Location: Commissioners' Conference Room, 1st fl. OWFN

Participants:	Presentation
<u> Panel 1 – External Stakeholders</u>	40 mins.*
 Alice Caponiti, Deputy Assistant Secretary, Office of Nuclear Fleet and Advanced Reactor Development, U.S. Department of Energy <u>Topic:</u> DOE perspectives on advanced reactor development, including Development and application of technology-inclusive, risk-informed, and performance-based methodologies 	10 mins.*
 Doug True, Chief Nuclear Officer and Senior Vice President, Nuclear Energy Institute <u>Topic:</u> Nuclear industry initiatives and priorities to support advanced reactor deployment including efforts on micro-reactors Status of NEI advanced reactor technology work groups (high temperature gas, fast reactors and molten salt reactors) 	10 mins.*
 Amir Afzali, Policy and Licensing Director - Next Generation Reactors, Southern Company Services <u>Topic:</u> Licensing Modernization Project efforts to develop technology- inclusive, risk-informed, and performance-based guidance for li advanced reactors Insights and lessons-learned from pilot applications 	10 mins.* censing

Edwin Lyman, Director of Nuclear Power Safety, Union of Concerned 10 mins.* Scientists

Topic:

• Stakeholder perspectives on advanced reactor regulatory and policy issues

Commission Q & A	40 mins.
Break	5 mins.
Panel 2 - NRC Staff	40 mins.*
Margaret Doane, Executive Director for Operations	

l opic:

• Opening Remarks

Robert Taylor, Deputy Director for New Reactors, NRR

<u>Topic:</u>

• Vision and direction for regulatory reviews of advanced reactors, key opportunities and challenges, and coordination with federal agencies and external stakeholders

Mike King, Director, Voglte Project Office, NRR

<u>Topic:</u>

- Vogtle project activities
- **John Segala**, Branch Chief, Advanced Reactor Policy Branch, Division of Advanced Reactors and Non-Power Production and Utilization Facilities, NRR

<u>Topic:</u>

- Licensing Modernization Project
 - Endorsement of guidance for a technology-neutral, risk-informed, and performance-based approach to licensing, and activities to meet NEIMA requirements

Ben Beasley, Branch Chief, Advanced Reactor Licensing Branch, Division of Advanced Reactors and Non-Power Production and

Utilization Facilities, NRR

<u>Topic:</u>

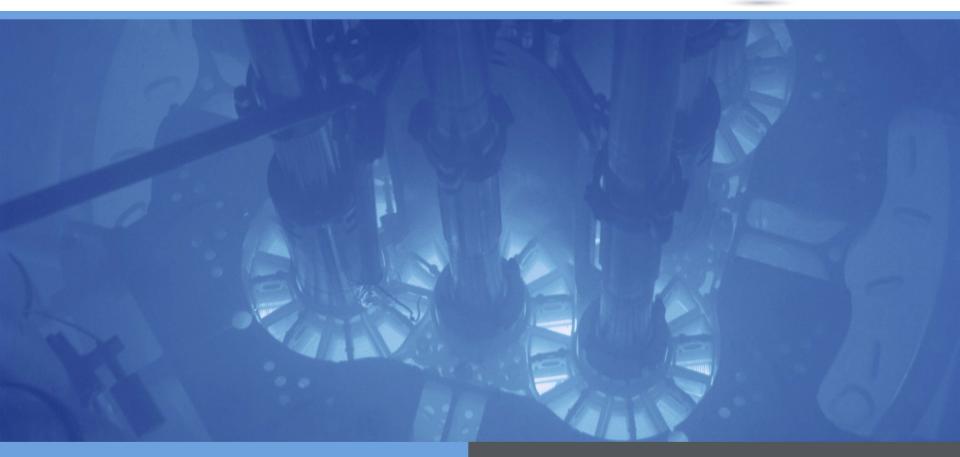
• Overview of readiness to review license applications, status of applications, and development of safety and environmental review regulatory guidance

Commission Q & A	40 mins.
Discussion – Wrap-up	5 mins.

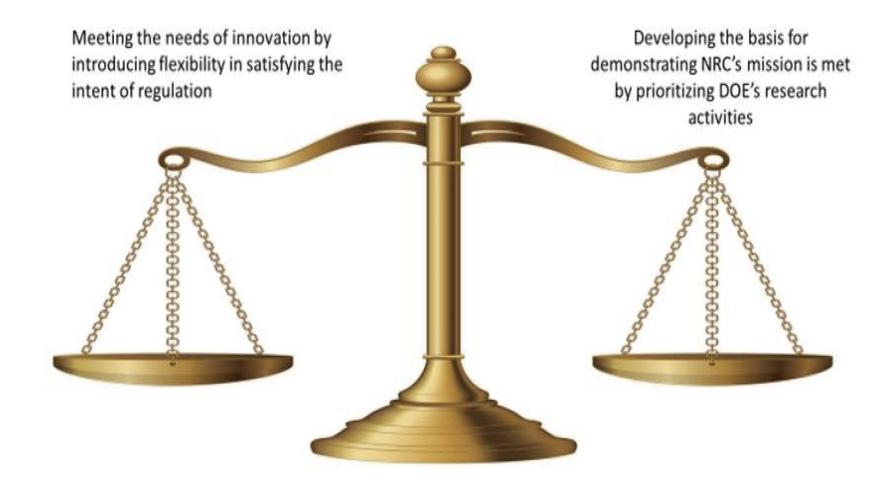
* For presentation only and does not include time for Commission Q & A's







DOE Perspectives on Advanced Reactor Development and Licensing Alice Caponiti Deputy Assistant Secretary for Reactor Fleet & Advanced Reactor Deployment Office of Nuclear Energy U.S. Department of Energy DOE Technology Inclusive Initiatives Can Expedite the Retirement of Regulatory Risk While Modernizing the Regulatory Framework



Why is DOE Interested in Assuring a Risk-Informed Performance Based (RIPB) Approach to Advanced Reactor Licensing ?

