

Entergy Operations, Inc.

River Bend Station 5485 U.S. Highway 61N St. Francisville, LA 70775 Tel. 225-381-4374

William F. Maguire Site Vice President River Bend Station

RBG-47876

June 21, 2018

Attn: Document Control Desk U.S. Nuclear Regulatory Commission 11555 Rockville Pike Rockville, MD 20852-2738

SUBJECT: Response to License Amendment Request (LAR) for Review and Approval Of

Relocation Of The Reactor Core Isolation Cooling (RCIC) Injection Point NRC

Request for Additional Information (RAI)

River Bend Station, Unit 1

Docket No. 50-458 License No. NPF-47

RBF1-18-0136

References: 1) Entergy Letter: License Amendment Request (LAR) (RBG-47823 dated January 29, 2018) (ADAMS Accession No. ML18029A187)

2) NRC email: River Bend Station, Unit 1, Request for Additional Information dated May 24, 2018 (ADAMS Accession No. ML18149A288)

Dear Sir or Madam:

In Reference 1, Entergy Operations, Inc (Entergy) submitted a request for the review and approval of relocation of the Reactor Core Isolation Cooling (RCIC) Injection Point from the Reactor Vessel Head Spray Nozzle to the 'A' Feedwater Line via the 'A' Residual Heat Removal (RHR) Shutdown Cooling Return Line. In an email dated May 24, 2018, (Reference 2) the NRC staff made a request for additional information (RAI), needed to complete the license amendment request. Enclosure 1 provides the responses to the RAIs. Enclosure 2 contains the pipe stress calculation updates AX-002H, AX-071AG, AX-076A, and AX-076B.

This letter does not contain any new commitments.

If you require additional information, please contact Mr. Tim Schenk at (225)-381-4177 or tschenk@entergy.com.

In accordance with 10 CFR 50.91(b)(1), Entergy is notifying the State of Louisiana and the State of Texas by transmitting a copy of this letter to the designated State Official.

RBG-47876 Page 2 of 2

I declare under penalty of perjury that the foregoing is true and correct. Executed on June 21, 2018.

Sincerely,

WFM/BAJ

Enclosure 1: Responses to Request for Additional Information

Enclosure 2: Pipe stress calculation updates AX-002H, AX-071AG, AX-076A, and AX-076B

cc: U.S. Nuclear Regulatory Commission

Region IV

1600 E. Lamar Blvd.

Arlington, TX 76011-4511

U.S. Nuclear Regulatory Commission

Attn: Ms. Lisa M. Regner, Project Manager

09-D-14

One White Flint North

11555 Rockville Pike

Rockville, MD 20852

NRC Senior Resident Inspector

Attn: Mr. Jeff Sowa

5485 U.S. Highway 61, Ste. NRC

St. Francisville, LA 70775

Department of Environmental Quality

Office of Environmental Compliance

Radiological Emergency Planning and Response Section

Ji Young Wiley

P.O. Box 4312

Baton Rouge, LA 70821-4312

Public Utility Commission of Texas

Attn: PUC Filing Clerk

1701 N. Congress Avenue

P.O. Box 13326

Austin, TX 78711-3326

RBG-47876

Enclosure 1

Responses to Request for Additional Information

REQUEST FOR ADDITIONAL INFORMATION RELOCATION OF THE REACTOR CORE ISOLATION COOLING (RCIC) INJECTION POINT RIVER BEND STATION, UNIT 1 DOCKET NO.: 50-458

Background

By letter dated January 29, 2018, Entergy submitted a license amendment application. The licensee stated that the 6-inch Reactor Core Isolation Cooling (RCIC) system injection line is routed to the 20-inch Feedwater A Loop by tapping into the 10-inch Reactor Heat Removal system shutdown cooling mode return line to feedwater inside the main steam tunnel just south of the jet impingement wall. The staff noted that the piping system has to be qualified in accordance with ASME Code Section III requirements and meet 10 CFR 50.55(a) and relevant GDCs. The staff requests the licensee to address the following items related to the piping analysis:

Question a

Piping analysis model: The staff noted that this change affects 3 different systems. Figure 3A.24-1 of USAR provides RCIC piping system used in analysis of water hammer loads. The staff noted that the piping model does not appear to address the RHR piping. Please explain. Also, describe the piping model used for other load cases.

Response to Question a

PX-1037 was updated with the modification and contains the water hammer analysis of the RCIC Piping; PX-1000 was updated and contains the water hammer analysis for the feedwater piping. Both analyses were updated to show injection into Residual Heat Removal (RHR) piping RHS-010-065 that connects to the Feedwater Mixing Tee and Feedwater Line FWS-020-047. Line RHS-010-065 is the only portion of the RHR system piping that is impacted by the RCIC reroute modification as it is protected from backflow by check valve E12-VF050A and the normally closed isolation valve E12-MOVF053A (ref. PID-27-07A).

Standard Load Cases for the requested RCIC and RHR system piping are provided in the response to Question b below. Additional analyses performed as a result of MR96-0069 can be provided on request.

Question b

Please provide the piping stress summary for RCIC and RHR systems.

Response to Question b

Pipe stress calculation updates to the affected RCIC and RHR piping as a result of MR96-0069 are provided in Enclosure 2: AX-002H, AX-071AG, AX-076A, and AX-076B.

RBG-47876 Enclosure 1 Page 3 of 3

Question c

Please discuss any thermal mode or transient changes and loading condition changes since the Cumulative Usage Factors (CUFs) and stresses are shown to lessen from previous values as specified below:

- 1. Table 3.6A-9a of the USAR lists the CUFs and the stresses for the feedwater system inside containment. The NRC staff noted that there is no routing change inside containment. Please explain what causes the reduction in stresses and CUFs.
- 2. Table 3.6A-10a of the USAR lists the stresses for the feedwater system outside of containment. The NRC staff noted that there is no routing change for the feedwater system outside of the containment. Please explain what causes the reduction in stresses.

Response to Question c Part 1

Stress and CUF were calculated in AX-017D revision 3K as part of the RCIC Reroute modification MR96-0069. The fluid transient load case used in this calculation update is based on RBS calculation PX-1000 Rev. 2 (also revised with MR96-0069) curves which are smaller than the design loads of PX-1000 Rev. 1 used in the previous analysis. The transient loads have reduced as a result of installing a nozzle check valve FWS-V3052 into the Feedwater system with MR96-0069. This nozzle check valve has a faster closing time compared to the previously modeled swing check valves. The reverse flow when the nozzle check valve closes would be lower producing smaller fluid transient loads. Smaller transient loads result in lower stress and fatigue values.

This revision also incorporated and superseded 6 additional addenda and markups of calculation AX-017D that were unrelated to the RCIC Reroute modification. The changes included a temperature change, piping schedule changes, a change in valve weight, and a change in damping curves that are unrelated to the changes made by modification MR96-0069; these incorporated changes may also have impacted the stress and CUF results.

Response to Question c Part 2

See the response to Part 1 above.

Note: The break points shown in current USAR Table 3.6A-10a have since been updated and no longer reflect the values changed by the subject RCIC modification MR96-0069.

RBG-47876

Enclosure 2

Pipe stress calculation updates AX-002H, AX-071AG, AX-076A, and AX-076B



ALC.	NO.	-		
	A	X-2H	Rev.	30

ATTACHMENT NO.:

JBI NO.: G13.10.2.2

PACE 1 O

ENTERGY ACT 22	DETTE		PAGE 1 OF 7	
TITLE: SUPERSEDES: N/A				
Stress Analysis for Main Steam Vent	SUPPLEMENTS: AX-2H Rev. 3			
CALCULATION STATUS:	□ APPROV	ED ■PEN	DING CANCELED	
SYSTEM NO.: 209	SYSTEM NO.: 209 MARK NO.:			
CLASSIFICATION: SAFETY RE	CLASSIFICATION: ■ SAFETY RELATED NON-SAFETY RELATED: □ QAPA □ NON-QAPA			
PURPOSE / SCOPE / OBJECTIVE:				
To document the changes implemented vessel head to the feedwater nozzle. A removed from the vent and spray asso	s a result of this		the RCIC injection from the reactor 6-1 (previously qualified in AX-76A) is	
	*		*	
CONCLUSION:				
The changes implemented by MR 96-0069 do not affect the piping qualified in AX-2H. The vent and spray assembly is now solely qualified by AX-2H. RPV nozzle number 7 and the vent and spray assembly remain qualified since the removal of line ICS-006-6-1 reduces the loads acting on the vent and spray assembly.				
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	JUN 0 9 199	98	*	
	CDC	*		
SOFTWARE USED FOR CALCULA	TION: YES	□ NO SDDF#:	N/A	
Manufacturer:	N	ameV	/ersion/Release No	
CONFIRMATIONS REQUIRED:	YES INO	CONFIRMATION CO	OMPLETE: DYES DNO DN/A	
KEYWORDS:	,			
RPV Vent Line, MR 96-0069, RCIC,	RPV nozzle			
	REVIEW	& APPROVAL		
Camile Lucder 6-23-97 (Signature/Date) Preparer Camilo Cuadra, KCN 0969 Camilo Cuadra, KCN 0969 Lother Kuhn KCN 1105 (Signature/Date) Camilo Cuadra, KCN 0969 Lother Kuhn KCN 1105				



CALC. NO. - REV. ADDENDUM
AX-2H Rev. 3C
ATTACHMENT NO.:
JBI NO.:
PAGE 2 OF

Stress Analysis for Main Steam Vent Line per MR 96-0069

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3.0	CALCULATION CHECKLIST	6
N/A	DESIGN VERIFICATION CHECKLIST	7

Total Pages: 7



CALC. NO. - REV. ADDENDUM

AX-2H Rev. 3C

ATTACHMENT NO.: (as required)

JBI NO.:

PAGE 3 OF

Stress Analysis for Main Steam Vent Line per MR 96-0069

		REVISION HISTORY
Revision No.	Paragraph No.	Description of Change
3C .	All	To document the changes implemented by MR 96-0069. This MR re-routes the RCIC injection from the reactor vessel head to the feedwater nozzle. As a result of this change, line ICS-006-6-1 (previously qualified in AX-76A) is removed from the vent and spray assembly.
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ENTERGY	K	IVER BEND STA	TION	PAGE 4 0	OF
	S	tress Analysis for Main Ste	am Vent Line per N	IR 96-0069	
		APPLICABLI	E DOCUMENTS		
L APPLICABLE C	ODES, ST	ANDARDS, & REFEREN	CES (NON-RBS):		
1. ASME Boiler &	Pressure V	essel Code, Section III, Divis	ion 1, 1974 Edition		
II. RBS REFEREN	CES:	:			
1. MR 96-0069			**		
2. AX-76A Rev. 3E	3				
III. AFFECTED D	OCUMEN	TS:			
DOC. NO.	REV. NO.	DOCUMENT TITLE		FECT vised, Initiated, etc.)	CHANGE DOC.
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CALC. NO. - REV. ADDENDUM
AX-2H Rev. 3C
ATTACHMENT NO.:
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ENTERGY PAGE 5 OF REMIONEO PER MR 96-0069 BLIND. FLANGE ADDED PER MR 96-0069



CALC. NO. - REV. ADDENDUM AX-2H Rev. 3C ATTACHMENT NO.: JBI NO.: PAGE 6 OF

CALCULATION CHECKLIST

YES	NO	N/A	FORMAT	EDP-AA-20 SECTION
X			Cover Page completed.	6.4.1
X			Table of Contents completed (as required).	6.4.2
X			Revision History Sheet completed (as required).	6.4.3
		×	Revisions are identified with revision lines in right margin.	6.4.3
X			Applicable Documents Page completed.	6.4.4
		×	Definitions established (as required).	6.4.5
X			Calculation/revision/addendum/page numbers are identified correctly.	6.4.10
			CONTENTS	
X			Previous calculation for the required analysis exists.	6.3
×			Calculation is appropriately titled for the intended scope.	6.4.1-2
X X			Purpose and scope are clearly and adequately established.	6.4.1-7, 7.1
X			Safety classification is correct for the identified scope.	6.4.1-6
X			Topics/documents/equipment for cross-reference/retrieval are identified.	6.4.1-11
x	:		Calculation is clear and comprehensible.	6.1
x			Applicable codes, standards, etc. are identified.	6.4.4-1
X			RBS references are identified.	6.4.4-2
		X	Affected documents are identified	6.4.4-3
X			Inputs and sources are identified, appropriate, and correct.	7.2.2-1
		×	Assumptions are identified and appropriate.	7.2.2-2
		X	Inputs derived from field walkdown have been witnessed/verified	7.2.2-5
		×	Engineering judgments are identified and appropriate.	7.2.2-6
		X	Calculation methodology is identified and supported by technical bases.	7.2.3
X			Conclusion is appropriate and is justified by calculation.	7.3
		×	Confirmations are identified and indicated as required on Cover Page.	7.5.7
		X	Directions for Confirmations are included.	7.5.7-3
X			Calculation data is appropriately included, attached, or referenced.	7.4
		X	Programs and software are identified and have been verified and validated.	8.0
X			Methods/calculations use to check results are identified and included.	192
\times			Results are accurate and in accordance with the established methodology.	
		X	Certification by Professional Engineer is required.	11.7
			VENDOR CALCULATIONS	
			Calculation is performed in accordance with EDP-AA-25.	10.2
			Calculation content and format are acceptable.	10.2
			Vendor, preparer, reviewer, and approver are clearly identified.	10.3
X			Design verification review has been completed (as applicable).	

Preparer (Signature/KCN or SSN/Date):

Call Cucolic (0969)

Reviewer (Signature/KCN or SSN/Date):



ATTACHMENT I

Page 1 of 2

ES-P-002-00

Typical Design Verification Record

	Page 7 of 7
Document Number AX-2H	Revision 3C
METHOD	
Verification methods to be used:	
Design Review Qualification Testing Alternate Calculations	
DOCUMENT(S) REVIEWED: (Attach Additional Sheet((s), if needed)
Document Number Revision	Document Title
AX-2H 3	Pipe Stress Calculation
	For Main Steam Vent Line
1.	
SUMMARY OF REVIEW: (Attach Additional Sheet(s), i	f needed)
	· .
Design Verification Completed By Lothor Kulm	Date: 6-23-97
Comment Resolutions Accepted By	Date:
Engineering Supervisor John Gupli	Date: 11-19-97



Record Revision Notice

ENGMSYS	Ax-002	AX-002H_ADDC		
Department	Document No.		Page 1 of 2	
413.10.2.2	30 11/19/97	11665077	(ASG Use Only) 510056/B98-0558	
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Reason For Revision		,		
MR 96-0069 has been ins	stalled.			
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Approvals:		1		
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Originator	W444	·.	Date	
Section Supervisor	A C. fella	0658	Date 6/23/99	

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CALCULATION COVER PAGE ENGINEERING DEPARTMENT RIVER BEND STATION

AX-2H Rev. 3C

ATTACHMENT NO.:

JBI NO.: G13.10.2.2

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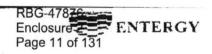
SUPERSEDES: N/A

RAJESH GUPTA, 1059

TITLE:

Stress Analysis for Main Steam Vent Line per MR 96-0069.				TS: AX-2H Rev. 3	
CALCULATION STATUS:	APPROV	VED	Am	0133/49	□ CANCELED
SYSTEM NO.: 209		- 1	MARK NO.:		
CLASSIFICATION: SAFETY RE	LATED	NO	N-SAFETY RELA	TED: QAPA	NON-QAPA
PURPOSE / SCOPE / OBJECTIVE:					
To document the changes implemented vessel head to the feedwater nozzle. A removed from the vent and spray asse	s a result of thi				
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	*			JUN 2 5 19	99
CONCLUSION:		,		SA TIC	3
The changes implemented by MR 96-0 assembly is now solely qualified by AX qualified since the removal of line ICS	K-2HL RPV noz	zie nu es the	mber 7 and the ve loads acting on the	ent and spray as	sembly remain
JUN 0 9 1998					
SOFTWARE USED FOR CALCULA	SDC	6	□ NO SDDF#:	N/A	
Manufacturer:			V		No
CONFIRMATIONS REQUIRED:					ES DNO DN/A
KEYWORDS: RPV Vent Line, MR 96-0069, RCIC, I	RPV nozzle		•		
	REVIEV	W & A	PPROVAL	<u> </u>	
(Signature/Date) Preparer Camilo Cuadra, KCN 0969	(Sig	gnatur er (No	Mn 6-23-97 e/Date) n-Safety) cation Reviewer		mpG ² /11-19-97 Enature/Date) Supervisor Iame, KCN, or SSN)

Lothar Kuhn, KCN 1105



Record Revision Notice

ENGMSYS	AX-07/A	16	
Department	Document No.	7	Page 1 of 2
613.10.2.2	4 3/35/85 Revision/Date	F#50404	ASG Use Only) ASG Use Only) Batch/Transmittal - Item No.
Revision Description			
Change calculation / adde	endum status from pending	to approved.	
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Reason For Revision		The Property of the Control of the C	
MR 96-0069 has been inst	helled		
WHY 90-0009 has been his	ancu.		7
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Approvals:			
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Originator	,		Date
Ω	1,11		1. 1.
Section Supervisor	+ (- Llle	0658	Date 6/23/99



CALC. NO. - REV. ADDENDUM AX-71AG, Rev. 4 ATTACHMENT NO .:

011307

TITLE: Pipe Stress Analysis for ICS function added through FWS system per MR 96-0069

SUPERSEDES: AX-71AG Rev. 3

SUPPLEMENTS:

CALCULATION STATUS:

APPROVED

CANCELED

SYSTEM NO.: 204, 209

MARK NO .:

CLASSIFICATION: SAFETY RELATED

NON-SAFETY RELATED: □ QAPA

☑ NON-OAPA

PURPOSE / SCOPE / OBJECTIVE:

See page \$7

RiG 11-20-97

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SDC

CONCLUSION:

This piping analysis demonstrates the acceptability of the piping system within acceptable design limits of ASME Section III Code for MR 96-0069.

