

**BOSTON EDISON**

Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

Ralph G. Bird

Senior Vice President — Nuclear
U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

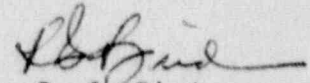
BEC0 89-178
December 14, 1989

License DPR-35
Docket 50-293

RESPONSE TO GENERIC LETTER 89-21:
REQUEST FOR INFORMATION CONCERNING STATUS OF
IMPLEMENTATION OF UNRESOLVED SAFETY ISSUE (USI) REQUIREMENTS

Generic Letter 89-21, dated 10/19/89, requested the implementation status of unresolved safety issue (USI) requirements applicable to Pilgrim Nuclear Power Station.

Attached is the information requested, in the format designated in Enclosure 1 to Generic Letter 89-21.


R. G. Bird

GGW/jcp/3886

Attachment

cc: Mr. D. McDonald, Project Manager
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation
Mail Stop: 14D1
U. S. Nuclear Regulatory Commission
1 White Flint North
11555 Rockville Pike
Rockville, MD 02852

U. S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406

Senior NRC Resident Inspector
Pilgrim Nuclear Power Station

A012
1/1

ATTACHMENT TO BECO LETTER 89-178

Response to Generic Letter 89-21:
Request for Information Concerning Status of
Implementation of Unresolved Safety Issue (USI) Requirements

(14 pages)

Pilgrim Nuclear Power Station - Response to Generic Letter 89-21:
Unresolved Safety issues For Which A Final Resolution Has Been Achieved

<u>USI/ MPA NUMBER</u>	<u>TITLE</u>	<u>REFERENCE DOCUMENT</u>	<u>APPLIC- ABILITY</u>	<u>STATUS *</u> / <u>DATE</u>	<u>REMARKS</u>
USI: A-1	Water Hammer	-NUREG-0927, Rev.1 -NUREG-0993, Rev.1 -NUREG-0737, item I.A.2.3 -NRC Inspection Report 88-11, 5/17/88 -NRC Inspection Report 88-19, 6/29/88	All	NC	Resolved with the publication of NUREG -0927, "Evaluation of Water Hammer in Nuclear Power Plants -Technical Findings Relevant to Unresolved Safety Issue A-1". The resolution of this USI required no changes to Pilgrim Station. In addition to the upgraded training program (NUREG-0737, item I.A.2.3), BEC implemented emergency operating procedures in accordance with BWROG- EPG, Rev. 4 (NEDO 31331, March 1987). The NRC verified the Pilgrim EOPs with Inspection Report 88-11, dated 5/17/88. Resulting open items were closed in Inspection Report 88-19, dated 6/29/88.
USI: A-2 MPA: D-10	Asymmetric Blowdown Loads on Reactor Primary Coolant Systems	-NUREG-O609 -GL 84-04	PWR	NA	PNPS is a BWR and is not affected by this issue.
USI: A-3	Westinghouse Steam Generator Tube Integrity	-NUREG-0844 -GL 85-02	W PWR	NA	PNPS is a BWR and is not affected by this issue.
USI: A-4	CE Steam Generator Tube Integrity	-NUREG-0844 -GL 85-02	CE PWR	NA	PNPS is a BWR and is not affected by this issue.

* C - Complete
i - Incomplete
NA - Not Applicable
NC - No Changes Necessary
E - Evaluating Actions Necessary

Pilgrim Nuclear Power Station - Response to Generic Letter 89-21:
Unresolved Safety Issues For Which A Final Resolution Has Been Achieved