- Reactor plant event sequence identification, evaluation, and safety system classification are a key source of regulatory uncertainty impeding advanced reactor deployment
- Difficult to consistently and confidently address uncertainty through a purely "ad-hoc" and expert judgment-based approach due to:
 - Wide variation in reactor technologies and safety case approaches
 - Financial uncertainty created by late-stage and less structured regulatory decisions
- RIPB approach is technology-inclusive and can be applied to the broad range of advanced reactor designs being supported by DOE
- Assists industry stakeholders and DOE in identifying vulnerabilities and uncertainties and focus research efforts in the most impactful areas

DOE Has a History of Promoting Risk-Informed Performance Based (RIPB) Methodologies

- Directly and consistently supported RIPB approaches to the design and licensing of advanced reactor technologies since the 1980's
 - General Atomics
 - Exelon Pebble Bed Modular Reactor
 - Next Generation Nuclear Plant (NGNP)
 - Licensing Modernization Project (LMP)
 - LMP Pilot Studies on a variety of designs
- Efforts resulted in the approach now being considered by the Commission
 - Per the Advisory Committee on Reactor Safeguards (ACRS), RIPB approach reflects the culmination of DOE, NRC and industry insights and "good practices"
- Strong partnerships and focus on RIPB methodologies continue between the private sector and Government

RIPB Approach Going Forward

- Ongoing DOE projects, are implementing the RIPB approach developed through the LMP in support of the DOE authorization process
- Continue DOE cost-shared initiatives addressing licensing (TICAP) and individual awards to industry
- Promote advanced reactor regulatory efficiency by assisting industry in developing NRC application development
- Continue collaborations with the international community on advanced reactor technologies

NRC Coordination & Engagement

- Joint advanced reactor deployment activities (LMP, TICAP)
- Nuclear Energy Institute Advanced Reactor Working Group and Technology-Specific Technical Working Groups
- Ongoing NRC-DOE partnerships to evaluate emerging technologies

Ongoing and Planned DOE Advanced Reactor Development and Deployment Efforts

- Advanced Reactor Technologies (ART) Program National Laboratory R&D Activities
- Versatile Test Reactor
- Advanced Reactor Demonstration Program
 - National Reactor Innovation Center (NRIC)
 - Advanced Reactor Demonstrations
 - Risk Reduction for Future Demonstrations
 - Regulatory Development
 - Advanced Reactor Safeguards
- ARC-20 Awards
- Industry Funding Opportunity Announcement Awards

Questions?



New & Advanced Reactors

Doug True Chief Nuclear Officer and Senior Vice President

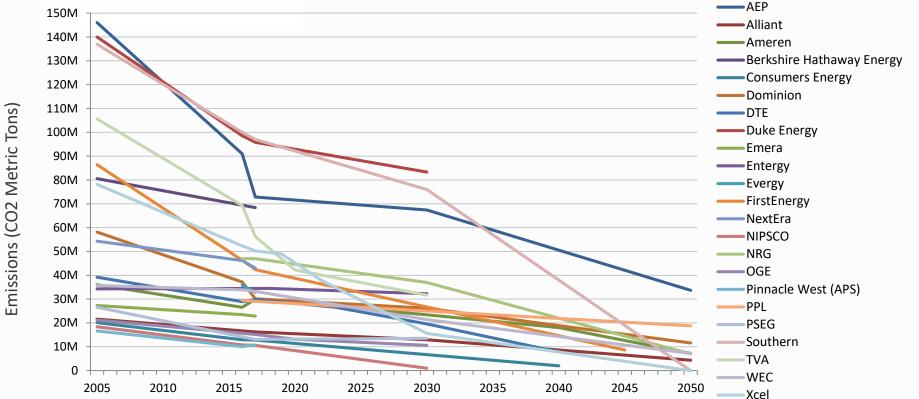
February 6, 2020

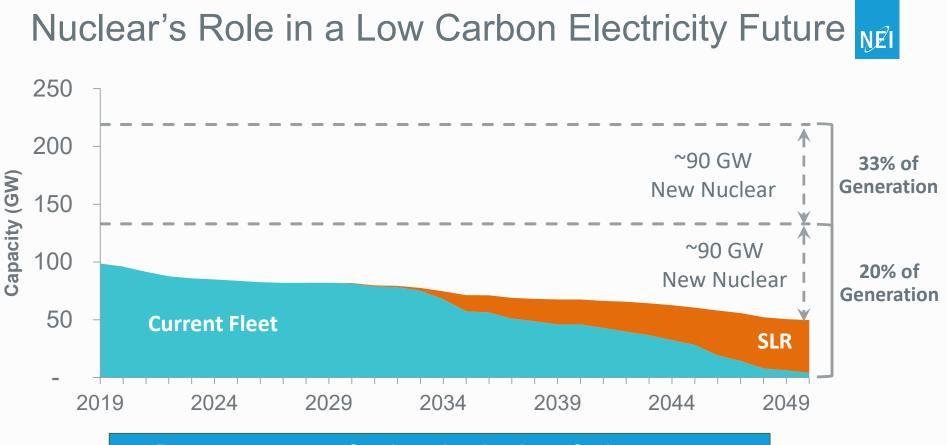




Decarbonization Trajectory of U.S. Utilities







Does not account for decarbonization of other sectors

The Time is Actually Now



- Achievement of Carbon Goals Won't Happen Overnight
- Renewable Share Will Grow, but is Limited
 - Even with storage
- Firm, Dispatchable Power Still Needed
 - Utilities making decisions today about conversions from coal
 - Cost competitive nuclear option
- Basic Timeline:



Regulatory Priorities

- 1. Construction and licensing decisions
 - Vogtle 3 and 4
 - Design and site applications
- 2. Advanced reactor policy decisions
 - Risk-informed regulatory framework
 - Technology inclusive
- 3. Streamlining processes
 - Significantly shorter reviews
 - Efficient Environmental review





Licensing Modernization Project (LMP)

Amir Afzali Licensing and Policy Director Southern Company

Licensing Modernization Project

Why: Reduce regulatory uncertainty to enable accelerated commercialization of advanced non-LWR reactors