SOFTWARE USED FOR CALCULATION: El YES | D NO SDDF #: 6229,400-100-001A

Version/Release No.

Manufacturer: Bechtel

Name ME101

CONFIRMATIONS REQUIRED: ☐ YES....EI NO. | CONFIRMATION COMPLETE: ☐ YES

XI N/A DNO

KEYWORDS: pipe, stress calculation, MR 96-0069

REVIEW & APPROVAL

(Signature/Date)

Preparer (Printed Name, KCN, or SSN) Rajni R. Patel (0861)

Lothar Kulm

6/18/97 (Signature/Date)

☐ Reviewer (Non-Safety) Design Verification Reviewer Lothar Kuhn(1105)

11-20-97

enature/Date) Supervisor

Rajesh Gupta(KCN) /059

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	ENTERGY

CALC. NO REV. ADDENDUM
AX-71AG, Rev. 4
ATTACHMENT NO.:
10/1307

SUPPLEMENTS:

TITLE: Pipe Stress Analysis for ICS function added through FWS system per MR 96-0069

SUPERSEDES: AX-71AG Rev. 3

CALCULATION STATUS:	□ APPROVED	⊠ PENDING	□ CANCELED
SYSTEM NO.: 204, 209		MARK NO.:	
CLASSIFICATION: SAFETY	RELATED NO	N-SAFETY RELATED: □ QA	APA ⊠ NON-QAPA
PURPOSE / SCOPE / OBJECTIVI	E:		

See page \$7 RJG 11-20-97

JUN 0 9 1998

SDC

CONCLUSION:

This piping analysis demonstrates the acceptability of the piping system within acceptable design limits of ASME Section III Code for MR 96-0069.

CONFIRMATIONS REQUIRED: ☐ YES ☑ NO CONFIRMATION COMPLETE: ☐ YES ☐ NO ☒ N/A

KEYWORDS: pipe, stress calculation, MR 96-0069

REVIEW & APPROVAL

Lothar Kulm

(Signature/Date)
Preparer
(Printed Name, KCN, or SSN)

Rajni R. Patel (0861)

(Signature/Date)

☐ Reviewer (Non-Safety)

☐ Design Verification Reviewer

Lothar Kuhn(1105)

6/18/97

(Signature/Date)
Supervisor
Rajesh Gupta(KCN) /659



CALC. NO. -REV. ADDENDUM AX-71AG, Rev. 4 ATTACHMENT NO.:

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	AX-71	IAG,	Rev. 4

ATTACHMENT NO.:

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REVISION HISTORY					
Revision No.	Paragraph No.	Description of Change			
4	All	Per MR 96-0069 the RCIC injection function was added to feedwater system by routing the renumbered RCIC injection line ICS-006-101-2 via RHR line RHS-010-65-2 into the West loop feedwater line. This revision incorporates all of former stress calc AX-76B into AX-71AG, except the portion of former line ICS-006-7-1 which was cut near node 78 and abandoned in place from node 86 to the penetration Z19.			
	-				
	4				
		-			



CALC. NO. - REV. ADDENDUM AX-71AG, Rev. 4

ATTACHMENT NO.:

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APPLICABLE DOCUMENTS

I. APPLICABLE CODES, STANDARDS, & REFERENCES (NON-RBS):

- 1. ASME Boiler and Pressure Vessel Code, Section III, 1974 edition
- ME101 Version N4 Pipe Stress Analysis Computer Program
- 3. Piping Design and Engineering by ITT Grinnell Industrial Piping, Inc. Fifth Edition
- Bergen-Paterson Pipe Supports, Catalog No. 82

II. RBS REFERENCES:

- 1. Pipe stress analysis summary, Calculation No. AX-76B Including Addenda
- EDS-ME-014, "Pipe Stress Analysis and Pipe Support Design Standard"
- Specification No. 228.000, Rev. 2 "Piping Engineering and Design"
- CHOC-EMTR-601, "Design and Installation of Seismic Small Bore Piping, Instrumentation Tubing & Supports for River Bend", dated Sept. 1980
- EMTR 605-0, "Procedure for Qualification of Flanged Joints for ASME III, Class 1, 2, and 3 Piping Systems"
- CHOC-EMDM 82-80, "Stress Intensification Factors and Stresses for Reduced Outlet Branch Connections."
- 7. CHOC-EMDM-80-4, Use of Pipe Support Stiffness Values in Pipe Stress analysis
- CHOC-EMDM-81-8, "Valve Modelling Procedure"
- LEM-34, "Frequencies and Acceptable Accelerations for Valve Qualification"
- Calculation RBI-EQ-352, "Acceptable Acceleration for Manual Valves"
- 11. Calculation RBI-EQ-416, "Valve Modelling and Acceptable Accelerations"
- Environmental Design Criteria No. 215.150, Rev. 2
- Design Criteria No. 228.010, "Pipe Stress and Pipe Support Design Criteria"
- Design Criteria No: 220.950/960, "High Energy Line Break Evaluation"
- Design Criteria No: 220,940, "Moderate Energy Line Crack Evaluation"
- Line Designation Table Change Notice LDTCN No.: 97-ICS-01
- 17. PICL Change No: AP-71-02
- PICL Change No: AP-76-01
- PICL Change No: AP-17-01
- Calculation G13.18.1.5*08, Rev. 1, "Peak Spread ARS for Seismic Events Including Curves with N-411-1 Damping"
- Calculation G13.18.1.5*09, Rev. 1, "Peak Spread ARS for SRV, Chugging, CO and Pool Swell events Including Curves with N-411-1 Damping"



CALC. NO. - REV. ADDENDUM AX-71AG, Rev. 4 ATTACHMENT NO.:

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APPLICABLE DOCUMENTS

II. RBS REFERENCES:

- 22. Calculation PX-334, Rev. 2, "1/2 SSE Structural Seismic Displacements Data and Input Procedure for Piping Analysis"
- Calculation PX-SB-316, "Pipe Stress Calc. for Vent, Drain & Instrument Connections"
- Calculation PX-SB-304, "Pipe Stress Calc for Vent/Drain Connections"
- 25. Calculation AX-17D, "Pipe Stress Calc. For F.W. PPG. in Reactor & Aux. Bldg"
- Calculation PX-873, "LPCI Pump Discharge Lines Water Hammer Analysis with Trapped air"
- 27. Calculation PX-1037, "Water Hammer Analysis on Reactor Core Isolation Cooling Piping Due to Pump Trip"
- -28. Calculation PX-897, "RCIC Pump Discharge Lines Water Hammer Analysis with Trapped Air"
- Calculation PX-1015, "Fluid Transient on RHS Piping Associated with Check Valve Slam Following a Pump Trip"
- Calculation PX-1024, "Fluid Transient on RHS Piping Associated with Suppression Pool Cleaning Mode with Trapped Air"
- 31. Calculation PX-888, "LPCI Pump Discharge Line Water Hammer Analysis with Trapped Air for Mode B"
- 32. Calculation PX-1005, "Water Hammer Analysis on RHR Piping Line During Shutdown Cooling"

III. AFFECTED DOCUMENTS:

DOC. NO.	REV. NO.	DOCUMENT TITLE	AFFECT (Superseded, Revised, Initiated, etc.)	CHANGE DOC.
G13.18.10.3*221	0	Pipe Support	Initiated	MR 96-0069
Z-76-2073A	0A	Pipe Support	Supplements	MR 96-0069
AX-76B	2	Pipe Sress	Revised	MR 96-0069



CALC. NO. - REV.

AX-71AG, Rev. 4

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PURPOSE / SCOPE / OBJECTIVE

MR 96-0069 reroutes the RCIC Injection Line from its original connection to the RPV Head to the FWS system through a short portion of the RHS piping covered by this AX. The objective of Revision 4 is to evaluate the subject piping for the new function of ICS injection in addition to the existing function of the RHS piping. A new run of 6" piping with a new check valve connecting line RHS-010-65-2 to the existing ICS piping is to be added. Since some of the original ICS piping beyond the diversion point is left inplace and was downgraded to ASME Class 2, the active ICS line was renumbered from ICS-006-7-1 to ICS-006-101-2. The design pressure in the lines RHS-010-65-2 and WCS-004-22-2 upto the first check valve were reduced from 1885 to 1525 psig. The piping is to be evaluated for thermal, fluid transient, dynamic, and hydrodynamic events associated with both the RHS and ICS system operation.

As a consequence, the piping of stress problem AX-71AG is now being extended top cover the ICS (RCIC) piping back to its nearest anchors. The added piping comprises all of the existing stress problem AX-76B, except the portion beyond the diversion point of which the first 6 ft is remained and the remainder abandoned in place is evaluated in calcualtion AX-76B. There are no support configuration changes; the rerouted ICS runs unsupported from its last existing support to the new check valve at the new junction to the RHS line.



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AX-71AG, Rev. 4

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METHOD OF ANALYSIS

The ICS piping was originally analyzed for hydrodynamic loads (SRV etc.) because it was connected to the containment penetration Z-19. Now it is connected to the reactor building via the feedwater line and therefore at least some of it has to be analyzed for the ARS transmitted from that end. The fluid transient due to ICS injection is also changed. There is no other physical or operational change to this stress problem.

The ICS piping in this stress problem includes CNS piping and tie back into RHS line of AX-71AG which previously in AX-76B had been modeled as a fictitious anchor. The thermal operating conditions of the incorporated piping from the AX-76B model have to be incorporated into those of the existing AX-71AG model.

ME101

The piping was analyzed using ME101, Version N4.

The input files have been annotated with comments describing the changes made from the existing ME101 input file. Such changes cover incorporation into the coding of various notes made in previous revisions as well as the changes/additions due to this Revision.

Fluid Transient Input

The piping experiences a fluid transient due to RHR injection into the feedwater line and another FT due to ICS injection into the feedwater line via the RHR line. In order to simplify the analysis by executing it in a single ME101 run, the second fluid transient is appended after the end of the first fluid transient, so that the piping response from the first transient has ceased and then responds to the second transient. The maximum loads of this load case correspond to the envelope of the individual fluid transient loads. The RCIC transient input file is derived from PX-1037 and run from t=0 to t=0.6 sec; the RHR transient input file is derived from PX-1015 and is delayed 0.6 sec. To implement this, the time histories of the segments affected by the RHR transient are delayed.

The fluid transient model of PX-1037 uses a preliminary routing of the RCIC line. The final routing breaks up the long north-south segment into a short segment running at 45 deg to this segment and the remainder. It also corrects the length of the west-east segment at El. 119'-9" from 9 ft to 3 ft. Addendum A to the PX calculation reconciles these differences by recommending an adjustment in the application. This is implemented by using conservative multipliers for the existing curves as follows:

LENGTH ADJUSTMENT

SEGMENT	PRELIMINARY LENGTH (FT)	FINAL LENGTH (FT)	FT MODEL PIPE SEGMENT	CURVE	APPLIED AX NODE	CURVE MULTIPLIER
X75-X78	9	3	6-6	36	X75	1.0
X79-X80	-	2.2	N/A	38	X79	0.5
X80-X82	12	9.8	7-2	38	X80	1.0



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METHOD OF ANALYSIS (cont.)

Response Spectra Input

The ARS input differs from Revision 3 mainly in that N-411 Code case spectra are now replacing 1% 2% spectra; but in addition, the Shield Building spectra are now also included, and no change to the piping mass points is needed to separate regions affected by Reactor Building spectra from those which experience only Auxiliary Building spectra.

The Reactor Building spectra that are picked up from the feedwater line connection have been input as follows:

The FWS penetration for AX-17D is at elevation 121'-6" and is rigidly connected to the drywell wall and the Shield Building wall at that elevation. The drywell wall is a rigid 6 way anchor (AX-17D node 200), while the Shield Building support is a 2 way guide (AX-17D node 183). The Jet Impingement wall also provides 2 way guide support to the Feedwater line at the Auxiliary Building (AX-17D node 280). The Feedwater line runs in the plant global X direction.

OBEA analysis uses the alternate Auxiliary Building displacement curves.

High energy Line Break has been addressed in the calculation no: 228.850-CBC-2024 Rev. 0. Since the flooding has been addressed in the HELB calculation, Moderate Energy Line Crack need not to be considered for this calculation.



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CONCLUSION

The completed analysis shows that the stresses and reactions on the subject piping are within acceptable limits in accordance with the requirements of the ASME Section III Code.

Valve accelerations are within acceptable limits.

Functional capability has been assured.

Stresses on vents, drains, and test connections are within acceptable limits.

Sleeve clearances are adequate.



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NOTES

- 1. The Coordinate system for all data expressed in the global system is defined as follows:
 - (-X) Is plant "called north" per piping drawing
 - (Y) Is vertical "up"
 - (Z) According to right hand rule

The local system is defined as follows for straight piping members:

- (X) Is along the axis of the pipe, positive in the direction of coding.
- (Y) In the plane of the local X-Axis and the global +Y-Axis unless the local X-Axis is aligned with the global Y-Axis; then the local Y-Axis will be parallel to the global X-Axis.
- (Z) According to right hand rule.
- 2. Forces and moments on restraints, supports, and equipment are those imposed by the piping systems (i.e., actions, not reactions).
- 3. Unless otherwise noted, forces and moments reported at an inline anchor represent the total forces and moments acting upon that anchor considering contributions from piping on all sides of that anchor. Reactions reported on terminal anchors are due to piping shown in this report only. For total reactions, values must be combined with those from continuation summaries shown on isometrics.
- Unless otherwise indicated, stresses at intersections have assumed an unreinforced branch connection.
- Piping support loads may be found in pipe support summary in Attachment B.
- For lines requiring seismic analysis, the reactions reported are cyclic in character, and reactions must be considered as acting in either the positive or negative direction.
- 7. On systems requiring time-history dynamic analysis for flow-induced transients, reactions and deflections reported are the maximum values in magnitude. Reactions and deflections that may occur in opposite directions are not reported in this summary, but are available if required.
- 8. Abbreviations

- The magnitude of the forces from flow transient PX-1015 are much larger than those from flow transient PX-888, PX-873, PX-1005 and PX-1024. Therefore, flow transient PX-1015 is enveloping the other transient cases and is the only one to be considered in the analysis.
- ME101 model and isometric has a small dimensional discrepancy between nodes 330 and 805 which
 is insignificant.



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ASSUMPTIONS

The weight of the new 6" check valve is assumed to be 430 lbs.



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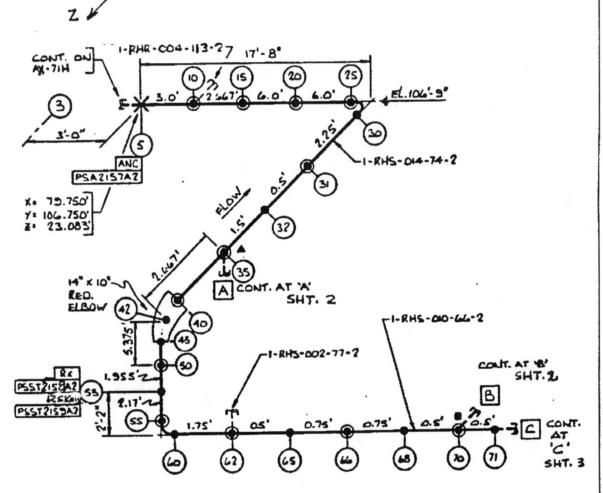
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WORK SKETCH:

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SHT. 1 0 9





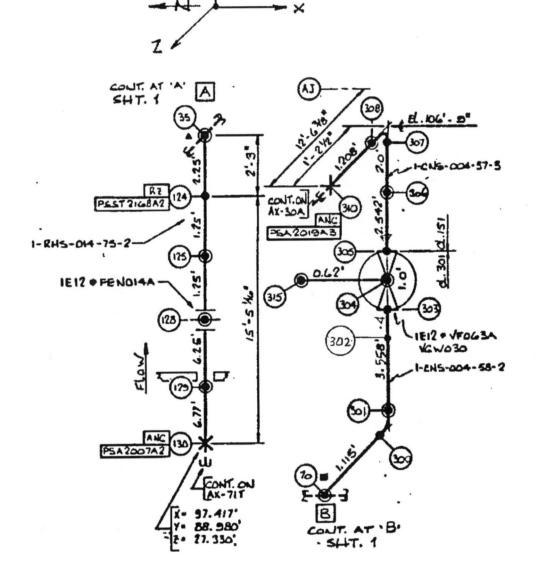
CALC. NO. - REV.

