

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

September 19, 2019

ANO Site Vice President Arkansas Nuclear One Entergy Operations, Inc. N-TSB-58 1448 S.R. 333 Russellville, AR 72802

SUBJECT:

ARKANSAS NUCLEAR ONE, UNIT 1 – CORRECTION TO ISSUANCE OF AMENDMENT NO. 264 TO ADOPT TECHNICAL SPECIFICATIONS TASK FORCE (TSTF) TRAVELER TSTF-425, REVISION 3, "RELOCATE SURVEILLANCE FREQUENCIES TO LICENSEE CONTROL – RITSTF INITIATIVE 5b" (EPID L-2018-LLA-0063)

Dear Sir or Madam:

By letter dated May 22, 2019 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML19098A955), the U.S. Nuclear Regulatory Commission (NRC) issued Amendment No. 264 to Renewed Facility Operating License No. DPR-51 for Arkansas Nuclear One, Unit 1 (ANO-1). The amendment adopted changes to the ANO-1 technical specifications (TSs) consistent with Technical Specifications Task Force (TSTF) Improved Standard Technical Specifications Change Traveler TSTF-425, Revision 3, "Relocate Surveillance Frequencies to Licensee Control – RITSTF [Risk-Informed TSTF] Initiative 5b," dated March 18, 2009 (ADAMS Accession No. ML090850642).

After the issuance of the amendment, the NRC staff was notified by Entergy Operations, Inc. (the licensee) of typographical errors unrelated to the license amendment that had been inadvertently included in revised TS page 3.4.14-2. Specifically, the two surveillance conditions in Surveillance Requirement 3.4.14.3 were incorrectly identified as items "c." and "d." They should have remained unchanged on TS page 3.4.14-2 as items "a." and "b.," as shown below:

- a. <= 340 psig for one valve; and
- b. <= 400 psig for the other valve

Upon further review, the NRC staff determined that the errors had been previously introduced during the preparation of Amendment No. 257, which was issued on March 10, 2017 (ADAMS Accession No. ML16165A423), and inadvertently carried forward without correction in Amendment No. 264. The NRC staff determined that these errors were introduced during the preparation of the license amendments and are entirely typographical in nature. The corrections do not change any of the conclusions in the safety evaluations associated with the amendments and do not affect the associated notices to the public.

The enclosure to this letter contains the corrected TS page 3.4.14-2. Please replace the corresponding page issued by Amendment No. 264 for Arkansas Nuclear One, Unit 1.

We regret any inconvenience this may have caused. If you have any questions, please contact me at (301) 415-4037 or by e-mail at Thomas.Wengert@nrc.gov.

Sincerely,

Thomas J. Wengert, Senior Project Manager

Plant Licensing Branch IV

Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket No. 50-313

Enclosure:

Corrected TS Page to Renewed Facility Operating License DPR-51 for Amendment No. 264

cc: Listserv

ENCLOSURE

FOR AMENDMENT NO. 264 ENTERGY OPERATIONS, INC. ARKANSAS NUCLEAR ONE, UNIT 1 DOCKET NO. 50-313

SURVEILLANCE REQUIREMENTS

	FREQUENCY	
SR 3.4.14.1	Not required to be performed in MODES 3 and 4. Verify leakage from each RCS pressure isolation check valve, or pair of check valves, as applicable, is less than or equal to an equivalent of the Allowable Leakage Limit identified below at a differential test pressure ≥ 150 psid. Pressure Isolation Allowable Leakage Limit Check Valve(s) Leakage Limit DH-14A ≤ 5 gpm DH-13A and DH-17 ≤ 5 gpm total DH-14B ≤ 5 gpm DH-13B and DH-18 ≤ 5 gpm total	In accordance with the INSERVICE TESTING PROGRAM AND Once prior to entering MODE 2 whenever the unit has been in MODE 5 for 7 days or more, if leakage testing has not been performed in the previous 9 months
SR 3.4.14.2	Verify DHR System autoclosure interlock prevents the valves from being opened with a simulated or actual high RCS pressure signal.	In accordance with the Surveillance Frequency Control Program
SR 3.4.14.3	Verify DHR System autoclosure interlock causes the valves to close automatically with a simulated or actual high RCS pressure signal: a. ≤ 340 psig for one valve; and b. ≤ 400 psig for the other valve.	In accordance with the Surveillance Frequency Control Program
SR 3.4.14.4	Verify DHR System autoclosure interlock prevents the valves from being opened with a simulated or actual Core Flood Tank isolation valve "not closed" signal.	In accordance with the Surveillance Frequency Control Program
SR 3.4.14.5	Verify DHR System autoclosure interlock causes the valves to close automatically with a simulated or actual Core Flood Tank isolation valve "not closed" signal.	In accordance with the Surveillance Frequency Control Program

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ARKANSAS NUCLEAR ONE, UNIT 1 - CORRECTION TO ISSUANCE OF AMENDMENT NO. 264 TO ADOPT TECHNICAL SPECIFICATIONS TASK

FORCE (TSTF) TRAVELER TSTF-425, REVISION 3, "RELOCATE SURVEILLANCE FREQUENCIES TO LICENSEE CONTROL - RITSTF INITIATIVE 5b" (EPID L-2018-LLA-0063) DATED SEPTEMBER 19, 2019

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ADAMS Accession No.: ML19232A379

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