NRC FORM 366 U. S. NUCLEAR REGULATORY COMMISSION (7.77) LICENSEE EVENT REPORT (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) CONTROL BLOCK: 10 0 LICENSE NUMBER CON'T 4 6 7 1 1 0 9 7 9 8 1 1 2 1 7 9 9 68 69 EVENT DATE 74 75 REPORT DATE 80 REPORT L 6 0 5 0 - 0 3 60 61 DOCKET NUMBER 0 1 SOURCE EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) 0 2 On 11/9/79 at 1400 hours during the review of NRC I&E Bulletin 79-21 response, it was discovered that our steam generator low level trip setpoints to Steam and Feedwater Rup- | 0 3 fture Control System Channels 1 and 2 were less conservative than the limits of T.S. 0 4 This finding is being reported in accordance with T.S. 6.9.1.8.h. A Mode 3 13.3.2.2. 0 5 restraint was placed on the unit until new values were determined and adjustments 0 6 made. A safety analysis is being made and will be forwarded as a revision. 0 7 (NP-32-79-12) 0 8 80 CAUSE COMP SYSTEM CAUSE VALVE CODE SUBCODE COMPONENT CODE SUBCODE CODE T | R | U |(14 E (15) B (12) CIHI S ZI (16) (13) 18 10 REVISION OCCURRENCE SEQUENTIAL REPORT EVENT YEAR REPORT NO. CODE TYPE LER/RO 19 1015 011 Т (17)REPORT NUMBER COMPONENT SHUTDOWN NPRD-4 FORM SUB. FRIME COMP ATTACHMENT ACTION FUTURE (22) HOURS MANUFACTURER TAKEN ACTION ON PLANT Z (21) Y 23 1110 N (24) Z N (25 5 6 Ø (26) (18)(19 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) The cause of the occurrence was an error in the analysis used in determining the 1 0 steam generator low level bistable setpoints. The analysis failed to take into 1 1 account the actual pressures and temperatures as seen by the steam generator during 1 2 operation. On 11/17/79, the adjustments were made and ST 5031.14 was successfully 1 3 performed. The restraint was lifted. 1 4 80 METHOD OF FACILITY OTHER STATUS (30) DISCOVERY DESCRIPTION (32) % POWER DISCOVERY [31] NRC Inspection Report 79-21 10 D NA 5 G (28) 80 ACTIVITY CONTENT LOCATION OF RELEASE (36) AMOUNT OF ACTIVITY (35) OF RELEASE RELEASED INA Z 33 Z 34 NA 6 80 10 11 PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER TYPE ø (37) Z (38) NA Ø PERSONNEL INJURIES 80 DESCRIPTION (41) NA (40) 80 11 12 LOSS OF OR DAMAGE TO FACILITY (43) DESCRIPTION ZI NA 1200 PUBLICITY NRC USE ONLY 7911270 418 DESCRIPTION (45) SSUED N (44) NA 419-259-5000, 235 Dan Trautman PHONE: DVR 79-163 NAME OF PREPARER.

TOLEDO EDISON COMPANY DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE SUPPLEMENTAL INFORMATION FOR LER NP-32-79-12

DATE OF EVENT: November 9, 1979

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: Steam Generator low level trip setpoints found to be less conservative than assumed.

<u>Conditions Prior to Occurrence</u>: The unit was in Mode 5, with Power (MWT) = 0, and Load (Gross MWE) = 0.

Description of Occurrence: On November 9, 1979 at 1400 hours, during the review of NRC I&E Bulletin 79-21 response, it was discovered that our steam generator low level trip setpoints to Steam and Feedwater Rupture Control System (SFRCS) Channels 1 and 2 were less conservative than the limits of Technical Specification 3.3.2.2. The steam generator low level bistable trip setpoint was 23 inches + 2 inches. However, the re-evaluation of the method used to determine the setpoints based on actual versus indicated level found that the actual level would be less conservative than the minimum 20 inches of water above the Jower tube sheet as required by Technical Specifications.

As a result, a Mode 3 restraint was placed on the unit until new values could be determined and adjustments to the bistable trip setpoint made. This occurrence is being reported in accordance with Technical Specification 6.9.1.8.h which requires prompt notification with a followup report within two weeks for an error in analysis, which would have permitted reactor operation in a manner less conservative than assumed.

Designation of Apparent Cause of Occurrence: The apparent cause of the occurrence was found to be an error in the analysis used in determining the steam generator low level bistable setpoints. The analysis failed to take into account the actual pressures and temperatures as seen by the steam generator during peration and in transients.

Analysis of Occurrence: A safety analysis is being performed and will be forwarded as a revision.

<u>Corrective Action</u>: On November 17, 1979, new steam generator low level bistable trip setpoints were established and adjustments were made. All four channels were proven operable by performance of ST 5031.14, "SFRCS Monthly Surveillance Test". The mode restraint relating this event was removed as of 1755 hours on November 19, 1979.

Failure Data: There have been no previous similar occurrences.

LER #79-105