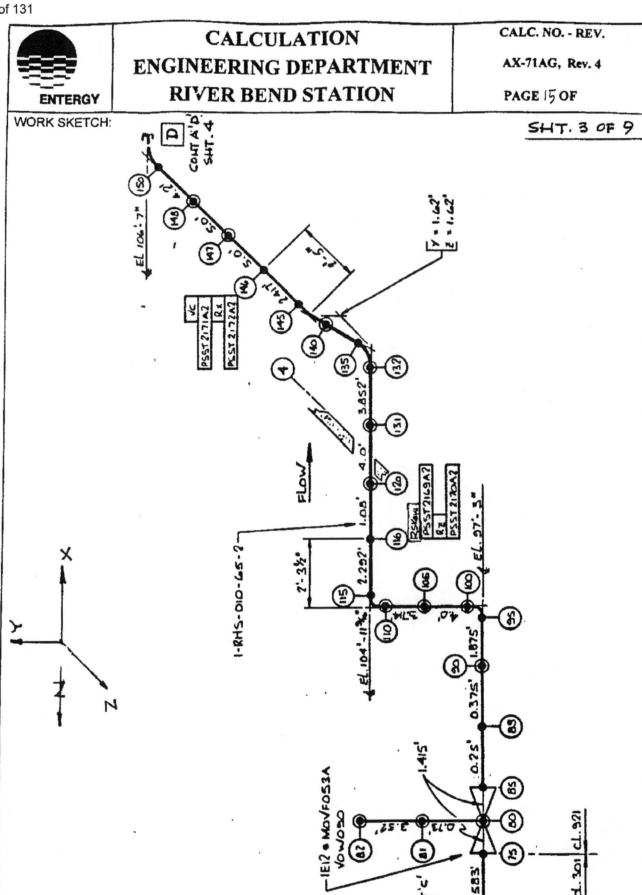
AX-71AG, Rev. 4

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WORK SKETCH:

SHT. 2 OF 9







CALC. NO. - REV.

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PAGE 16 OF WORK SKETCH: SHT. 4 OF 9 SHT.5 RSKEW B-B RSKEW PSST 2177A2 1-RHS-010-65-2-RSKEW PSST2174A2 RSKEL PSST 2175A2 FLOW. 1.8' 1.75 CONT. AT 'D'

SHT. 3



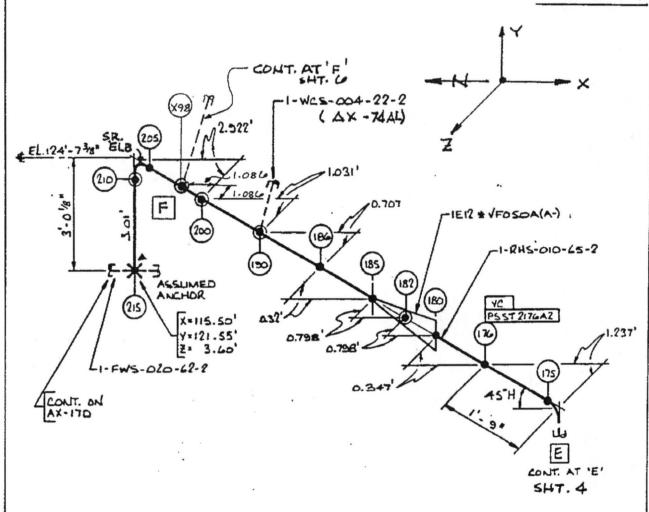
CALC. NO. - REV.

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WORK SKETCH:

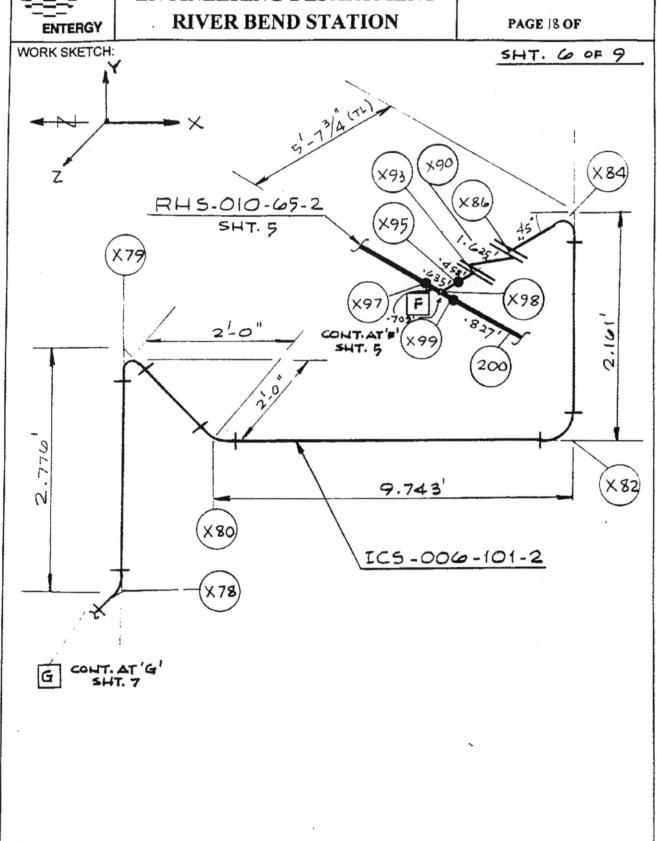
SHT. 5 01 9





CALC. NO. - REV.

AX-71AG, Rev. 4



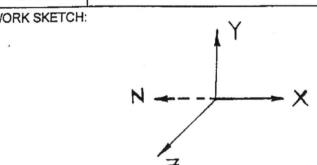


CALC. NO. - REV.

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SHT. 7 of 9

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DIRECTION COSINE OF SKEW SUPPORTS

NOD	E	Yx	Yy	Υ _Z
47	RSK	_	-9973	0729.
48	SSK	9976	-	.0690
71	ssk	.9848	-	-1736
100	REK	- 1194	.9928	_=

NOTE: ALL SNUBBERS ... PSA-3

AV6. t = . 598 G 1.583 SSKEW 1. 117 4.75

4.75

V.C.

HVY WALL

1.25 HVY WALL SZ PSA3 RZ AVE. t = . 635 -1.25 SSKEW PSA3 4.0 3.656 EL. 104.75 <u>(59</u> RSKEW HVY WALL AVG. t . . 620

COUT. AT 'H' SHT. 8

ADD 'X' IN FRONT OF NODE POINT NOTE: FOR MEIOI MATH MODEL



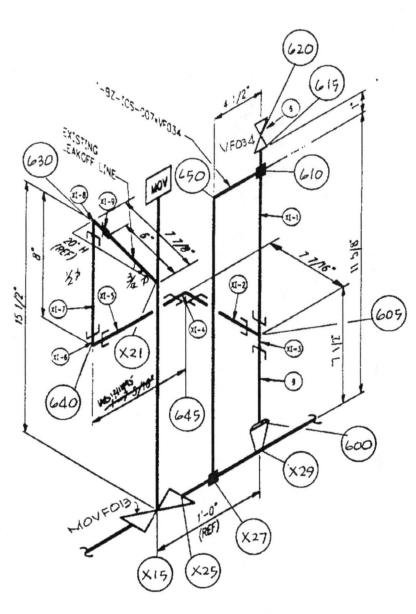
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WORKSKETCH

SHT. 9 OF 9



DETAIL "A" (SHT.8)

									A	LOWAB	F				
	COMPONENT	NOM PIPE	WALL	CLASS		SIGN ITIONS		ATING	ı ~	STRESS (KSI)			WE 10	115) /F	r
ITEM	IDENTIFICATION	SIZE (IN)	THKN (IN)	MATERIAL AND GRADE	TEMP (F)	PRESS (PSI)	TEMP (F)	PRESS (PSI)	sc	SH	SA	PIPI	FLUIO	INSU	101AL
1	RHS-014-74-2	14.	•375	301 SA106GRB	358	500	298	425	15.	15.	22.5	54.6	59.7	4.61	(18.9)
2	RHS-014-75-2							,							1
თ	RHS-010-66-2	10-75	-365		480	500	298	425				40.5	34.1	3.37	77.97
4	RHS-010-65-2		·843	921 SA106GRB	450	1525	1215	1348				<i>89.</i> 2	28,0	4.79	121.99
5	CNS-004-57-3	4.5	-237	151 SA106GKB	120	135	100	80				10.79	5.51	-	16.30
6	3/4" SCH 160	1.05	.218	911/921 SA 10648B	450	1885	300	1348				1.94	·128	-	2.04
7	3/4" SCH 80	1.05	-154	301 SA106GRB	358	500	298	425				1.47	188	-	1.658
8	ICS-006-101-2	6-629	-432	911 SAIOGGEB	450	1525	300	1348				2857	11 29	-	39.84
9	ICS-006-8-2	V	.562	921 SA 106GRB	170		170	1190				36.42	10.29	-	46.69
10	RHS-004-90-2	4.5	·337	911 SA 106 GRB	358		300	1215				14.98	4.98	-	19.96
11	RHS-004-113-2		.237	301 5A106GRB		500		425				10.79	5.51	-	16.3
12	CNS-004-52-2				,			583				10.79	5.51	1.63	17-93
13	CNS-004-115-3	1		151 SA1064RB	120	135	100	115				10.79	5.51	-	16.3
14	1/2" 55.4 80	-840	. 147	301 SA 106GEB	358	500	300	425				1,09	101	-	1.19



ENGINEERING DEPARTMENT RIVER BEND STATION CALCULATION

BASIC ANALYTICAL DATA

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BASIC ANALYTICAL DATA

	COMPONENT	100	NON	SWEC	CONDITIONS	TONS	OPERATING	MUM	₹	STRESS	.		*	WE ICHTS	
	IDENTIFICATION	13 (K)	Z Z	AND CRADE	TEMP (F)	PPE SS (PSJ)	FE (F)	PRESS (PSI)	S	₹ ¥	₹s	ą.	FLUID	(LIBS)	IOTAL
-	ELBOWS NP X31-X32	6.425	808.	911 SA 106 GEB	575	1525	300	1348	15.	-5.	22.5			2	58.74
-	X 43-X45		. 633									1		i	50.29
	×49-×50		.639												50.39
	09x-65X		029.												49.64
	X 73-X75	-	.598		->-	-	-	-		-					48.53
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				VALVE	DATA			*
NODE NO.	NOMINAL SIZE(IN)	VALVE NO.	S&W DESCRIPTION NO.	TYPE	S&W FILE NO.	VALVE LENGTH (FT.)	VALVE WEIGHT (LB.)	OPER. WT. (LB.
80	10	IER+ HONFOSSA	YOW-090-6-275	blobe	12210 - 228 - 2/2. 047 - 083 E	2.83	1900	1100
182	10	IEIZ & VFOSOA	VOW-090-D-27	CHECK	12210.228.211 · 049 · 001E	2.256	1850	_
304	4	IER+VF063A	VGW-030-C-2KS	GATE	12210-228-211- 049-007K	1.0	350	_
200	3/4	IRHS+VIIZ	V05.150-G-2	GLOBE	122/0.228-213- 058 - 011L	.334	١٥	
200		IRHS #VII3						_
90		1E12 + VF058A	VOS_150-G-2					_
90		IEIR OVFOSAA			ļ	+	+	_
31	+	IRHS=V129	VOS-060-B-2		12210-228-213- -58-008T	.327	6	
128	1/2"	RHS#VSI	V05060-5-2		228 - 213 - 453-4	087		
128	1/2"	R#5# V52	11		п			
50	12"×2"	1E12 * RV025A			247-521-207-0265		45.0	_
X90	6"			CHECK		'	430	_

	V.		,	VALVE	DATA			
NODE NO.	NOMINAL SIZE(IN)	VALVE NO.	S&W DESCRIPTION NO.	TYPE	S&W FILE NO.	VALVE LENGTH (FT.)	VALVE WEIGHT (LB.)	OPER. WT. (LB.)
20/22	6	VGW090-A-145C	1E51*MOVF013	GATE	228.212-047-0039	1.666	750	360
1370	6	KWOPO D IMSA	156/#AOVEOGS	CHECK	208-012-008F	1.166	307	123
350/ 352	4	VON090-A-1M59	IEIR#MONFO23	GLOBE	218.212-047-084	1.166	325	210
520	4	VCW090-D-IM	IEI2*VF019	CHECK	228.211-049-00lF	1.166	145	_
520	4	YGW080-C-2KS	1E12* VF086	GATE	228.21-019-007K	1.0	-	350
725	3/4"	V05150 4 2	1265#VF084	GOBE	208.R13-058-011L	.334	10	
745	3/4	VOW/50-G-2	IICS*VF085	GLOBE	208.213-058-01L	.334	10	
610	3/4	VOW150-G-2M	1E51*VF034		288.213-058-044	.292	10	_
625	3/4	VOW150-G-2M	1651#VF035			.292	10	_
1220	3/4	VOW/50-G-2M	13CS # 1302			.3/25	10	
1820	3/4	VOW 150-9-2M	1715#V100			.333	10	-
815	3/4	VOW150-G-2M	IEPZ#VFOGI			.3125	10	-
830	3/4	VOW150-9-2M	1E12#VF062			.3/25	10	_
915	1/2	V05060-B-2	IRHS* V74		288.213-058-0081	,25	6	_
965	1/2	VOSOX60-B-2	IRH5*V73	1	228,213-058-0081	.2.66	6	_



CALCULATION

CALC. NO. - REV. AX-71AG, Rev. 4

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CALCUL	ATION	SHEET
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		CALCULAT	ION IDENTIFICATION N	JMBER	26
J. O.	OR W.O. NO.	DIVISION & GROUP	CALCULATION NO.	OPTIONAL TASK CODE	PAGE
	15510	EH/NP(T)	12210-AX-71AG	N/A	
255			REV.4		
REF.					
	CALCULA	TION OF THE EQ	UNALENT SECTION	FOR VALVE YOKE (1.	E12 * HOVFO53
8 &/	VALVE	SIZE = 10"	が	1100 183 THOTOR	OPER. C.G.
120	VALVE	3/12 2 /0			
8/	VALVE	WT = 2350 #	(F+ VH)	-51	
23				COMBIN	ED VALVE/OPER
	- HOTOR O	PER. WT. = 1100#		4=22-22" 1000 1h	
*	VALUE	C V (2350+1100)	x27.27-//00 X51	1900 lb	
	VALVE C.	6. $\chi = \frac{(2350+1100)}{23}$	50	YIT WALV	£ 6.0.
		= 8.75	3-7	700 00 85	,
		***	_		Υ
	: = DI.	STANCE 81-82		THE RESIDENCE IN COLUMN 2 AND ADDRESS OF THE PARTY OF THE	****
		* 8.75 = 42.25 *		N	x
-	= 2/	- 0.13- 42.23		12	GLoBA
	HOHEN	T OF INERTIA F	OR YOKE		AXES
	(FREQUE	NCY=72HZ) =	1 (27188/21)		Makes on the stand street correlates where
				Many no er	
			(211 x 72 x 42-25/	27.9×106×3864	4:
		. 5	25.0 in "	and the same of th	
				-	
	$I_{x} = I_{z}$	$=\frac{\pi}{6\pi}(0,4-1)$	0,4)		
	~ 220 ·	0= 1 (10.754	- Di)		
	: D. :	7.18 iN			*
	. S. THIC	KNESS OF EQUIL	MIENT SECTION.	OF PIPE O.D. 10.	75"
		· · · · /	2.75 -7.18	705	
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			-	, ar summed form, all committees all settingers in	
	NOTE	FROM REF # 23	OPER. NT (INCLU	DING PARTOF YOU	(E) = 1300 t
		L	VALVE WT = 220	· · · · · · · · · · · · · · · · · · ·	
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		BY ENG. JUDO	CITENIS MILL	DATE INSIDATERI	- / 5/ - 50
		_		E THAT THE NIETA	WE TO C
		ONTHE ANAL	YSIS CONSIDERIN	U JAMI ING DISIM	wee 10 Cit
		ON THE ANAL.		ECREASE DUE TO EFFE	

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STORE & WEBSTER ENGINEERING CORPORATION CALCULATION SHEET



	CALCULATI	ON IDENTIFICATION N	JMBER	27
J.O. OR W.O. NO.	DIVISION & GROUP		OPTIONAL TASK CODE	PAGE
12210	E.M./Pipe Stress	AX-71AG		

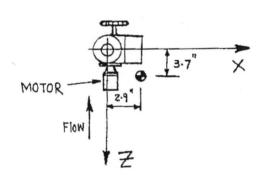
REV. 4

VALVE C.G. (Ref.

