

TABLE 3.2.6

POST-ACCIDENT INSTRUMENTATION

Minimum Number of Operable Instrument Channels	Parameter	Type of Indication	Instrument Range
2	Drywell Atmospheric Temperature (Note 1)	Recorder #16-19-45 Recorder #TR-1-149	0-300° F 0-300° F
2	Drywell Pressure (Note 1) Torus Pressure (Note 1)	Recorder #16-19-44	0-80 psia 0-80 psia
2	Torus Water Level (Note 3)	Meter #16-19-46A Meter #16-19-46B	0-3 ft. 0-3 ft.
2	Torus Water Temperature	Meter #16-19-48	60-180° F
2	Reactor Pressure (Note 1)	Recorder #6-97 Meter #6-90A Meter #6-90B	0-1200 psig 0-1200 psig 0-1200 psig
2	Reactor Vessel Water Level (Note 1)	Meter #2-3-91A Meter #2-3-91B	(-150)-0-(+150)"H ₂ O (-150)-0-(+150)"H ₂ O
1	Control Rod Position (Note 1, 2)	Meter	0-48" RPIS
1	Neutron Monitor (Note 1, 2)	Meter	0-125% Rated Flux
1	Torus Air Temperature (Note 1)	Recorder #TR-16-19-45	0-300° F
2/valve	Safety/Relief Valve Position from pressure switches (Note 4)	Lights (SRV 2-71-A through D)	Closed - Open
1/valve	Safety Valve Position from Acoustic Monitor (Note 5)	Meter Z1-2-1A/B	Closed - Open

Note 1 - From and after the date that one of these parameters is not indicated in the control room, continued reactor operation is permissible during the next seven days. If reduced to one indication of a parameter operation is permissible for 30 days.

Note 2 - Control rod position and neutron monitor instruments are considered to be redundant to each other.

TABLE 3.2.6 NOTES

- Note 3 - From and after the date that this parameter is reduced to one indication in the control room, continued reactor operation is permissible during the next thirty days. If both channels are inoperable and indication cannot be restored in six hours, an orderly shutdown shall be initiated and the reactor shall be in a hot shutdown condition in six hours and a cold shutdown condition in the following 18 hours.
- Note 4 - From and after the date that safety/relief valve position from pressure switches is unavailable, reactor operation may continue provided safety/relief valve position can be determined from recorder 2-166 (thermocouple, 0-600°F) and meter 16-19-48 (torus water temperature, 60-180°F). If both indications are not available, the reactor shall be in a hot shutdown condition in six hours and a cold shutdown condition in the following 18 hours.
- Note 5 - From and after the date that safety valve position from the acoustic monitor is unavailable, reactor operation may continue provided safety valve position can be determined from recorder 2-166 (thermocouple, 0-600°F) and recorder 16-19-44 (drywell pressure 0-80 psia). If both indications are not available, the reactor shall be in a hot shutdown condition in six hours and in a cold shutdown condition in the following 18 hours.

6.9 ENVIRONMENTAL QUALIFICATION

- A. By no later than June 30, 1982 all safety-related electrical equipment in the facility shall be qualified in accordance with the provisions of Division of Operating Reactors "Guidelines for Evaluating Environmental Qualification of Class 1E Electrical Equipment in Operating Reactors" (DOR Guidelines); or, NUREG-0588 "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment", December 1979. Copies of these documents are attached to Order for Modification of License DPR-28 dated October 24, 1980.
- B. By no later than December 1, 1980, complete and auditable records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Thereafter, such records should be updated and maintained current as equipment is replaced, further tested, or otherwise qualified.

6.10 INTEGRITY OF SYSTEMS OUTSIDE CONTAINMENT

A program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels will be implemented. This program shall include the following:

- 1) Provisions establishing preventive maintenance and periodic visual inspection requirements, and
- 2) System leakage inspections, to the extent permitted by system design and radiological conditions, for each system at a frequency not to exceed refueling cycle intervals. The systems subject to this testing are (1) Residual Heat Removal, (2) Core Spray, (3) Reactor Water Cleanup, (4) HPCI, (5) RCIC, and (6) sample systems.

6.11 IODINE MONITORING

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas* under accident conditions will be implemented. This program shall include the following:

- 1) Training of personnel,
- 2) Procedures for monitoring, and
- 3) Provisions for maintenance of sampling and analysis equipment.

* Areas requiring personnel access for establishing hot shutdown condition.