



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION II  
245 PEACHTREE CENTER AVENUE N.E., SUITE 1200  
ATLANTA, GEORGIA 30303-1200

May 10, 2019

Ms. Tanya Hamilton  
Site Vice President  
Shearon Harris Nuclear Power Plant  
5413 Shearon Harris Road  
Mail Code HNP01  
New Hill, NC 27562-9300

SUBJECT: SHEARON HARRIS UNIT 1 – NUCLEAR REGULATORY COMMISSION  
INTEGRATED INSPECTION REPORT 05000400/2019001

Dear Ms. Hamilton:

On March 31, 2019, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at your Shearon Harris Unit 1. On April 10, 2019, the NRC inspectors discussed the results of this inspection with you and other members of your staff. The results of this inspection are documented in the enclosed report.

No NRC-identified or self-revealing findings were identified during this inspection.

However, inspectors documented a licensee-identified violation which was determined to be of very low safety significance in this report. The NRC is treating this violation as a non-cited violation (NCV) consistent with Section 2.3.2.a of the Enforcement Policy.

If you contest the violation or significance of the NCV, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555-0001; with copies to the Regional Administrator, Region II; the Director, Office of Enforcement; and the NRC resident inspector at the Shearon Harris Nuclear Power Plant.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with 10 CFR 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

*/RA/*

Lundy F. Pressley, Acting Chief  
Reactor Projects Branch 4  
Division of Reactor Projects

Docket No.: 05000400  
License No.: NPF-63

Enclosure:  
Inspection Report 05000400/2019001

cc: Distribution via ListServ

SUBJECT: SHEARON HARRIS UNIT 1 – NUCLEAR REGULATORY COMMISSION  
 INTEGRATED INSPECTION REPORT 05000400/2019001, May 10,2019

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**ADAMS ACCESSION NUMBER: ML 19130A153**

OFFICE	RII:DRP	RII:DRP	RII:DRP	RII:DRP	RII:DRP	RIIDRS	RIIDRP
NAME	JZeiler	APatz	JDodson	DJackson	PNiebaum	SDowney	LPressley
DATE	4/30/2019	4/30/2019	4/30/2019	4/30/2019	4/30/209	4/30/2019	5/9/2019

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**U.S. NUCLEAR REGULATORY COMMISSION**

**Inspection Report**

Docket Number: 05000400

License Number: NPF-63

Report Number: 05000400/2019001

Enterprise Identifier: I-2019-001-0025

Licensee: Duke Energy Progress, LLC

Facility: Shearon Harris, Unit 1

Location: New Hill, NC 27562

Inspection Dates: January 01, 2019 to March 31, 2019

Inspectors: J. Zeiler, Senior Resident Inspector  
A. Patz, Resident Inspector  
S. Downey, Senior Reactor Inspector (Section 71152.2)

Approved By: Lundy F. Pressley, Acting Chief  
Reactor Projects Branch 4  
Division of Reactor Projects

## **SUMMARY**

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting a quarterly inspection at Harris, Unit 1 in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information. Findings and violations being considered in the NRC's assessment are summarized in the table below. A licensee-identified non-cited violation is documented in report section 71152.

### **List of Findings and Violations**

No findings were identified.

### **Additional Tracking Items**

None.

## PLANT STATUS

Unit 1 operated at or near rated thermal power for the entire inspection period.

## INSPECTION SCOPE

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors performed plant status activities described in IMC 2515 Appendix D, "Plant Status" and conducted routine reviews using IP 71152, "Problem Identification and Resolution." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

## REACTOR SAFETY

### 71111.04 - Equipment Alignment

#### Partial Walkdown (IP Section 02.01) (3 Samples)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

- (1) 'A' boric acid transfer system during 'B' boric acid transfer pump unavailability on January 3, 2019
- (2) 'B' emergency service water (ESW) pump and 'B' ESW booster pump while 'A' ESW pump and 'A' ESW booster pump was out-of-service for preventive maintenance on January 15, 2019
- (3) 'A' containment spray (CS) pump while 'B' CS pump was out-of-service for preventive maintenance on February 5, 2019

