



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 8, 2019

Mr. Thomas D. Ray
Site Vice President
McGuire Nuclear Station
Duke Energy Carolinas, LLC
12700 Hagers Ferry Road
Huntersville, NC 28078-8985

SUBJECT: MCGUIRE NUCLEAR STATION, UNITS 1 AND 2 – ISSUANCE OF AMENDMENT NOS. 316 AND 295 TO CORRECT EDITORIAL ERROR IN TECHNICAL SPECIFICATION SURVEILLANCE REQUIREMENT 3.0.5 AND REMOVE EXPIRED FOOTNOTES FROM TS 3.5.2, 3.6.6, 3.7.5, 3.7.6, 3.7.7, 3.7.9, 3.7.11, 3.8.1, AND 3.8.4 (EPID L-2019-LLA-0022)

Dear Mr. Ray:

The U.S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment Nos. 316 and 295 to Renewed Facility Operating License Nos. NPF-9 and NPF-17 for the McGuire Nuclear Station, Units 1 and 2, respectively. The amendments revise the Technical Specifications (TSs) in response to the application from Duke Energy Carolinas, LLC dated February 5, 2019.

The amendments correct an editorial error in section 3.0, "SR [Surveillance Requirement] APPLICABILITY," specifically, SR 3.0.5. The amendments also modify Technical Specification (TS) 3.5.2, "ECCS [Emergency Core Cooling System] - Operating," TS 3.6.6, "Containment Spray System," TS 3.7.5, "Auxiliary Feedwater (AFW) System," TS 3.7.6, "Component Cooling Water (CCW) System," TS 3.7.7, "Nuclear Service Water System (NSWS)," TS 3.7.9, "Control Room Area Ventilation System (CRAVS)," TS 3.7.11, "Auxiliary Building Filtered Ventilation Exhaust System (ABFVES)," TS 3.8.1, "AC [Alternating Current] Sources - Operating," and TS 3.8.4, "DC [Direct Current] Sources – Operating" to remove expired TS footnotes.

A copy of the related Safety Evaluation is also enclosed.

T. Ray

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A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

If you have any questions, please contact me at 301-415-3867 or Michael.Mahoney@nrc.gov.

Sincerely,

A handwritten signature in black ink, appearing to read "Michael Mahoney", with a long horizontal flourish extending to the right.

Michael Mahoney, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-369 and 50-370

Enclosures:

1. Amendment No. 316 to NPF-9
2. Amendment No. 295 to NPF-17
3. Safety Evaluation

cc: Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE ENERGY CAROLINAS, LLC

DOCKET NO. 50-369

MCGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 316
Renewed License No. NPF-9

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility), Renewed Facility Operating License No. NPF-9, filed by Duke Energy Carolinas, LLC (the licensee), dated February 5, 2019, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-9 is hereby amended to read as follows:

- (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 316, are hereby incorporated into this renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 120 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed License
and Technical Specifications

Date of Issuance: August 8, 2019



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE ENERGY CAROLINAS, LLC

DOCKET NO. 50-370

MCGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 295
Renewed License No. NPF-17

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility), Renewed Facility Operating License No. NPF-17, filed by the Duke Energy Carolinas, LLC (the licensee), dated February 5, 2019, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-17 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 295, are hereby incorporated into this renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 120 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed License
and Technical Specifications

Date of Issuance: August 8, 2019

ATTACHMENT

AMENDMENT NO. 316 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-9

AMENDMENT NO. 295 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NOS. 50-369 AND 50-370

Operating Licenses

Replace the following pages of the Renewed Facility Operating Licenses with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

NPF-9, page 3
NPF-17, page 3

Insert

NPF-9, page 3
NPF-17, page 3

Appendix A, "Technical Specifications," of Operating Licenses

Replace the following pages of the Appendix A Technical Specifications (TSs) with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

TS 3.0-6
TS 3.5.2-1
TS 3.6.6-1
TS 3.7.5-1
TS 3.7.5-2
TS 3.7.6-1
TS 3.7.7-1
TS 3.7.7-2
TS 3.7.9-1
TS 3.7.9-3
TS 3.7.11-1
TS 3.8.1-4
TS 3.8.4-1