<u>USI/ MPA NUMBER</u>	<u>TITLE</u>	<u>REFERENCE DOCUMENT</u>	<u>APPLIC- ABILITY</u>	<u>STATUS * / DATE</u>	<u>REMARKS</u>
USI: A-5	B&W Steam Generator Tube Integrity	•NUREG-0844 •GL 85-02	B&W PWR	NA	PNPS is a BWR and is not affected by this issue.
USI: A-6 MPA: A-05	Mark I Containment Short Term Program	•NUREG-0408. •BEC0 letter, dated 10/14/75 Final Report for PNPS •BEC0 letter, dated 12/1/76 proposed TS changes •NRC letter, dated 6/21/78 issued Amendment 31. •BEC0 letter, dated 5/31/89 SIMS Update	Mark I BWR	C 6/21/78	By letter dated 10/14/75, BECo endorsed the GE Final Report on the Mark I Short Term Program (NEDC 20989). This report provided the GE analysis, for plants referenced in the report, to confirm structural and functional capability of the containment suppression chamber and attached piping to withstand newly-identified suppression pool hydrodynamic loading conditions not considered in the original design analysis. Subsequent to the Final Report, BECo requested a technical specification amendment to assure that normal plant operating conditions are within the envelope of those considered in the GE analysis. The NRC staff approved the proposed technical specification and issued amendment 31 by letter dated June 21, 1978. The supporting Safety Evaluation concluded that the issuance of the amendment provided the necessary assurance that the plant's operating conditions remain within the envelope assumed in the GE Final Report in accordance with the Mark I Short Term Containment Program for the Pilgrim Nuclear Power Station, Unit I. Verification of closeout of this issue at Pilgrim was provided in the BECo SIMS update letter, dated 5/31/89.

- * C - Complete
 I - Incomplete
 NA - Not Applicable
 NC - No Changes Necessary
 E - Evaluating Actions Necessary

Pilgrim Nuclear Power Station - Response to Generic Letter 89-21:
Unresolved Safety Issues For Which A Final Resolution Has Been Achieved

<u>USI/ MPA NUMBER</u>	<u>TITLE</u>	<u>REFERENCE DOCUMENT</u>	<u>APPLIC- ABILITY</u>	<u>STATUS * / DATE</u>	<u>REMARKS</u>
USI: A-7 MPA: D-01 TAC: 7948	Mark I Long Term Program	•NUREG -0661 •NUREG -0661, Suppl. 1 •Order, dated 1/13/81 •BEC0 letter, dated 1/15/82 proposed TS change •NRC letter, dated 2/5/82 issued Amendment 53 •BEC0 letter, dated 10/26/83 PUA for PNPS •NRC letter, dated 11/7/84 issued Amendment 83. •NRC letter, dated 1/30/85 SE for PUA for PNPS •BEC0 letter, dated 4/12/85 completion of Mark I Long Term Program. •BEC0 letter, dated 10/30/87 SIMS/MPA Update	Mark I BWR	C 4/12/85	Subsequent to the issuance of the Orders, dated 1/13/81, and the submittal of the final Plant Unique Analysis (PUA) by BECo, dated 10/26/83, the NRC staff and technical consultants reviewed the PUA and supplemental information provided by BECo. The NRC safety evaluation, dated 1/30/85, indicated it had completed its assessment of the PNPS, Unit 1 PUA against the generic acceptance criteria contained in NUREG-0661 and its supplements. The NRC staff concluded that the dynamic loads utilized by BECo were conservative and found acceptable. The letter transmitting the safety evaluation requested BECo to submit proposed technical specification changes because of the Mark I containment program modifications at PNPS. BECo letter, dated 4/12/85, indicated all modifications were complete and all technical specifications had been requested and approved by amendment 53, dated 2/5/82, and by amendment 83, dated 11/7/84, thus implementing all requirements of the Mark I Long Term Program.
USI: A-8	Mark II Containment Pool Dynamic Loads	•NUREG-0808 •NUREG-0487, Supp. 1 / 2 •NUREG-0802	Mark II BWR	NA	NA for PNPS. This USI is addressed specifically to BWR plants with Mark II containments.

- * C - Complete
 I - Incomplete
 NA - Not Applicable
 NC - No Changes Necessary
 E - Evaluating Actions Necessary

Pilgrim Nuclear Power Station - Response to Generic Letter 89-21:
Unresolved Safety Issues For Which A Final Resolution Has Been Achieved

