



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
245 PEACHTREE CENTER AVENUE NE, SUITE 1200
ATLANTA, GEORGIA 30303-1257

November 9, 2018

Dennis R. Madison
Vice President
Southern Nuclear Operating
Company, Inc.
Joseph M. Farley Nuclear Plant
7388 North State Highway 95
Columbia, AL 36319

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT – NRC INTEGRATED INSPECTION
REPORT 05000348/2018003 AND 05000364/2018003

Dear Mr. Madison:

On September 30, 2018, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at your Joseph M. Farley Nuclear Plant, Units 1 and 2. On October 16, 2018, NRC inspectors discussed the results of this inspection with you and other members of your staff. The results of this inspection are documented in the enclosed report.

NRC inspectors documented one Licensee Identified Violation and one Severity Level (SL) IV violation with no associated finding. The NRC is treating the SL IV violation as a non-cited violation (NCV) consistent with Section 2.3.2.a of the Enforcement Policy. If you contest the violations or significance of this NCV, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555-0001; with copies to the Regional Administrator, Region II; the Director, Office of Enforcement; and the NRC resident inspector at the Joseph M. Farley Nuclear Plant, Units 1 and 2.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with 10 CFR 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

/RA/

Alan Blamey, Branch Chief
Reactor Projects Branch 2
Division of Reactor Projects

Docket Nos.: 50-348, 50-364
License Nos.: NPF-2, NPF-8

Enclosure:
IR 05000348/2018003, 05000364/2018003

cc Distribution via ListServ

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT – NRC INTEGRATED INSPECTION
REPORT 05000348/2018003 AND 05000364/2018003

DISTRIBUTION:

S. Price, RII
M. Kowal, RII
K. Sloan, RII
OE Mail
RIDSNNRRDIRS
PUBLIC
RidsNrrPMFarleyResource

ADAMS ACCESSION NUMBER: [Accession Number](#)

OFFICE	RII/ DRP	RII/ DRP	RII/ DRP	RII/ DRP	RII/ DRP	RII/ DRP	
NAME	K. Miller	A. Blamey	P. Niebaum	N. Staples	D. Mas	M. Schwieg	
DATE	10/31/2018	11/9/2018	10/31/2018	10/31/2018	11/1/2018	10/31/2018	

**U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report**

Docket Number(s): 50-348, 50-364

License Number(s): NPF-2, NPF-8

Report Number(s): 05000348/2018003; and 05000364/2018003

Enterprise Identifier: I-2018-003-0038

Licensee: Southern Nuclear Operating Company, Inc.

Facility: Joseph M. Farley Nuclear Plant

Location: Columbia, Alabama

Inspection Dates: July 1, 2018 to September 30, 2018

Inspectors: P. Niebaum, Senior Resident Inspector
M. Schweg, Senior Resident Inspector
K. Miller, Resident Inspector
D. Mas-Peñaranda, Project Engineer

Approved By: A. Blamey, Chief
Reactor Projects Branch 2
Division of Reactor Projects

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring licensee’s performance by conducting baseline inspections at Joseph M. Farley, Units 1 and 2 in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC’s program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information. NRC and self-revealed findings, violations, and additional items are summarized in the table below

List of Findings and Violations

Unit 1 Pressurizer Safety Valve Lift Pressure Outside of Technical Specification Tolerance Band			
Cornerstone	Severity	Crosscutting	Report Section
Not Applicable	Severity Level (SL) IV NCV 05000348/2018003-01 Closed	Not Applicable	71153 – Follow-up of Events and Notices of Enforcement Discretion
A self-revealed SL IV NCV of TS 3.4.10, “Pressurizer Safety Valves,” was identified when a routine lift pressure test revealed that pressurizer safety valve Q1B13V0031C was lower than allowed by TS SR 3.4.10.1 for a duration that was longer than the condition’s TS required action completion time.			

Additional Tracking Items

Type	Issue number	Title	Report Section	Status
LER	05000348/2018-001-00	Pressurizer Safety Valve Lift Pressure Outside of Technical Specification Limits due to Spring Relaxation	71153 – Follow-up of Events and Notices of Enforcement Discretion	Closed

PLANT STATUS

Unit 1 began the report period at or near 100 percent rated thermal power (RTP). On September 8, Unit 1 was reduced to approximately 16 percent RTP to support a containment entry for an oil addition to the 1C reactor coolant pump motor upper bearing oil reservoir. Following this work, Unit 1 achieved 100 percent RTP on September 9. Unit 1 stayed at or near 100 percent RTP until September 24, when the reactor was shut down to support a planned maintenance outage to rework a main steam isolation valve (MSIV) actuator. The reactor was started on September 28 and RTP was increased to approximately 72 percent by the end of the report period.