Insert

TS 3.0-6
TS 3.5.2-1
TS 3.6.6-1
TS 3.7.5-1
TS 3.7.5-2
TS 3.7.6-1
TS 3.7.7-1
TS 3.7.7-2
TS 3.7.9-1
TS 3.7.9-3
TS 3.7.11-1
TS 3.8.1-4
TS 3.8.4-1

- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
 - (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproducts and special nuclear materials as may be produced by the operation of McGuire Nuclear Station, Units 1 and 2, and;
 - (6) Pursuant to the Act and 10 CFR Parts 30 and 40, to receive, possess and process for release or transfer such byproduct material as may be produced by the Duke Training and Technology Center.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level
The licensee is authorized to operate the facility at a reactor core full steady state power level of 3469 megawatts thermal (100%).
 - (2) Technical Specifications
The Technical Specifications contained in Appendix A, as revised through Amendment No. 316, are hereby incorporated into this renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.
 - (3) Updated Final Safety Analysis Report
The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on December 16, 2002, describes certain future activities to be completed before the period of extended operation. Duke shall complete these activities no later than June 12, 2021, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.

The Updated Final Safety Analysis Report supplement as revised on December 16, 2002, described above, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following issuance of this renewed operating license. Until that update is complete, Duke may make changes to the programs described in such supplement without prior Commission approval, provided that Duke evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts, 30, 40 and 70, to possess, but not separate, such byproducts and special nuclear materials as may be produced by the operation of McGuire Nuclear Station, Units 1 and 2; and,
- (6) Pursuant to the Act and 10 CFR Parts 30 and 40, to receive, possess and process for release or transfer such by product material as may be produced by the Duke Training and Technology Center.

C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or thereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at a reactor core full steady state power level of 3469 megawatts thermal (100%).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 295, are hereby incorporated into this renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Updated Final Safety Analysis Report

The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on December 16, 2002, describes certain future activities to be completed before the period of extended operation. Duke shall complete these activities no later than March 3, 2023, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.

The Updated Final Safety Analysis Report supplement as revised on December 16, 2002, described above, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following issuance of this renewed operating license. Until that update is complete, Duke may make changes to the programs described in such supplement without prior Commission approval, provided that Duke evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59, and otherwise complies with the requirements in that section.

3.0 SR APPLICABILITY (continued)

SR 3.0.4 Entry into a MODE or other specified condition in the Applicability of an LCO shall only be made when the LCO's Surveillances have been met within their specified Frequency, except as provided by SR 3.0.3. When an LCO is not met due to Surveillances not having been met, entry into a MODE or other specified condition in the Applicability shall only be made in accordance with LCO 3.0.4.

This provision shall not prevent entry into MODES or other specified conditions in the Applicability that are required to comply with ACTIONS or that are part of a shutdown of the unit.

SR 3.0.5 Surveillance Requirements shall apply to each unit individually unless otherwise indicated as stated in LCO 3.0.9 for individual Specifications or whenever certain portions of a Specification contain surveillance parameters different for each unit, which will be identified in parentheses or footnotes.

3.5 EMERGENCY CORE COOLING SYSTEMS (ECCS)

3.5.2 ECCS — Operating

LCO 3.5.2 Two ECCS trains shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

-----NOTE-----
 In MODE 3, both safety injection (SI) pump or RHR pump flow paths may be isolated by closing the isolation valves for up to 2 hours to perform pressure isolation valve testing per SR 3.4.14.1.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more trains inoperable. <u>AND</u> At least 100% of the ECCS flow equivalent to a single OPERABLE ECCS train available.	A.1 Restore train(s) to OPERABLE status.	72 hours
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3.	6 hours
	<u>AND</u> B.2 Be in MODE 4.	12 hours

3.6 CONTAINMENT SYSTEMS

3.6.6 Containment Spray System

LCO 3.6.6 Two containment spray trains shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One containment spray train inoperable.	A.1 Restore containment spray train to OPERABLE status.	72 hours
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3.	6 hours
	<u>AND</u> B.2 Be in MODE 5.	84 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.6.6.1 -----NOTE----- Not required to be met for system vent flow paths opened under administrative control. ----- Verify each containment spray manual and power operated valve in the flow path that is not locked, sealed, or otherwise secured in position is in the correct position.	In accordance with the Surveillance Frequency Control Program

(continued)

3.7 PLANT SYSTEMS

3.7.5 Auxiliary Feedwater (AFW) System

LCO 3.7.5 Three AFW trains shall be OPERABLE.

