

LICENSEE EVENT REPORT

CONTROL BLOCK: \_\_\_\_\_ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	I	L	D	R	S	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4	5		
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33		
LICENSEE CODE														LICENSE NUMBER						LICENSE TYPE					CAT 58			

0	1	L	0	5	0	0	0	2	3	7	0	7	1	9	8	3	0	7	2	8	8	3	9	
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	
REPORT SOURCE											DOCKET NUMBER						EVENT DATE				REPORT DATE			

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During normal operation a 1/8 inch by 1/16 inch hole was discovered on a bellows seal  
 0 3 | on the torus to drywell vacuum breaker line that contains vacuum breakers 2-1601-32A  
 0 4 | and B. Calculations indicated that primary containment leakage did not exceed the  
 0 5 | Tech Spec limit (3.7.A.2) of 13.7 SCFM. Safety significance was considered minimal  
 0 6 | since combined leakage total was less than the maximum allowable Tech Spec limit  
 0 7 | (13.7 SCFM). There was no effect on public health or safety. Last occurrence of  
 0 8 | arc strike reported by R.O. 83-13 on Docket 50-237.

0	9	S	A	A	F	P	E	N	E	T	R	X	Z				
7	8	9	10	11	12	13	14	15	16	17	18	19	20				
SYSTEM CODE			CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE					COMP. SUBCODE		VALVE SUBCODE			
17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32		
LER RC REPORT NUMBER		EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.							
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER	
R		X		A		A		0038		Y		N		N		P090	
30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 7 | The arc strike was considered accidental. The cavity was filled to the original  
 1 8 | section thickness and the surface was buffed to remove stress risers. The repair  
 1 9 | welds were examined by visual and liquid penetrant methods and the line was air  
 1 10 | tested to verify no leakage. Walkdown of construction areas will be implemented  
 1 11 | to review the need for precautions to prevent recurrence.

1	5	E	0	9	2	N/A	A	Visual Observation
7	8	9	10	11	12	13	14	15
FACILITY STATUS		% POWER		OTHER STATUS			METHOD OF DISCOVERY	
ACTIVITY CONTENT		RELEASED OF RELEASE		AMOUNT OF ACTIVITY			LOCATION OF RELEASE	
PERSONNEL EXPOSURES		PERSONNEL INJURIES		LOSS OF OR DAMAGE TO FACILITY			PUBLICITY ISSUED	
000		000		Z			Z	
000		000		Z			Z	
Z		N/A		N/A			N/A	
N		N/A		N/A			N/A	
N		N/A		N/A			N/A	

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PDR ADOCK 05000237  
S PDR

NRC USE ONLY

NAME OF PREPARER David Tang Wee

PHONE: (815) 942-2920 x483

ATTACHMENT TO LICENSEE EVENT REPORT #83-55/01T-0  
COMMONWEALTH EDISON COMPANY (CWE)  
DRESDEN UNIT 2 (ILDRS-2)  
DOCKET # 050-237

During normal operation, a 1/8 inch by 1/16 inch hole was discovered on the bellows in the torus to drywell vacuum breaker line for vacuum breakers 2-1601-32A and B. The leakage through the hole was calculated to be 6.9 SCFM based on 48 PSIG and 98°F. The combined leakage total based on the as left leakage after the refueling outage from the primary containment was 11.84 SCFM. These calculations indicate that the leakage from the primary containment to the secondary containment was less than 1.6 weight percent per day at 48 PSIG (Tech Spec 3.7.A.2); the maximum allowable limit is 13.7 SCFM. Based on the breach of primary containment an unusual event was declared at 1525 hours on 7/19/83 and Unit 2 was shutdown. Reactor temperature was less than 212°F and the unusual event was terminated at 0537 hours on 7/20/83. Safety significance was considered minimal since the combined leakage total was less than the maximum allowable limit, and all ECCS systems were available. The last occurrence involving an arc strike to the containment was reported by R.O. 83-13 on Docket 50-237.

According to Station Construction personnel construction work for the Mark I modification in the vicinity of the leak was in progress from 6/13/83 until 6/24/83. On 6/17/83, the unit was shutdown for nitrogen leaks and a snubber inspection and returned to service on 6/19/83. One day later a U-2 generator problem forced the unit to shutdown. Generator repairs were completed and the unit was critical on 7/13/83. On 7/19/83 the arc strike was discovered. From the facts above, the maximum number of days that the primary containment was required is 12.