### 71111.05Q - Fire Protection

#### Quarterly Inspection (IP Section 03.01) (5 Samples)

The inspectors evaluated fire protection program implementation in the following selected areas:

- (1) ESW intake structure and yard area (fire zones 12-I-ESWPA, 12-I-ESWPA-BAL, 12-I-ESWPB, 12-I-ESWPB-BAL, and 12-I-ESW-BAL) on January 15, 2019
- (2) Reactor auxiliary building (RAB) 190' elevation 'A' residual heat removal (RHR) and CS pump room (fire zone 1-A-1-PA) on February 5, 2019
- (3) RAB 216' elevation mechanical penetration room, south corridor, and ESW pipe tunnel (fire zones 1-A-2-MP, 1-A-2-COR, and 1-A-2-PT) on February 6, 2019

- (4) RAB 286' elevation 'B' electrical switchgear and ventilation rooms (fire zones 1-A-SWGRB and 1-A-5-HVB) on February 13, 2019
- (5) RAB 236' elevation chemical and volume control system (CVCS) area (fire zone 1-A-3-COMB) on February 15, 2019

#### 71111.06 - Flood Protection Measures

##### Inspection Activities - Internal Flooding (IP Section 02.02a.) (1 Sample)

The inspectors evaluated internal flooding mitigation protections in the RAB 216' elevation on February 14, 2019.

#### 71111.11Q - Licensed Operator Requalification Program and Licensed Operator Performance

##### Licensed Operator Performance in the Actual Plant/Main Control Room (IP Section 03.01) (1 Sample)

The inspectors observed and evaluated licensed operator performance in the control room during 'A' train reactor trip breaker and solid state protection system testing on March 18, 2019.

##### Licensed Operator Requalification Training/Examinations (IP Section 03.02) (1 Sample)

The inspectors observed and evaluated a simulator scenario for operator requalification training involving a small-break loss-of-coolant accident on February 19, 2019.

#### 71111.12 - Maintenance Effectiveness

##### Routine Maintenance Effectiveness Inspection (IP Section 02.01) (2 Samples)

The inspectors evaluated the effectiveness of routine maintenance activities associated with the following equipment and/or safety significant functions:

- (1) Containment ventilation isolation radiation monitor 3561C failed to alarm when tested on November 7, 2018
- (2) Loose fan belts on 'A' RHR/CS pump room air handler system resulting in fan speed below minimum requirements on December 31, 2018

#### 71111.13 - Maintenance Risk Assessments and Emergent Work Control

##### Risk Assessment and Management Sample (IP Section 03.01) (5 Samples)

The inspectors evaluated the risk assessments for the following planned and emergent work activities:

- (1) Elevated (Green) risk during planned unavailability of the 'A' ESW pump and 'A' ESW booster pump for preventive maintenance on January 15, 2019

- (2) Elevated (Green) risk during planned unavailability of the 'B' CS pump for seal water piping inspections on February 5, 2019
- (3) Elevated (Green) risk during emergent repair of power range nuclear instrument channel NI-41 following inoperability due to anomalous upper detector high current on February 11, 2019
- (4) Elevated (Green) risk during planned unavailability of the 'B' emergency safeguards sequencer for relay calibration and replacement on February 13, 2019
- (5) Elevated (Green) risk during planned unavailability of the 'A' motor driven auxiliary feedwater (AFW) pump for preventive maintenance on February 19, 2019

#### 71111.15 - Operability Determinations and Functionality Assessments

##### Operability Determinations and Functionality Assessments Sample (IP Section 02.01) (4 Samples)

The inspectors evaluated the following operability determinations and functionality assessments:

- (1) Leakage of diaphragm valves in the emergency core cooling system (ECCS) recirculation flow path on January 15, 2019 (nuclear condition reports (NCRs) 02252508 and 02252515)
- (2) Increased levels of particulate found in oil of 'A' component cooling water (CCW) pump inboard bearing on January 31, 2019 (NCR 02255195)
- (3) Gas voids found in piping upstream of 'B' CS pump discharge containment isolation valve 1CT-88 on February 20, 2019 (NCR 02258496)
- (4) 'A' train emergency diesel generator auxiliary lube oil pump tripped after starting on March 21, 2019 (NCR 02264131)