-----NOTE-----
Only one AFW train, which includes a motor driven pump, is required to be OPERABLE in MODE 4.

APPLICABILITY: MODES 1, 2, and 3,
MODE 4 when steam generator is relied upon for heat removal.

ACTIONS

-----NOTE-----
LCO 3.0.4.b is not applicable when entering MODE 1.

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>A. One steam supply to turbine driven AFW pump inoperable.</p> <p><u>OR</u></p> <p>-----NOTE----- Only applicable if MODE 2 has not been entered following refueling. -----</p> <p>One turbine driven AFW pump inoperable in MODE 3 following refueling.</p>	<p>A.1 Restore affected equipment to OPERABLE status.</p>	<p>7 days</p> <p><u>AND</u></p> <p>10 days from discovery of failure to meet the LCO</p>
<p>B. One AFW train inoperable in MODE 1, 2 or 3 for reasons other than Condition A.</p>	<p>B.1 Restore AFW train to OPERABLE status.</p>	<p>72 hours</p> <p><u>AND</u></p> <p>10 days from discovery of failure to meet the LCO</p>

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>C. Required Action and associated Completion Time for Condition A or B not met.</p> <p><u>OR</u></p> <p>Two AFW trains inoperable in MODE 1, 2, or 3.</p>	<p>C.1 Be in MODE 3.</p> <p><u>AND</u></p> <p>C.2 Be in MODE 4.</p>	<p>6 hours</p> <p>12 hours</p>
<p>D. Three AFW trains inoperable in MODE 1, 2, or 3.</p>	<p>D.1 -----NOTE----- LCO 3.0.3 and all other LCO Required Actions requiring MODE changes are suspended until one AFW train is restored to OPERABLE status. -----</p> <p>Initiate action to restore one AFW train to OPERABLE status.</p>	<p>Immediately</p>
<p>E. Required AFW train inoperable in MODE 4.</p>	<p>E.1 Initiate action to restore AFW train to OPERABLE status.</p>	<p>Immediately</p>

3.7 PLANT SYSTEMS

3.7.6 Component Cooling Water (CCW) System

LCO 3.7.6 Two CCW trains shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>A. One CCW train inoperable.</p>	<p>A.1 -----NOTE----- Enter applicable Conditions and Required Actions of LCO 3.4.6, "RCS Loops — MODE 4," for residual heat removal loops made inoperable by CCW. ----- Restore CCW train to OPERABLE status.</p>	<p>72 hours</p>
<p>B. Required Action and associated Completion Time of Condition A not met.</p>	<p>B.1 Be in MODE 3. <u>AND</u> B.2 Be in MODE 5.</p>	<p>6 hours 36 hours</p>

3.7 PLANT SYSTEMS

3.7.7 Nuclear Service Water System (NSWS)

LCO 3.7.7 Two NSWS trains shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>A. One NSWS train inoperable.</p>	<p>A.1 -----NOTES----- 1. Enter applicable Conditions and Required Actions of LCO 3.8.1, "AC Sources— Operating," for emergency diesel generator made inoperable by NSWS. 2. Enter applicable Conditions and Required Actions of LCO 3.4.6, "RCS Loops—MODE 4," for residual heat removal loops made inoperable by NSWS. ----- Restore NSWS train to OPERABLE status.</p>	<p>72 hours</p>
<p>B. Required Action and associated Completion Time of Condition A not met.</p>	<p>B.1 Be in MODE 3. <u>AND</u> B.2 Be in MODE 5.</p>	<p>6 hours 36 hours</p>