Unit 2 began the report period at approximately 100 percent RTP and maintained that power level for the entire report period.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards."

REACTOR SAFETY

71111.01 - Adverse Weather Protection

External Flooding (1 Sample)

The inspectors verified that flood protection barriers, mitigation plans, procedures, and equipment were consistent with the licensee's design requirements and risk analysis assumptions for coping with external flooding for the Service Water Intake Structure.

71111.04 - Equipment Alignment

Partial Walkdown (3 Samples)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

- (1) Unit 2 containment spray system, 'A' train, on August 13, 2018.
- (2) Unit 2 'A' motor driven auxiliary feedwater pump, on August 14, 2018.
- (3) Unit 2 residual heat removal system, 'A' train, on August 28, 2018.

Complete Walkdown (1 Sample)

The inspectors evaluated system configurations during a complete walkdown of the Unit 2 auxiliary feedwater system on August 30, 2018.

71111.05AQ - Fire Protection Annual/Quarterly

Quarterly Inspection (5 Samples)

The inspectors evaluated fire protection program implementation in the following selected areas on August 8, 2018:

- (1) Fire Area 072, Service Water Pump Room.
- (2) Fire Area 073, Service Water Intake Structure Battery Room Train B.
- (3) Fire Area 074, Service Water Intake Structure Battery Room Train A.
- (4) Fire Area 075, Service Water Intake Structure 5kV Switchgear Room B & West Stairs.
- (5) Fire Area 076, Service Water Intake Structure 5kV Switchgear Room A & East Stairs.

Annual Inspection (1 Sample)

The inspectors evaluated fire brigade performance on August 16, 2018.

71111.11 - Licensed Operator Requalification Program and Licensed Operator Performance

Operator Requalification (1 Sample)

The inspectors observed and evaluated a crew of licensed operators in the plant's simulator during the licensed operator annual simulator exam on August 1, 2018.

Operator Performance (1 Sample)

The inspectors observed and evaluated the power ascension of Unit 1 following an oil addition to the 1C reactor coolant pump on September 8, 2018.

71111.12 - Maintenance Effectiveness

Routine Maintenance Effectiveness (2 Samples)

The inspectors evaluated the effectiveness of routine maintenance activities associated with the following equipment and/or safety significant functions:

- (1) The maintenance rule program periodic (a)(3) evaluation on Aug. 29, 2018.
- (2) Unit 1 and 2 shared '1-2A' diesel generator degraded mode selector switch on Sept. 6, 2018.

71111.13 - Maintenance Risk Assessments and Emergent Work Control (4 Samples)

The inspectors evaluated the risk assessments for the following planned and emergent work activities:

- (1) Unit 2 elevated risk due to '2B' emergency diesel generator corrective maintenance on August 10, 2018.
- (2) Unit 2 elevated risk due to '2B' containment spray pump preventive maintenance on August 13, 2018.
- (3) Units 1 and 2 elevated risk due to modification work in the high voltage switchyard on September 5, 2018.
- (4) Unit 1 MSIV 'A' emergent work on September 26, 2018.

71111.15 - Operability Determinations and Functionality Assessments (5 Samples)

The inspectors evaluated the following operability determinations and functionality assessments:

- (1) The '2B' residual heat removal (RHR) pump radial bearing reservoir leaked oil during performance of the FNP-2-STP-11.2, "2B RHR Pump Quarterly Inservice Test" on July 16, 2018.
- (2) Unit 2 pressurizer pressure transmitter PT-457 drifted high on July 1, 2018.
- (3) Mounting bolt broken during replacement of mechanism operated cell switch on circuit breaker Q2R15BKRDH07 on July 24, 2018.
- (4) Unit 1, reactor coolant pump under voltage relay test review on July 28, 2018.
- (5) Unit 1, impacts of potential oil leak on 1C reactor coolant pump motor upper bearing oil reservoir on July 6, 2018.

