

LICENSEE EVENT REPORT

CONTROL BLOCK: \_\_\_\_\_ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	0	H	D	B	S	1	2	0	0	-	0	0	0	0	0	0	0	0	3	4	1	1	1	1	4		5				
7	8	9	LICENSEE CODE						14	15	LICENSE NUMBER										25	26	LICENSE TYPE				30	57	CAT	58		5

0	1	REPORT SOURCE	L	6	0	5	0	0	0	3	4	6	7	0	1	1	5	8	3	8	0	5	0	5	8	3	9
7	8	CON'T		60	61	DOCKET NUMBER						68	69	EVENT DATE				74	75	REPORT DATE				80			

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | (NP-33-83-03) On 1/15/83, 1/18/83, 4/9/83 and 4/10/83, the Reactor Coolant System (RCS)

0 3 | Dose Equivalent I-131 exceeded the Technical Specification 3.4.8 limit of 1.0 uCi/gm.

0 4 | The peak levels measured were: 2.67 uCi/gm on 1/15/83, 1.39 uCi/gm on 1/18/83 and

0 5 | 4/9/83, and 1.95 uCi/gm on 4/10/83. There was no danger to the health and safety of

0 6 | the public or station personnel. At no time did the specific activity exceed the

0 7 | allowable limit of Tech Spec 3.4.8 which accommodates possible iodine spiking pheno-

0 8 | menon that may occur following a change in thermal power.

0	9	SYSTEM CODE	R	C	11	CAUSE CODE	E	12	CAUSE SUBCODE	C	13	COMPONENT CODE	F	U	E	L	E	X	14	COMP. SUBCODE	Z	15	VALVE SUBCODE	Z	16																	
7	8	9	10	11	12	13	14	15	16	17	18	19	20	EVENT YEAR		21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | The cause was a slight leakage of fission products through the fuel cladding. Some

1 1 | leakage is normal following a reactor trip due to RCS temperature and pressure changes.

1 2 | The I-131 level was monitored until it dropped below the Technical Specification

1 3 | limit.

1 4 | \_\_\_\_\_

1	5	FACILITY STATUS	X	28	% POWER	0	0	0	29	OTHER STATUS	NA	30	METHOD OF DISCOVERY	A	31	DISCOVERY DESCRIPTION	Per Table 4.4-4 of Tech Spec 3.4.8	32
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25

1	6	ACTIVITY CONTENT RELEASED	Z	33	AMOUNT OF ACTIVITY	NA	35	LOCATION OF RELEASE	NA	36
7	8	9	10	11	12	13	14	15	16	17

1	7	PERSONNEL EXPOSURES	0	0	0	37	DESCRIPTION	NA	39
7	8	9	10	11	12	13	14	15	16

TOLEDO EDISON COMPANY  
DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE  
SUPPLEMENTAL INFORMATION FOR LER NP-33-83-03

DATE OF EVENT: January 15 and 18, 1983

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: Reactor Coolant System (RCS) Dose Equivalent I-131 Exceeded Technical Specification Limits

Conditions Prior to Occurrence: The unit was in Mode 3, with Power (MWT) = 0 and Load (Gross MWE) = 0

Description of Occurrence: On January 15, 1983 after a reactor trip, the specific activity of the primary coolant exceeded the Technical Specification 3.4.8 limit of 1.0  $\mu\text{Ci/gm}$  Dose Equivalent I-131. The level peaked at 2.67  $\mu\text{Ci/gm}$  at 2004 hours on January 15, 1983.

A similar event occurred after a reactor trip on January 18, 1983 with the dose equivalent level peaking at 1.39  $\mu\text{Ci/gm}$  at 2301 hours.

On April 9, 1983, at 0420 hours, following a power reduction to repair a failed steam generator level transmitter, the Dose Equivalent I-131 exceeded the limit of Technical Specification 3.4.8. The level peaked at 1.39  $\mu\text{Ci/gm}$  at 0620 hours on April 10, 1983.

The following information is supplied per reporting requirements:

Power History

On January 15, 1983, reactor power was approximately 100% until a reactor/turbine trip at 1700 hours.