1 MOV FOI3 (NODEXIS)

VALVE ONLY: - - - - - - - - - - - Y = 10.14" = .845

OPERATOR ONLY :



$$X = 2.9'' = .242'$$

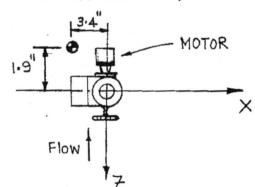
$$Y = (5.2 + 30.437) - 10.14$$

= 25.497 = 2.125

@ MOV FO23 (NODE 345)

VALVE ONLY : - - - - - Y = 63 " = .516'

OPERATOR ONLY



$$X = -3.4'' = -.283'$$

$$Y = (6.0 + 18.562) - 6\frac{3}{6}$$

= $18.375''$ = $1.531'$

$$Z = -1.9'' = -.158'$$

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CALCULATION ENGINEERING DEPARTMENT RIVER BEND STATION

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CONCENTRATED WEIGHTS

ITEM	COMPONENT DESCRIPTION	WEIGHT (LBS)	NODE POINT
1	VALVE BODY, 1E51*MOVF013	690	X20
2	VALVE OPERATOR, 1E51-MOVF013	420	X22
3	VALVE, 1E51*VF034	10	610
8	VALVE, 1E12*VF019	145	320
9	VALVE, 1E12*VF061	10	815
10	VALVE, 1E12*VF062	10	835
11	VALVE BODY, 1E12*MOVF023	280	350
12	VALVE OPERATOR, 1E12*MOVF023	255	352
13	SUPPORT COMPONENT WT.	10	382
14	SUPPORT COMPONENT WT.	66	410
15	FLANGE WEIGHT	30	415
16	FLANGE WEIGHT	30	417
17	VALVE, 1E12*VF086	350	520
18	1E12-FEV014A(A-)	450	128
19	Valve Operator 1E12*MOVF053A	1100	82
20	Valve Weight 1E12*MOVF053A	1900	80
21	1E12*VF050A	1850	182
22	Check Valve	430	X98
23	1E12*VF063A	350	304

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CALCULATION ENGINEERING DEPARTMENT RIVER BEND STATION

CALC. NO. - REV.

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RELATIVE THERMAL ANCHOR DISPLACEMENTS

SUPPORT PT.	SUPPORT TYPE	LOCATION	THER MODE	SUPPORT ATTACEMENT ELEVATION (ft)	AX (in)	AY (in)	∆ Z (<u>i</u> n)
215	ASSUMED	AUX.	1-8	121'- 65%	1-631	- 100	.092
			2		-160	-009	007



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RELATIVE SEISMIC ANCHOR DISPLACEMENTS

SUPPORT PT:	SUPPORT TYPE	LOCATION	SUPPORT ATTACHMENT ELEVATION (ft)	△ X(in)	AY (in)	△Z(in)
5 *	ANCHOR	BLPG.	106-9"	-27	. 124	-240
53 *	RX ,RZ		99'-5"	-25	-	-23
116 🛪	VC, RZ		104'-119/6	_	-124	.235
146 *	VC,RX		106-7"	-265	-124	_
166 *	R× , RZ		108-45	.270	1	.240
169 *	RX、RZ		123-05/	-325	1	.275
176 *	vc		124-73/8"	_	- 1245	_
215 **	ASSUMED		121- 6 5%	.690	. 240	.295
124 *	RZ		104-6"	-		-235
130 *	ANCHOR		88'-1134"	-21	-1235	-205
310 *	ANCHOR	+	106-9"	-27	124	-24

REF: PX-334

^{**} Reference for the seismic anchor movements: Absolute/displacements of the Auxiliary Building. All restraints have Auxiliary Building displacement except the fictitious anchor for the feedwater line (node 215) which has Reactor Building and orbital motion relative to Auxiliary Building restraints.

STONE & WEBSTER ENGINEERING CORPORATION

CALCULATION SHEET

	CALCULATION IDEN	TIFICATION NUMBER		31
JO. OR WO. NO. 12210	DIVISION & GROUP NP(C)	CALCULATION NO.	OPTIONAL TASK CODE	PAGE

REF.

DESIGN INPUT (CONT'D)

RELATIVE SEISMIC ANCHOR AND/OR SUPPORT DISPLACEMENTS

-(* SEE CALCULATION ON THE FOLLOWING PAGE(S) 55-6/ .)

NODE	SUPT.	BUILDING	SUPPORT ATTACHMENT		ACEMENTS *		REMARKS
NO.	TYPE	(STRUCTURE)	ELEVATION (FT)	DX	DY	DZ	
×1	ANCHOR	AUX. BLOG.	91.719	0.2550	0.1800	0.2370	44
X 9	RX		97.25	0.2750			
×47	RSKEW		104.75		0.1810	0.2680	
×48	SSKEW		104.75	0.3070		0.2680	
X 52	5₹		104.75			0.2680	
× 61	RZ		106.0			0.2720	
× 71	Sskew		118.167	0.3620		0.3030	
×76	Vc		119.75		0.1810		
-90	RZ		120.75			0.3100	
100	RSKEW		122.526	0.3800	0.1810		
121	RZ		182,526			0.3130	
122	Ve		122.526		0.1810		
141	VE, RZ	+	122,526		0.1810	0.3130	
220	VE, RZ	SHEED BLOG	122.60		0.4110	0.8530	
285	ANKHOR	STEEL CONT.	122.76	0.9870	0.4390	0.8690	

STONE & WEBSTER ENGINEERING CORPORATION

CALCULATION SHEET

	CALCULATION IDEN	NTIFICATION NUMBER		32
JO. OR WO. NO. 12210	DIVISION & GROUP NP(C)	CALCULATION NO. AX-71AG REV.4	OPTIONAL TASK CODE	PAGE

REF.

DESIGN INPUT (CONT'D)

RELATIVE SEISMIC ANCHOR AND/OR SUPPORT DISPLACEMENTS

(* SEE CALCULATION ON THE FOLLOWING PAGE(S) 55-6/

NODE	SUPT.			SUPPORT ATTACHMENT	DISPLAC	CEMENTS *		REMARKS
NO.	TYPE	(STRUC	TURE)	ELEVATION (FT)	DX	DY	DZ	
-260	AVEHOR	DRYWELL		123.17	1.0460	0.4780	0.9310	
338	Rove	AUX.	BLOG.	98.75	0.2800	0.1800		
375	RZ			100.385			0.2570	
380	Rx			103.677	0.3020			
382	54			104.72		0.1810		
410	Ve			106.75		0.1810		
445	ANCHER			106.75	0.3150	0.1810	0.2740	
537	VegRz			98.39		0.1800	0.2520	
575	ANCHOR	1		103.016	0.3000	0.1800	0.2640	



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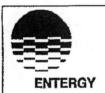
PAGE 33 OF

Basic Analytical Data Thermal Modes:

ICS and RHS is injecting - or not injecting - while FWS is hot or old, i.e, while feedwater connection node 215 has either the static displacement, for feedwater hot or cold. Injection at 122 deg F bounds "no injection" at 120 deg F.

Modes 1-9 represent the different flow conditions. Modes 6-9 were not analyzed. They can be considered bounded by modes 1-5.

Region	Mode (Temperature °F)									
	1	2	3	4	5	6	7	8	9	Piping Line No.
1	122	40	40	298	185	122	40	122	122	RHS-014-74-2
2	122	40	40	298	40	122	40	122	122	RHS010-66-2
3	122	40	40	298	40	122	40	122	122	RHS-010-65-2
4	425	40	425	298	425	122	40	170	170	RHS-010-65-2 (AFTER CK)
5	425	40	425	298	425	122	40	170	170	ICS-006-101-2 (AFTER CK)
6	122	40	40	122	40	122	40	170	170	ICS-006-101/7-2, RHS-004-90-2
7	122	40	40	122	40	122	40	170	170	ICS-006-8-2
8	122	40	40	122	40	122	122	122	122	RHS-004-113-2, CNS-004-52-2
9	122	40	40	100	40	122	100	100	122	CNS-004-415-3
10	122	40	40	298	185	122	40	122	122	RHS-014-75-2
11	122	40	40	298	40	122	40	122	122	CNS-004-58-2
12	122	40	40	100	185	122	40	122	122	CNS-004-57-3
	425	40	425	425	425	425	425	425	70	Feedwater Line Temp



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DYNAMIC LOADS FLUID TRANSIENT FILE NAMES:

FROM	TO	FILE NAME	
678	1178	/home3/ywu90/px1037/frcic.29	
1180	1680	/home3/ywu90/px1037/frcic.30	
1682	2182	/home3/ywu90/px1037/frcic.31	
2184	2684	/home3/ywu90/px1037/frcic.32	
2686	3186	/home3/ywu90/px1037/frcic.33	
3188	3688	/home3/ywu90/px1037/frcic.34	
3690	4190	/home3/ywu90/px1037/frcic.35	
4192	4692	/home3/ywu90/px1037/frcic.36	
4694	5194	/home3/ywu90/px1037/frcic.37	
5196	5696	/home3/ywu90/px1037/frcic.38	
5698	6198	/home3/ywu90/px1037/frcic.39	
6200	6700	/home3/ywu90/px1037/frcic.40	
6702	7202	/home3/ywu90/px1037/frcic.41	
7204	7704	/home3/ywu90/px1037/frcic.42	
7706	8206	/home3/ywu90/px1037/frcic.43	
8222	8721	/home3/riverbend/ft/px1015/p1015f04.017	
8723	9222	/home3/riverbend/ft/px1015/p1015f04.018	
9224	9723	/home3/riverbend/ft/px1015/p1015f04.019	
9725	10224	/home3/riverbend/ft/px1015/p1015f04.020	
10226	10725	/home3/riverbend/ft/px1015/p1015f04.021	
10727	11226	/home3/riverbend/ft/px1015/p1015f04.022	
11228	11727	/home3/riverbend/ft/px1015/p1015f04.023	



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DYNAMIC LOADS FILE NAMES

Uniform ARS method is used. All supports except the fictitious anchor at node 215 are in the Auxiliary Building; therefore, the seismic ARS for the elevation range from highest to the lowest Auxiliary Building attachments are input. Node 215 transmits the Reactor Building ARS of Drywell and Shield Building to the piping model. The feedwater penetration Z-3A is attached to these structures at EL. 121.85'. For the seismic ARS, a structural point above and below have to be input; for the hydrodynamic ARS, the structural model has a node at the penetration elevation.

TABLE 1

Structural Attachment	Elevation (FT)	Curve	Number
		OBE	SSE
Auxiliary Building	70	auxbon01.ace	auxbsn02.ace
Auxiliary Building	95	auxbon03.ace	auxbsn04.ace
Auxiliary Building	114	auxbon05.ace	auxbsn06.ace
Auxiliary Building	141	auxbon07.ace	auxbsn08.ace
Drywell	114	rbdwon41.ace	rbdwsn42.ace
Drywell	133	rbdwon43.ace	rbdwsn44.ace
Shield Building	112.67	rbsbon03.ace	rbsbsn04.ace
Shield Building	133	rbsbon05.ace	rbsbsn06.ace

TABLE 2

Structural Attachment	Elevation (ft)	Curve Number						
		SRV1V	SRV16V	снсо	PS			
Drywell	121	*	rbdwan43.ace	rbdwcn43.ace	rbdwpn43.ace			
Shield Building	121	rbsbtn03.ace	rbsban03.ace	rbsbcn03.ace	rbsbpn03.ace			

* BOUNDED BY SEVICY CASE



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MAXIMUM STRESS LEVELS

ASME SECT. III, CLASSES 2 6 3

DESIGN CONDITION	LEVEL		CATION MAXIMUM ELEMENT	MAXIN COMPUT STRESS (ED	STR		COMPUTED ALLOWABLE
SUSTAINED LOADS EQN. 8		X25	X27 X25	716	56	Si 150	H 000	.478
OCCASIONAL LOADS EQN. 9	В	X35	X35 280	1567	4	1.2		.971
OCCASIONAL LOADS EQN. 9	c	303	302 303	2069	2	1.8		.766
OCCASIONAL LOADS EQN. 9	D	303	302 303	2101	2.4 SH 1011 36000		.584	
THERMAL EXPANSION EQN. 10		505	21\$ M 505	2209		225	500	.982

Based on industry practice the regions which are isolated from the Reactor Building by at least 3 equivalent dynamic restraints in each direction can be considered unaffected by the Reactor Building response spectra. These regions experience only OBE and SSE ARS, and not hydrodynamic ARS. The boundaries of the RB affected region are beyond nodes (or exclude boundary nodes) 116, X1, and X338. Therefore, the stresses in these regions can be corrected to exclude the hydrodynamic loading. The maximum stresses for level C and D were not corrected, since they are within the allowables.

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FUNCTIONAL CAPABILITY EVALUATION:

The maximum stress level for Equation 9 emergency condition and Equation 9 faulted condition are below the Equation 9 Normal/Upset allowable, and OD/t is less than 50 for piping associated with this calculation. Therefore, the piping system ensures the functional capability. No further detail evaluation is required.

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PIPE BREAK LOCATIONS

The only applicable region is from the feedwater connection at node 215 upstream to the first check valve in the RHR line, the RCIC line and the RWCU line (not modelled) i.e., nodes 185-215 and X93-X98.

Conclusion:

Besides the mandatory new break at node X93 (the interface of High Energy and Moderate Energy piping - see calculation no: 228.850-CBC-2024 REV.0), no new High Energy breaks need be postulated. The stresses in the High Energy region are well below the allowbles.



CALCULATION

CALC. NO. - REV. AX-71AG, Rev. 4

VALVE IDENTIFICATION NUMBER: 1E12*MOVF053A

Loading	Acceleration in	Global Coord	inate System	Acceleration in	Global Coord	inate System	Accept	Acceptable Acceleration (g)		
Case	at C	.G. of operator NP 82	(g)	at C.C	G of Valve Bod NP 81	y (g)				
	Ax	Ay	Az	Ax	Ay	Az	Ax	Ay	Az	
OBEI	0.162	0.131	0.238	0.094	0.13	0.237	2	2	. 2	
SSEI	0.323	0.259	0.475	0.187	0.259	0.486	2	2	2	
SRV1V	1.137	0.783	1.277	0.617	0.781	0.347	3.5	3.5	3.5	
SRV16V	1.563	1.112	1.759	0.858	1.11	0.545	4	4	4	
CHUG/CO	0.411	0.153	0.224	0.198	0.151	0.307				
PS	0.115	0.086	0.103	0.058	0.086	0.034	4	4	4	
FT	0.093	0.083	0.151	0.094	0.083	0.087	4	4	4	
LIDOET	4 57440004	4.40070477	4 704 40004	0.00000700	4.40000450					
UPSET	1.57412261	1.12276177	1.78143931	0.86823729	1.12066453	0.6006355	6	6		
FAULTED	1.65071742	1.15495584	1.84192372	0.90508176	1.15276667	0.79689334	7.21110255	7.21110255	7.2111025	

Valve Spec No: 228.212-047-083E

LEGEND:

UPSET = SRSS(OBEI,FT,SRVMAX)

FAULTED = SRSS(SSEI, FT, SRVMAX.,MAX(CHCO, PS))

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CALCULATION

AX-71AG, Rev. 4

CALC. NO. - REV.

		AVI	AL IDENTIFIC	CATION NUME	JEIN IEIZ VI	019				
Loading	Acceleration in	Global Coord	linate System	Acceleration in	Global Coord	linate System	Accept	Acceptable Acceleration (g)		
Case	at C.	G. of operator	(9)	at C.0	G of Valve Bod	ly (g)				
		NP			NP 320					
	Ax	Ay	Az	Ax	Ay	Az	Ax	Ay	Az	
OBEI				0.159	0.161	0.119	2	2		
SSEI				0.315	0.32	0.236	2	2		
SRV1V				1.306	1.636	0.958	3.5	3.5	3.	
SRV16V				1.717	1.921	1.127	4	4		
CHUG/CO				1.086	1.37	0.884				
PS				0.148	0.188	0.114	4	4		
FT				1.377	1.849	0.6	4	4		
UPSET	0	0	0	2.20669413	2.67113515	1.28229872	6	6		
FAULTED	0	0	0	2.47443711	3.01468771	1.5707581	7.21110255	7.21110255	7.2111025	

Valve Spec No: 228.211

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CALCULATION

CALC. NO. - REV.