71111.19 - Post Maintenance Testing (4 Samples)

The inspectors evaluated the following post maintenance tests:

- (1) FNP-0-STP-80.2, "Diesel Generator 1C Operability Test," after replacement of MOC switch on circuit breaker Q2R15BKRDH07 on July 24, 2018.
- (2) FNP-1-STP-80.20, "Diesel Generator 1-2A 1000 KW Load Rejection Test," after replacement of an automatic voltage regulator control card on August 22, 2018.
- (3) FNP-2-STP-201.6, "Pressurizer Pressure Q2B31PT0457 Loop Calibration and Operational Test," after replacement of pressurizer pressure transmitter on September 11, 2018.
- (4) FNP-1-STP-21.2, "MSIV Air System Leak Test," after replacement of air actuator on September 26, 2018.

71111.20 - Refueling and Other Outage Activities (1 Sample)

The inspectors evaluated Unit 1 forced outage activities from September 24, 2018 to September 28, 2018.

71111.22 - Surveillance Testing

The inspectors evaluated the following surveillance tests:

Routine (1 Sample)

- (1) Unit 1, FNP-1-STP-109.1, Power Range Neutron Flux Channel Calibration Using The Plant Computer performed on July 16, 2018.

In-Service Testing (1 Sample)

(1) Unit 1, FNP-1-STP-22.1, Auxiliary Feedwater Pump 1A Inservice Test, on July 24, 2018.

RCS Leakage Detection (1 Sample)

(1) FNP-1-STP-9.0, RCS Leakage Test, on October 24, 2018.

71114.06 - Drill Evaluation

Emergency Planning Drill (1 Sample)

The inspectors evaluated an emergency preparedness drill that involved the failure of the reactor to trip which led to an Alert Emergency, the loss of two fission product barriers which lead to a Site Area Emergency and the subsequent loss of the third barrier which led to a General Emergency on August 14, 2018.

OTHER ACTIVITIES – BASELINE

71151 - Performance Indicator Verification

The inspectors verified licensee performance indicators submittals listed below for the period from July 2017, through June 2018. (6 Samples)

- (1) Safety System Functional Failure (SSFF), Units 1 and 2.
- (2) RCS specific activity, Units 1 and 2.
- (3) RCS leakage, Units 1 and 2.

71152 - Problem Identification and Resolution

Semiannual Trend Review (1 Sample)

The inspectors reviewed the licensee's corrective action program for trends that might be indicative of a more significant safety issue.

- (1) Condition Report 10529513, Trend in Missed Fire Watches.

Annual Follow-up of Selected Issues (1 Sample)

The inspectors reviewed the licensee's implementation of its corrective action program related to the following issues:

- (1) Condition Report 10527200, Failure of Emergency Diesel Generator 1-2A to Run on August 20, 2018.

71153 - Follow-up of Events and Notices of Enforcement Discretion

Licensee Event Reports (1 Sample)

The inspectors evaluated the following licensee event reports which can be accessed at <https://lersearch.inl.gov/LERSearchCriteria.aspx>:

- (1) Licensee Event Report (LER) 05000348/2018-001-00, Pressurizer Safety Valve Lift Pressure Outside of Technical Specification Limits due to Spring Relaxation

OTHER ACTIVITIES – TEMPORARY INSTRUCTIONS, INFREQUENT AND ABNORMAL

60855.1 - Operation of an Independent Spent Fuel Storage Installation

The inspectors evaluated the licensee's independent spent fuel storage installation cask loadings on September 20, 2018.