#### 71111.18 - Plant Modifications

##### Temporary Modifications and/or Permanent Modifications (IP Section 03.01 and/or 03.02) (1 Sample)

The inspectors evaluated the following temporary or permanent modification:

- (1) Installation of wireless electronic equipment in the nuclear plant power block including control room

#### 71111.19 - Post Maintenance Testing

##### Post Maintenance Test Sample (IP Section 03.01) (6 Samples)

The inspectors evaluated the following post maintenance tests:

- (1) Operations surveillance test (OST)-1214, ESW System Operability Train A Quarterly Interval Modes 1-2-3-4-5-Defueled, following scheduled preventive maintenance on the 'A' ESW pump and 'A' ESW booster pump on January 15, 2019



- (2) OST-1122, Train A 6.9 kV Emergency Bus Undervoltage Trip Actuating Device Operational Test and Contact Check, following corrective replacement of the turbine driven AFW pump governor positioner on January 19, 2019
- (3) Technical Procedure Testing (OPT)-1512, Train B Essential Chilled Water Turbopak Units Quarterly Inspection and Checks, following preventive maintenance on the 'B' essential services chilled water (ESCW) chiller on January 29, 2019
- (4) OST-1119, Containment Spray Operability Train B Quarterly Interval Modes 1-4, following scheduled preventive maintenance on the 'B' CS pump on February 5, 2019
- (5) OST-1211, Auxiliary Feedwater Pump 1A-SA Operability Test Quarterly Interval Modes 1-4, and OST-1077, Auxiliary Feedwater Valves Operability Test Quarterly Interval Mode 4-5-6, following scheduled preventive maintenance on the 'A' motor driven AFW pump motor and discharge pressure control valve on February 19, 2019
- (6) Operating Procedure (OP)-145, Component Cooling Water, for operating the 'B' CCW pump following pump inrush current testing on March 4, 2019

#### 71111.22 - Surveillance Testing

The inspectors evaluated the following surveillance tests:

##### FLEX Testing (IP Section 03.02) (1 Sample)

Work orders (WO) 20281145-02 and 20281145-03, Perform quarterly functional tests of FLEX diesel generators 1FLX-E042 and 1FLX-E043, on March 6, 2019

##### In Service Testing (IST) (IP Section 03.01) (1 Sample)

OST-1093, CVCS/Safety Injection System Operability Train B Quarterly Interval Modes 1-4, on January 3, 2019

##### Reactor Coolant System (RCS) Leak Detection (IP Section 03.01) (1 Sample)

OST-1026, Reactor Coolant System Leakage Evaluation, Computer Calculation, Daily Interval, Modes 1-4, on March 22, 2019

##### Surveillance Testing (IP Section 03.01) (2 Samples)

- (1) OST-1076, Auxiliary Feedwater Pump 1B-SB Operability Test Quarterly Interval Modes 1-4, on January 7, 2019
- (2) OPT-1538, Emergency Safeguards Sequencer System Test - Train B Quarterly Interval Modes 1-6, on February 13, 2019

#### **OTHER ACTIVITIES – BASELINE**

##### 71151 - Performance Indicator Verification

The inspectors verified licensee performance indicators submittals listed below:

IE01: Unplanned Scrams per 7000 Critical Hours Sample (IP Section 02.01) (1 Sample)

Unit 1 (January 1, 2018 - December 31, 2018)

IE03: Unplanned Power Changes per 7000 Critical Hours Sample (IP Section 02.02) (1 Sample)

Unit 1 (January 1, 2018 – December 31, 2018)

IE04: Unplanned Scrams with Complications (USwC) Sample (IP Section 02.03) (1 Sample)

Unit 1 (January 1, 2018 – December 31, 2018)

71152 - Problem Identification and Resolution

Annual Follow-up of Selected Issues (IP Section 02.03) (2 Samples)

The inspectors reviewed the licensee's implementation of its corrective action program related to the following issues:

- (1) TS 3.0.3 entered for both trains of containment vacuum relief rendered inoperable during maintenance on containment ventilation isolation radiation monitors (NCR 02244746)
- (2) Operating experience smart sample (OpESS) 2018/01, 10 CFR Part 21 notification of the potential existence of defects related to control rod drive mechanism (CRDM) thermal sleeves (NCRs 02220429, 02208989)

## INSPECTION RESULTS

Licensee-Identified Non-Cited Violation	71152
<p>This violation of very low safety significance was identified by the licensee and has been entered into the licensee corrective action program and is being treated as a Non-Cited Violation, consistent with Section 2.3.2 of the Enforcement Policy.</p>	
<p>Violation: Technical Specification (TS) 3.6.5, "Vacuum Relief System," requires two containment vacuum relief trains shall be operable in Modes 1, 2, 3, and 4. The associated action statement for TS 3.6.5 with one inoperable containment vacuum relief train requires the inoperable train to be restored to operable status within 72 hours or be in at least hot standby within the next 6 hours and in cold shutdown within the following 30 hours. Since there is no TS action statement addressing inoperability of both containment vacuum relief trains, the limiting condition for operation associated with TS 3.0.3 applies. TS 3.0.3 requires, within one hour, action shall be initiated to place the unit in a mode in which (TS 3.6.5) does not apply and be in hot standby within the next 6 hours, hot shutdown within the following 6 hours, and cold shutdown within the subsequent 24 hours.</p> <p>Contrary to the above, on November 20, 2018, while in Mode 1, the licensee failed to follow the action requirements of TS 3.0.3, in that, within one hour of entering TS 3.0.3, after both containment vacuum relief trains were rendered inoperable, actions were not initiated to place the unit in a mode in which TS 3.6.5 did not apply. TS 3.0.3 was exited 3 hours and 18 minutes from the time both containment vacuum relief valves were initially rendered inoperable. Both containment vacuum relief trains were rendered inoperable during planned maintenance to replace the electrical breakers to containment ventilation isolation radiation monitors 3561A and 3561C. When both radiation monitors were de-energized for this planned work, a containment ventilation isolation signal (CVIS) was generated as designed, which prevents opening containment ventilation systems, including both 'A' and 'B' train containment vacuum relief valves. While reviewing procedural guidance for returning the radiation monitors to service and resetting CVIS after the work was completed, the operators realized that the containment vacuum relief system had been impacted and TS 3.0.3 had been applicable.</p> <p>Significance: Green. The inspectors assessed the significance of the finding using IMC 0609, Appendix A, "Significance Determination Process for Findings At-Power," dated June 19, 2012. The finding was screened using Exhibit 3, "Barrier Integrity Screening Questions," since it was associated with a degraded containment pressure control system. The finding was determined to be of very low safety significance (Green) because it did not represent an actual open pathway in the physical integrity of containment or partial failure of containment isolation systems or containment heat removal components; and Harris does not have hydrogen igniters.</p> <p>Corrective Action Reference: NCR 02244746</p>	

## EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On April 10, 2019, the inspectors presented the quarterly resident inspector inspection results to Ms. Tanya Hamilton, Site Vice President, and other members of the licensee staff.