		VAL	VE IDENTIFIC	ATION NUMB	ER: 1E12*VF0	63A			
Loading Case	Acceleration in at C	Global Coord G. of operator NP 304			Global Coord G of Valve Bod NP 315	, ,	Acceptable Acceleration (g)		
	Ax	Ay	Az	Ax	Ay	Az	Ax	Ay	Az
OBEI	0.4	0.112	0.342	0.4	0.117	0.402	2	2	. 2
SSEI	0.792	0.224	0.68	0.793	0.233	0.797	2	2	
SRV1V	2.337	0.773	2.125	2.338	0.85	2.357	3.5	3.5	3.5
SRV16V	3.739	1.046	3.088	3.741	1.116	3.546	4	4	-
CHUG/CO	0.159	0.328	1.132	0.16	0.421	0.376			
PS	0.196	0.086	0.169	0.196	0.093	0.198	4	4	-
FT	0.257	0.092	0.264	0.257	0.088	0.324	4	4	
UPSET	3.76910732	1.05599432	3.11807697	3.77109135	1.12556164	3.58339169	6	6	
FAULTED	3.83560295	1.12264865	3.36886687	3.83775911	1.21849497	3.66819806	7.21110255	7.21110255	7.2111025

Valve Spec No: 228.211-049-007K



CALCULATION

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VALVE IDENTIFICATION NUMBER: 1E12*VF050A

Loading	Acceleration is	n Global Coord	linate System	Acceleration in	Global Coord	linate System	Acceptable Acceleration (g)		ion (g)
Case	at C	.G. of operato	r (g)	at C.0	G of Valve Bod	ly (g)			
		NP			NP 182				
	Ax	Ay	Az	Ax	Ay	Az	Ax	Ay	Az
OBEI				0.119	0.102	0.121	2	2	. 2
SSEI				0.236	0.204	0.239	2	2	2
SRV1V				0.735	0.797	0.574	3.5	3.5	3.5
SRV16V				0.965	1.072	0.885	4	4	4
CHUG/CO				0.212	0.489	0.168			
PS				0.081	0.097	0.065	4	4	4
FT				0.963	1.462	1.19	4	4	4
UPSET	0	0	0	1.36848639	1.81577311	1.48794019	6	6	6
FAULTED	0	0	0	1.3997264	1.88874694	1.51151249	7.21110255	7.21110255	7.21110255

Valve Spec No: 228.211-049-001E



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		VALV	E IDENTIFICA	ATION NUMBE	R: 1E51*MOV	F013				
Loading	Acceleration in Global Coordinate System			Acceleration in	Global Coord	linate System	Accept	Acceptable Acceleration (g)		
Case	at C	.G. of operator NP X22	(g)	at C.0	G of Valve Bod NP X20	y (g)				
	Ax	Ay	Az	Ax	Ay	Az	Ax	Ay	Az	
OBEH	0.254	0.145	0.208	0.133	0.153	0.159	2	2	. 2	
SSEI	0.503	0.287	0.413	0.264	0.302	0.314	2	2	2	
SRV1V	1.633	0.995	1.24	0.874	1.082	0.797	3.5	3.5	3.5	
SRV16V	2.262	1.28	1.73	1.149	1.361	1.041	4	4	4	
CHUG/CO	0.351	0.37	0.691	0.765	0.406	0.307				
PS	0.168	0.105	0.149	0.096	0.114	0.08	4	4	4	
FT	1.055	1.257	1.456	0.705	1.259	0.908	4	4	4	
	-									
UPSET	2.50882144	1.79985388	2.27070474	1.35459034	1.86032551	1.3904769	6	6		
FAULTED	2.57019046	1.85410841	2.40018458	1.57230627	1.92183298	1.44947922	7.21110255	7.21110255	7.2111025	

Valve Spec No: 228.212

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CALCULATION

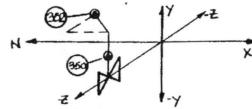
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		VALV	E IDENTIFICA	ATION NUMBE	R: 1E51* M OV	F023			
Loading Case	Acceleration in Global Coordinate System at C.G. of operator (g) NP 352			Acceleration in Global Coordinate System at C.G of Valve Body (g) NP 350			Acceptable Acceleration (g)		
	Ax	Ay	Az	Ax	Ау	Az	Ax	Ay	Az
OBEI	0.367	0.179	0.306	0.128	0.154	0.126	2	2	. 2
SSEI	0.726	0.356	0.607	0.254	0.306	0.249	2	2	2
SRV1V	1.695	1.506	2.139	0.945	1.026	0.962	3.5	3.5	3.5
SRV16V	3.052	2.115	3.167	1.312	1.529	1.16	4	4	4
CHUG/CO	0.505	1.115	1.091	0.889	0.55	0.842			
PS	0.18	0.187	0.254	0.102	0.126	0.112	4	4	4
FT	0.772	1.254	1.01	0.985	0.764	0.617	4	4	4

UPSET	3.16944427	2.4653158	3.33820685	1.64558591	1.71617394	1.31991098	6	6	6
FAULTED	3.26998303	2.72317866	3.55087581	1.88319038	1.82144805	1.58027023	7.21110255	7.21110255	7.21110255



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ENGINEERING DEPARTMENT RIVER BEND STATION CALCULATION

7.21110255

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		VA	LVE IDENTIFI	CATION NUME	BER: 1E12*VF	086			
Loading Case				Acceleration in Global Coordinate System at C.G of Valve Body (g) NP 520			Acceptable Acceleration (g)		
	Ax	Ay	Az	Ax	Ау	Az	Ax	Ay	Az
OBEI				0.228	0.148	0.131	2	2	. 2
SSEI				0.451	0.294	0.261	2	2	2
SRV1V				0.963	0.927	1.07	3.5	3.5	3.5
SRV16V				1.308	1.436	1.581	4	4	4
CHUG/CO				0.528	0.72	0.797			
PS				0.086	0.115	0.155	4	4	4
FT				0.489	0.572	0.918	4	4	4

1.55279876

1.73035141

1.83287916

2.01137142

7.21110255

1.41490954

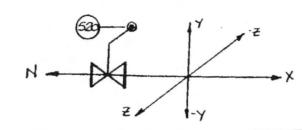
1.5595416

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Valve Spec No: 228.211

UPSET

FAULTED



7.21110255



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FLANGE QUALIFICATION

LINE NO. RHS-014-75-2
SUBJECT: QUALIFICATION OF FLANGED JOINTS (TABLE A)
Based On: EMTR-605-0 (REF#5)
ASME III Class 2 and 3
FLANGE TYPE: R.F. WELDING NECK SIZE: 14"
FLANGE MATERIAL: SA-105
BOLTING MATERIAL: SA - 193 GR. 37 NO. OF BOLTS: 20
S = 25000 psi \(\array{20,000 psi} \)
S_ = 31200 psi
C = 20.25 in.
A _b = <u>.763 × 20 = 15.26</u> in. 2
$D_{f} = 16.25$ in.
$F_s = S_y / 36000 = -867 \le 1.0$
F _{sc} = F _s .C = 17.56 iv.
S: Minimum allowable value of stress for bolting material at 100°F from table I-7.3, subsection NA, ASME III (psi)
Sy: Yield strength of flange material at the design temperature from table I-2.2, I-7.1 subsection NA, ASME III (psi)
C: Diameter of bolt circle. (in.)
Ab: Total cross-sectional area of the bolts at the root of the threads. (in)
D _f : Outside diameter of raised face of flange. (in.)
A = ACCEPTABLE X = UNACCEPTABLE



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FLANGE QUALIFICATION

LINE NO. RHS-014-75-2 NP: 128

SUBJECT: QUALIFICATION OF FLANGED JOINTS (TABLE B)

Based On: EMTR-605-0 (REF#5)

Design, Normal and Upset Conditions

a-1 Deadweight and Thermal

Local Homents	M _x (ft-1b)	My (ft-lb)	M _z (ft-1b)
DEADWEIGHT	- 657	-699	3203
THERMAL **	1152	1156	2689
STEADY STATE **			
TOTAL (A) *	+495	699	5892

1. Bending moment =
$$\sqrt{(M_y)^2 + (M_z)^2}$$

= $\sqrt{(699)^2 + (589)^2} = 5933$. ft 1b

3. Allowable moment =
$$\frac{3125}{12}$$
 F_{sc} . A_b

$$= \frac{3125}{12} \times 17.56 \times 15.26 = 69783$$
 ft 1b

$$\frac{\mathsf{M}_{\mathsf{fs}} \leq \frac{3125}{12}}{\frac{36000}{36000}} \quad \mathsf{C} \quad \mathsf{A}_{\mathsf{b}}$$

(A)

Mfs: The maximum of 1 or 2 = 5933. ft.lb

- * Total (A) is the maximum of the (deadweight + steady state) or (deadweight + thermal + steady state) algebraic combination.
- ** Haximum components from all Thermal cases input.



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FLANGE QUALIFICATION

LINE NO. RHS-014-75-2 NP: 128

SUBJECT: QUALIFICATION OF FLANGED JOINTS (TABLE C)

Based On: EMTR-605-0 (REF# 5)

a-2 Desdweight, Thermal and Dynamic Loads

Local Moments	M _x (ft-lb)	Hy (ft-1b)	M _z (ft-lb)
TOTAL (A)	+ 495	699	5892
N+U Dynamic **	3 37	611	1100
OBEA	261	445	237
TOTAL (B) *	1093	17 55.	7229

4. Bending Moment =
$$\sqrt{(M_y)^2 + (M_z)^2}$$

= $\sqrt{(1755)^2 + (7225)^2}$ = 7439 ft-lb

5. Torsional moment = M_x · = 1093ft-lb

6. Allowable moment = $\frac{6250}{12}$ F_{sc} A_b = $\frac{6250}{12} \times 17.56 \times 15.26 = 139565$ ft-1b

 $H_{fd} \leq \frac{6250}{12} \frac{(Sy) C}{36000} A_b$

(A)

Mfd: the maximum of 4 or 5. = 7439 Pf-LB

* Absolute combination of A, N+U dynamic and OBEA

** Maximum value of all dynamic load combinations for normal and upset condition (i.e., OBE, SRV, Fluid Transient, etc)

NOTE: SRV, FT DOES NOT APPLY, SINCE

ISOLATED FROM RB.



CALC. NO. - REV.

AX-71AG, Rev. 4

PAGE 49 OF

FLANGE QUALIFICATION

LINE NO. RHS-014-75-2 NP: 128

SUBJECT: QUALIFICATION OF FLANGED JOINTS (TABLE D)

Based On: EMTR-605-0 (REF# 5)

b. Emergency Condition

b-1 Deadweight, Thermal and Dynamic Loads

Local Moments	H _x (ft-lb)	My (ft-lb)	M _z (ft-lb)
TOTAL (A)	495	699	5892
EMERGENCY DYNAMIC **	337	611	1100
TOTAL (C) *	832	1310	6992

7. Bending moment =
$$\sqrt{(\frac{M_y}{2})^2 + (\frac{M_z}{2})^2}$$
 = $\sqrt{(1310)^2 + (4992)^2}$ = 7114 ft-1b

8. Torsional Moment = M_x = 832 ft-lb

9. Operating pressure = Po = 500pSi < 1.5p = 1.5 × 500 = 750 pSi

10. Allowable moment = $(11250 \text{ A}_{\text{b}} - \frac{(\pi)}{16} \text{ D}_{\text{f}}^2 \text{ P}_{\text{e}}) F_{\text{sc}}/12$ = $(11250 \times 15.26 - \frac{17}{16} \times 16.25^2 \times 500) \times \frac{17.56}{12} = 213282$

$$M_{fd} \le (11250 A_b - \frac{(\pi)}{16}) D_f^2 P_E \frac{C}{12} \frac{(Sy)}{36000}$$

Po: Maximum operating pressure during emergency condition (psi)= 500 PSi

P: Design pressure (psi) = 500 psi

PE: Emergency condition pressure concurrent with Mfd (psi) = 500 PSi

. M. The maximum of 7 or 8 = 7114 ft. 16

* Absolute combination of A and emergency dynamic

** Maximum value of all dynamic load combinations for emergency condition

CALC. NO. - REV.

AX-71AG, Rev. 4

PAGE 50 OF

FLANGE QUALIFICATION

LINE NO.

RHS-014-75-2 NP: 128

SUBJECT: QUALIFICATION OF FLANGED JOINTS (TABLE E)

Based On: EMTR-605-0 (REF# 5)

Faulted Condition

Deadweight, Thermal and Dynamic Loads c-1

Local Moments	M _x (ft-1b	M _y (ft-1b)	M _z (ft-lb)
TOTAL (A)	495	699	5.897
FAULTED DYNAMIC **	662	1191	2102
TOTAL (D) *	1157	1890	7.994

11. Bending moment =
$$\sqrt{(\text{M}_y)^2 + (\text{M}_z)^2}$$

= $\sqrt{(1890)^2 + (7924)^2}$ = 824 ft-1b

12. Torsional Moment = M

13. Operating pressure = Po = 500 psi ≤2.0 P= 2 x 500 = 1000 psi

14. Allowable moment = (11250 $A_b - \frac{(\pi)}{16}$ D_f^2 P_f) $F_{sc}/12$

= (11250 x 15.26 - T x 16.252 x 500) x 17.56 = 213282 FT. LB

$$H_{fd} \leq (11250 A_b - \frac{(7)}{16}) D_f^2 P_f \frac{C}{12} \frac{(Sy)}{36000}$$

Po: Maximum operating pressure during faulted condition (psi) = 500 ρSi

P: Design pressure (psi) = 500 psi

Pf: Faulted condition pressure concurrent with Mfd (psi) = 500 psi

Mfd: The maximum of 11 or 12 = 8214 Ff- LRs

Absolute combination of A and faulted dynamic

** Maximum value of all dynamic load combinations for faulted condition



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FLANGE QUALIFICATION

LINE	NO. 1-RHS-004-113-2
SUBJ	ECT: QUALIFICATION OF FLANGED JOINTS (TABLE A)
Base	d On: EMTR-605-0
	MAX, OF
ARME	III Class 2 and 3
-	
	GE MATERIAL: 54-105
BOLT	ING MATERIAL: 54-193, 58.87 NO. OF BOLTS: 8 @ 3/4 D/A.
S	= <u>25000</u> psi
S	= 36000 psi
•	= <u>7.88</u> in.
A _L	= 2.416 in.2
-	= 6.19 in.
-	= S _y / 36000 = //O \(\leq 1.0 \)
-	= F.C = 7.88 W.
	-
S:	Minimum allowable value of stress for bolting material at 100°F from table I-7.3, subsection NA, ASME III (psi)
Sy:	Yield strength of flange material at the design temperature from table I-2.2, I-7.1 subsection NA, ASME III (psi)
C:	Diameter of bolt circle. (in.)
A _b :	Total cross-sectional area of the bolts at the root of the threads. (in)
Dg:	Outside dismeter of raised face of flange. (in.)
<u> </u>	ACCEPTABLE X = UNACCEPTABLE



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FLANGE QUALIFICATION

LINE NO. RHS-004-113-2 NP: 415/417

SUBJECT: QUALIFICATION OF FLANGED JOINTS (TABLE B)

Based On: EMTR-605-0 (REF#5)

Design, Normal and Upset Conditions

a-1 Deadweight and Thermal

Local Moments	M _x (ft-lb)	My (ft-1b)	M _z (ft-lb)
DEADWEIGHT	- 47	5	29
THERMAL **	- 91	50	155
STEADY STATE **	_		
TOTAL (A) *	- 138	55	184

1. Bending moment =
$$\sqrt{(\text{M}_y)^2 + (\text{M}_z)^2}$$

= $\sqrt{(55)^2 + (184)^2} = 192$. ft 1b

3. Allowable moment =
$$\frac{3125}{12}$$
 F_{sc}. A_b

$$M_{fg} \le \frac{3125}{12} \frac{(S_y)}{36000} C A_b$$

(A)

Mfg: The maximum of 1 or 2 = 192. ft.1h

- * Total (A) is the maximum of the (deadweight + steady state) or (deadweight + thermal + steady state) algebraic combination.
- ** Haximum components from all Thermal cases input.

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CALCULATION ENGINEERING DEPARTMENT RIVER BEND STATION

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FLANGE QUALIFICATION

LINE NO. __ RHS-004-113-2 NP: 415/417

SUBJECT: QUALIFICATION OF FLANGED JOINTS (TABLE C)

Based On: EMTR-605-0 (REF# 5)

a-2 Deadweight, Thermal and Dynamic Loads

Local Moments	H _x (ft-lb)	My (ft-1b)	M _z (ft-lb)
TOTAL (A)	- 138	55	184
N+U Dynamic **	36	210	153
OBEA	72	56.	172
TOTAL (B) *	246	321	509

4. Bending Moment =
$$\sqrt{(H_y)^2 + (H_z)^2}$$

= $\sqrt{(32+)^2 + (509)^2}$ = 602 ft-lb

6. Allowable moment =
$$\frac{6250}{12}$$
 F_{sc} A_b
= $\frac{6250}{12} \times 17.56 \times 15.26 = 139565$ ft-1b

M.	$\overline{}$	6250	(Sy)	C	A
Id	_	12	360	00	. 0

(A)

M_{fd}: the maximum of 4 or 5. = 602 P4-LB

Absolute combination of A, N+U dynamic and OBEA

** Maximum value of all dynamic load combinations for normal and upset condition (i.e., OBE, SRV, Fluid Transient, etc)



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FLANGE QUALIFICATION

LINE NO. RHS-004-113-2 NP: 415/417

SUBJECT: QUALIFICATION OF FLANGED JOINTS (TABLE D)

Based On: EMTR-605-0 (REF# 30)

b. Emergency Condition

b-1 Deadweight, Thermal and Dynamic Loads

Local Homents	M _x (ft-lb)	My (ft-1b)	M _z (ft-lb)
TOTAL (A)	138	55	184
(08EI,OCCE) EMERGENCY DYNAMIC **	36	210	153
TOTAL (C) *	17.4	269	337

7. Bending moment =
$$\sqrt{(H_y)^2 + (M_z)^2}$$

= $\sqrt{(265)^2 + (337)^2}$ = 429 ft-1b

10. Allowable moment =
$$(11250 \text{ A}_b - \frac{(\pi)}{16} \text{ D}_f^2 \text{ P}_e) \text{ F}_{sc}/12$$

= $(11250 \times 15.26 - \frac{\pi}{16} \times 16.25^2 \times 500) \times \frac{17.56}{12} = 213282$
FT-LE

$$M_{fd} \le (11250 A_b - \frac{(\pi)}{16} D_f^2 P_E) \frac{C}{12} \frac{(Sy)}{36000}$$

(A)

Po: Maximum operating pressure during emergency condition (psi)= 500 pSi

P: Design pressure (psi) = 500 PSi

 P_E : Emergency condition pressure concurrent with Mfd (psi) = 500 ρ Si

. Mfd: The maximum of 7 or 8 = 174 Pt-16

* Absolute combination of A and emergency dynamic

** Maximum value of all dynamic load combinations for emergency condition



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FLANGE QUALIFICATION

LINE NO.