INSPECTION RESULTS

Unit 1 Pressurizer Safety Valve Lift Pressure Outside of Technical Specification Tolerance Band			
Cornerstone	Severity	Crosscutting	Report Section
Not Applicable	SL IV NCV 05000348/2018003-01 Closed	Not Applicable	71153 – Follow-up of Events and Notices of Enforcement Discretion
A self-revealed SL IV NCV of TS 3.4.10, "Pressurizer Safety Valves," was identified when a routine lift pressure test revealed that pressurizer safety valve Q1B13V0031C was lower than allowed by TS SR 3.4.10.1 for a duration that was longer than the condition's TS required action completion time.			
<p><u>Description:</u> On April 12, 2018, pressurizer safety valve Q1B13V0031C was removed from service at Farley Nuclear Plant Unit 1, and on April 16, 2018, the valve was tested with steam at an offsite facility. As-found lift testing determined that the valve opened at 2420 psig steam pressure, which was low outside the plant technical specification allowable lift setting range of 2460 psig to 2510 psig. The valve had been installed and placed in service at Farley Nuclear Plant Unit 1 on October 8, 2013, and remained in service during three complete 18-month fuel cycles. Upon removal of valve Q1B13V0031C from Unit 1 on April 12, 2018, it was replaced with a similar operable refurbished valve. Licensee Event Report (LER) 05000348/2018-001-00, "Pressurizer Safety Valve Lift Pressure Outside of Technical Specification Limits due to Spring Relaxation," was submitted by the licensee for this event.</p> <p>Corrective Action(s): The valve was replaced with a similar operable refurbished valve during the refueling outage prior to plant startup.</p> <p>Corrective Action Reference(s): The licensee entered this issue into their corrective actions program as CR10483091 – "PZR safety valve test results."</p> <p><u>Performance Assessment</u> The inspectors determined the condition was not reasonably foreseeable and preventable by the licensee and therefore was not a performance deficiency. Specifically, random setpoint drift is a recognized valid phenomenon that can occur despite routine testing and maintenance.</p>			

Enforcement:

Severity: Traditional Enforcement is being used to disposition this violation with no associated Reactor Oversight Process performance deficiency per NRC Memorandum "Interim Guidance for Dispositioning Severity Level IV Violations with No Associated Performance Deficiency" (ML18158A220). The inspector assessed the severity of the violation using Section 6.1 of the Enforcement Policy and determined the significance is appropriately characterized as Severity Level IV, due to the inappreciable potential safety consequences. The licensee determined that the safety valve low as-found lift set-point did not have an adverse impact on reactor coolant system over-pressurization protection, since the valve continued to perform its reactor coolant system over-pressure protection function to prevent the system from exceeding the design pressure of 2485 psig. Therefore, the plant remained bounded by the accident analysis in the FSAR, based on the as-found condition.

Violation: Farley Nuclear Plant Unit 1 TS LCO 3.4.10, "Pressurizer Safety Valves," required three operable pressurizer safety valves with lift settings between 2460 psig and 2510 psig, while the Unit was in modes 1, 2, and 3. With one pressurizer safety valve inoperable, Action Statement, Condition "A." Required Action "A.1," required restoration of the valve to operable status within 15 minutes. If the required action and associated completion time is not met, Action Statement, Condition "B," required that the unit be in mode 3 within 6 hours. Contrary to the above, the licensee determined the pressurizer safety valve setting was outside the TS limits longer than 6 hours and 15 minutes during the last three operating cycle, between October 8, 2013 and April 12, 2018, while the Unit was in modes 1, 2, and 3.

Disposition: This violation is being treated as an NCV, consistent with Section 2.3.2 of the Enforcement Policy.

Licensee Identified Non-Cited Violation	71152
This violation of very low safety significance was identified by the licensee and has been entered into the licensee corrective action program and is being treated as a Non-Cited Violation, consistent with Section 2.3.2 of the Enforcement Policy.	
Violation: Farley Unit 1 Operating License Condition 2.C.(4) and Unit 2 Operating License Condition 2.C.(6), Fire Protection, required in part that Plant Farley shall implement and maintain in effect all provisions of the approved fire protection program that comply with 10 CFR 50.48(c) and NFPA 805. NFPA 805 section 3.2.3 stated, in part, procedures to accomplish compensatory actions implemented when fire protection systems and other systems credited by the fire protection program and this standard cannot perform their intended function shall be established. Licensee procedure FNP-0-SOP-0.4, "Fire Protection Operability and LCO Requirements" section 4.0 establishes compensatory action when fire protection systems and other systems credited by the fire protection program cannot perform their intended functions.	
Contrary to the above, since January 16, 2018 through August 28, 2018, the licensee failed to establish compensatory measures (fire watches) as required by licensee procedure FNP-0-SOP-0.4 on thirteen occasions. The cause of the fire watch discrepancies were mainly because Farley Operations staff lacked an adequate understanding and ownership of the fire watch implementation process.	

Significance/Severity Level: The inspectors evaluated this finding in accordance with NRC's inspection manual chapter (IMC) 0609 App F, "Fire Protection Significance Determination Process, issued May 2, 2018. The finding screened to Green (very low safety significance) because the inspectors determined that the missed fire watches would not adversely impact equipment important to safe shutdown of the unit in the respective fire areas.

