

B 06/09/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL 50-260

REC: OREILLY J P
NRC

ORG: FOX H S
TN VALLEY AUTH

DOC DATE: 05/30/78
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DOCTYPE: LETTER NOTARIZED: NO
SUBJECT:

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LTR 1 ENCL 1

FORWARDING LICENSEE EVENT REPT (RO 50-260/78-010) ON 05/04/78 CONCERNING
DURING SURVEILLANCE TESTING, RELIEF VALVE ON STANDBY LIQUID CONTROL PUMP "B"
OPENED AT 900 PSIG, VIOLATING TECH SPEC LIMIT OF 4.4 A. 2. A.

PLANT NAME: BROWNS FERRY - UNIT 2

REVIEWER INITIAL: XJM
DISTRIBUTOR INITIAL: DL

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

INCIDENT REPORTS
(DISTRIBUTION CODE A002)

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- LPDR'S
- ATHENS, AL**W/ENCL
- TIC**W/ENCL
- NSIC**W/ENCL
- ACRS CAT B**W/16 ENCL

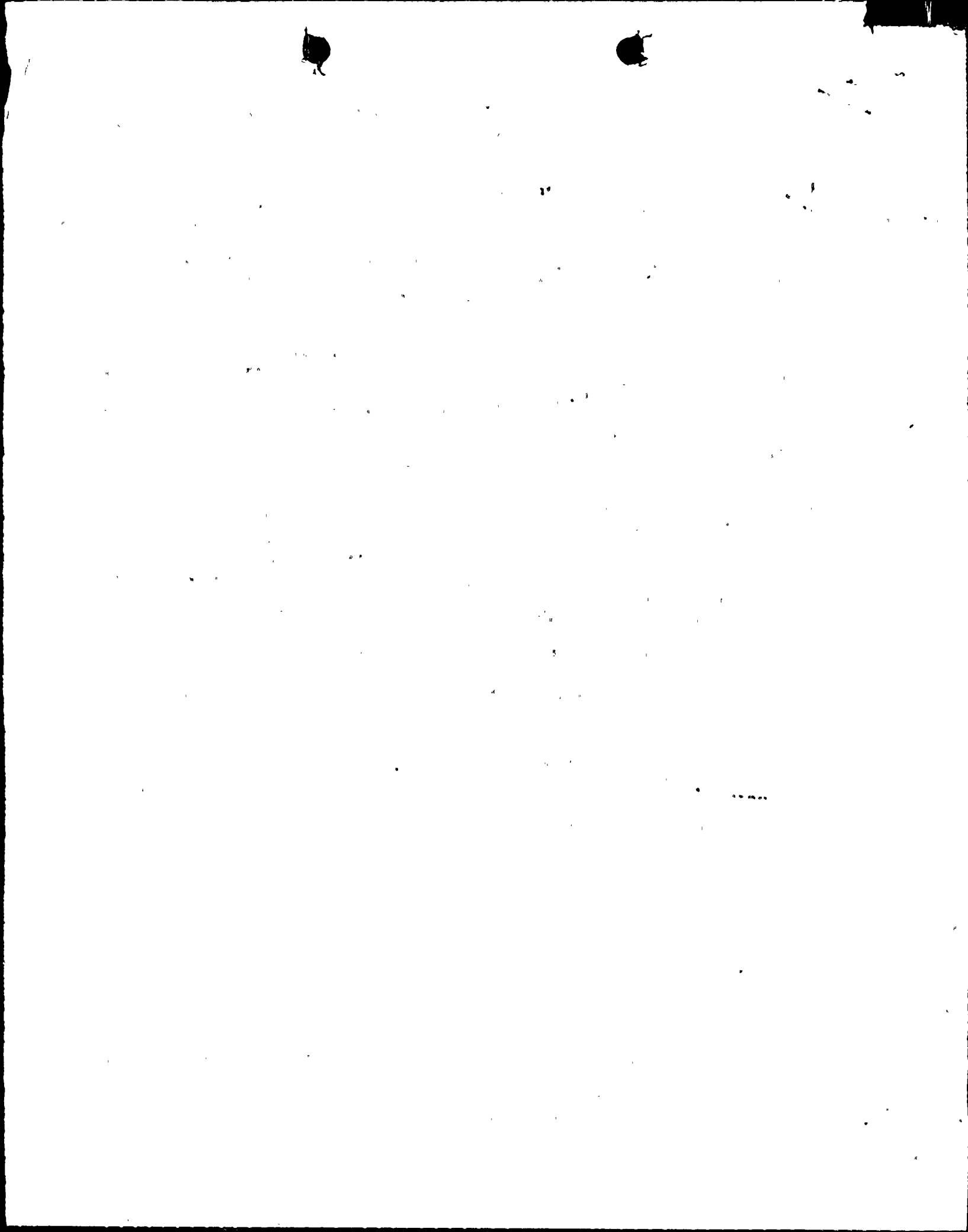
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REGULATORY GUIDE 10.1