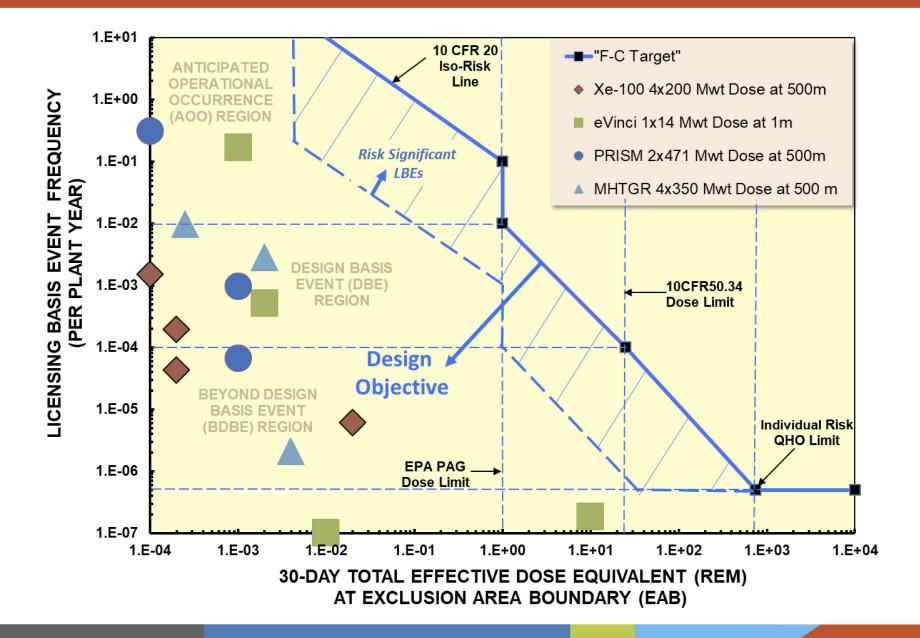
- Consistent with the Commission's long-standing effort to transition to risk-informed, performance-based regulations
- Key to achieving modern risk-informed regulation as envisioned in the agency's Transformation Initiative.

How: Develop transparent, systematic, risk-informed, performancebased, and predictable methodology

What: NEI 18-04 and four supporting reports are intended to:

- Select and evaluate Licensing Basis Events (LBEs)
- Classify Structures, Systems and Components (SSCs) based on their holistic and realistic contribution to risk
- Determine Defense-in-Depth (DiD) adequacy

LBE Evaluation Chart



LMP Tabletop Insights

Results of the Tabletop exercises confirmed that:

- The LMP process can be effectively executed for a spectrum of different non-LWR concepts
- Design decisions can be optimized through an integrated and realistic analysis of the plant's response
- Information obtained through the LMP-based design evaluation can be used for building a strong operational risk management program

Summary

- The LMP methodology, presented in NEI 18-04, is developed based on:
 - over 20 years of industry interactions with the NRC staff on risk-informed regulatory approaches, including many public reviews and discussions
 - lessons learned from a number of industry tabletop exercises, covering different technologies and designs
- Positive support of Draft Guide-1353 which endorses NEI 18-04:
 - ACRS letter of support, following a number of meetings with the industry and NRC staff.
- Next steps:
 - **Critical** Approval of SECY-19-0117
 - Valuable- NRC endorsement of ASME non-LWR PRA standard.

SECY-19-0117 key step in support of Commission's longstanding effort to transition to risk-informed, performancebased regulations and regulatory modernization

Acronyms

- LMP– Licensing Modernization Project
- NEI- Nuclear Energy Institute
- LBE- Licensing Basis Events
- DBE- Design Basis Events
- DBA- Design Basis Accidents
- BDBE- Beyond Design Basis Events
- DiD– Defense-in-Depth
- QHO- Quantitative Health Objective
- EPA– Environmental Protection Agency
- PAG– Protection Active Guide
- EAB- Exclusion Area Boundary
- Mwt- Megawatt Thermal
- F-C- Frequency-Consequence

UCS Perspectives on Advanced Reactor Regulatory and Policy Issues

February 6, 2020 Dr. Edwin Lyman Director of Nuclear Power Safety Climate and Energy Program Union of Concerned Scientists

What is an "advanced reactor"?

- NRC has not adopted a consistent definition for the term "advanced reactor"—this is problematic
- "This regulatory basis recognizes that the phrase 'advanced reactors' has different meanings in different documents ..." -- Draft Regulatory Basis, Rulemaking for Physical Security for Advanced Reactors, July 2019
 - Physical Security Regulatory Basis: "Light-water small modular reactors and non-light-water reactors"
 - could include large CANDUs
 - 2008 Advanced Reactor Policy Statement: any design other than an LWR licensed before 1997
 - could include large LWRs (AP1000)
- In any case, these definitions do not include the NRC's enhanced safety and security expectations in the Advanced Reactor Policy Statement

NEIMA's definition

• More refined (but also problematic) definition in the Nuclear Energy Innovation and Modernization Act:

"...a fission ...or fusion reactor ... with significant improvements compared to commercial nuclear reactors under construction as of ..." January 14, 2019 (date of enactment), including improvements such as—

(A) additional inherent safety features;

- (B) significantly lower levelized cost of electricity;
- (C) lower waste yields;
- (D) greater fuel utilization;
- (E) enhanced reliability;
- (F) increased proliferation resistance;
- (G) increased thermal efficiency; or
- (H) ability to integrate into electric and nonelectric applications.

NRC's NEIMA obligations

- [NRC] shall develop and implement, where appropriate, strategies for the increased use of risk-informed, performance-based licensing evaluation techniques and guidance for commercial advanced nuclear reactors within the existing regulatory framework"
 - Gives the NRC full discretion to determine where such strategies are "appropriate" but limits application to reactors "with significant improvements" compared to the AP1000
- However, NEIMA does not make clear if this definition applies to reactors that have both significant improvements and significant disadvantages compared to current commercial reactors
 - Non-LWRs will generally have some improvements and some disadvantages compared to LWRs
- Nor does it specify how the NRC should make such determinations

Common-cause failure

- The proposed framework for risk-informing advanced reactors lacks defense-in-depth because the accuracy of the probabilistic risk assessment (PRA) is a common-cause failure mode
- PRAs for non-LWR designs are largely academic exercises and lack sufficient data for validation
- The PRA (with mechanistic source term) may be used to justify
 - Siting in densely populated urban areas
 - Elimination of off-site radiological emergency planning
 - Reduction in number of armed responders
 - Reduction in number of operators
 - No containment structure
 - No safety-related electrical power
 - Reduction in NRC oversight
- What is the cumulative impact of these regulatory rollbacks?