Corrective Action Reference(s): Condition report (CR) 10529513 is the trend CR that documented several missed fire watches. Corrective actions are planned and currently in progress to address this issue.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

The inspectors confirmed that proprietary information was controlled to protect from public disclosure.

- On October 16, 2018, the inspectors presented the quarterly baseline inspection results, respectively to Dennis Madison, Site Vice President, and Scott Briggs, Plant Manager, and other members of the licensee staff.

DOCUMENTS REVIEWED

71111.01: Adverse Weather Protection

Procedures:

FNP-0-AOP-21, Severe Weather, Rev. 46.1

NMP-OS-017, Severe Weather, Ver. 1.1

NMP-AD-014, Requirements for Compliance with NERC Standards, Ver. 6.1

NMP-AD-014-GL01, Guidelines for Compliance with NERC Standards, Ver. 6.0

Documents:

Regulatory Guide 1.27, Ultimate Heat Sink

D-171417, Pond Fill Discharge Structure, Rev. 3

D-170178, Piping- River Water Pumps Discharge To Storage Pond, Rev. 19

D-176981, Storage Dam and Dike – General Plan, Rev. 8

D-171419, River Water discharge to pond structure, Rev. 0

71111.04: Equipment Alignment

Procedures:

FNP-2-SOP-9.0, Containment Spray System, Ver. 38.1

FNP-2-SOP-9.0A, Containment Spray System, Ver. 8

FNP-2-SOP-7.0, Residual Heat Removal System, Ver. 99.0

FNP-2-SOP-7.0A, Residual Heat Removal System, Ver. 11.0

FNP-2-SOP-22.0, Auxiliary Feedwater System, Ver. 77.2

FNP-2-SOP-22.0A, Auxiliary Feedwater System, Ver. 14.0

Drawings:

D-205007, Unit 2 P&ID – Auxiliary Feedwater System, Sheet 1, Ver. 29.0

D-205033, Unit 2 P&ID – Main Steam and Auxiliary Steam Systems, Sheet 1,
Ver. 39.0

D-205033, Unit 2 P&ID – Main Steam and Auxiliary Steam Systems, Sheet 2, Ver. 25.0
D-205038, Unit 2 P&ID – Safety Injection System, Sheet 1, Ver. 39
D-205038, Unit 2 P&ID – Safety Injection System, Sheet 2, Ver. 24
D-205038, Unit 2 P&ID – Safety Injection System (Containment Spray), Sheet 3, Ver. 34
D-205041, Unit 2 P&ID – Residual Heat Removal System, Sheet 1, Ver. 20

Condition Reports:

10530528 10530671

71111.05: Fire Protection Annual/Quarterly

Procedures:

FNP-0-FPP-3.0, Owner Controlled Area Pre-Fire Plan, Ver. 2.0
FNP-2-FPP-1.0, Unit 2 Auxiliary Building Pre-Fire Plan, Ver. 1.0
NMP-TR-425-F14, Fire Drill Package, Ver. 1.0

Condition Reports:

10508072 10508079 10516723 10517136 10523359 10523468
10523467

Work Orders:

SNC913159 SNC719746 SNC885164 SNC641901 SNC881904 SNC644350
SNC538103 SNC776088 SNC737915 SNC952320 SNC952321

Drawings:

D513635, Unit No. 1/2 – Fire Barriers and Fire Boundaries – Service Water Intake Structure (SWIS), Ver. 1.0

Documents:

A-181805, NFPA 805 Fire Protection Program Design Basis Document, Ver. 5.0

71111.11: Licensed Operator Regualification Program

Procedures:

FNP-1-UOP-3.1, Power Operation, Ver. 131.0

Documents:

Operations Training Simulator Exam Scenario #31, Approved on July 26, 2018

71111.12: Maintenance Effectiveness

Procedures:

NMP-ES-027, Maintenance Rule Program, Ver. 8

Documents:

EVAL-F-Y41-04230, MR evaluation for 1-2A diesel generator MR function Y41-F05
Maintenance Rule Expert Panel (MREP) Meeting 18-16 Agenda, August 29, 2018.
Technical Evaluation (TE) 1016165
MREP meeting minutes from Meeting 18-16 on August 29, 2018.