A04

DISTRIBUTION: LTR 45 ENCL 45
SIZE: 1P+1P

CONTROL NBR: 781600292

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TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

MAY 30 1978

REGULATORY DOCKET FILE COPY

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
230 Peachtree Street, NW., Suite 1217
Atlanta, Georgia 30303

U.S. NRC
REGULATORY SERVICES
BRANCH

1978 JUN 7 PM 12 32

U.S. NRC
REGULATORY SERVICES
BRANCH

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 -
DOCKET NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE
OCCURRENCE REPORT BFRO-50-260/7810

The enclosed report provides details concerning the relief valve on standby liquid control pump "B" which opened at 900 psig (which is lower than the designed setting of 1425 ± 75 psig as designated by Technical Specification 4.4.A.2.a) during surveillance testing. This report is submitted in accordance with Browns Ferry unit 2 Technical Specification 6.7.2.b.(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

H. S. Fox
Director of Power Production

Enclosure (3)

cc (Enclosure):

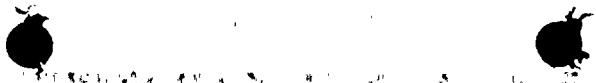
Director (3)

Office of Management Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Director (40)

Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

4002
5/11



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LICENSEE EVENT REPORT

CONTROL BLOCK: 1 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | ALBRF2 | 200-000000-003 | 411111 | 4 | 5

CON'T
 REPORT SOURCE L6050-02607 | 0504788 | 053178 | 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

02 | During surveillance testing, the relief valve on standby liquid control pump "B" opened
03 | at 900 psig, which is lower than the designed setting of 1425 ± 75 psig as designated
04 | by T.S. 4.4.A.2.a. Refueling operations were in progress on the unit; there was no
05 | fuel in the reactor vessel; and there were no effects on public health or safety.
06 | Similar events have been previously reported on BFRO-50-296/771, BFRO-50-296/778, and
07 | BFRO-50-296/7713.

08 | _____

SYSTEM CODE SH11 | CAUSE CODE E12 | CAUSE SUBCODE B13 | COMPONENT CODE VALVEX14 | COMP. SUBCODE X15 | VALVE SUBCODE B16
 LER/RO REPORT NUMBER 78 | EVENT YEAR 78 | SEQUENTIAL REPORT NO. 010 | OCCURRENCE CODE 03 | REPORT TYPE L | REVISION NO. 0
 ACTION TAKEN E18 | FUTURE ACTION Z19 | EFFECT ON PLANT Z20 | SHUTDOWN METHOD Z21 | HOURS 0000 | ATTACHMENT SUBMITTED N23 | NPRD-4 FORM SUB. Y24 | PRIME COMP. SUPPLIER N25 | COMPONENT MANUFACTURER C71026

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

10 | The relief valve is a Crosby Valve and Gage Co., size 1 x 2, style JMWK. The valve's
11 | set pressure was adjusted properly and it was retested and met surveillance testing
12 | requirements.
13 | _____
14 | _____

FACILITY STATUS H28 | % POWER 00029 | OTHER STATUS N/A30 | METHOD OF DISCOVERY B31 | DISCOVERY DESCRIPTION N/A32

ACTIVITY RELEASED Z33 | CONTENT OF RELEASE Z34 | AMOUNT OF ACTIVITY N/A35 | LOCATION OF RELEASE N/A36

PERSONNEL EXPOSURES NUMBER 00037 | TYPE Z38 | DESCRIPTION N/A39

PERSONNEL INJURIES NUMBER 00040 | DESCRIPTION N/A41

LOSS OF OR DAMAGE TO FACILITY TYPE Z42 | DESCRIPTION N/A43

PUBLICITY ISSUED N44 | DESCRIPTION N/A45

NRC USE ONLY

NAME OF PREPARER _____ PHONE: _____