<u>Date</u>	<u>Time</u>	<u>Reactor Power</u>
1/14/83	0000 hours	99.4%
	1600 hours	99.5%
1/15/83	1600 hours	98.7%
	1700 hours	TRIP

On January 18, 1983, reactor power was approximately 100% until a reactor/turbine trip at 1925 hours.

<u>Date</u>	<u>Time</u>	<u>Reactor Power</u>
1/16/83	1600 hours	84.3%
1/17/83	0000 hours	87.3%
	0800 hours	97.9%
	1600 hours	99.2%

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SUPPLEMENTAL INFORMATION FOR LER NP-33-83-03  
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1/18/83	0000 hours	99.3%
	0800 hours	99.4%
	1600 hours	99.6%
	1925 hours	TRIP

On April 8, 1983, a power reduction from approximately 100% was initiated for a short maintenance outage.

<u>Date</u>	<u>Time</u>	<u>Reactor Power</u>
4/8/83	2000	99.1%
4/9/83	0100	48.5%
	0700	6.4%
	1600	21.7%
	2400	42.0%
4/10/83	0300	79.8%
	0345	TRIP

Fuel Burnup By Core Region: See attached computer printout.

Cleanup Flow History 48 hours prior to the first limiting exceeding sample, the average letdown flow during this time was:

January 13, 1983	- 60 gpm
January 14, 1983	- 61 gpm
January 15, 1983	- 86 gpm
January 16, 1983	- 109 gpm
January 17, 1983	- 101 gpm
January 18, 1983	- 60 gpm
April 7, 1983	- 60 gpm
April 8, 1983	- 69 gpm
April 9, 1983	- 93 gpm
April 10, 1983	- 95 gpm

There was no degassing operation.

Table 1 lists the specific activity analysis, the time duration the specific activity exceeded the Technical Specification, and the maximum level reached for each occurrence.

Designation of Apparent Cause of Occurrence: The cause was a slight leakage of fission products through the fuel cladding. Some leakage is normal following a reactor trip due to RCS temperature and pressure changes.

Analysis of Occurrence: There was no danger to the health and safety of the public or station personnel.

At no time did the specific activity exceed the allowable limit of Technical Specification 3.4.8, which accommodates possible iodine spiking phenomenon that may occur following a change in thermal power as shown in Figure 3.4-1.

1 | Corrective Action: Per the action statement requirements of Technical Specification 3.4.8, sampling of the primary coolant was performed at least once every four hours. Monitoring of the I-131 level continued until it dropped below the Technical Specification limit of 1.0  $\mu\text{Ci}/\text{gm}$ . Since iodine spikes are typical following a power change, no additional corrective action is applicable.

Failure Data: Previous occurrences of the RCS dose equivalent I-131 exceeding the Technical Specification limit have been reported in Licensee Event Reports NP-33-80-114 (80-088), NP-33-81-15 (81-016), NP-33-81-37 (81-031).

LER #83-002

TABLE 1

Date of Occurrence	Specific Activity Analysis			Approximate Time Duration Tech Spec Level Exceeded	Maximum Level $\mu\text{Ci/gm}$
	Date	Time	Activity ( $\mu\text{Ci/gm}$ ) Dose Equivalent I-131		
January 15, 1983	1/15/83	0810	0.17	21 hours	2.67
		2004	2.67		
		2155	2.54		
	1/16/83	0102	2.33		
		0400	2.40		
		0730	1.59		
		0930	1.58		
1330		1.14			
1730	0.73				
January 18, 1983	1/18/83	1354	0.14	8 hours	1.39
		2301	1.39		
	1/19/83	0256	1.11		
		0645	0.81		
April 9, 1983	4/8/83	1207	0.09	11 hours	1.39
	4/9/83	0420	1.06		
		0806	1.37		
		1145	1.39		
		1531	0.96		
April 10, 1983	4/10/83	0130	0.41	15 hours	1.95
		0620	1.95		
		1002	1.82		
		1341	1.31		
		1710	1.30		
		2110	0.89		

TOLEDO EDISON COMPANY  
 DAYTS-BESSE NUCLEAR POWER STATION UNIT ONE  
 SUPPLEMENTAL INFORMATION FOR IER NP-33-83-03  
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FUEL BURNUP BY CORE REGION FOR  
JANUARY 15, 1983 AND JANUARY 18, 1983 EVENTS

The fuel burnup is given for each fuel assembly as designated by the computer assigned assembly number. For the location of the assemblies, see the attached core map. Units are given in  $\frac{\text{Megawatt Days}}{\text{Metric Ton}} \times 10^{-3}$ .

