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Anthony J. Vitale Site Vice President

NL-18-044

June 25, 2018

U.S. Nuclear Regulatory Commission Document Control Desk 11545 Rockville Pike, TWFN-2F1 Rockville, MD 20852-2738

SUBJECT:

Supplemental Information Regarding the Review of the Indian Point Nuclear Generating Unit Nos. 2 and 3 License Renewal Application (LRA) (CAC Nos. MD5407 and MD5408)

Docket Nos. 50-247 and 50-286 Licenses Nos. DPR-26 and DPR-64

REFERENCES:

- Entergy Letter, "Reply to Request for Additional Information for the Review of the Indian Point License Renewal Application RAI SET 2017-03 (CAC Nos. MD5407 and MD5408)," dated May 18, 2017
- 2) Entergy Letter, "Request for Additional Information for the Review of the Indian Point License Renewal Application RAI SET (CAC Nos. MD5407 and MD5408)," dated May 19, 2017
- 3) Entergy Letter, "Comments on Proposed Director's Decision for 10 CFR 2.206 Petition on Indian Point Baffle Former Bolts Filed by the Union of Concerned Scientists Docket Nos. 50-247 and 50-286 (License Nos. DPR-26 and DPR-64)," dated February 09, 2017

Dear Sir or Madam:

By email date June 06, 2018, the Nuclear Regulatory Commission (NRC) identified "several questions" that needed answers to support completion of the license renewal application Safety Evaluation Report for Indian Point Nuclear Generating Unit Nos. 2 and 3 (Indian Point). The questions asked by the NRC and the Indian Point responses are included in Attachment 1 of this letter.

AIZ8 NRR Entergy is making no new commitments in this letter. Should you have any questions regarding this matter, please contact Mr. Robert Walpole, Manager, Regulatory Assurance, Indian Point Energy Center at (914) 254-6710.

Sincerely,

AJV/trj

Attachment: 1. Response to email from NRC date June 06, 2018, Regarding the

License Renewal Application - Safety Evaluation Report

cc: Mr. David Lew, Acting Regional Administrator, NRC Region I

Mr. Sherwin E. Turk, NRC Office of General Counsel, Special Counsel

Mr. William Burton, NRC Senior Project Manager, Division of License Renewal

Mr. Richard V. Guzman, NRR Senior Project Manager

Ms. Bridget Frymire, New York State Department of Public Service

Ms. Alicia Barton, President and CEO NYSERDA

NRC Resident Inspector's Office

ATTACHMENT 1

to NL-18-044

RESPONSE TO EMAIL FROM NRC DATE JUNE 06, 2018 REGARDING THE LICENSE RENEWAL APPLICATION - SAFETY EVALUATION REPORT

ENTERGY NUCLEAR OPERATIONS, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 & 3

DOCKET NOS. 50-247 AND 50-286

Response to email from NRC date June 06, 2018, Regarding the License Renewal Application - Safety Evaluation Report

By email dated June 06, 2016, the NRC asked the following questions:

- 1. Question: By letter dated May 18, 2017, IPEC stated that it would implement an enhancement to the Bolting Integrity Program before May 31, 2018. Has this been done?
 - a. Answer: Yes, we are in the implementation phase of this enhancement. A program has been developed and a number of potential opportunistic inspections selected.
- 2. Question: By letter dated May 19, 2017 IPEC identified an enhancement to the "detection of aging effects" program element in the Steam Generator Integrity program to revise applicable procedures to specify a general visual inspection of the steam generator channel head (Commitment No. 54), consistent with LR-ISG-2016-01, by December 31, 2017. Has this been done?
 - Answer: Yes, the steam generator integrity program procedures have been revised to specify a general visual inspection of the steam generator channel head.
- 3. Question: By letter dated February 6, 2017, IPEC that the IP3 baffle-bolt inspections previously scheduled to be performed in 3R20 (spring 2019) will be performed in 3R19 (spring 2017). Visual and UT inspections on 100 percent of all accessible BFBs, and a visual inspection of the baffle-edge bolts and baffle-former assembly, will be performed in 3R19. Has this been done?
 - a. Answer: The actual date of the letter was February 9, 2017. During the 3R19 we inspected all baffle bolts and replaced all of the bolts which either had flaws or could not be inspected. All baffle bolts were inspected visually and with UT. All baffle bolts that had any indications of flaws were replaced as documented in EC 71081. All baffle bolt location contained either an existing baffle bolt that showed no indications of flaws OR were replaced with a new design. We also performed a visual inspection of the baffle-edge bolts and baffle-former assembly and no unacceptable indications were identified.