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>SR 3.7.7.1 -----NOTE----- Isolation of NSWS flow to individual components does not render the NSWS inoperable. -----</p> <p>Verify each NSWS manual, power operated, and automatic valve in the flow path servicing safety related equipment, that is not locked, sealed, or otherwise secured in position, is in the correct position.</p>	<p>In accordance with the Surveillance Frequency Control Program</p>
<p>SR 3.7.7.2 Verify each NSWS automatic valve in the flow path servicing safety related equipment, that is not locked, sealed, or otherwise secured in position, actuates to the correct position on an actual or simulated actuation signal.</p>	<p>In accordance with the Surveillance Frequency Control Program</p>
<p>SR 3.7.7.3 Verify each NSWS pump starts automatically on an actual or simulated actuation signal.</p>	<p>In accordance with the Surveillance Frequency Control Program</p>

3.7 PLANT SYSTEMS

3.7.9 Control Room Area Ventilation System (CRAVS)

LCO 3.7.9 Two CRAVS trains shall be OPERABLE.

-----NOTE-----
The control room envelope (CRE) boundary may be opened intermittently under administrative control.

APPLICABILITY: MODES 1, 2, 3, 4, 5, and 6,
During movement of irradiated fuel assemblies,
During CORE ALTERATIONS.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One CRAVS train inoperable for reasons other than Condition B.	A.1 Restore CRAVS train to OPERABLE status.	7 days
B. One or more CRAVS trains inoperable due to inoperable CRE boundary in MODE 1,2,3, or 4.	B.1 Initiate action to implement mitigating actions.	Immediately
	<u>AND</u>	24 hours
	B.2 Verify mitigating actions ensure CRE occupant exposures to radiological, chemical, and smoke hazards will not exceed limits.	90 days
C. Required Action and associated Completion Time of Condition A or B not met in MODE 1, 2, 3, or 4.	B.3 Restore CRE boundary to OPERABLE status.	
	<u>AND</u>	
C. Required Action and associated Completion Time of Condition A or B not met in MODE 1, 2, 3, or 4.	C.1 Be in MODE 3.	6 hours
	C.2 Be in MODE 5.	36 hours (continued)

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3.7 PLANT SYSTEMS

3.7.11 Auxiliary Building Filtered Ventilation Exhaust System (ABFVES)

LCO 3.7.11 Two ABFVES shall be OPERABLE.

-----NOTE-----
The Auxiliary Building pressure boundary may be opened intermittently under administrative controls.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One ABFVES inoperable.	A.1 Restore ABFVES to OPERABLE status.	7 days
B. Two ABFVES inoperable.	B.1 Restore one ABFVES to OPERABLE status.	24 hours
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 3.	6 hours
	<u>AND</u> C.2 Be in MODE 5.	36 hours

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>B. (continued)</p>	<p>B.5 Evaluate availability of Emergency Supplemental Power Source (ESPS).</p> <p><u>AND</u></p> <p>B.6 Restore DG to OPERABLE status.</p>	<p>1 hour</p> <p><u>AND</u></p> <p>Once per 12 hours thereafter</p> <p><u>AND</u></p> <p>72 hours from discovery of unavailable ESPS</p> <p><u>AND</u></p> <p>24 hours from discovery of Condition B entry ≥ 48 hours concurrent with unavailability of ESPS</p> <p><u>AND</u></p> <p>14 days</p> <p><u>AND</u></p> <p>17 days from discovery of failure to meet LCO 3.8.1.a or LCO 3.8.1.b</p>

3.8 ELECTRICAL POWER SYSTEMS

3.8.4 DC Sources — Operating

LCO 3.8.4 The four channels of DC sources shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>A. One channel of DC source inoperable.</p>	<p>A.1 Restore channel of DC source to OPERABLE status.</p>	<p>2 hours</p>
	<p><u>OR</u></p> <p>A.2.1 Verify associated bus tie breakers are closed between DC channels.</p>	<p>2 hours</p>
	<p><u>AND</u></p> <p>A.2.2 Restore channel of DC source to OPERABLE status.</p>	<p>72 hours</p>
<p>B. Required Action and Associated Completion Time not met.</p>	<p>B.1 Be in MODE 3.</p>	<p>6 hours</p>
	<p><u>AND</u></p> <p>B.2 Be in MODE 5.</p>	<p>36 hours</p>