								FUEL ASSEMBLIES*
PHISAD	27542E+01	35514E+01	37802E+01	35488E+01	27457E+01	29172E+01	39053E+01	1 through 7
	11194E+02	22218E+02	17817E+02	22230E+02	11193E+02	38853E+01	28741E+01	8 through 14
	13450E+02	45135E+01	24701E+02	24023E+02	15512E+02	15337E+02	15520E+02	15 through 21
	23979E+02	25082E+02	44478E+01	13404E+02	29487E+01	45411E+01	26106E+02	22 through 28
	14377E+02	15475E+02	16534E+02	51114E+01	16536E+02	15474E+02	14349E+02	29 through 35
	26059E+02	44299E+01	28539E+01	39650E+01	25144E+02	14499E+02	24406E+02	36 through 42
	12306E+02	27081E+02	28112E+02	27098E+02	12296E+02	24368E+02	14431E+02	43 through 49
	24681E+02	38423E+01	27937E+01	11046E+02	27667E+02	15591E+02	12185E+02	50 through 56
	13853E+02	26236E+02	13941E+02	26236E+02	13844E+02	12157E+02	15551E+02	57 through 63
	23477E+02	10931E+02	27054E+01	35837E+01	22635E+02	15195E+02	16319E+02	64 through 70
	27439E+02	26967E+02	13561E+02	22297E+02	13566E+02	26989E+02	27494E+02	71 through 77
	16303E+02	15143E+02	22564E+02	34890E+01	37867E+01	17817E+02	15302E+02	78 through 84
	50180E+01	28009E+02	13888E+02	22272E+02	15965E+02	22294E+02	13917E+02	85 through 91
	28070E+02	50464E+01	15290E+02	17769E+02	37149E+01	35332E+01	22574E+02	92 through 98
	15124E+02	16265E+02	27458E+02	26941E+02	13520E+02	22247E+02	13530E+02	99 through 105
	26987E+02	27506E+02	16310E+02	15137E+02	22563E+02	34842E+01	27244E+01	106 through 112
	19929E+02	23423E+02	15508E+02	12086E+02	13768E+02	26177E+02	13878E+02	113 through 119
	26153E+02	13790E+02	12141E+02	15548E+02	23476E+02	10917E+02	26961E+01	120 through 126
	38357E+01	25225E+02	14377E+02	24304E+02	12232E+02	27029E+02	28035E+02	127 through 133
	26938E+02	12198E+02	24330E+02	14401E+02	25009E+02	38225E+01	28543E+01	134 through 140
	44076E+01	26031E+02	14309E+02	15442E+02	16491E+02	50199E+01	16460E+02	141 through 147
	15423E+02	14297E+02	26028E+02	44067E+01	28479E+01	13399E+02	44260E+01	148 through 154
	24797E+02	23939E+02	15484E+02	15300E+02	15461E+02	23929E+02	25181E+02	155 through 161
	44202E+01	13398E+02	28705E+01	38691E+01	11172E+02	22207E+02	17793E+02	162 through 168
	22196E+02	11163E+02	38608E+01	28678E+01	27357E+01	35349E+01	37673E+01	169 through 175
	35331E+01	27324E+01						176 through 177