RHS-004-113-2 NP: 415/417

SUBJECT: QUALIFICATION OF FLANGED JOINTS (TABLE E)

Based On: EMTR-605-0 (REF#)

Faulted Condition

Deadweight, Thermal and Dynamic Loads c-1

Local Moments LOADS	M _x (ft-lb	H _y (ft-1b)	M _z (ft-lb)
TOTAL (A)	138	5 5	184
FAULTED DYNAMIC **	54.	326	236
TOTAL (D) *	192	381	420

11. Bending moment =
$$\sqrt{(M_y)^2 + (M_z)^2}$$

= $\sqrt{(38)^2 + (420)^2}$ = 567 ft-1b

12. Torsional Moment = M

13. Operating pressure = Po = 500 psi <2.0 P= 2 x 500 = 1000 psi

14. Allowable moment = (11250 $A_b - \frac{(\pi)}{16}$ D_f^2 P_f) $F_{sc}/12$

= (11250 x 15.26 - T x 16.252 x 500) x 17.56 = 213282

$$H_{fd} \le (11250 A_b - \frac{(\pi)}{16}) D_f^2 P_f \frac{C}{12} \frac{(Sy)}{36000}$$

Po: Maximum operating pressure during faulted condition (psi) = 500 psi

Design pressure (psi) = 500 psi

Pg: Faulted condition pressure concurrent with Mfd (psi) = 500 psi

Mfd: The maximum of 11 or 12 = .567 ff-LBs

Absolute combination of A and faulted dynamic

** Maximum value of all dynamic load combinations for faulted condition



CALC. NO. - REV.

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VENT & DRAIN QUALIFICATION:

The following Vents & Drains are qualified generically by Calculation PX-SB-316:

Valve No:

RHS*V1229

RHS*V112 and V113

RHS*V51

RHS&V52

ICS*VF084 & VF085

ICS*VF061 & VF062

ICS*VF073

ICS*VF074

The following Vents & Drains are qualified with tie-back support:

Valve No:

ICS*VF034

1E12*VF058A & VF059A

RHS*V112 & V113



CALC. NO. - REV.

AX-71AG, Rev. 4

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SLEEVE CLEARANCES

Piping displacements due to reanalysis of Rev. 4 have not changed significantly; either they have decreased at certain locations or increased insignificantly at some locations. The thermal load cases and Seismic Anchor Movement load case movements are not altered, and this analysis has used N411 damping values, which reduces the dynamic displacements. The original values from previous revisions are still applicable.

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STONE & WEBSTER ENGINEERING CORPORATION

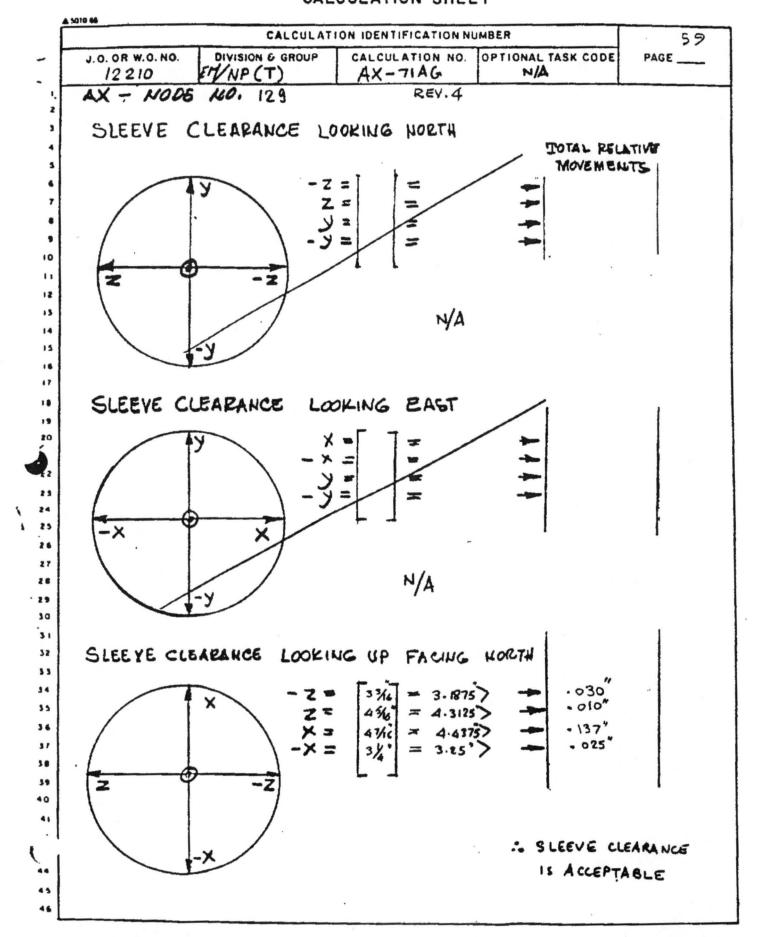
CALCULATION SHEET

			TION IDENTIFE				58
J.O. OR W.O. NO. 12210					PAGE		
AX- HODE NO.		AVTS IN	SLEEVE	REV. 4	TOATIO	u c	
					INPUT		
LOAD CASE	CASE	NUPIPE DE	FLECTION		PATION	SHU AF	RELATIVE
DESCRIPTION	NO.		MYT	MY		MOYEN	
DERDWEIGHT	5	X	-007			THE RESERVE OF THE PERSON NAMED IN	07
THERMAL (N+U)							
THERMAL (MAX)	9	×	·112			-1	12
THERMAL (MIN)	8	×	014				014
BUILDING SETTLEMENT 1	*						
BUILDING SETILE MENT 2	-						
OBEA	18	The second name of the second	· 230	± -23			.005
558 A (2088A)		×	.460	± -47	0	+	.010
OBEI	12	× +	.003	J -44	_	±	.009
1	13	×	-005				.016
DYNAMIC	-	2 ·	.009		_	+1 +1	-00 g
DYNAMIC	20	2	.010			# #	·010
OCCE	21	2	·010			#	-010
DYNAMIC OCCF	27		+ .010			±	.010
TOTAL RELATIVE MOVE MOUTS	= Di	ut ± Ther.	BLDG.	± 4	_ A38C	DYNAM!	e ecs)
OR ** INSIGNIFICAN		wt 1 Ther	BLDG.	T = 5	± ABC	DYNAMIC (OCCF)	-

NOTES! I. THE SIGN OF THE TOTAL RELATIVE MOVEMENTS MUST BE THE BIGH OF THE MAXIMUM COMBINATION OF DWT., THERMAL AND BLDG. SETTLEMENT.

2. THE MAXIMUM OF EITHER EQUATION SHOWN ABOVE IS RECORDED ON THE NEXT PAGE TO BE COMPARED WITH THE SLEEVE CLEARANCE.

CALCULATION SHEET



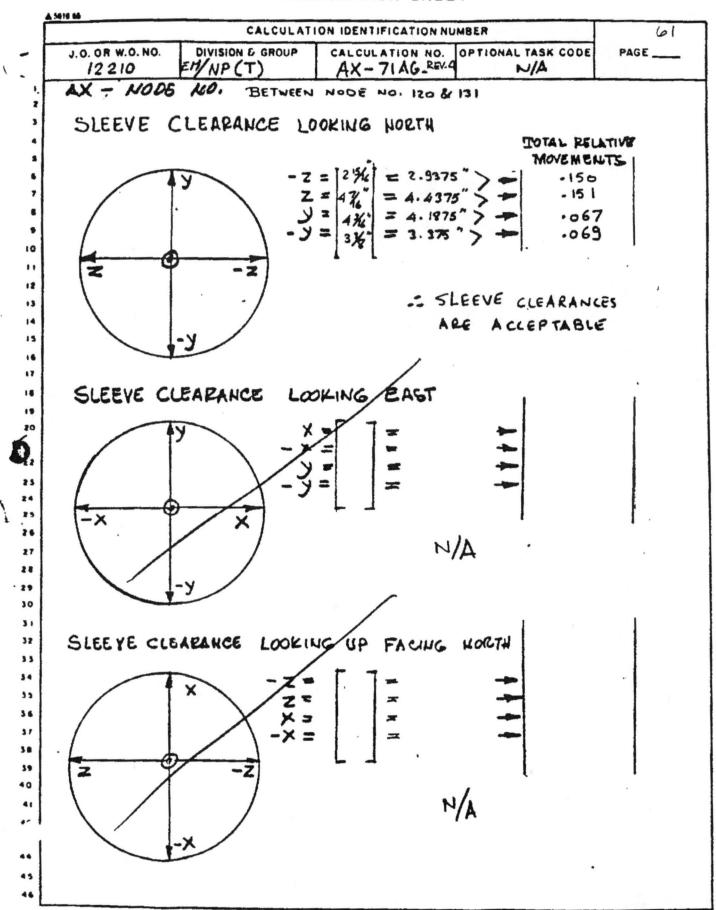
STONE & WEBSTER ENGINEERING CORPORATION CALCULATION SHEET

	A 5418 86											
	CALCULATION IDENTIFICATION NUMBER 60											
_	J.O. OR W.O. NO.		ON & GROUP	CALCULA	TION NO.		TASK CODE	PAGE				
•	12210	M/NE	(T)	AX-7	AG_	2	IA					
3	AX- NODE NO.			SLEEVE		TRATIO	u S					
5												
,	1010 0100	10000	Nupipe (NAME AND ADDRESS OF THE OWNER, WHEN PERSON AND ADDRESS OF THE PERSON A		PATION		0.0.45.45				
	LOAD CASE	CASE	PIPE DE	HYT	MY		*** * *	RELATIVE				
,	DESCRIPTION	1001		.013/002	14/4		MOVEN	the same of the sa				
10	DEADWEIGHT	.5	The second secon	0.00/.004			013					
11	Jan Jan J	 	-	0.007.004			. 0.00	7.000				
12	THERMAL (N-U)											
13	minama (ma)	9	X	-037			.03	THE RESERVE OF THE PERSON NAMED IN COLUMN TWO IS NOT THE PERSON NAMED IN COLUMN TWO IS NOT THE PERSON NAMED IN				
14	THERMAL (MAX)	1	2	.110			.//					
15		10		024		_	02	marine the latter than the same of the sam				
16	THERMAL (MIN)		7	//3			11	3				
16	BUILDING SETTLEMENT 1	*										
19	BUILDING											
20	SETILE MENT 2	-						Bullion, and the				
6			Y ±	- 128	-12	4	1.0	004				
D	OBEA	18	Z +	. 241	-23	35	4 .0	006				
23			Y±	.256	-24	48	4.0	800				
24	SSEA (2088A)		2 1	-482	-4	70	±.0	12				
23		12		.014			+ • •	714				
24	DBEI	12	2	\$.013			4.0	13				
27	1	13	Y	.023	-	- 10	± .	ാ				
5.0	SSEI	1,3	2	±.022			<u> </u>	022				
29	DYNAMIC	0-	~	± .016		-	+1	016				
30	oeau	20		± .018		-	+ .	018				
31	DYNAMIC	21	~	± .016	-		±.	016				
35	OCCE	21	2	± .018		-	± .	018				
33	DYNAMIC	27	~	±.024	1	- II.	± .	024				
34	OCCF	27	2	± . 075	J	-	+	.025				
35	TOTAL PELATIVE	1-		RLDG.	1		DYNAMI	5				
37	MOVE MENTS	= 0	ut ± 146R.5	BLDG.	1 ± 6	DBEA :	(occu /o					
38		1		-3.4	1,		(000)					
19		1.		RIDG.	1		DYNAMIC	_ `				
40	OR	= 0	wit Ther:	SETTLEMEN	T = =	± A366	(OCCF)					
41		1 24. /	NSIGNIFICA	NT (SEE P				•				
42	NOTES: 1. THE					THEMP	-	F THE BIGH				

OF THE MAXIMUM CONSINATION OF DWT., THERMAL AND BLOG. SETTLEMENT. 2. THE MAXIMUM OF EITHER EQUATION SHOWN AROVE IS RECORDED

ON THE NEXT PAGE TO BE COMPARED WITH THE SLEEVE CLEARANCE.

STONE & WEBSTER ENGINEERING CURPORATION CALCULATION SHEET



CALCULATION SHEET

	62			
JO. OR WO. NO. 12210	DIVISION & GROUP	CALCULATION NO. AX-7/AG REV.4	OPTIONAL TASK CODE	PAGE

REF.

SLEEVE CLEARANCE EVALUATION

	1				1
NODE 5		טא	PIPE Output	NUPIPE Input	
Load Case	Case	Pip	e Deflection	Penetration	Sum of Relative
Description	No.	Dir.	Movement	Movement	Movements
	2,	×	.005		-005
Deadweight	3/	근	012		0/2
	700	×	003	0	003
Thermal (N+U)	20	곷	.004	0	.004
		×	007	0	007
Thermal (Max.)	26	곤	013	0	013
	00	×	001	0	00/
Thermal (Min.)	23	돈	0	0	0
Building	19-CLASS	×	0	0	0
Settlement 1	2 RW	2	0	0	0
Building	20-CLASS	×	0	0	0
Settlement 2	2 RUN	군	0	0	0
	27	×	.268	.255	.013
OBEA	37	7	.249	. 237	.012
		×			.026
SSEA (2*OBEA)		Z			.024
	1 ,_ 1	X	.009		.009
OBEI	40	~	.024	_	.024
	11	×	.017	-	.017
SSEI	41	Z	.042	_	-042
	100	×	.072	_	.072
Dynamic OCCU	45	モ	.126		.126
	0=	×	.074	-	.074
Dynamic OCCE	35	Z	.126	_	.126
	24	×	.075	-	.075
Dynamic OCCF	36	-2	./3/		./3/

- Notes: 1. The sign of the total relative movements must be the sign of the maximum combination of Deadweight, Thermal and Building Settlement.
 - The maximum of either equation shown above is recorded on the next page to be compared with the sleeve clearance.

CALCULATION SHEET

		CALCULATION ID					63
JO. OR WO. 12210	NO.	DIVISION & GROUP		TION NO.		TASK CODE	PAGE
REF.							
		SLEEVE	CLEARANCE	EVALUATIO	N		
	5100	ve Clearance Look	ing North				
		Y ,					Relative
		1	2	- 1		- 1	1
			2	-		_	
	z		·z	_		_	
-			-,	-	-	-	
		-y					
	Slee	ve Clearance Look	ing East				
		Y					
		1	-,	- 1	1 -	- 1	1
			,	-	-		
	×		· x			-	
		ĺ				_	
		1				I	
		- y					
	Slee	ve Clearance Look	ing Up Fac	ing North	No	de 5	
4		×		MIN.			
		1	-2	- 11/	z" •	1.5"-	.180
			2	- 14	z -	1.5"-	.180"
	2		·z ·	23	8 -	2.375-	.105° : 0k
		ļ	-,	= 25	8 =	2.375-	.180° .180° .105° :: 0k
		1		, 1		i	1
-		- x					
Conclus	ion:	: OK					
					х.		

CALCULATION SHEET

CALCULATION IDENTIFICATION NUMBER						
JO. OR WO. NO. 12210	DIVISION & GROUP	CALCULATION NO.	OPTIONAL TASK CODE	PAGE		

REF.