The arc strike was considered to be accidental due to the type of work in the immediate area. A repair program was prepared and approved by the station. The cavity was filled to the original section thickness and the surface was buffed to remove stress risers. The repair was examined by visual and liquid penetrant method. The vacuum breaker line was also air tested and the repaired area was tested for leaks.

Station Construction management will conduct a review of each job and conduct job site walkdowns of work in progress to inform contractor management personnel of systems and for components within a given work area which impact on plant safety, such that additional precautions to personnel working in said areas can be issued.

In addition, a directive has been issued to all contractor management personnel stating that no installed structural members can be removed during the course of any job without specific owner permission.



Commonwealth Edison

DEVIATION REPORT

DVR NO. 12 \* 2 \* 83 \* 102  
STA UNIT YEAR NO.

PART 1 TITLE OF DEVIATION OCCURRED  
7/19/83 1525  
DATE TIME

Vac. Breaker 2-1601-32A & B Bellows Leak

SYSTEM AFFECTED 1600 PLANT STATUS AT TIME OF EVENT TESTING  
Primary Containment MODE Run, PWR(MWT) 2368, LOAD(MWE) 759 YES NO

DESCRIPTION OF EVENT An 1/8 inch by 1/16 inch hole was discovered on the torus to drywell vacuum breaker line bellows. Hole was located on line containing vacuum breakers 2-1601-32 A & B. Unusual event declared and orderly unit shutdown commenced in accordance with DGP 2-1.

EQUIPMENT FAILURE D29167 10 CFR50.72 NRC RED PHONE YES NO  
NOTIFICATION MADE YES NO  
RESponsible SUPERVISOR M. Korchynsky DATE 7/19/83

PART 2 OPERATING ENGINEER'S COMMENTS  
The leakage through the hole was calculated to be 6.9 SCFM at the accident pressure of 48 psig. The Tech Spec limit for primary containment is 13.7 SCFM. Reactor temperature was less than 212°F, and the unusual event was terminated at 0537 on July 20, 1983.

EVENT OF PUBLIC INTEREST  
TECH. SPEC. VIOLATION  
NON REPORTABLE OCCURRENCE  
14 DAY REPORTABLE/T.S. 6.6.B.1.c  
30 DAY REPORTABLE/T.S.  
ANNUAL/SPECL REPORT REQ'D  
24-HOUR NRC NOTIFICATION REQ'D  
TELEPH T. Tongue 7/19/83 1525  
REGION III DATE TIME  
TELEGM/TELECOPY J.G. Keppler 7/20/83 1000  
REGION III DATE TIME  
Revised Telecopy 7/20/82 1707  
CECO CORPORATE NOTIFICATION MADE  
IF ABOVE NOTIFICATION IS PER 10CFR21  
5-DAY WRITTEN REPORT REQ'D PER 10CFR21  
Telecopy Revised telecopy 7/20/83 1600  
Dennis P. Galle 7/20/83 1005  
CECO CORPORATE OFFICER DATE TIME

PRELIMINARY REPORT COMPLETED AND REVIEWED John M. Almer 7/20/83  
OPERATING ENGINEER DATE

INVESTIGATED REPORT & RESOLUTION ACCEPTED BY STATION REVIEW  
J. Brunner 8/1/83 John M. Almer 8/1/83

RESOLUTION APPROVED AND AUTHORIZED FOR DISTRIBUTION  
D.M. Tagan 8/1/83  
STATION SUPERINTENDENT DATE



**Commonwealth Edison**  
Dresden Nuclear Power Station  
R.R. #1  
Morris, Illinois 60450  
Telephone 815/942-2920

July 28, 1983

DJS Ltr #83-738

James G. Keppler, Regional Administrator  
Region III  
U.S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, IL 60137

Reportable Occurrence Report #83-55/01T-0, Docket #050-237 is being submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.1.(c), abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

D.J. Scott  
Station Superintendent  
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: Director of Inspection & Enforcement  
Director of Management Information & Program Control  
U.S. NRC, Document Management Branch  
File/NRC

AUG 02 1983

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