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO

AMENDMENT NO. 316 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-9

AND

AMENDMENT NO. 295 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-17

DUKE ENERGY CAROLINAS, LLC

MCGUIRE NUCLEAR STATION, UNITS 1 AND 2

DOCKET NOS. 50-369 AND 50-370

1.0 INTRODUCTION

By letter dated February 5, 2019 (Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML19042A117), Duke Energy Carolinas, LLC (the licensee) submitted a license amendment request to change the Technical Specifications (TSs) for the McGuire Nuclear Station (McGuire), Units 1 and 2. The amendments correct an editorial error in section 3.0, "SR [Surveillance Requirement] APPLICABILITY," specifically, SR 3.0.5. The proposed amendments also revise TS 3.5.2, "ECCS [Emergency Core Cooling System] - Operating," TS 3.6.6, "Containment Spray System," TS 3.7.5, "Auxiliary Feedwater (AFW) System," TS 3.7.6, "Component Cooling Water (CCW) System," TS 3.7.7, "Nuclear Service Water System (NSWS)," TS 3.7.9, "Control Room Area Ventilation System (CRAVS)," TS 3.7.11, "Auxiliary Building Filtered Ventilation Exhaust System (ABFVES)," TS 3.8.1, "AC [Alternating Current] Sources - Operating," and TS 3.8.4, "DC [Direct Current] Sources - Operating" to remove expired TS footnotes.

2.0 REGULATORY EVALUATION

2.1 Licensee's Proposed Changes

Current TS SR 3.0.5 is as follows:

Surveillance Requirements shall apply to each unit individually unless otherwise indicated as stated in LCO [limiting condition for operation] 3.0.8 for individual Specifications or whenever certain portions of a Specification contain surveillance parameters different for each unit, which will be identified by parentheses or footnotes.

The licensee proposed to revise TS SR 3.0.5 as follows:

Surveillance Requirements shall apply to each unit individually unless otherwise indicated as stated in LCO 3.0.9 for individual Specifications or whenever certain portions of a Specification contain surveillance parameters different for each unit, which will be identified by parentheses or footnotes.

The licensee proposed to delete the following two footnotes from TS 3.5.2:

* 'A' Train ECCS is allowed to be inoperable for a total of 14 days to address a non-conforming condition on the 'A' Train NSWS supply piping from the Standby Nuclear Service Water Pond (SNSWP). The 14 days may be taken consecutively or in parts until completion of the activity, or by March 31, 2019, whichever occurs first. During the period in which the 'A' Train NSWS supply piping from the SNSWP is not available, the 'A' Train NSWS will remain aligned to Lake Norman until the system is ready for post maintenance testing. Any maintenance that is performed on the remaining portions of 'A' Train NSWS during the period in which the 'A' Train NSWS from the SNSWP supply piping is not available will be limited to a 72 hour completion time. The latter will not count against the 14 day completion time. Allowance of the extended Completion Time is contingent on meeting the Compensatory Measures described in MNS [McGuire Nuclear Station] LAR submittal correspondence letter MNS-17-031.

† For Unit 1 only, the Completion Time for Required Action A.1 may be extended one-time to 10 days during the 1A RHR AHU repair evolution and is contingent on meeting the compensatory measures described in MNS correspondence letter MNS-15-093. Upon completion of the repair evolution, this footnote is no longer applicable and will expire on March 31, 2016.

The licensee proposed to delete the following footnote from TS 3.6.6:

* 'A' Train Containment Spray is allowed to be inoperable for a total of 14 days to address a non-conforming condition on the 'A' Train NSWS supply piping from the Standby Nuclear Service Water Pond (SNSWP). The 14 days may be taken consecutively or in parts until completion of the activity, or by March 31, 2019, whichever occurs first. During the period in which the 'A' Train NSWS supply piping from the SNSWP is not available, the 'A' Train NSWS will remain aligned to Lake Norman until the system is ready for post maintenance testing. Any maintenance that is performed on the remaining portions of 'A' Train NSWS during the period in which the 'A' Train NSWS from the SNSWP supply piping is not available will be limited to a 72 hour completion time. The latter will not count against the 14 day completion time. Allowance of the extended Completion Time is contingent on meeting the Compensatory Measures described in MNS LAR submittal correspondence letter MNS-17-031.