<u>USI/ MPA NUMBER</u>	<u>TITLE</u>	<u>REFERENCE DOCUMENT</u>	<u>APPLIC- ABILITY</u>	<u>STATUS * / DATE</u>	<u>REMARKS</u>
USI: A-9 MPA: A-20 C-02 TAC: 59127 65493	Anticipated Transients Without Scram	•NUREG-0460, Vol. 4 •10CFR 50.62 •BECO letter, dated 4/7/80, proposed TS changes for ATWS RPT/ARI mods. •NRC letter, dated 5/12/80, issued Amendment 42. •GL 85-03 •GL 85-06 •BECO letter, dated 5/29/87, proposed TS change. •NRC letter, dated 8/5/87, issued Amendment 102. •NRC letter, dated 9/8/88, issued Amendment 121. •NRC letter, dated 6/6/89, post-implementation review	All	I See note under Remarks	PNPS ATWS related modifications (ARI/RPT) required technical specification changes in 1980. NRC letter, dated 5/12/80, issued Amendment 42 including TS changes for ATWS ARI/RPT mods. Amendment 62, dated 8/5/82, included clarification of the LCO's for the ATWS related TS actions. Because of SLCS modifications required by 10 CFR 50.62, BECO proposed TS changes, dated 5/29/87. The NRC issued Amendment 102, dated 8/5/87. Modifications to the standby liquid control system were completed on 9/16/88 by plant design change (PDC) 86-75. On 9/8/88, the NRC issued a requested TS change (Amendment 121) that clarifies surveillance requirements for ATWS RPT/ARI instruments at PNPS. Note: This USI remains open for Pilgrim due to an unresolved issue regarding diversity requirements of the ATWS rule. The BWROG has appealed the NRC staff position on these diversity requirements. However, ATWS related systems at Pilgrim are operational.

- * C - Complete
 I - Incomplete
 NA - Not Applicable
 NC - No Changes Necessary
 E - Evaluating Actions Necessary

Pilgrim Nuclear Power Station - Response to Generic Letter 89-21:
Unresolved Safety Issues For Which A Final Resolution Has Been Achieved

<u>USI/ MPA NUMBER</u>	<u>TITLE</u>	<u>REFERENCE DOCUMENT</u>	<u>APPLIC- ABILITY</u>	<u>STATUS * / DATE</u>	<u>REMARKS</u>
USI: A-10 MPA: B-25 TAC: 8841 6866	BWR Feedwater Nozzle Cracking	•NUREG-0619 •NRC Letter, dated 11/13/80 •GL 81-11 •NRC Letter, dated 5/29/84 (SER)	BWR	C 1/11/85 See note under Remarks	The NRC safety evaluation, dated 5/29/84, concluded that appropriate modifications were being made at Pilgrim to limit crack growth in the feedwater nozzles. By cutting and capping the CRD return line, BECo eliminated potential cracking in the CRD return line nozzle. The staff indicated that BECo should continue to perform the routine inspections of the feedwater nozzles and spargers at the intervals specified in Table 2 of NUREG-0619 to ensure crack growth does not exceed ASME limits. These inspections are part of the augmented inspections conducted in conjunction with Pilgrim's ISI Program. Note: Modifications, procedures, and training, required by this USI (NUREG-0619) were completed by plant design change (PDC) 83-45 on 1/11/85.
USI: A-11 MPA: A-23 TAC: 71533	Reactor Vessel Material Toughness	•NUREG-0744, Rev. 1 •10 CFR 50.60 •GL 82-26	All	NC	BECo removed the first set of reactor vessel specimens during the 1980 refueling outage. These samples were analyzed by Southwest Research Institute and the results were submitted to the NRC with the 1981 Pilgrim In-Service Inspection (ISI) report. The report indicated that the material toughness remains above 50 ft.-lbs., as required by Appendix G to 10 CFR 50.

- * C - Complete
- I - Incomplete
- NA - Not Applicable
- NC - No Changes Necessary
- E - Evaluating Actions Necessary

Pilgrim Nuclear Power Station - Response to Generic Letter 89-21:
Unresolved Safety Issues For Which A Final Resolution Has Been Achieved