Acceptance review

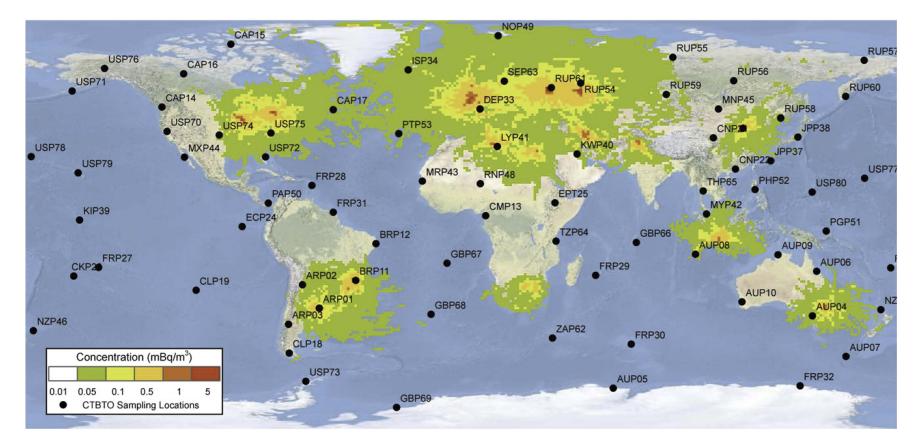
- Staff have proposed that the required content of non-LWR applications and the level of detail of NRC's review themselves be "risk-informed"—that is, also based on the PRA
- This could lead to circular reasoning: systems, structures and components (SSCs) that the applicant asserts are less risksignificant would receive less review, making it more difficult for staff to determine if the SSCs were properly classified in the first place
- NRC should develop acceptance criteria that new reactor applicants would have to meet to enable independent confirmation that their designs are likely to be significantly safer and more secure *before* allowing them to use risk-informed licensing processes

Changing urban siting policy

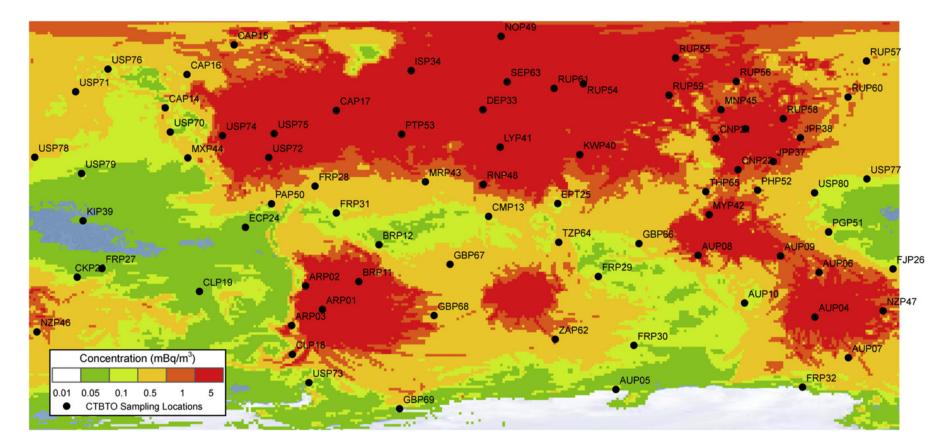
- UCS does not support the staff's proposed changes to longstanding NRC policy to allow siting of "advanced" reactors in densely populated areas—and certainly not through a mere change to a regulatory guide
- Changes to siting policy should not be based only on individual risk limits but also technically sound societal risk metrics
 - Land contamination/relocation standard
 - Population dose limit
- "... a power plant could be located in Central Park and still meet the Commission's quantitative offsite release standard." – Separate Views of Commissioner Bernthal on Safety Goals Policy, 1986
- Any such changes would be of great consequence and should occur only through rulemaking

Example of a less safe "advanced" reactor: the MSR

- One of the advantages of the molten salt-fueled reactor (MSR) is the flexibility provided by a circulating liquid fuel
- Noble gas fission products are stripped from the fuel by sparging with helium gas
- MSR vendors assert that they will be able to trap and retain noble gas fission products
 - few details provided on the specifications, practicality, efficiency, reliability, and cost of off-gas processing systems
- Xenon (Xe) releases from MSRs could pose problems not only for public health and safety, but for Comprehensive Test Ban Treaty verification
- 40 to 90 percent of cesium-137 generated would be released from the core into the off-gas system under NORMAL conditions



Global maximum calculated concentration of ¹³³Xe expected emission from current isotope producers, assuming releases of 5x10⁹ Bq/day (T.W. Bowyer et al., *Journal of Environmental Radioactivity* 115 (2013) 192-200)



Global maximum calculated concentration of 133 Xe expected emission from current and future isotope producers, assuming releases of 1×10^{12} Bq/day (T.W. Bowyer et al., *Journal of Environmental Radioactivity* 115 (2013) 192-200)

Controlling the xenon background

- Unacceptable IMS interference occurs at Xe emission levels below those needed to meet safety limits
- A seminal study determined that a maximum average Xe-133 emission rate of 5x10⁹ Becquerels/day (0.14 curies/day) per facility would be adequate to control the problem
- 400 MW_{th} Terrestrial Energy molten salt reactor would generate 1x10¹⁷ becquerels/day of ¹³³Xe
 - Source term is seven orders of magnitude greater than the 5x10⁹ Bq/day level
- The NRC should require MSRs to comply with this limit (or a technically justified alternative)
 - Jeopardizing CTBT verification would be "inimical to the common defense and security"

Acronyms

- MSR: Molten-Salt [Fueled] Reactor
- NEIMA: Nuclear Energy Innovation and Modernization Act
- PRA: Probabilistic Risk Assessment
- SSC: Structures, Systems, and Components
- UCS: Union of Concerned Scientists