The licensee proposed to delete the following footnote from TS 3.7.5:

* 'A' Train AFW is allowed to be inoperable for a total of 14 days to address a non-conforming condition on the 'A' Train NSWS supply piping from the Standby Nuclear Service Water Pond (SNSWP). The 14 days may be taken consecutively

or in parts until completion of the activity, or by March 31, 2019, whichever occurs first. During the period in which the 'A' Train NSWWS supply piping from the SNSWP is not available, the 'A' Train NSWWS will remain aligned to Lake Norman until the system is ready for post maintenance testing. Any maintenance that is performed on the remaining portions of 'A' Train NSWWS during the period in which the 'A' NSWWS from the SNSWP supply piping is not available will be limited to a 72 hour completion time. The latter will not count against the 14 day completion time. Allowance of the extended Completion Time is contingent on meeting the Compensatory Measures described in MNS LAR submittal correspondence letter MNS-17-031.

The licensee proposed to delete the following footnote from TS 3.7.6:

* 'A' Train CCW is allowed to be inoperable for a total of 14 days to address a non-conforming condition on the 'A' Train NSWWS supply piping from the Standby Nuclear Service Water Pond (SNSWP). The 14 days may be taken consecutively or in parts until completion of the activity, or by March 31, 2019, whichever occurs first. During the period in which the 'A' Train NSWWS supply piping from the SNSWP is not available, the 'A' Train NSWWS will remain aligned to Lake Norman until the system is ready for post maintenance testing. Any maintenance that is performed on the remaining portions of 'A' Train NSWWS during the period in which the 'A' NSWWS from the SNSWP supply piping is not available will be limited to a 72 hour completion time. The latter will not count against the 14 day completion time. Allowance of the extended Completion Time is contingent on meeting the Compensatory Measures described in MNS LAR submittal correspondence letter MNS-17-031.

The licensee proposed to delete the following footnote from TS 3.7.7:

* 'A' Train NSWWS is allowed to be inoperable for a total of 14 days to address a non-conforming condition on the 'A' Train NSWWS supply piping from the Standby Nuclear Service Water Pond (SNSWP). The 14 days may be taken consecutively or in parts until completion of the activity, or by March 31, 2019, whichever occurs first. During the period in which the 'A' Train NSWWS supply piping from the SNSWP is not available, the 'A' Train NSWWS will remain aligned to Lake Norman until the system is ready for post maintenance testing. Any maintenance that is performed on the remaining portions of 'A' Train NSWWS during the period in which the 'A' NSWWS from the SNSWP supply piping is not available will be limited to a 72 hour completion time. The latter will not count against the 14 day completion time. Allowance of the extended Completion Time is contingent on meeting the Compensatory Measures described in MNS LAR submittal correspondence letter MNS-17-031.

The licensee proposed to delete the following footnote from TS 3.7.9:

* 'A' Train CRAVS is allowed to be inoperable for a total of 14 days to address a non-conforming condition on the 'A' Train NSWWS supply piping from the Standby Nuclear Service Water Pond (SNSWP). The 14 days may be taken consecutively or in parts until completion of the activity, or by March 31, 2019, whichever occurs first. During the period in which the 'A' Train NSWWS supply piping from the SNSWP is not available, the 'A' Train NSWWS will remain aligned

to Lake Norman until the system is ready for post maintenance testing. Any maintenance that is performed on the remaining portions of 'A' Train NSW during the period in which the 'A' NSW from the SNSWP supply piping is not available will be limited to a 72 hour completion time. The latter will not count against the 14 day completion time. Allowance of the extended Completion Time is contingent on meeting the Compensatory Measures described in MNS LAR submittal correspondence letter MNS-17-031.

Note that subsequent to this proposed license amendment request, TS 3.7.9 was revised by amendment nos. 313 and 292, issued on April 18, 2019 (ADAMS Accession No. ML19050A297). TS 3.7.9, Limiting Condition for Operation, Condition G, "One or more CRAVS train(s) heater inoperable," and its associated Required Actions and Completion Times were deleted.