SLEEVE CLEARANCE EVALUATION

NODE 71		טע	PIPE Output	NUPIPE Input	
Load Case	Case	-	e Deflection	Penetration	Sum of Relative
Description	No.	Dir.	Movement	Movement	Movements
	31	<u>×</u>	0		0
Deadweight	101	근	002	-	-,002
	20	×	.285	0	.285
Thermal (N+U)	20	근	. 044	0	.014
	0.	×	. 767	0	.767
Thermal (Max.)	26		.207	0	.207
	40	×	.064	0	.064
Thermal (Min.)	23 -	근	002	0	002
Building	19-CLASS -	*	.019	0	.019
Settlement 1		- 근	-,00/	0	001
Building	20-CLASS		008	0	7,008
Settlement 2	1	ᠽ	0	0	0
	2-	×	.370	.362	.008
OBEA	37	2	. 307	- 303	.004
		×			.016
SSEA (2*OBEA)		る			.008
		×	.∞3	-	.003
OBEI	40	7	.005	-	.005
	1,1	×	.006	_	,006
SSEI	41	2	.010	—	.0/0
	1-	×	.058	_	.058
Dynamic OCCU 45	45	=	.108		.108
		×	.058	_	.058
Dynamic OCCE	35	老	.108		.108
		×	.058		.058
Dynamic OCCF	36	Z	.109		./09

- Notes: 1. The sign of the total relative movements must be the sign of the maximum combination of Deadweight, Thermal and Building Settlement.
 - The maximum of either equation shown above is recorded on the next page to be compared with the sleeve clearance.

$$\chi = /0 + .767 + .019 / + .016 + .058 = .860"$$

 $z = /-.002 + .207 - .001 / + .008 + .109 = .321"$

CALCULATION SHEET

	CALCULATION IDEN	ITIFICATION NUMBER		65
JO. OR WO. NO. 12210	DIVISION & GROUP	CALCULATION NO. AX-71 AG 1284.4	OPTIONAL TASK CODE	PAGE
REF.	SLEEVE	CLEARANCE EVALUATIO	M	
s	leeve Clearance Looki	ing North		
	Y			Relative
	2	-z = z = y = -y =		
sı	-y Leeve Clearance Looki	ng East		
	Y			
	×x	-y =		
	-у			
S1	eeve Clearance Looki		,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,	
	x	MIN, FOR A DIRECTIO	W.	nn, * 1
	zz	z = 23 $x = 13$	$\begin{vmatrix} 2.375^{2} \\ 8 \end{vmatrix} = 2.375^{2} $ $= 2.375^{2} $ $= 1.875^{2} $ $= 1.875^{2} $.321° .860°
Conclusion	- x : : ok	//		

CALCULATION SHEET

CALCULATION IDENTIFICATION NUMBER					
JO. OR WO. NO. 12210	DIVISION & GROUP NP(C)	CALCULATION NO. AX-71 AG REV.4	OPTIONAL TASK, CODE	PAGE	

REF.

SLEEVE CLEARANCE EVALUATION

NODE 100			PIPE Output	NUPIPE Input	
Load Case	Case	The state of the s	e Deflection	Penetration	Sum of Relative
Description	No.	Dir.	Movement	Movement	Movements
	3/	<u>y</u>	014		014
Deadweight	101	3	002		-,002
	20	7	.054	0	.054
Thermal (N+U)	20	7	.003	0	.003
	26	<u>Y</u>	.118	0	.118
Thermal (Max.)	20	근	.003	0	.003
	23	7	.015	0	.015
Thermal (Min.)		근	.002	0	,002
Building	19-CLASS	7	.004	0	.004
Settlement 1	2 RW	근	.0	0	0
Building	20.000	Y	272	270	-,002
Settlement 2	2 RUN	己	0	0	0
	37	<u>Y</u>	.195	.181	.014
OBEA	13/	군	.3/4	.5/3	.00/
					.028
SSEA (2*OBEA)		<u>-</u>			.002
	10	<u> </u>	.004		.004
OBEI	40	곷	.007		.007
	41	Y	.008		.008
SSEI	41	- 군	.013		.013
	45	<u>Y</u>	.050		.050
Dynamic OCCU	40	콘	. 075		. 075
	25	<u>Y</u>	.050		.050
Dynamic OCCE	35	ᠽ	.075	-	.075
	21	Υ	.05/		.05/
Dynamic OCCF	36	근	. 076		.076

- Notes: 1. The sign of the total relative movements must be the sign of the maximum combination of Deadweight, Thermal and Building Settlement.
 - The maximum of either equation shown above is recorded on the next page to be compared with the sleeve clearance.

CALCULATION SHEET

	CAI	CULATION SHEE	: 1	
		ITIFICATION NUMBER	_	47
JO. OR WO. NO. 12210	DIVISION & GROUP MP(C)	AX-71 AG REV.4	OPTIONAL TASK CODE	PAGE
REF.				
	SLEEVE (LEARANCE EVALUATIO	DN .	
			-	
Sle	eve Clearance Looki	ing North Node	100	
	У			Relative
		MIN		
	ĺ	-z = //7	8 = 7.625	77
z		z = //	8 =/.625-	017 ok
		y = 1/3/	16 =1.812 -	/87"
		-y = //3	= .625 . . .	187"
	-y		. 7	
Slee	eve Clearance Looki	ng East		
	y			
į	1	-x = 1	1 = -1	1
. ж		v =	• -	
	1	-y -		
	-y			
Slee	eve Clearance Looki	ng Up Facing North	ı	
	×			
	1	-z =	1 - 1	1
		z =		
z	-2			
	ł			
	-x			•
Conclusion	: OK			
	, -			

CALCULATION SHEET

	68			
JO. OR WO. NO. 12210	DIVISION & GROUP MP(C)	CALCULATION NO.	OPTIONAL TASK CODE	PAGE

REF.

SLEEVE CLEARANCE EVALUATION

NODE 395		טא	PIPE Output	NUPIPE Input	
Load Case Description	Case No.	Pip Dir.	e Deflection Movement	Penetration Movement	Sum of Relative Movements
	4.	×	.010		.010
Deadweight	3/	Y	03/		03/
	7-	X	.078	0	.078
Thermal (N+U)	20	7	.004	0	.004
	21	×	.212	0	.212
Thermal (Max.)	26	Y	.088	0	.088
	7.4	×	.035	0	. 035
Thermal (Min.)	23	Y	015	0	015
Building	19-cuss	X	0	0	0
Settlement 1	2 RUN	Y	001	0	00/
Building	20-046	*	0	0	0
Settlement 2	2 RUN	Y	270	-,270	0
	0-1	*	.314	.315	7,001
OBEA	87	Y	-180	.181	-,00/
		×			7,002
SSEA (2*OBEA)		Y			-,002
	1-		050		.020
OBEI	40	Y	.062		.062
	41	X	. 033		,033
SSEI	1-7/	Y	.099		,099
	45	_x_	.112		.112
Dynamic OCCU	122	<u>Y</u>	.216		.216
	35	×	.112		.112
Dynamic OCCE	100	Y	.216		.2/6
	36	×	.115		.115
Dynamic OCCF	100	<u> </u>	.229		.229

- Notes: 1. The sign of the total relative movements must be the sign of the maximum combination of Deadweight, Thermal and Building Settlement.
 - The maximum of either equation shown above is recorded on the next page to be compared with the sleeve clearance.

CALCULATION SHEET

	CALCULATION IDEN	TIFICATION NUMBER		69
JO. OR WO. NO. 12210	DIVISION & GROUP NP(C)	CALCULATION NO. AX-7 (AG REV. 4	OPTIONAL TASK CODE	PAGE
REF.	SLEEVE C	LEARANCE EVALUATION	<u>и</u>	
	Y			Relative vements
z		-z = z = y = -y =		
Slee	-y ve Clearance Looki y	7006	DE 395	
x	-x	-x = 32 x = 32 y = 32 -y = 3	w. 4" = 3.25"- 4" = 3.0"- " = 3.0"-	.339" .339" .288" .288"
Slee	-y ve Clearance Looki x	ng Up Pacing North	1	
2	-2	-z = z = x =		
Conclusion :	-x .			



CALCINO	P
	- REV. ADDENDUM
AX-71AG, Re	The state of the s
ATTACHMEN	T NO.:
PAGE	E 70 OF

CALCULATION CHECKLIST

YES	NO	N/A	FORMAT	EDP-AA-20 SECTION
x			Cover Page completed.	6.4.1
x			Table of Contents completed (as required).	6.4.2
x			Revision History Sheet completed (as required).	6.4.3
		x	Revisions are identified with revision lines in right margin.	6.4.3
x			Applicable Documents Page completed.	6.4.4
		x	Definitions established (as required).	6.4.5
x			Calculation/revision/addendum/page numbers are identified correctly.	6.4.10
			CONTENTS	
x			Previous calculation for the required analysis exists.	6.3
x			Calculation is appropriately titled for the intended scope.	6.4.1-2
x			Purpose and scope are clearly and adequately established.	6.4.1-7, 7.1
x			Safety classification is correct for the identified scope.	6.4.1-6
x			Topics/documents/equipment for cross-reference/retrieval are identified.	6.4.1-11
x			Calculation is clear and comprehensible.	6.1
x			Applicable codes, standards, etc. are identified.	6.4.4-1
x			RBS references are identified.	6.4.4-2
x			Affected documents are identified	6.4.4-3
x			Inputs and sources are identified, appropriate, and correct.	7.2.2-1
I			Assumptions are identified and appropriate.	7.2.2-2
		x	Inputs derived from field walkdown have been witnessed/verified	7.2.2-5
x			Engineering judgments are identified and appropriate.	7.2.2-6
x			Calculation methodology is identified and supported by technical bases.	7.2.3
x			Conclusion is appropriate and is justified by calculation.	7.3
		x	Confirmations are identified and indicated as required on Cover Page.	7.5.7
		x	Directions for Confirmations are included.	7.5.7-3
x			Calculation data is appropriately included, attached, or referenced.	7.4
x			Programs and software are identified and have been verified and validated.	8.0
x			Methods/calculations use to check results are identified and included.	
x			Results are accurate and in accordance with the established methodology.	
		x	Certification by Professional Engineer is required.	11.7
			VENDOR CALCULATIONS	
		X	Calculation is performed in accordance with EDP-AA-25.	10.2
		X	Calculation content and format are acceptable.	10.2
		x	Vendor, preparer, reviewer, and approver are clearly identified.	10.3
x			Design verification review has been completed (as applicable).	

Preparer (Signature/KCN or SSN/Date):

Reviewer (Signature/KCN or SSN/Date):

Lothor Kuhn (1:05) 6/18/97

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Name of Street
====
T
ENTERGY

CALCULATION ENGINEERING DEPARTMENT RIVER BEND STATION

CAL	C. NO REV. ADDENDUM
	AX-71AG, Rev. B
ATTA	CHMENT NO.:

PAGE 71 OF

	REVIEWER'S CHECKLIST								
Item	Calc Page			Comment	3		Respo	onse/Resolution	
				MANAGEMENT			IIII.AANUUUNAAAAAAAAAA		
					*	-		***************************************	

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-									
				VI VI VIII VIII III III III					

	***************************************						W. M. W.		

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			·						
Comme	nts Provide	d By		Comme	nts Resolved By	7	Resolu	tion Accepted	
			×						
Loth	ar Kul	n (1105)	6/18/97					14 CHA A A A A A A A A A A A A A A A A A A	
Reviewe	er	KCN	Date	Preparer	KCN	Date	Reviewer	KCN	Date

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ENTERGY

CALCULATION

CALC. NO REV. ADDENDUM
AX-71AG, Rev. 4
ATTACHMENT NO.:

	ENGINEERING DEPARTMENT	ATTACHMENT NO.:	
ENTERGY	RIVER BEND STATION	PAGE 72 OF	
	Design Verification Record	d	
Document No. A	AX-71AG, Rev. 4		
метнор			
Verification met	hods to be used:		
xr	Design Review		
	Qualification Testing		
A	Alternate Calculations		
DOCUMENT(S)	REVIEWED: (Attach Additional Sheet(s), if nee	eded)	
Document No.	Revision D	ocument Title	
AX-71AG	4	Pipe Stees Analysis	
		Cox ics function backer	2
		per 842 96-0069.	

	REVIEW: (Attach Additional Sheet(s), if needed))	
Acceptable as is.			
Design Verification (Completed By Lothar Kulm (1105)	Date 6/18/97	
Comments Resolution	n Accepted By	Date	
Engineering Supervis	sor Royash Gupls	Date 11-20-97	
4			



CALCULATION COVER PAGE ENGINEERING DEPARTMENT

CALC. NO. - REV. ADDENDUM

AX-71AG, Rev. 4A

ATTACHMENT NO.: (as required)

ENTERGY RIVER BEND STATION		CATION	JBI NO.: G13.10.2.2
		PAGE 1 OF 6	
Title: Pipe Stress Analysis F	For ICS Function Added Thro	ugh FWS System Per MR	96-0069
CALCULATION		IMPLEMENTATION OF ME CONFIRMATIONS	R 96-0069
SYSTEM NO.: 107/FWS	,	MARK NO.: RHS.	010-065-2
CLASSIFICATION: SA	FETY RELATED	NON-SAFETY RELATI	ED: QAPA D NON-QAPA
PURPOSE / SCOPE / OBJI	ECTIVE:		
To qualify the welding tee du	e to a reduction in the wall th	ickness which was requir	ed for final fit up during construction.
CONCLUSION:			
The piping remains within the	e code allowables.		
SOFTWARE USED FOR C	ALCHIATION: DVE	S ■NO SDDF#:	
	Name Version/Rel		
KEYWORDS:		icensing/Design Basis Im	
		Addressed in DCA No.	
FWS, ICS		l Licensing/Design Basis l None - Reason:	Impact Checklist included
		11000 110000	
	_	A STATE OF THE STA	
	REVIEV	W & APPROVAL	
Brian Lanka, KCN 13 Preparer	☐ Reviewer () ☑ Design Ver	nature/Date) Non-Safety) ification Reviewer me, KCN, or SSN)	(Signature/Date) Supervisor (Printed Name, KCN, or SSN) RAJESH GUPTA, 1059

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CALC.	NO REV. ADDENDUM
·	AX-71AG, Rev. 4A

ATTACHMENT NO.: (as required)

JBI NO.: G13.10.2.2

PAGE 2

DEVISION DISTORY

Revision No.	Paragraph No.	Description of Change
4A		To qualify the welding tee due to a reduction in the wall thickness during final fit up during construction.
F		



CALC. NO. - REV. ADDENDUM AX-71AG, Rev. 4A

ATTACHMENT NO.: (as required)

JBI NO.: G13.10.2.2 PAGE 3

REFERENCE DOCUMENTATION

I. Control Document Cross References (Page four of NORMS): Other Calculations, Procedures, Drawings, etc. Identify how documents are affected.

				EFFECT	(Check Or	ie)
DOC. NO.(Control Xrefs)	REV. NO.	TYPE	Supplemented	Superseded	Voided	Reference Only
1. AX-71AG	4	CALCSW	X			
744						

II. Record Cross References (Page five of NORMS): RBS change documents, ERs, MRs, Calculation Addenda, etc. Identify how documents are affected.

	·			EFFECT	(Check On	e)
RECORD NO. (Rec Xref)	REV NO	TYPE	Supplemented	Superseded	Voided	Reference Only
1. MR96-0069	0	MR				X
A						
				 	 	

- III. Miscellaneous References (Misc Ref): Applicable codes, standards, & references (non-RBS):
- 1. ASME B &PV Code, 1974 Edition.
- 2. NAVCO Piping Datalog, 10th Edition.



CALC. NO. - REV. ADDENDUM AX-71AG, Rev. 4A

ATTACHMENT NO.: (as required)

JBI NO.: G13.10.2.2 PAGE 4

Calculation:

During installation of the 10" welding tee (sch 160) at node point X98, the ends had to be machined to schedule 120 to allow proper fit up to pipe line RHS-010-065-2 (Sch 120).

This tee was modeled and designed as a sch. 160 tee. The ME101 analysis (Ref. I.1) considered the tee to be schedule 160. For the tee to be welded to the pipe, the ends had to be machined to sch 120. This calculation will qualify the tee stress as if it was schedule 120, which is conservative. Since only the thickness of the end of the tee is reduced, the change in weight will be negligible.

Design Input:

Properties are per Ref. III.2.

Pipe Size	Schedule	Thickness	r – mean radius
10"	120	0.843"	4.9535"
10"	160	1.125"	4.8125"

Pipe Stress

The reduction in wall thickness affects mechanical stress and pressure stress. Pressure stress is calculated by PD/4t and mechanical stress is calculated by iM/z

By changing the thickness from 1.125" to 0.843" the pressure stress will increase 1.125 / 0.843 or 33%. The mechanical stress is calculated by i M / Z. Per NC-3652.4, $Z = \pi r^2 t$. Where r is the mean radius. Thus

$$Z 120 = \pi (4.9535)^2 (0.843) = 65.0 \text{ in}^3$$

$$Z 160 = \pi (4.8125)^2 (1.125) = 81.9 \text{ in}^3$$

Thus, the section modulus will change by 81.9 / 65 or 26%. The change in thickness will also affect the SIF for a welding tee. Per Ref. III.1, SIF = $0.9 / (4.4 \text{ t/r})^{667}$ Thus

SIF
$$120 = 0.9 / (4.4 * 0.843 / 4.9535)^{.667} = 1.09$$

SIF
$$160 = 0.9 / (4.4 * 1.125 / 4.8125)^{667} = 0.88 = 1.00$$
 Since the SIF can not be less than 1.00.

Thus the mechanical stress will increase by 1.09 * 1.26 = 1.37. This value envelops the pressure stress term and will conservatively be used to increase the equation stresses. Existing stresses per Ref. I.1.

Equation	Node	Existing Stress (Psi)	New Stress (1.37 * Existing Stress)	Allowable	Margin	
Equation 8	X99-X97	4209	5766	15000	0.38	
Equation 9B	X99-X97	6185	8473	18000	0.47	
Equation 9C	X99-X97	6196	8489	27000	0.31	
Equation 9D	X99-X97	6241	8550	36000	0.24	
Equation 10	X99-X97	5269	7219	22500	0.32	
High Energy Eq 9B + 10	Not Applicable per Ref. I.1					

. Pipe stresses are within their allowables.



