



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

November 26, 2018

Mr. Bryan C. Hanson
Senior Vice President
Exelon Generation Company, LLC
President and Chief Nuclear Officer
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT – CORRECTION TO AMENDMENT NO. 321 REVISING TECHNICAL SPECIFICATIONS TO ADOPT TSTF-542, REVISION 2, “REACTOR PRESSURE VESSEL WATER INVENTORY CONTROL” (EPID L-2017-PMP-0010)

Dear Mr. Hanson:

By letter dated August 24, 2018, the U.S. Nuclear Regulatory Commission (NRC) issued Amendment No. 321 to Renewed Facility Operating License No. DPR-59 for the James A. FitzPatrick Nuclear Power Plant (FitzPatrick). The amendment was in response to your application dated October 2, 2017,¹ as supplemented by letters dated January 22, 2018, and April 19, 2018.²

The amendment revised existing FitzPatrick Technical Specification (TS) requirements related to “operations with a potential for draining the reactor vessel,” with new requirements on reactor pressure vessel water inventory control to protect the FitzPatrick TS Safety Limit 2.1.1.3, which stated, “Reactor vessel water level shall be greater than the top of active irradiated fuel.”

Subsequent to issuance of the amendment, the NRC staff was notified by Exelon Generation Company, LLC (the licensee) of a typographical error that was introduced in its submission of revised TS pages 3.3.5.1-10 and 3.3.5.1-12 with the amendment request. The licensee’s submittal stated the following for note (c) at the bottom of each of these two pages:

c. With reactor steam dose pressure > 150 psig.

¹ Agencywide Documents Access and Management System (ADAMS) Accession No. ML17275A520.

² ADAMS Accession Nos. ML18022A829 and ML18109A371, respectively.

The word "dose" should have been "dome" in the licensee's revised TS pages 3.3.5.1-10 and 3.3.5.1-12 in its submittal. This error was inadvertently carried over by NRC staff to Amendment No. 321. The NRC staff has determined that this typographical error was made inadvertently and is entirely editorial in nature. This correction does not change any of the conclusions in the safety evaluation associated with the issuance of Amendment No. 321 and does not affect the associated notice to the public. Enclosed are the two replacement TS pages.

If you have any questions, please contact me at 301-415-2934 or Booma.Venkataraman@nrc.gov.

Sincerely,

A handwritten signature in black ink, appearing to read "V. Boome", with a horizontal line underneath.

Booma Venkataraman, Project Manager
Plant Licensing Branch I
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-333

Enclosure:
Corrected pages 3.3.5.1-10 and 3.3.5.1-12

cc: Listserv

Enclosure

Corrected pages 3.3.5.1-10 and 3.3.5.1-12

Table 3.3.5.1-1 (page 3 of 5)
Emergency Core Cooling System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER FUNCTION	CONDITIONS REFERENCED FROM REQUIRED ACTION A.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
2. LPCI System (continued)					
g. Low Pressure Coolant Injection Pump Discharge Flow – Low (Bypass)	1, 2, 3	1 per subsystem	E	SR 3.3.5.1.5 SR 3.3.5.1.6	≥ 1040 gpm and ≤ 1665 gpm
h. Containment Pressure - High	1, 2, 3	4	B	SR 3.3.5.1.3 SR 3.3.5.1.6	≥ 1 psig and ≤ 2.7 psig
3. High Pressure Coolant Injection (HPCI) System					
a. Reactor Vessel Water Level – Low Low (Level 2)	1, 2 ^(c) , 3 ^(c)	4	B	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.4 SR 3.3.5.1.5 SR 3.3.5.1.6	≥ 126.5 inches
b. Drywell Pressure - High	1, 2 ^(c) , 3 ^(c)	4	B	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.4 SR 3.3.5.1.5 SR 3.3.5.1.6	≤ 2.7 psig
c. Reactor Vessel Water Level – High (Level 8)	1, 2 ^(c) , 3 ^(c)	2	C	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.4 SR 3.3.5.1.5 SR 3.3.5.1.6	≤ 222.5 inches
d. Condensate Storage Tank Level - Low	1, 2 ^(c) , 3 ^(c)	4	D	SR 3.3.5.1.3 SR 3.3.5.1.6	≥ 59.5 inches
e. Suppression Pool Water Level – High	1, 2 ^(c) , 3 ^(c)	2	D	SR 3.3.5.1.3 SR 3.3.5.1.6	≤ 14.5 ft
f. High Pressure Coolant Injection Pump Discharge Flow - Low (Bypass)	1, 2 ^(c) , 3 ^(c)	1	E	SR 3.3.5.1.5 SR 3.3.5.1.6	≥ 475 gpm and ≤ 800 gpm
g. High Pressure Coolant Injection Pump Discharge Pressure – High (Bypass)	1, 2 ^(c) , 3 ^(c)	1	E	SR 3.3.5.1.3 SR 3.3.5.1.6	≥ 25 psig and ≤ 80 psig

(continued)

(c) With reactor steam dome pressure > 150 psig.

Table 3.3.5.1-1 (page 5 of 5)
Emergency Core Cooling System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER FUNCTION	CONDITIONS REFERENCED FROM REQUIRED ACTION A.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
5. ADS Trip System B (continued)					
c. Reactor Vessel Water Level - Low (Level 3)	1, 2 ^(c) , 3 ^(c)	1	F	SR 3.3.5.1.1 SR 3.3.5.1.2 SR 3.3.5.1.4 SR 3.3.5.1.5 SR 3.3.5.1.6	≥ 177 inches
d. Core Spray Pump Discharge Pressure - High	1, 2 ^(c) , 3 ^(c)	2	G	SR 3.3.5.1.3 SR 3.3.5.1.6	≥ 90 psig and ≤ 110 psig
e. Low Pressure Coolant Injection Pump Discharge Pressure - High	1, 2 ^(c) , 3 ^(c)	4	G	SR 3.3.5.1.3 SR 3.3.5.1.6	≥ 105 psig and ≤ 145 psig

(c) With reactor steam dome pressure > 150 psig.

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OFFICE	NRR/DORL/LPL1/PM	NRR/DORL/LPL1/LA	NRR/DORL/LPL1/BC	NRR/DORL/LPL1/PM
NAME	BVenkataraman	LRonewicz	JDanna	BVenkataraman
DATE	11/23/2018	11/23/2018	11/26/2018	11/26/2018

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