The licensee proposed to delete the following footnote from TS 3.7.11:

* 'A' Train ABFVES is allowed to be inoperable for a total of 14 days to address a non-conforming condition on the 'A' Train NSW supply piping from the Standby Nuclear Service Water Pond (SNSWP). The 14 days may be taken consecutively or in parts until completion of the activity, or by March 31, 2019, whichever occurs first. During the period in which the 'A' Train NSW supply piping from the SNSWP is not available, the 'A' Train NSW will remain aligned to Lake Norman until the system is ready for post maintenance testing. Any maintenance that is performed on the remaining portions of 'A' Train NSW during the period in which the 'A' NSW from the SNSWP supply piping is not available will be limited to a 72 hour completion time. The latter will not count against the 14 day completion time. Allowance of the extended Completion Time is contingent on meeting the Compensatory Measures described in MNS LAR submittal correspondence letter MNS-17-031.

The licensee proposed to delete the following footnote from TS 3.8.1:

* 'A' Train EDGs are allowed to be inoperable for a total of 14 days to address a non-conforming condition on the 'A' Train NSW supply piping from the Standby Nuclear Service Water Pond (SNSWP). The 14 days may be taken consecutively or in parts until completion of the activity, or by March 31, 2019, whichever occurs first. During the period in which the 'A' Train NSW supply piping from the SNSWP is not available, the 'A' Train NSW will remain aligned to Lake Norman until the system is ready for post maintenance testing. Any maintenance that is performed on the remaining portions of 'A' Train NSW during the period in which the 'A' NSW from the SNSWP supply piping is not available will be limited to a 72 hour completion time. The latter will not count against the 14 day completion time. Allowance of the extended Completion Time is contingent on meeting the Compensatory Measures described in MNS LAR submittal correspondence letter MNS-17-031.

Note that subsequent to this proposed license amendment request, TS 3.8.1 was revised by amendment nos. 314 and 293, issued on June 28, 2019 (ADAMS Accession No. ML19126A030). Required Action B.4 was renamed to B.6 in those license amendments, as well as other changed to TS 3.8.1.

The licensee proposed to delete the following footnote from TS 3.8.4:

* The Completion Time that one channel of DC source can be inoperable as specified by Required Action A.2.2 may be extended beyond the "72 hours" for up to 14 days as part of the battery replacement project. This allowance may be used one-time for each of the four DC channels. Upon completion of the battery replacement project, this footnote is no longer applicable and will expire on December 31, 2016.

2.2 Applicable Regulations and Guidance

In Title 10 of the *Code of Federal Regulations* (CFR) 50.36, the Commission established its regulatory requirements related to the content of TSs. Pursuant to 10 CFR 50.36, TS are required to include items in the following five specific categories related to station operation: (1) safety limits, limiting safety system settings, and limiting control settings; (2) LCOs; (3) Surveillance Requirements (SRs); (4) design features; and (5) administrative controls.

3.0 TECHNICAL EVALUATION

3.1 Change to SR 3.0.5

By letter dated March 29, 2007 (ADAMS Accession Package No. ML063100507), the NRC approved license Amendment Nos. 238 and 220, which added a new TS Limiting Condition for Operation (LCO) 3.0.8. The existing TS LCO 3.0.8 at that time was renumbered to TS LCO 3.0.9. However, the licensee inadvertently failed to provide the corresponding change to SR 3.0.5, which referenced the old TS LCO 3.0.8 (now TS LCO 3.0.9).

The proposed change to SR 3.0.5 would correct an editorial error and correct the reference LCO 3.0.8 (which should be LCO 3.0.9) in SR 3.0.5. The addition of LCO 3.0.8 was inserted as part of NRC-approved license amendments. The licensee inadvertently failed to provide the corresponding change to SR 3.0.5, and the NRC staff concludes that the proposed change corrects the editorial oversight and is, therefore, acceptable.

3.2 Deletion of TS Footnotes

By letter dated September 10, 2014 (ADAM Accession No. ML14231A634), the NRC approved license Amendment Nos. 274 and 254, which allowed temporary changes for TS 3.8.4. Specifically, the license amendments added a TS footnote which allowed a one-time extension to the CT for Required Action 2.2, to support replacement of the existing shared 125 Vdc vital batteries. The TS footnote expired on December 31, 2016. The NRC staff concludes that removal of the expired footnote is administrative in nature and is, therefore, acceptable.