\*AS READ FROM LEFT TO RIGHT

FUEL BURNUP BY CORE REGION FOR  
APRIL 9, 1983 AND APRIL 10, 1983 EVENTS

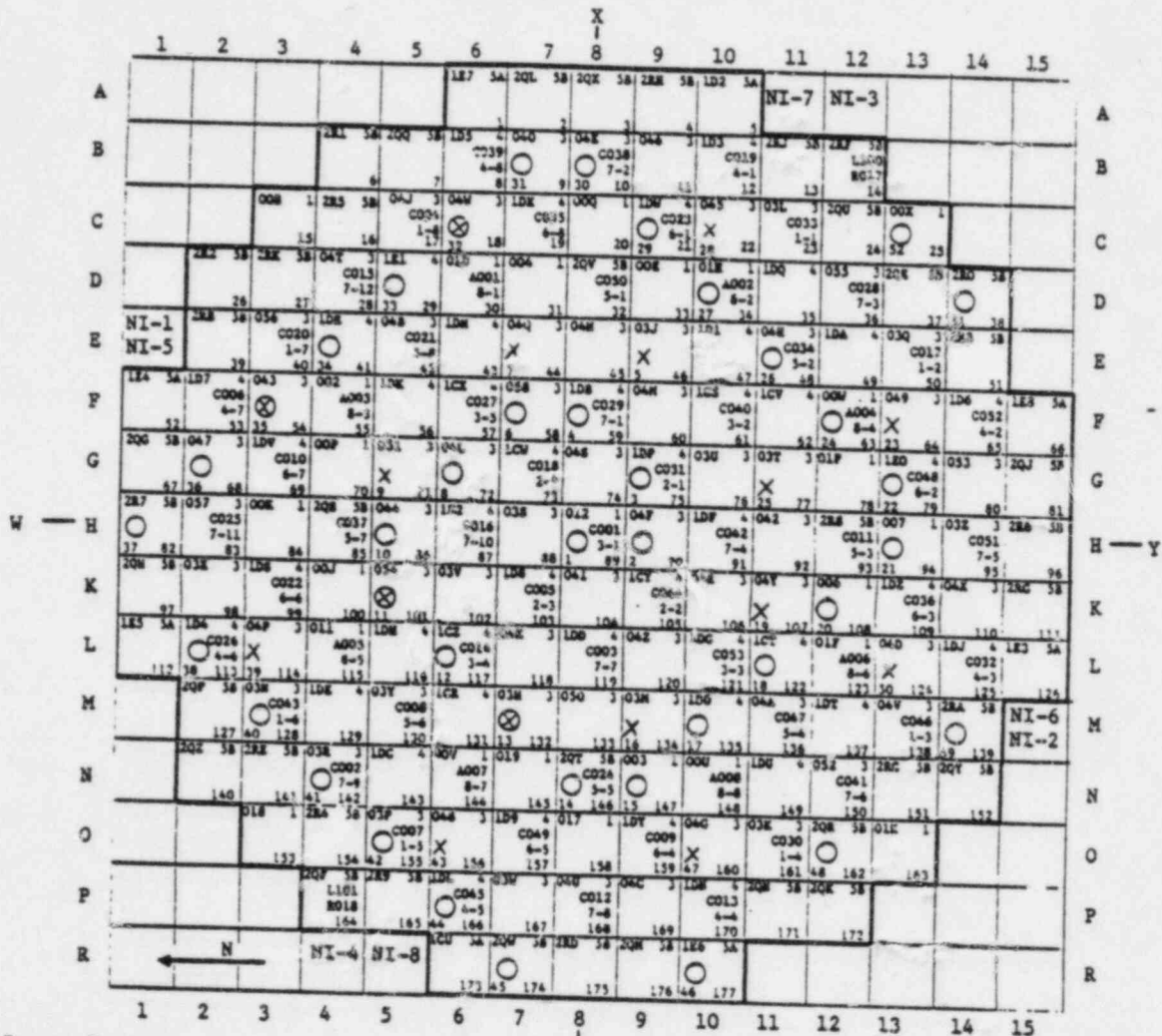
The fuel burnup is given for each fuel assembly as designated by the computer assigned assembly number.  
 For the location of the assemblies, see the attached core map. Units are given in  $\frac{\text{Megawatt Days}}{\text{Metric Ton}} \times 10^{-3}$