Condition Reports:
10500679

71111.13: Maintenance Risk Assessments and Emergent Work Evaluation

Procedures:

NMP-OS-010-001, Farley Protected Equipment Logs, Ver. 15
NMP-GM-031-001, Online Maintenance Rule (a)(4) Risk Calculations, Ver. 5

Condition Reports:
10532482

Documents:

Unit 2 EOOS Operator's Risk Report for August 10, 2018
Unit 2 EOOS Operator's Risk Report for August 13, 2018
Unit 1 EOOS Operator's Risk Report for September 5, 2018
Unit 2 EOOS Operator's Risk Report for September 5, 2018

71111.15: Operability Determinations and Functionality Assessments

Procedures:

FNP-2-STP-11.2, 2B RHR Pump Quarterly Inservice Test, Ver. 43.0
FNP-0-GMP-30.1, Tribology Program, Ver. 25.0
FNP-0-GMP-37.0, Residual Heat Removal Pump Lubrication Procedure, Ver. 4.0
FNP-0-M-50, Master List of Surveillance Requirements, Ver. 30

Documents:

U418156, Instruction Manual RHR Pumps, Ver. 4.0
Administrative Tracking Item (ATI) 1705, dated May 22, 2018
A508632, Pressurizer Pressure Protection Scaling Document, Ver. 5.0
A-177048, Relay Settings, Sheet 328, Rev. 1
WCAP-1375-P, Westinghouse Setpoint Methodology for Protection Systems Farley Nuclear Plant Units 1 and 2, Rev. 2
ATI-1705, Administrative Tracking Item for drifting PT-457
ODMI-18-04, 1C RCP motor bearing oil reservoir low level alarm, Ver. 3.0

Condition Reports:

10515924 10495878 10489880 10518144 10518931 10518992
10520001

Work Orders:

SNC880406 SNC830329 SNC810896 SNC810891 SNC955428

71111.19: Post Maintenance Testing

Procedures:

FNP-0-EMP-1313.12, Maintenance of Siemens-Allis 4.16 kV Metal-Clad Switchgear MOC Switch, Ver. 9.3
FNP-0-EMP-1313.19, Inspection and Adjustment of Cutler Hammer 4.16 kV Circuit Breakers Type MA-VR350, Ver. 17.0
FNP-0-EMP-1313.20, Enhanced Inspection of Cutler Hammer 4.16 kV Circuit Breakers Type MA-VR350, Ver. 24.1
FNP-0-STP-80.2, Diesel Generator 1C Operability Test, Ver. 67.0

FNP-1-STP-80.20, Diesel Generator 1-2A 1000 KW Load Rejection Test, Ver. 23.1
FNP-2-STP-201.6, Pressurizer Pressure Q2B31PT0457 Loop Calibration and Operational Test,
Ver. 53.0
FNP-1-STP-21.2, MSIV Air System Leak Test, Ver. 18.0

Condition Reports:

10518144 10518931 10527200

Work Orders:

SNC830329 SNC964137 SNC955428

71111.20: Refueling and Other Outage Activities

Procedures:

FNP-1-UOP-2.4 Planned Reactor Shutdown and Cooldown to Cold Shutdown_

Condition Reports:

10540912 10541123

Documents:

NMP-OS-003-F01, Operational-Making Issue Worksheet, Ver. 6.4

71111.22: Surveillance Testing

Procedures:

FNP-1-STP-22.1, Auxiliary Feedwater Pump 1A Inservice Test, Ver. 45.0
FNP-1-STP-109.0, Power Range Neutron Flux Channel Calibration, Ver. 61
FNP-1-STP-9.0, RCS Leakage Test, Ver. 51.5

Condition Reports:

10515973 10515992 10520944

Work Orders:

SNC918430

Documents:

Integrated Plant Computer (IPC) printouts of reactor thermal power, reactor coolant loop delta-
temperature 1 min. average from July 11 through July 18, 2018
S-2018-014, Ver. 1, Standing order for CR10515992

71114.06: Drill Evaluation

Procedures:

NMP-EP-141, Event Classification, Ver. 2.0
NMP-EP-144, Protective Actions, Ver. 3.0
NMP-EP-141-001-F01, Farley – Hot Initiating Condition Matrix, Ver. 2.0

Documents:

Emergency Preparedness Drill Controller Guide, August 14, 2018
Reactor Plant Event Notification Worksheet, Drill message 1, August 14, 2018
Reactor Plant Event Notification Worksheet, Drill message 2, August 14, 2018
Reactor Plant Event Notification Worksheet, Drill message 3, August 14, 2018
Reactor Plant Event Notification Worksheet, Drill message 4, August 14, 2018