By letter dated February 3, 2016 (ADAM Accession No. ML16004A352), the NRC approved license Amendment No. 281 for McGuire, Unit 1, which allowed temporary changes for TS 3.5.2. Specifically, the license amendments added a TS footnote which allowed a one-time extension to the Completion Time (CT) of Required Action A.1 to support maintenance on Residual Heat Removal (RHR) air handler unit 1A. The TS footnote associated with the one-time extension to the CT of Required Action A.1 to support maintenance on RHR air handler

unit 1A expired on March 31, 2016. The NRC staff concludes that removal of the expired footnote is acceptable.

By letter dated February 5, 2018 (ADAMS Accession No. ML18030A682), the NRC approved license Amendment Nos. 308 and 287, which allowed temporary changes for TS 3.5.2, 3.6.6, 3.7.5, 3.7.6, 3.7.7, 3.7.9, 3.7.11, and 3.8.1. Specifically, the license amendments added a TS footnote to the subject TSs which permitted the 'A' Train of NSWS to be inoperable for a total of 14 days to address a non-conforming condition on the NSWS 'A' Train supply piping from the Standby Nuclear Service Water Pond (SNSWP). The TS footnote expired on March 31, 2019. The NRC staff concludes that removal of the expired footnotes is an administrative change and is, therefore, acceptable.

The proposed changes to TSs 3.5.2, 3.6.6, 3.7.5, 3.7.6, 3.7.7, 3.7.9, 3.7.11, 3.8.1 and 3.8.4 would remove temporary TS footnotes which were inserted as part of NRC-approved license amendments. The TS footnotes have expired; therefore, the NRC staff concludes the removal of TS footnotes TSs 3.5.2, 3.6.6, 3.7.5, 3.7.6, 3.7.7, 3.7.9, 3.7.11, 3.8.1 and 3.8.4 is acceptable.

3.3 NRC Staff Conclusion

The NRC staff finds that the proposed changes to TS 3.5.2, 3.6.6, 3.7.5, 3.7.6, 3.7.7, 3.7.9, 3.7.11, 3.8.1 and 3.8.4 and SR 3.0.5 are administrative changes which are non-technical in nature.

The NRC staff concludes that with the deletion of the expired TS footnotes in TSs 3.5.2, 3.6.6, 3.7.5, 3.7.6, 3.7.7, 3.7.9, 3.7.11, 3.8.1 and 3.8.4 and the correction of the editorial error in SR 3.0.5, McGuire, Units 1 and 2, continue to meet the regulatory requirements of 10 CFR 50.36 and is, therefore, the proposed changes are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the NRC staff notified the North Carolina State official of the proposed issuance of the amendments on July 2, 2019 (ADAMS Accession No. ML19183A476). The NRC staff confirmed on July 3, 2019, that the North Carolina State official had no comments.

5.0 PUBLIC COMMENTS

On April 23, 2019 the NRC staff published a "Notice of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing," in the *Federal Register*, regarding the license amendment request (84 FR 16893). In accordance with the requirements in 10 CFR 50.91, the notice provided a 30-day period for public comment on the proposed no significant hazards consideration determination. One comment from a member of the public was received, however it was not related to the no significant hazards consideration determination nor the LAR. The comment can be found at www.regulations.gov, reference NRC-2019-0099-0002.

6.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20 and change inspections or SRs. The NRC staff has determined that the amendments involve no significant

increase in the amounts and no significant change in the types of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there was one public comment on this finding, however it was not related to the no significant hazards consideration determination nor the license amendment request (84 FR 16893; April 23, 2019). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: M. Mahoney, NRR

Date: August 8, 2019

SUBJECT: MCGUIRE NUCLEAR STATION, UNITS 1 AND 2 – ISSUANCE OF AMENDMENT NOS. 316 AND 295 TO CORRECT EDITORIAL ERROR IN TECHNICAL SPECIFICATION 3.0 AND REMOVE EXPIRED FOOTNOTES FROM TS 3.5.2, 3.6.6, 3.7.5, 3.7.6, 3.7.7, 3.7.9, 3.7.11, 3.8.1, AND 3.8.4 (EPID L-2019-LLA-0022) DATED AUGUST 8, 2019

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