							FUEL ASSEMBLIES*
44936E+01	57712E+01	61204E+01	57816E+01	44983E+01	47002E+01	62491E+01	1 through 7
13825E+02	24556E+02	20118E+02	24577E+02	13846E+02	62568E+01	46772E+01	8 through 14
14713E+02	72375E+01	26855E+02	26354E+02	18153E+02	17396E+02	18151E+02	15 through 21
26274E+02	27256E+02	71974E+01	14675E+02	47347E+01	72681E+01	28439E+02	22 through 28
17058E+02	17350E+02	18727E+02	82091E+01	18736E+02	17362E+02	17048E+02	29 through 35
28404E+02	71681E+01	46475E+01	63162E+01	27311E+02	17281E+02	26992E+02	36 through 42
15293E+02	29397E+02	30291E+02	29426E+02	15294E+02	26988E+02	17246E+02	43 through 49
26851E+02	61934E+01	45356E+01	13502E+02	26025E+02	17535E+02	15060E+02	50 through 56
16763E+02	28583E+02	16523E+02	28587E+02	16764E+02	15046E+02	17502E+02	57 through 63
25777E+02	13386E+02	44434E+01	57979E+01	24898E+02	17628E+02	18349E+02	64 through 70
29679E+02	25402E+02	16332E+02	25019E+02	16342E+02	29437E+02	29779E+02	71 through 77
18341E+02	17580E+02	24849E+02	56999E+01	61106E+01	20082E+02	17341E+02	78 through 84
80721E+01	30159E+02	16454E+02	24963E+02	18336E+02	25018E+02	16495E+02	85 through 91
30239E+02	81254E+01	17337E+02	20037E+02	60362E+01	57401E+01	24829E+02	92 through 98
17527E+02	18289E+02	29756E+02	29373E+02	16287E+02	24966E+02	16304E+02	99 through 105
29412E+02	29815E+02	18349E+02	17571E+02	24823E+02	56923E+01	44577E+01	106 through 112
13373E+02	25665E+02	17444E+02	14955E+02	16671E+02	28517E+02	16454E+02	113 through 119
28498E+02	16730E+02	15031E+02	17518E+02	25773E+02	13367E+02	44283E+01	120 through 126
61737E+01	27403E+02	17149E+02	26882E+02	15190E+02	29310E+02	30204E+02	127 through 133
29186E+02	15187E+02	26919E+02	17183E+02	27172E+02	61634E+01	46353E+01	134 through 140
71281E+01	28340E+02	16992E+02	17318E+02	18659E+02	81061E+01	18628E+02	141 through 147
17278E+02	16983E+02	28363E+02	71310E+01	46315E+01	14663E+02	71529E+01	148 through 154
26980E+02	26271E+02	18128E+02	17356E+02	18105E+02	26259E+02	27341E+02	155 through 161
71503E+01	14641E+02	46628E+01	62239E+01	13791E+02	24550E+02	20071E+02	162 through 168
24538E+02	13803E+02	62159E+01	46581E+01	44822E+01	57618E+01	61114E+01	169 through 175
57592E+01	44776E+01						176 through 177

\*AS READ FROM LEFT TO RIGHT

DAVIS-BESSE UNIT 1, CYCLE 3

March 18, 1982



Inc core Detector  
 ○ = Total Core Monitor  
 X = Symmetry Monitor  
 ⊗ = Total Core and Symmetry Monitor

Inc core Detector String Number  
 Regenerative Sources at P4 and B12  
 Computer Assigned Assembly Number

Fuel Assembly ID  
 All ID's preceded by "NJ0"

Fuel Assembly Batch ID, (No. in Core)  
 1 (25) Former Batch A - once burned  
 3 (60) Former Batch C - twice burned  
 4 (44) Former Batch D - once burned  
 5A (8) New Fuel - 3.04% enriched  
 5B (40) New Fuel - 2.99% enriched

Control Component/Source ID and Type  
 A = APSRA C = CRA  
 R - Regenerative Neutron Source

Control Component Group/Rod

Prepared By Daniel B. Kelly Approved By D. J. Doherty