<u>USI/ MPA NUMBER</u>	<u>TITLE</u>	<u>REFERENCE DOCUMENT</u>	<u>APPLIC- ABILITY</u>	<u>STATUS * / DATE</u>	<u>REMARKS</u>
USI: A-12	Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports	•NUREG-0577, Rev. 1	PWR	NA	NA for PNPS. This USI is addressed by PWRs.
USI: A-17	Systems Interactions	•AEC letter to BECo, dated 8/3/72 (Quad Cities Flooding Event) •NUREG -1174 •NUREG-1229 •NUREG/CR-3922 •NUREG/CR-4261 •NUREG/CR-4470 •GL 88-20, (and Supp. 1) •GL 89-18 •BECo letter, dated 10/27/89, response to GL 88-20, Supp. 1 •NUREG/CR-5420	All	NC	While USI A-17, as resolved by GL 89-18, dated 9/6/89, did not require any licensee actions, it did provide 5 lessons learned from the review of the systems interaction issue. One lesson referenced was for licensees to continue to review events occurring at other operating nuclear power plants, per NUREG-0737 TMI task action plan (TAP) Item I.C.5. This TAP for PNPS was completed on 3/31/82, (see BECo letter, dated 4/18/89). Also, in response to GL 88-20, Supp. 1, on 10/27/89, BECo indicated we would perform a level 1 PRA in accordance with NRC guidance.

- * C - Complete
- I - Incomplete
- NA - Not Applicable
- NC - No Changes Necessary
- E - Evaluating Actions Necessary

Pilgrim Nuclear Power Station - Response to Generic Letter 89-21:
Unresolved Safety Issues For Which A Final Resolution Has Been Achieved

<u>USI/ MPA NUMBER</u>	<u>TITLE</u>	<u>REFERENCE DOCUMENT</u>	<u>APPLIC- ABILITY</u>	<u>STATUS * / DATE</u>	<u>REMARKS</u>
USI: A-24 MPA: B-60 TAC: 42477 56821 59761	Qualification of Class 1E Safety- Related Equipment	•NUREG-0588, Rev. 1 •10 CFR 50.49 •GL 82-09 •GL 84-24 •GL 85-15 •NRC letter, dated 3/26/85 •NRC letter, dated 4/15/86	All	C 11/30/85 See note under Remarks	The NRC safety evaluation, dated 3/26/85, concluded that BECo's electrical equipment environmental qualification program (EQ) was in compliance with the requirements of 10 CFR 50.49. BECo informed the NRC that we were in compliance as of 11/30/85. Note: A post-implementation inspection of the Pilgrim EQ Program was conducted on 12/9/85 through 12/13/85. Inspection Report 85-35, dated 4/15/86, determined that, as of 11/30/85, BECo had implemented an EQ program to meet the requirements of 10 CR 50.49. Open items from Inspection Report 85-35 were closed by Inspection Report 87-37, dated 11/3/87.
USI: A-26 MPA: B-04	Reactor Vessel Pressure Transient Protection	•DOR Letters to Licensees, dated 8/76 •NUREG-0224 •NUREG-0371 •GL 88-11	PWR	NA	NA for PNPS. This USI is addressed to PWRs.
USI: A-31	Residual Heat Removal Shutdown Requirements	•NUREG-0606 •RG 1.139	All OLS (after 1/79)	NA	NA for PNPS. The operating license for Pilgrim Station was issued prior to 1979.

- * C - Complete
 I - Incomplete
 NA - Not Applicable
 NC - No Changes Necessary
 E - Evaluating Actions Necessary

Pilgrim Nuclear Power Station - Response to Generic Letter 89-21:
Unresolved Safety Issues For Which A Final Resolution Has Been Achieved