Protecting People and the Environment

Advanced Reactors and New Reactors Topics Briefing

Commission Meeting

February 6, 2020





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Opening Remarks

Margaret Doane

Executive Director for Operations



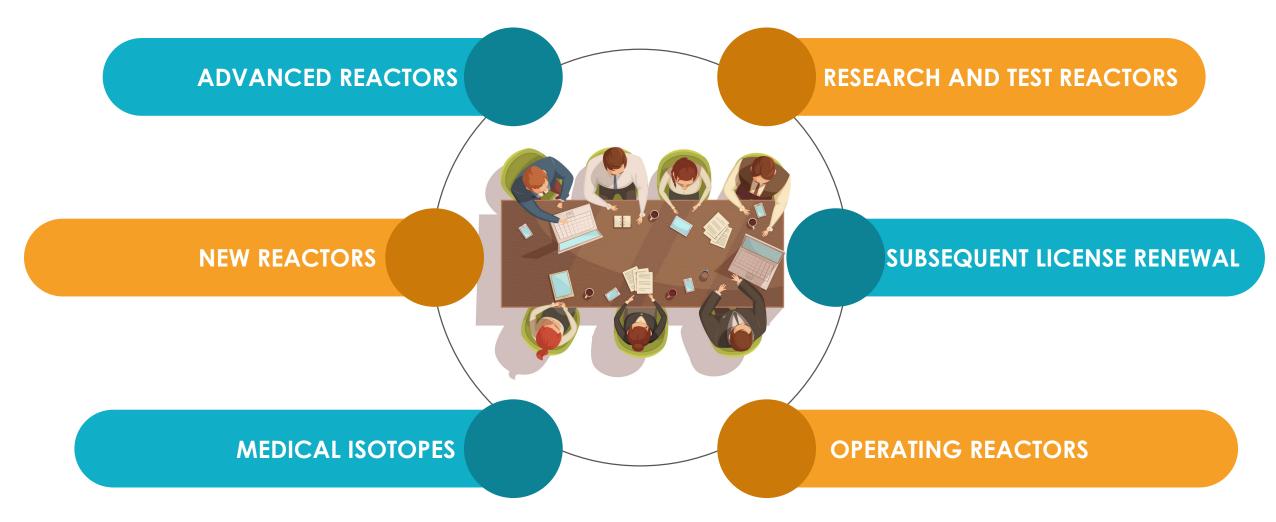
Vision and Direction for Regulatory Reviews of New and Advanced Reactors

Robert Taylor

Deputy Office Director for New Reactors

UNIFIED NRR

We Make SAFE Use of Nuclear Technology POSSIBLE



SAFE, TIMELY, AND EFFECTIVE LICENSING & OVERSIGHT OF VOGTLE 3&4



Photos Courtesy of Southern Nuclear Operating Company

Overseeing Vogtle Construction

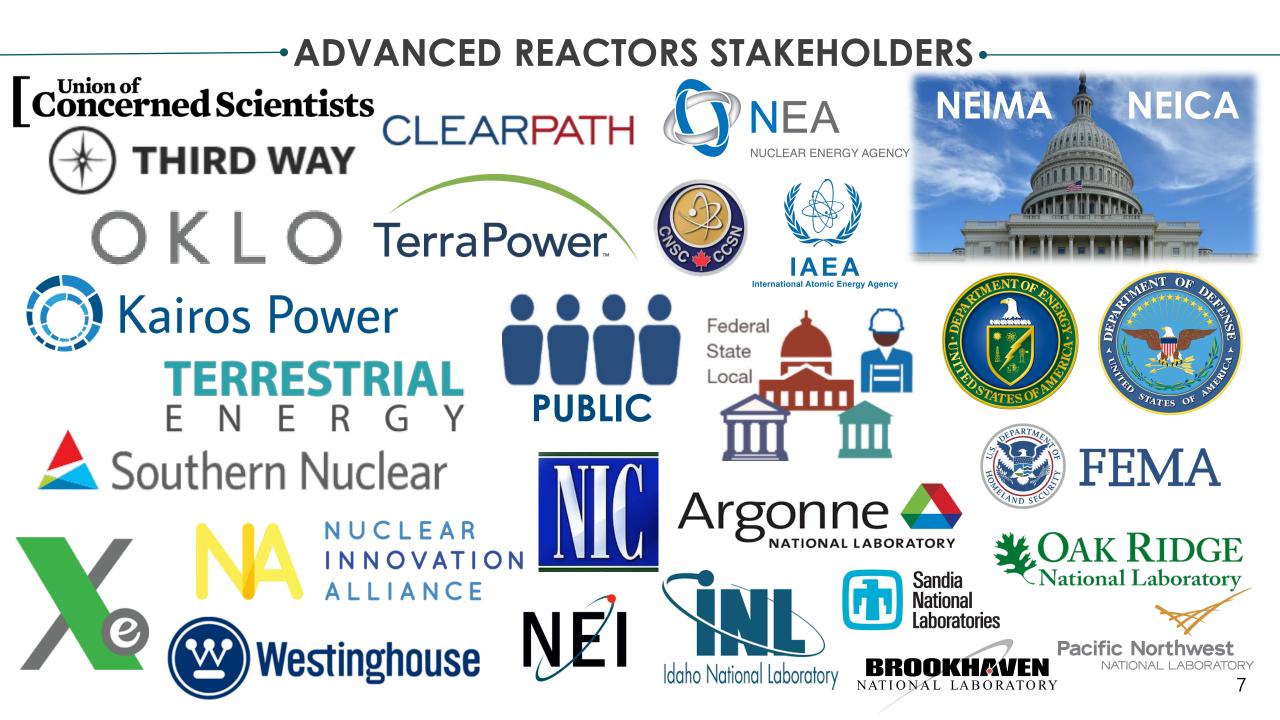
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Closing ITAAC

Resolving License Amendment Requests

SUSTAINING SMALL MODULAR REACTOR (SMR) PROGRESS





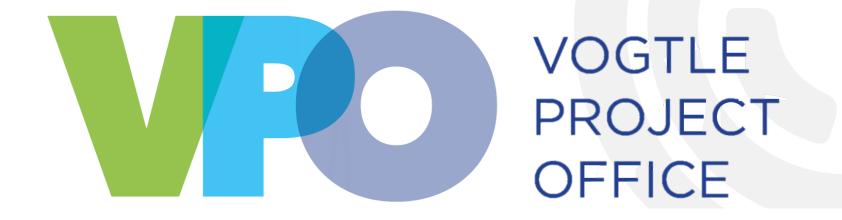
ENABLING the **SAFE Use of New NUCLEAR** Technologies PREPARATION

EXECUTION

SUCCESS

COMMITMEN

8



Michael King, Director Vogtle Project Office Office of Nuclear Reactor Regulation