**DOCUMENTS REVIEWED**

Inspection Procedure	Type	Designation	Description or Title	Revision or Date	
71111.04	Procedures	OP-107	Chemical and Volume Control	Revision 117	
		OP-112	Containment Spray System	Revision 46	
		OP-139	Service Water System	Revision 136	
71111.05Q	Fire Plans	CSD-HNP-PFP-RAB-190-216	Reactor Auxiliary Building Elevations 190 and 216 Pre-Fire Plan	Revision 1	
		CSD-HNP-PFP-RAB-236	Reactor Auxiliary Building Elevation 236 Pre-Fire Plan	Revision 2	
		CSD-HNP-PFP-SEC	Out Building Pre-Fire Plan	Revision 4	
		CSD-HNP-RAB-286	Reactor Auxiliary Building Elevation 286 Pre-Fire Plan	Revision 1	
	Miscellaneous	Fire Protection Permit and Impairment Log Entries			
	Procedures	AD-EG-ALL-1520	Transient Combustible Control	Revision 11	
		FPP-001	Fire Protection Program Manual	Revision 44	
		FPP-013	Fire Protection - Minimum Requirements, Mitigation Actions and Surveillance Requirements	Revision 99	
	71111.06	Calculations	PRA-F-E-004	Reactor Auxiliary Building Unit 1 Elevation 190' & 216' Internal Flooding Analysis	Revision 8
	71111.11Q	Procedures	AD-OP-All-0106	Conduct of Infrequently Performed Test or Evolution	Revision 4
AD-OP-ALL-1000			Conduct of Operations	Revision 15	
AD-TQ-ALL-0420			Conduct of Simulator Training and Evaluation	Revision 14	
AOP-025			Loss of One Emergency AC Bus (6.9kV) or One Emergency DC Bus (125V)	Revision 42	
AP-002			Plant Conduct of Operations	Revision 73	
EOP-E-0			Reactor Trip or Safety Injection	Revision 12	
MST-I0001			Train A Solid State Protection System Actuation Logic & Master Relay Test	Revision 38	
OMM-001			Operations Administrative Requirements	Revision 114	
OMM-014	Operation of the Work Coordination Center	Revision 73			

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		OP-104	Rod Control System	Revision 43
	Work Orders	WO 20281152	Conduct SSPS Testing on 'A' Train	
71111.12	Calculations	HNP-F/PSA-0020	HNP PRA Evaluation of Risk Significant SSCs for Maintenance Rule	Revision 1
		HNP-F/PSA-0065	HNP PRA - System Notebooks	Revision 8
	Corrective Action Documents	NCR 02241966	Containment ventilation isolation radiation monitor 3561C failed to alarm when tested	
		NCR 02250497	Loose fan belts on 'A' RHR/CS pump room air handler system	
	Miscellaneous	NUMARC 93-01	Industry Guideline for Monitoring the Effectiveness of Maintenance at Nuclear Power Plants	Revision 4A
	Procedures	AD-EG-ALL-1210	Maintenance Rule Program	Revision 1
		EOP-E-1	Reactor Trip Response	Revision 5
		OWP-RM	Radiation, Effluent, and Explosive Gas Monitoring	Revision 45
	Work Orders	WO 20287202	Perform MST-I0347 to determine cause of electronic spikes on RM-01CR-3561ASA	
		WO 20303302	Repair loose belts on 'A' RHR/CS pump room air handler	
71111.13	Procedures	AD-NF-ALL-0501	Electronic Risk Assessment Tool (ERAT)	Revision 3
		AD-OP-ALL-0201	Protected Equipment	Revision 5
		AD-OP-ALL-1000	Conduct of Operations	Revision 15
		AD-WC-ALL-0200	On-Line Work Management	Revision 13
		AD-WC-ALL-0240	On-Line Risk Management Process	Revision 1
		AD-WC-ALL-0410	Work Activity Integrated Risk Management	Revision 7
		OMM-001	Conduct of Operations	Revision 114
	Work Orders	WO 10622005	Lubrication of 'A' ESW booster pump coupling	
		WO 12296955	Tighten seal water piping connection on 'A' containment spray pump	
		WO 20193828	AFW pump motor 1A-SA testing	
		WO 20237424	Preventive maintenance on 'A' ESW stuffing box and seal water cooling system	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		WO 20266256	Replace actuator oil in 'A' AFW discharge control valve	
		WO 20311779	Repair emergent issue with N-41 upper detector reading high	
		WOs 20181791, 20266115, and 20266116	'B' train sequencer relay calibrations and replacement	
71111.15	Corrective Action Documents	NCR 01959624	Copper detected in CCW pump bearing oil sample	
		NCR 02203899	Oil sample particulate results for the 'A' CCW pump	
	Procedures	AD-OP-ALL-0105	Operability Determinations and Functionality Assessments	Revision 4
		AD-OP-ALL-0202	Aggregate Operator Impact Assessment	Revision 2
		DBD-103	Chemical and Volume Control System	Revision 24
		DBD-106	Containment Spray System	Revision 17
		DBD-131	Component Cooling Water System	Revision 17
		DBD-201	Emergency Diesel Generator System	Revision 18
		OST-1013	1A-SA Emergency Diesel Generator Operability Test Monthly Interval Modes 1-6	Revision 43
		OST-1107	ECCS/Containment Spray Flow Path and Piping Filled Verification Monthly Interval Modes 1-6	Revision 36
		PM-M0102	3/8 - 6 Inch Diaphragm Valve Nonmetallic Component Replacement	Revision 6
Work Orders	WO 20179904	Evaluate gas voids		
71111.18	Engineering Changes	EC 413085	Non-Controlled Plant Equipment Wireless Emitting Device Installation	Revision 1
		EC 414212	Electromagnetic Interference Proximity for Transceivers	Revision 3
	Miscellaneous	NEI 96-07	Guidelines for 10 CFR 50.59 Evaluations Endorsed by Regulatory Guide 1.187	Revision 1
		Regulatory Guide 1.187	Guidance for Implementation of 10 CFR 50.59, dated November 2000	
	Procedures	AD-EG-ALL-1110	Design Review Requirements	Revision 6
		AD-EG-ALL-1130	Activation of Engineering Changes	Revision 2
		AD-EG-ALL-1132	Preparation and Control of Design Change Engineering Changes	Revision 13
		AD-EG-ALL-1137	Engineering Change Product Selection	Revision 5