<u>USI/ MPA NUMBER</u>	<u>TITLE</u>	<u>REFERENCE DOCUMENT</u>	<u>APPLIC- ABILITY</u>	<u>STATUS * / DATE</u>	<u>REMARKS</u>
USI: A-36 MPA: C-10 C-15 TAC: 10817 52257	Control of Heavy Loads Near Spent Fuel	•NUREG-0612 •GL 81-07 •GL 83-42 •GL 85-11 •NRC letter, dated 12/22/80 •NRC letter, dated 3/6/85, SE for Phase I, NUREG-0612 •BECO letter, dated 5/31/89, SIMS update	All	C 10/2/85 See note under Remarks	The NRC safety evaluation, dated 3/6/85, concluded that Pilgrim had complied with NUREG-0612, sections 5.1.1 and 5.1.3. and that Phase I of the Control of Heavy Loads was complete. GL 85-11, dated 6/28/85, determined that no further action was necessary. Also, the PNPS technical specifications, section 5.5.E, prohibits loads exceeding one thousand pounds from traveling over fuel assemblies stored in the spent fuel pool. Note: Modifications, procedures, and training, required by this USI, were completed by plant design change (PDC) 83-83 on 10/2/85.
USI: A-39 TAC: 7948	Determination of SRV Pool Dynamic Loads and Pressure Transients	•NUREG-0802 •NUREG-0763 •NUREG-0783 •NUREG-0661	BWR	NC See note under Remarks	Note: SRV acceptance criteria for Mark I containments are presented in NUREG-0661, and are dealt with as part of USI A-7. See implementation and status summary for USI A-7.
USI: A-40	Seismic Design Criteria	•NUREG/CR-4776 •NUREG/CR-3480 •NUREG/CR-1582 •NUREG/CR-1161 •NUREG-1233 •NUREG/CR-3805 •NUREG/CR-5347 •NUREG/CR-3509	All	NA See note under Remarks	USI A-40 does not apply to Pilgrim Station. Note: The seismic design concerns of USI A-40 are being addressed at PNPS by USI A-46, "Seismic Qualification of Equipment in Operating Plants". PNPS is a participant utility of the Seismic Qualification Utility Group (SQUG).

- * C - Complete
I - Incomplete
NA - Not Applicable
NC - No Changes Necessary
E - Evaluating Actions Necessary

Pilgrim Nuclear Power Station - Response to Generic Letter 89-21:
Unresolved Safety Issues For Which A Final Resolution Has Been Achieved

<u>USI/ MPA NUMBER</u>	<u>TITLE</u>	<u>REFERENCE DOCUMENT</u>	<u>APPLIC- ABILITY</u>	<u>STATUS * / DATE</u>	<u>REMARKS</u>
USI: A-42 MPA: B-05 B-84 TAC: 46673 56835 69153	Pipe Cracks in Boiling Water Reactors	<ul style="list-style-type: none"> •NUREG-0313, Rev. 1 •GL 81-04 •BECo letter, dated 7/8/81 •BECo letter, dated 5/20/83 •NRC letter, dated 6/20/84, SER for NUREG-0313,Rev.1 •BECo letter, dated 5/31/89, SIMS update <p>IGSCC Expanded Issues:</p> <ul style="list-style-type: none"> •GL 84-11 •NRC letter, dated 6/20/84, SER refers to GL 84-11 •BECo letter, dated 6/4/84, pipe replacement •BECo letter, dated 2/4/85, TS change per GL 84-11 •BECo letter, dated 5/6/87, augmented inspection plan •GL 88-01 •NUREG-0313, Rev. 2 •NRC letter, dated 3/8/88, closed GL 84-11 for PNPS •BECo letter, dated 8/4/88, response to GL 88-01 •NRC letter, dated 3/31/89, GL 88-01 RAI •BECo letter, dated 6/19/89, response to GL 88-01 RAI 	BWR	C 6/20/84	<p>The NRC SER, dated 6/20/84, concluded that the guidance of NUREG-0313, Rev.1 was moot because of newly identified IGSCC concerns with larger bore piping. Further staff review would come under Generic Letter 84-11. This SER closed USI A-42 for Pilgrim.</p> <p>IGSCC Expanded Issues: On 2/4/85, BECo proposed TS changes to address reactor coolant leak detection limits as specified in GL 84-11. This generic letter was subsequently superceded in its entirety by GL 88-01, which enclosed NUREG-0313, Rev.2. Augmented IGSCC inspections, required by GL 84-11, are now performed under GL 88-01 (NUREG-0313,Rev.2) criteria. The NRC staff did not complete its action on BECo's TS request pending the development of revised staff positions regarding the IGSCC problems as provided in GL 88-01. On 3/8/88, the NRC informed BECo that TAC number 69153 was assigned to the NRC review of this response as well as BECo's TS charge, dated 2/4/85. BECo provided responses to GL 88-01, and the associated RAI, on 8/4/88 and 6/19/89, respectively.</p>

- * C - Complete
I - Incomplete
NA - Not Applicable
NC - No Changes Necessary
E - Evaluating Actions Necessary