Photo Courtesy of Southern Nuclear Operating Company



Leveraging technology and innovation to ensure safe oversight of Vogtle project



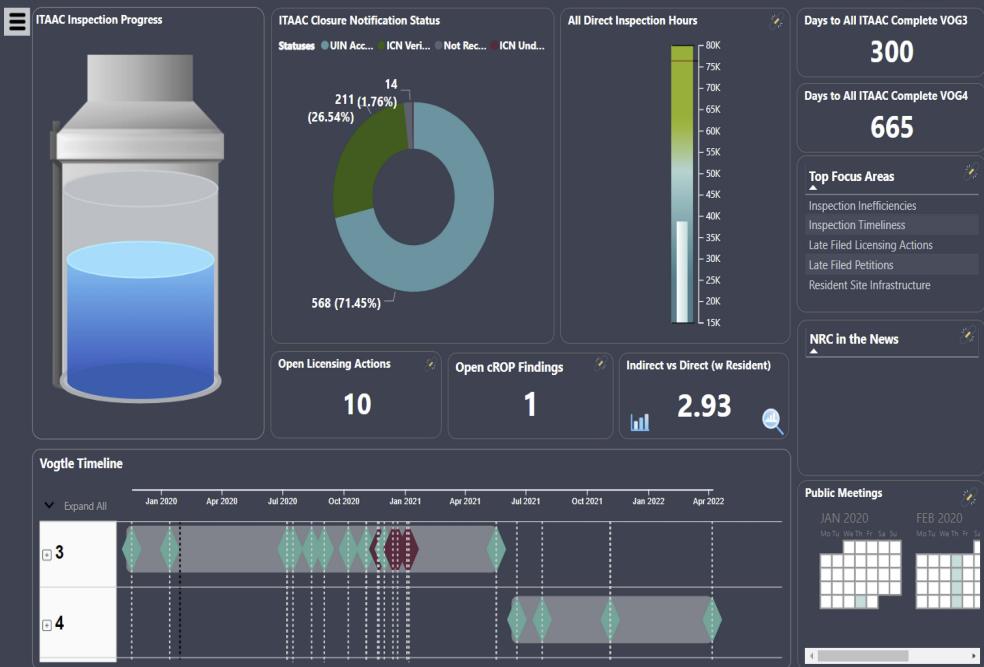
Photos Courtesy of Southern Nuclear Operating Company

ITAAC CLOSURE

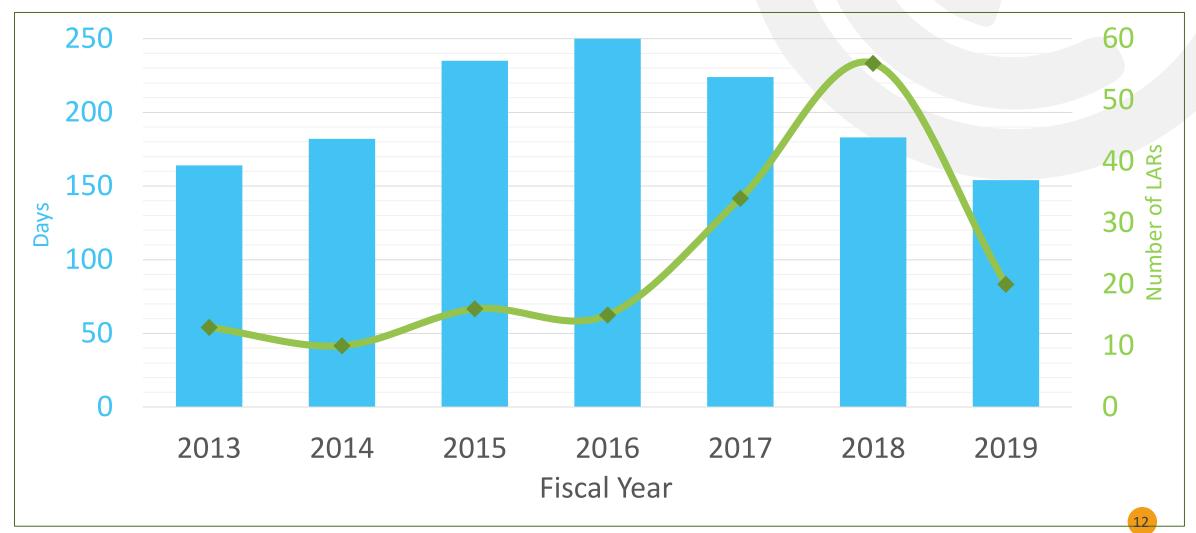
Vogtle Units 3 and 4 Dashboard

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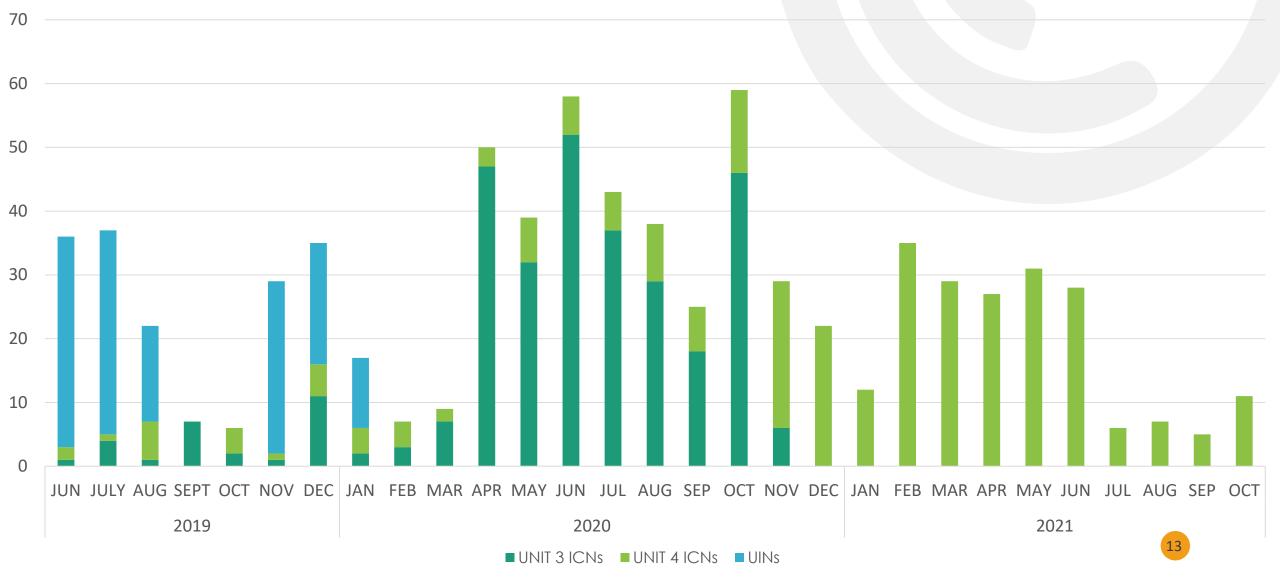
.ast Refresh