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		AD-LS-ALL-0008	10 CFR 50.59 Review Process	Revision 0
	Work Orders	WO 20288719	Install wireless emitting device and hardware for main control room	
71111.19	Procedures	PLP-400	Post Maintenance Testing	Revision 64
	Work Orders	WO 10622005	Lubrication of 'A' ESW booster pump coupling	
		WO 12296955	Tighten seal water piping connection on 'A' containment spray pump	
		WO 20191986	Obtain one set of inrush current data	
		WO 20193828	AFW pump motor 1A-SA testing	
		WO 20237424	Preventive maintenance on 'A' ESW stuffing box and seal water cooling system	
		WO 20266256	Replace actuator oil in 'A' AFW discharge control valve	
		WO 20285410	VMS-2, Determine available adjustment for future modification	
WO 20307647	TDAFW pump tripped while starting during OST-1044			
71111.22	Procedures	CSD-EG-HNP-8888	Flexible Response to Extended Loss of ALL AC Power	Revision 0
		ISI-800	Inservice Testing of Pumps	Revision 30
		OST-1093	Completed procedures on January 14, 2015, and January 12, 2017	
		PD-OP-ALL-0500	Diverse and Flexile Coping Strategies FLEX Program	Revision 1
71151	Miscellaneous	2018 Licensee Event Reports		
		2018 Operator Log Entries		
		NEI 99-02	Regulatory Assessment Performance Indicator Guideline	
	Procedures	AD-LS-ALL-0004	NRC Performance Indicators and Monthly Operating Report	Revision 3
71152	Engineering Changes	412571	HNP RFO-21 Thermal Sleeve Flange Upper Flange Wear Inspection Results	Revision 0
	Procedures	AD-LS-ALL-0006	Notification/Reportability Evaluation	Revision 1
		AD-PI-ALL-0100	Corrective Action Program	Revision 20
		AD-PI-ALL-0102	Apparent Cause Evaluation	Revision 4

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		AD-PI-ALL-0104	Prompt Investigation Response Team	Revision 4
		AD-PI-ALL-0106	Cause Investigation Checklists	Revisions 2 and 3
		AD-PI-ALL-0108	Performance Analysis	Revision 0
		DBD-136	Containment Ventilation & Cooling Systems	Revision 12
		OWP-RM	Radiation, Effluent, and Explosive Gas Monitoring	Revisions 44 and 45
	Work Orders	WO 20191928	Replace breaker to radiation monitor RM-3561A	
		WO 20191929	Replace breaker to radiation monitor RM-3561C	