Pilgrim Nuclear Power Station - Response to Generic Letter 89-21:
Unresolved Safety Issues For Which A Final Resolution Has Been Achieved

<u>USI/ MPA NUMBER</u>	<u>TITLE</u>	<u>REFERENCE DOCUMENT</u>	<u>APPLIC- ABILITY</u>	<u>STATUS * / DATE</u>	<u>REMARKS</u>
USI: A-43	Containment Emergency Sump Performance	•NUREG-0869, Rev. 1 •NUREG-0897, Rev. 1 •R.G. 1.82, Rev. 1 •GL 85-22	All	NC	This USI was closed by GL 85-22, dated 12/3/85. The NRC concluded in the regulatory analysis performed for this USI, NUREG-0869, Rev. 1, dated October, 1985, and in the technical findings regarding this USI, presented in NUREG-0897, Rev. 1, also dated October, 1985, that no new generic requirements are needed for operating plants to address the concerns of this USI. Revised guidance, provided in R.G. 1.82, Rev. 1, allows licensees to perform 50.59 reviews to determine thermal insulation needs for use on primary system components. Therefore, this USI requires no action by PNPS.

- * C - Complete
 I - Incomplete
 NA - Not Applicable
 NC - No Changes Necessary
 E - Evaluating Actions Necessary

Pilgrim Nuclear Power Station - Response to Generic Letter 89-21:
Unresolved Safety Issues For Which A Final Resolution Has Been Achieved

<u>USI/ MPA NUMBER</u>	<u>TITLE</u>	<u>REFERENCE DOCUMENT</u>	<u>APPLIC- ABILITY</u>	<u>STATUS * / DATE</u>	<u>REMARKS</u>
USI: A-44 MPA: B-63 TAC: 68585 40577 43890	Station Blackout	<ul style="list-style-type: none"> •R.G. 1.155 •NUREG-1032 •NUREG-1109 •10 CFR 50.63 •GL 81-04 •BECo letter, dated 8/12/81, response to GL 81-04 •NUMARC letter, dated 9/28/88, generic response to SBO •NRC letter, dated 10/7/88 approved NUMARC response •BECo letter, dated 4/17/89, response to blackout rule 	All	I See note under Remarks	<p>In response to GL 81-04, procedure changes and training were identified to address the loss of all AC power at PNPS. During the extended refueling outage (RFO-7), a non-class 1E station blackout diesel generator (SBO-DG) was installed as part of a self-initiated safety enhancement program (SEP). In the 4/17/89 response to 10 CFR 50.63 (c)(3), BECo addressed the NRC approved NUMARC document, dated 9/28/88. BECo also indicated the SPO-DG will be designated as an alternative AC power source when the proposed modifications are completed.</p> <p>In the 4/17/89 letter, BECo projected an RFO 8 (spring of 1991) completion date in anticipation of receiving NRC acceptance notification shortly after the 4/17/89 submittal.</p> <p>Note: The NRC review was scheduled to be completed in the fall of 1989, however, due to problems identified during inspections at other plants related to SBO, a new review schedule, and additional guidance for complying with Station Blackout, will be provided to licensees by the NRC staff.</p>

- * C - Complete
- I - Incomplete
- NA - Not Applicable
- NC - No Changes Necessary
- E - Evaluating Actions Necessary

Pilgrim Nuclear Power Station - Response to Generic Letter 89-21:
Unresolved Safety Issues For Which A Final Resolution Has Been Achieved

<u>USI/ MPA NUMBER</u>	<u>TITLE</u>	<u>REFERENCE DOCUMENT</u>	<u>APPLIC- ABILITY</u>	<u>STATUS * / DATE</u>	<u>REMARKS</u>
USI: A-45 TAC: 74451	Shutdown Decay Heat Removal Requirements	•NUREG-1289 •NUREG/CR-5230 •GL 88-20 •GL 88-20,Supp.1 •BECo letter, dated 10/27/89, response to GL 88-20, Supp.1	All	NC	This USI was resolved by NUREG-1289, "Regulatory and Backfit Analysis: USI A-45", dated November 1988. No new requirements are imposed by this USI. The NRC has subsumed A-45 into its Severe Accident Policy as the most effective way to achieve resolution of the concerns of A-45. During the extended refueling outage (RFO7), a limited IPE was performed and several plant modifications implemented as part of a self-initiated Safety Enhancement Program (SEP). Several of these modifications enhance Pilgrim's capability to remove decay heat; for example, additional water sources for core spray and reactor water, additional diesel fire pump for supplying these water sources, improved containment spray nozzles, and ADS system logic changes. In the GL 88-20, Supp.1, response dated 10/27/89, BECo committed to perform a Level 1 PRA consistent with NRC guidance. The schedule for this PRA will be provided in the Long Term Plan updates, required by section 3.H of the PNPS operating license.