Modern Risk-Informed Licensing for Faster Decision-Making



All Vogtle UINs Are Accepted Enabling Efficient ICN Reviews



Continuously Adapting Inspection to Incorporate Lessons Learned from First Part 52 Construction



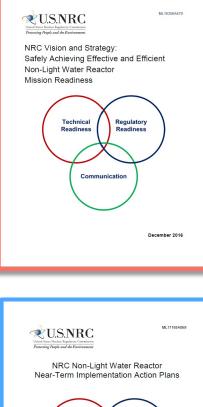


Initiatives to Modernize Advanced Reactor Licensing

John Segala

Chief, Advanced Reactor Policy Branch

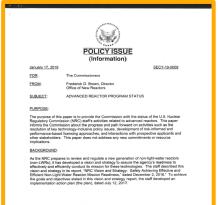
Executing the Vision











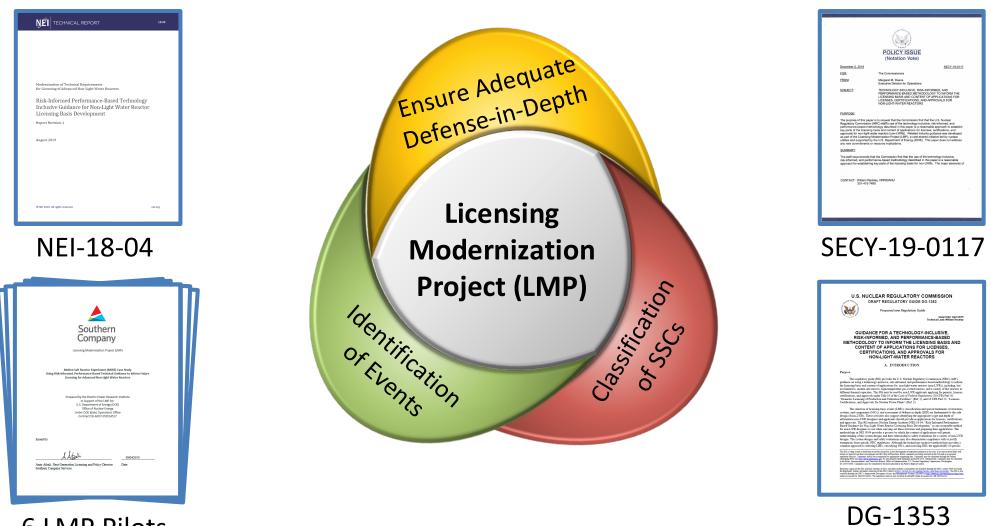
CONTACT: Lucieann Vechioli, NRO/DSRA 301-415-6035

See "NRC Vision and Stategy: Safely Achieving Effective and Efficient Non-Light Water Reactor Mission Readiness," dated December 21, 2016 (Agencywidd Documents Access and Management System (ADMRS) Accession No. ML163564AC70).

Facilitating Innovation with an Integrated Safety Approach

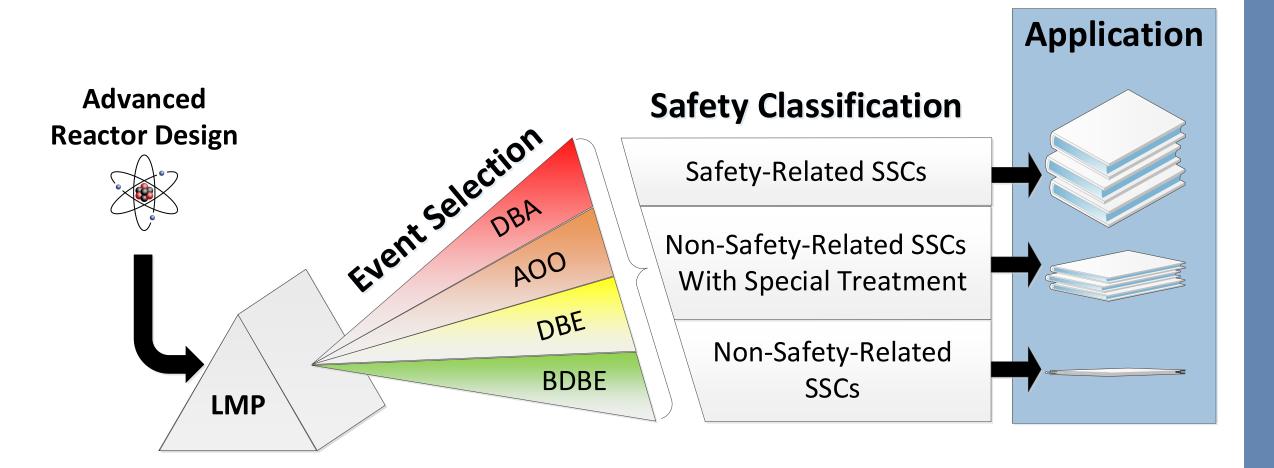


Developing a Modern Risk-Informed Framework



6 LMP Pilots

Risk-Informing the Content of Applications



Cooperating Internationally



Sharing Regulatory Experience

Advancing Risk-Informed and Performance-Based Licensing Approaches

Implementing NEIMA Provisions





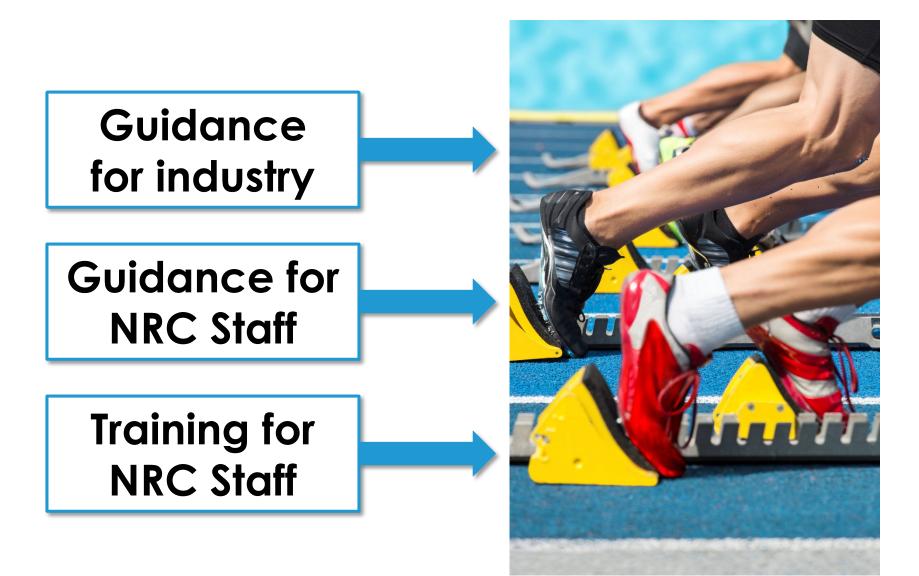
- Issued Reports to Congress
 - Staged Licensing Process
 - Risk-Informed Guidance
- Developing Guidance within Existing Framework
 - Source Term
 - Event Selection
 - Containment Performance
 - Emergency Preparedness
- Developing a Technology-Inclusive, Risk-Informed and Performance-Based Regulatory Framework
 - 10 CFR Part 53