Reactor Plant Event Notification Worksheet, Drill message 5, August 14, 2018
Reactor Plant Event Notification Worksheet, Drill message 6, August 14, 2018

71151: Performance Indicator Verification

Procedures:

FNP-0-AP-54.0, Preparation and Reporting of NRC Performance Indicator Data and NRC Operating Data, Ver. 15
FNP-1-STP-746, Primary Coolant System Dose Equivalent Iodine-131 Determination, Ver. 28
FNP-2-STP-746, Primary Coolant System Dose Equivalent Iodine-131 Determination, Ver. 28
FNP-0-CCP-25, Dose Equivalent Iodine-131 Determination, Ver. 15
FNP-1-CCP-42, Primary Coolant Liquid Gamma Spectroscopy Analysis, Ver. 27
FNP-2-CCP-42, Primary Coolant Liquid Gamma Spectroscopy Analysis, Ver. 27
FNP-1-CCP-651.1, Routine Sampling of the Reactor Coolant System, Ver. 18.2
FNP-1-STP-9.0, RCS Leakage Test, Ver. 51.5
FNP-2-STP-9.0, RCS Leakage Test, Ver. 47.5
FNP-0-SOP-0.13, Recording Limiting Conditions For Operations, Ver. 35
FNP-1-STP-21.2, MSIV Air System Leak Test, Ver. 18.0
FNP-1-STP-45.7, MSIV Air System Leak Test, Ver. 24.0

Documents:

Main Control Room logs, various dates
Chemistry Surveillance logs, various dates
Farley Key Performance Indicators Report, August 2018

Licensee Event Reports (LERs):

05000348/2016007-02
05000348/2016008-00
05000348, 364/2016009-00
05000348/2017001-00
05000364/2017001-00
05000364/2017002-00
05000364/2017003-00
05000364/2017004-00
05000364/2017005-00

71152: Problem Identification and Resolution

Procedures:

NMP-ES-006-GL02, Preventative Maintenance Change Requests, Version 11.0
NMP-GM-002-001, Corrective Action Program Instructions, Version 3.0
NMP-GM-008, Operating Experience Program, Version 3.0
NOS -105, Internal Nuclear Oversight Audits, Version 5.6
NOS-104-001-F07, Engineering Audit Planning Guide, Version 2.0
NOS-104-001F17, Maintenance Audit Planning Guide, Version 3.0
PS-004, Vendor Technical Information Program, Version 4.0

Condition Reports:

10527200	2007100531	10540109	10540120	10540234	10540246
10540350	10529513	10513247	10529663	10529242	10525415
10450707	10463785	10464160	10464292	10464719	10481328
10529963					

Work Orders:
SNC812826

Documents:

CR 10527200 Support Refute Matrix, dated 8/30/18
Fairbanks Morse Owners' Group Fax Query 0709-09b, IEN Engineering Additional Follow-up
Information Item concerning SBSR Voltage Regulator Part 21, dated 9/21/07
Nuclear Oversight Audit of Engineering 1/4/17
Nuclear Oversight Audit of Maintenance 11/6/17
PMCR 22107, Extend EDG 1-2A PM Frequency, dated 8/4/10
Technical Decision Making Form, dated 8/30/18
CARs 272781, 274383

71153: Event Follow-up

Documents:

Licensee Event Report (LER) 05000364/2018-001-00, Pressurizer Safety Valve Lift Pressure
Outside of Technical Specification Limits during to Spring Relaxation

Condition Reports:

10483091

Work Orders:

SNC594727

60855.1 - Operation of an Independent Spent Fuel Storage Installation

Procedures:

FNP-0-MP-110.0, Dry Fuel Storage Campaign Guidelines, Ver. 19.0
FNP-0-MP-111.1, Hi-Storm System Site Transportation, Ver. 19.0
FNP-0-MP-111.11, MPC Helium Dehydration System Operation, Ver. 3.0
FNP-0-MP-111.12, Forced Helium Dehydration System Operation, Ver. 10.0
FNP-0-MP-111.13, Supplemental Cooling System Operation, Ver. 7.0
FNP-0-MP-111.2, Hi-Storm System Preparation and Loading Operation, Ver. 23.0
FNP-0-MP-111.3, MPC Fuel Loading Operations, Ver. 29.0