- * C - Complete
- I - Incomplete
- NA - Not Applicable
- NC - No Changes Necessary
- E - Evaluating Actions Necessary

Pilgrim Nuclear Power Station - Response to Generic Letter 89-21:
Unresolved Safety Issues For Which A Final Resolution Has Been Achieved

<u>USI/ MPA NUMBER</u>	<u>TITLE</u>	<u>REFERENCE DOCUMENT</u>	<u>APPLIC- ABILITY</u>	<u>STATUS * / DATE</u>	<u>REMARKS</u>
USI: A-46 MPA: B-105 TAC: 69741	Seismic Qualification of Equipment in Operating Plants	•NUREG-1030 •NUREG-1211 •GL 87-02 •BECO letter, dated 10/11/88, response to GL 87-02 •NRC letter, dated 7/29/88, SER for SQUG GIP •NRC letter, dated 6/7/89	All	E	USI A-46 was resolved with the issuance of GL 87-02 on 2/19/87. In addition, on 7/29/88, the NRC issued a safety evaluation for the Generic Implementation Procedure (GIP) developed by the Seismic Qualification Utility Group (SQUG). The SE identified several open issues which needed resolution. The BECO response to GL 87-02, dated 10/11/88, provided an initial schedule for implementation of the seismic verification program and also indicated BECO membership in SQUG. BECO noted the schedule for implementation of the GIP is contingent upon resolution and approval of the open issues by the NRC. In the 10/11/88 response, BECO also listed various actions to be conducted during operating cycle 8, prior to the end of 1990. This schedule is expected to be extended to cycle 9 and RFO 9. The actual implementation schedule for PNPS will be included in updates to the BECO Long Term Plan in accordance with license condition 3.H of the PNPS operating license.
USI: A-47 MPA: B-113 TAC: 74981	Safety Implication of Control Systems	•NUREG-1217 •NUREG-1218 •GL 89-19	All	E	A BECO response is due by 3/20/90, according to the requirements of GL 89-19, dated 9/20/89. Schedules for implementation will be determined in accordance with the Long Term Plan, license condition 3.H, of the PNPS operating license.

- * C - Complete
 I - Incomplete
 NA - Not Applicable
 NC - No Changes Necessary
 E - Evaluating Actions Necessary

Pilgrim Nuclear Power Station - Response to Generic Letter 89-21:
Unresolved Safety Issues For Which A Final Resolution Has Been Achieved

<u>USI/ MPA NUMBER</u>	<u>TITLE</u>	<u>REFERENCE DOCUMENT</u>	<u>APPLIC- ABILITY</u>	<u>STATUS * / DATE</u>	<u>REMARKS</u>
USI: A-48	Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment	•10 CFR 50.44 •GL 84-09 •NUREG-1370	All, except PWRs with large dry containments	NC	No changes to PNPS are required by this USI since PNPS has had an inert containment since initial startup in December of 1972. The original Technical Specification, section 3.7.A.5. "Oxygen Concentration", required the primary containment atmosphere oxygen concentration to be less than 5% oxygen within 24 hours of placing the reactor in Run mode. Amendment 87, dated 4/27/85, reduced the oxygen concentration to less than 4% for compliance with 10 CFR 50.44. This is the present operating condition for PNPS.
USI: A-49	Pressurized Thermal Shock	•RG 1.154 •RG 1.99 •10 CFR 50.61 •GL 88-11	PWR	NA	NA for PNPS. This USI is addressed to PWRs.

- * C - Complete
- I - Incomplete
- NA - Not Applicable
- NC - No Changes Necessary
- E - Evaluating Actions Necessary