Readiness to Review Non-LWR Applications

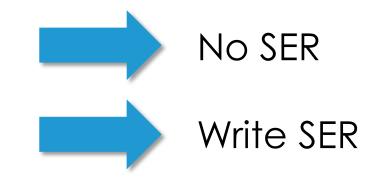
Ben Beasley, Chief Advanced Reactor Licensing Branch

Readiness: Staff Preparations



Readiness: New Approaches

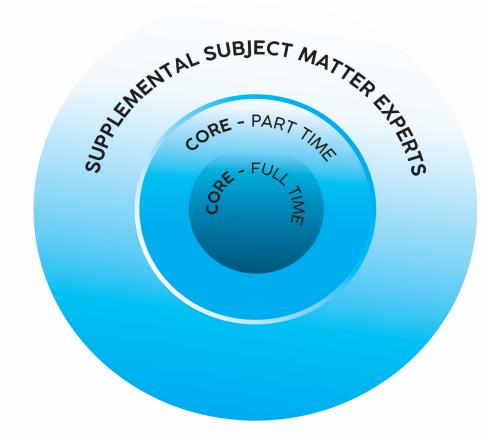
- Pre-application interaction:
 - White paper, audit
 - Topical report, Preliminary Safety Information Document



- Integrated approach to reasonable assurance of adequate protection
- Environmental reviews

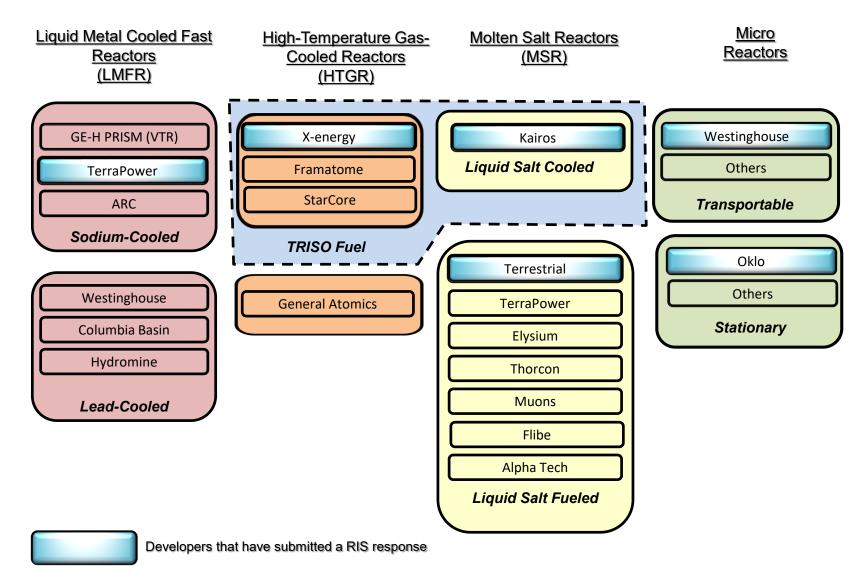
Readiness: Project Team

OKLO INTEGRATED REVIEW — PROJECT TEAM STRUCTURE





Potential Applicants



Current and Anticipated Work

		2020											2021											
Applications	Jan	Feb	Mar	April	May	June	July	Aug	Sept	Oct	Nov	Dec	Jan	Feb	Mar	April	May	June	July	Aug	Sept	Oct	Nov	Dec
Oklo Aurora 1 COL		_	_			_								—							_			
Aurora 2 COL										-							_	_	_	—	—	-		
Aurora 3 COL												_	_								_	_		
Kairos PSID															-		-							
Topical Reports																								
Kairos (12 reports)	-		-						->	_			—	_										
Oklo QA Program	_	_	+																					
X-energy QA Program		-																						
General Atomics Fuel	-						-																	
TerraPower QA Program												-		•										
White Papers																								
Terrestrial (10 papers)		-						-	-				- •	-		_	- >	-	-			_		
TerraPower (2 papers)										_		• •	-	-	- •									

Oklo Review Status



Photo Courtesy of Oklo, Inc

Oklo Aurora Powerhouse

- AOO anticipated operational occurrence
- **BDBE** beyond design basis event
- **CFR** Code of Federal Regulations
- **COL** combined license
- DBA design basis accident
- DBE design basis event
- **DG** draft guide
- **GE-H** GE Hitachi Nuclear Energy
- HTGR high-temperature gas-cooled reactor

- ICN ITAAC closure notification
- **ITAAC** inspections, tests, analyses, and acceptance criteria
- LAR license amendment request
- MSR molten salt reactor
- NEICA Nuclear Energy Innovation Capabilities Act
- NEIMA Nuclear Energy Innovation and Modernization Act
- NRR Office of Nuclear Reactor Regulation
- LMP licensing modernization project

- LMFR liquid metal cooled fast reactor
- LWR light-water reactor
- NEI Nuclear Energy Institute
- NRC U.S. Nuclear Regulatory Commission
- **QA** quality assurance
- **PISD** preliminary safety information document
- **RIS** regulatory issue summary
- SER safety evaluation report
- SMR small modular reactor

- SSC systems, structures and components
- **TRISO** tristructural-isotropic
- **UAMPS** Utah Associated Municipal Power Systems
- **UIN** uncompleted ITAAC notification
- VPO Vogtle Project Office
- VRG Vogtle Readiness Group
- VTR versatile test reactor
- WGSAR Working Group on the Safety of Advanced Reactors