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ATTACHMENT NO.: (as required)

JBI NO.: G13.10.2.2

PAGE 5

CALCULATION CHECKLIST

YES	NO	N/A	FORMAT	EDP-AA-20 SECTION
X			Cover Page completed.	6.4.1
		X	Table of Contents completed (as required).	6.4.2
X			Revision History Sheet completed (as required).	6.4.3
		X	Revisions are identified with revision lines in right margin.	6.4.3
X			Applicable Documents Page completed.	6.4.4
		X	Definitions established (as required).	6.4.5
X			Calculation/revision/addendum/page numbers are identified correctly.	6,4.10
			CONTENTS	
X	***		Previous calculation for the required analysis exists.	6.3
X			Calculation is appropriately titled for the intended scope.	6.4.1-2
X			Purpose and scope are clearly and adequately established.	6.4.1-7, 7.1
X			Safety classification is correct for the identified scope.	6.4.1-6
X			Topics/documents/equipment for cross-reference/retrieval are identified.	6.4.1-11
X			Calculation is clear and comprehensible.	6.1
X			Applicable codes, standards, etc. are identified.	6.4.4-1
X			RBS references are identified.	6.4.4-2
X			Affected documents are identified	6.4.4-3
X			Inputs and sources are identified, appropriate, and correct.	7.2.2-1
X			Assumptions are identified and appropriate.	7.2.2-2
		X	Inputs derived from field walkdown have been witnessed/verified	7.2.2-5
X			Engineering judgments are identified and appropriate.	7.2.2-6
X			Calculation methodology is identified and supported by technical bases.	7.2.3
X			Conclusion is appropriate and is justified by calculation.	7.3
		X	Confirmations are identified and indicated as required on Cover Page.	7.5.7
		X	Directions for Confirmations are included.	7.5.7-3
X			Calculation data is appropriately included, attached, or referenced.	7.4
		X	Programs and software are identified and have been verified and validated.	8.0
		X	Methods/calculations use to check results are identified and included.	
X			Results are accurate and in accordance with the established methodology.	
		X	Certification by Professional Engineer is required.	11.7
			VENDOR CALCULATIONS	
		X	Calculation is performed in accordance with EDP-AA-20.	10.2
	-	X	Calculation content and format are acceptable.	10.2
		X	Vendor, preparer, reviewer, and approver are clearly identified.	10.2
		X	Design verification review has been completed (as applicable).	

Preparer (Signature/KCN or SSN/Date):

Reviewer (Signature/KCN or SSN/Date): 6

4-15-99

(0969)



CALCULATION ENGINEERING DEPARTMENT ATTACHMENT NO.: (as required) JBI NO.: G13.10.2.2

CALC.	NO	REV.	ADD	END	UM

AX-71AG, Rev. 4A

ENTERGY	RIVER BEND STATION	PAGE 6					
Design Verification Record							
Document No. AX-	71AG, Rev. 4A						
метнор							
Verification method	ds to be used:						
X	Design Review						
	Qualification Testing						
	Alternate Calculations						
DOCUMENT(S) R	EVIEWED: (Attach Additional Sheet(s), if needed)						
Document No.	Revision	Document Title					
AX-71AG,	4A		-				
	arrangement and a second and a		-				
			-				
			-				
SUMMARY OF RE	EVIEW: (Attach Additional Sheet(s), if needed)						
Acceptable as is.			y				
Design Verification	Completed By Family Towoollan (0969)	Date 5/	3/99				
Comments Resolution	on Accepted By	Date					
Engineering Supervi	sor Rajish Gupti / 5-3-99/ken	105% Date	-3-99				



Record Revision Notice

1			
Department DE	Document No. A	C-71AG Re	V 4/4 Page 1 of 2
	6-22-99 1 100	130 33740	(ASG Use Only)
IDIV VI	0111111	FileNet Location	11311-00
JBIX No. Revision Description	Revision/Date	Filelyet Location	Batch/Transmittal - Item No.
* * *			
Change the status of the calculati	on from Pending to Approv	ed.	
			T T
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Reason For Revision			
MD07.0070			
MR96-0069 was implemented du	ring Kr-08.		
			RECEIVED
			JUN 2 3 1999
•			SA TIC
Approvals:			
	*		
100) .		1 /22
Originator Originator	KCN:	1389	Date (p/22/99
	1 11		,
Ω	11/	,	. / /
Section Supervisor	T (Jelly	ECN 0658	Date 6/21/98



CALC. NO. - REV. ADDENDUM

AX-71AG, Rev. 4A

ATTACHMENT NO.: (as required)

JBI NO.: G13.10.2.2

PAGE 1 OF 6

Title: Pipe Stress Analysis For ICS Function Added Through FWS System Per MR96-0069						
☐ CANCELED						
□ NON-QAPA						
ring construction.						
cluded						
REVIEW & APPROVAL						
sture/Date) Dervisor ne, KCN, or SSN) A, 1059						
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CALCULATION COVER PAGE ENGINEERING DEPARTMENT

CALC.	NO	
	AX-76A Rev. 3E	
ATTA	CHMENT NO.:	
JBI NO	O.: G13.10.2.2	
	PAGE 1 OF 9	

ENTERGY RIVER BEND ST.	JBI NO.: G13.10.2.2							
ENTERGY RIVER BEND ST	ALIUN	PAGE 1 OF 9						
TITLE:		SUPERSEDES: N/A						
Stress Analysis for ICS Piping in the Reactor Building p	er MR 96-0069	SUPPLEMENTS: AX-76A Rev. 3						
CALCULATION STATUS: □ APPROVED ■ PENDING □ CANCELI								
SYSTEM NO.: 209	MARK NO.:							
CLASSIFICATION: ■ SAFETY RELATED	NON-SAFETY RELA	ATED: QAPA NON-QAPA						
PURPOSE / SCOPE / OBJECTIVE:	* ,							
	To document the changes implemented by MR 96-0069. This MR re-routes the RCIC injection from the reactor vessel head to the feedwater nozzle. As a result of this changes, line ICS-006-6-1 is removed and ICS-006-57-1 is downgraded to ASME class 2 and abandoned in place.							
*								
	*							
CONCLUSION:								
The changes implemented by MR 96-0069 are acceptable RPV head nozzle and the Drywell Bellows Penetration I greatly reduces the stress acting on the piping and the s KJB*Z19, and pipe supports. Therefore, the piping systallowables of the ASME Section III Code.	ORB*Z130. Likewise upport loads acting of	the abandonment of ICS-006-57-2 on penetration DRB*Z130, penetration						
	- Ty	* **						
	•							
SOFTWARE USED FOR CALCULATION: YES	□ NO SDDF#:	N/A						
Manufacturer: Na	meV	ersion/Release No						
CONFIRMATIONS REQUIRED: ☐ YES ■ NO C	ONFIRMATION CO	OMPLETE: GYES GNO GN/A						
KEYWORDS:								
ICS, MR 96-0069, RCIC								
REVIEW & APPROVAL								
Preparer ☐ Reviewer (Camilo Cuadra, KCN 0969 ☐ Design Ver	ture/Date) Non-Safety) rification Reviewer te, KCN, or SSN)	(Signature/Date) Supervisor (Printed Name, KCN, or SSN) RAJESH GUPTA 1059						



CALC. NO. - REV. ADDENDUM AX-76A Rev. 3E ATTACHMENT NO.:

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PAGE 2 OF

Pipe Stress Analysis for ICS in the Reactor Building per MR 96-0069

TABLE OF CONTENTS

SECTION	DESCRIPTION	PAGE NO.		
N/A	REVISION HISTORY			3
N/A	APPLICABLE DOCUMENTS		•	4
1.0	ISOMETRICS	*		5
2.0	CALCULATION DETAILS			7
3.0 ,	CALCULATION CHECKLIST			8
N/A	DESIGN VERIFICATION CHECKLIST			9

Total Pages: 9



CALC. NO. - REV. ADDENDUM
AX-76A Rev. 3E
ATTACHMENT NO.: (as required)
JBI NO.:
PAGE 3 OF

Pipe Stress Analysis for ICS in the Reactor Building per MR 96-0069

		REVISION HISTORY				
Revision No.	Paragraph No.	Description of Change				
3A	AII	Per MR 96-0069, the piping is rerouted through the RHR system. As a result of this change, the piping in this calculation is either removed or abandoned in place.				
	,					



CAL	C. NO REV. ADDENDUM
	AX-76A Rev. 3E
ATTA	CHMENT NO.:
JBI N	O.:
	PAGE 4 OF

Pipe Stress Analysis for ICS in the Reactor Building per MR 96-0069

APPLICABLE DOCUMENTS

L APPLICABLE CODES, STANDARDS, & REFERENCES (NON-RBS):

1. ASME Boiler & Pressure Vessel Code, Section III, Division 1, 1974 Edition

II. RBS REFERENCES:

1. MR 96-0069

III. AFFECTED DOCUMENTS:

DOC. NO.	REV. NO.	DOCUMENT TITLE	AFFECT (Superseded, Revised, Initiated, etc.)	CHANGE DOC.
Z-76-3009A	0	Pipe Support Calculation	VOID	MR 96-0069
Z-76-3009A	- 0A	Pipe Support Calculation	VOID	MR 96-0069
Z-76-3012A	0	Pipe Support Calculation	VOID	MR 96-0069
Z-76-3012A	0A	Pipe Support Calculation	VOID	MR 96-0069
Z-76-3023A	0	Pipe Support Calculation	VOID	MR 96-0069
Z-76-3023A	0A	Pipe Support Calculation	VOID	MR 96-0069
12210-N-SR-140	1A	Support Stress Report	REVISED	MR 96-0069
12210-N-SR-143	1A	Support Stress Report	REVISED	MR 96-0069
12210-N-SR-153	1A	Support Stress Report	REVISED	MR 96-0069
12210-N-SR-144	1A	Support Stress Report	REVISED	MR 96-0069
12210-N-SR-145	1A	Support Stress Report	REVISED	MR 96-0069
12210-N-SR-146	1A	Support Stress Report	REVISED	MR 96-0069
12210-N-SR-147	1A	Support Stress Report	REVISED	MR 96-0069
12210-N-SR-148	1A	Support Stress Report	REVISED	MR 96-0069
12210-N-SR-149	1A	Support Stress Report	REVISED	MR 96-0069
12210-N-SR-150	1A	Support Stress Report	REVISED	MR 96-0069
12210-N-SR-151	1A	Support Stress Report	REVISED	MR 96-0069
12210-N-SR-155	1A	Support Stress Report	REVISED	MR 96-0069
AX-002H	3C	Pipe Stress Calculation	GENERATED	MR 96-0069
	1			
				, , , , , , , , , , , , , , , , , , , ,
				· ·

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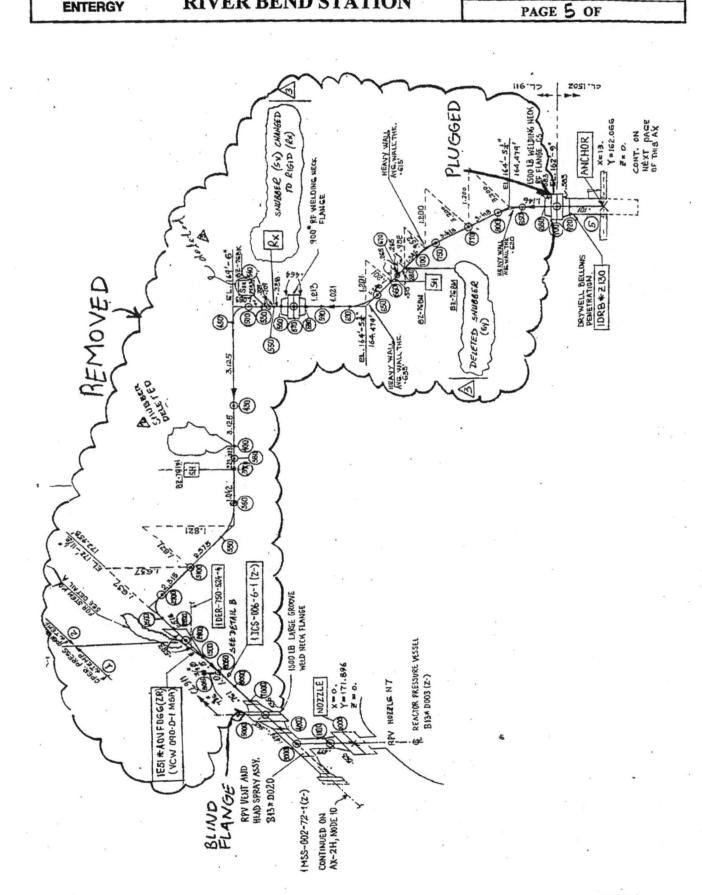
CALCULATION WORK SHEET ENGINEERING DEPARTMENT RIVER BEND STATION

CALC. NO. - REV. ADDENDUM

AX-76A Rev. 3E

ATTACHMENT NO.:

JBI NO.:





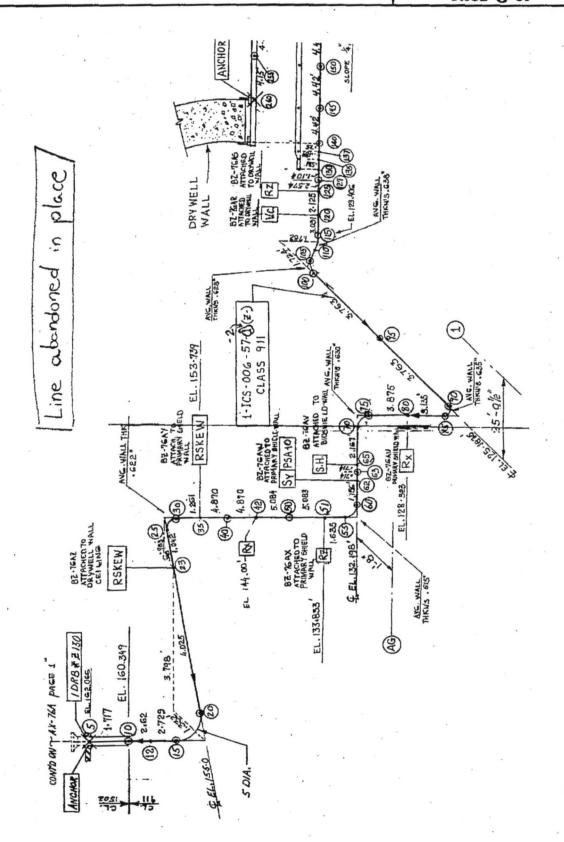
CALC. NO. - REV. ADDENDUM

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CALCULATION WORK SHEET ENGINEERING DEPARTMENT RIVER BEND STATION

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JBI NO.: PAGE 7 OF

CALCULATION DETAILS

MR 96-0069 re-outes the RCIC Injection Point from the Reactor Head Spray connection to the A Loop of Feedwater via RHR piping. As a result of this change, pipe line ICS-006-6-1 (Z-) is removed from the RPV vent and head spray assembly to the drywell bellows penetration DRB*Z130. The flange at the RPV assembly is replaced by a blind flange, and the flange at DRB*Z130 replaced by plug as shown in the isometrics. Pipe supports associated with this pipe are removed. See isometrics. The remaining line ICS-006-57-2 (Z-) from DRB*Z130 to penetration 1KJB*Z19 is abandoned in place.

The abandoned pipe, ICS-006-57-1(Z-), will be empty and capped at both ends which results in a considerable reduction in dead weight and nullification of the thermal and fluid transient loads. Therefore, the structural integrities of the piping, the pipe supports, and the penetrations are assured. The pipe supports associated with this line are downgraded to Class 2.

Removal of ICS-006-6-1 (Z-) eliminates the loads on one side of the RPV vent and head spray assembly. Line MSS-002-72-1(Z-) connected to the other side of the assembly is not affected by MR 96-0069. Thus, RPV nozzle N7 is now solely qualified in AX-2H. Since the total load acting on the nozzle is reduced, the nozzle remains qualified.

The following table lists the pipe supports affected by this MR:

Support No.	Change
ICS*PSSH-3009-A1	Removed
ICS*PSSH-3023-A1	Removed
ICS*PSSP-3012-A1	Removed
ICS-PSR-3013-A2	Downgraded to class 2
ICS-PSR-3014-A2	Downgraded to class 2
ICS-PSSH-3017-A2	Downgraded to class 2
ICS-PSSP-3018-A2	Downgraded to class 2
ICS-PSST-3016-A2	Downgraded to class 2
ICS-PSST-3019-A2	Downgraded to class 2
ICS-PSST-3020-A2	Downgraded to class 2
ICS-PSST-3021-A2	Downgraded to class 2
ICS-PSST-3027-A2	Downgraded to class 2



CALC. NO. - REV. ADDENDUM AX-76A Rev. 3E ATTACHMENT NO.: JBI NO.: PAGE 8 OF

CALCULATION CHECKLIST

YES	NO N/A FORMAT			
X			Cover Page completed.	6.4.1
X			Table of Contents completed (as required).	6.4.2
X			Revision History Sheet completed (as required).	6.4.3
		X	Revisions are identified with revision lines in right margin.	6.4.3
X			Applicable Documents Page completed.	6.4.4
		X	Definitions established (as required).	6.4.5
X			Calculation/revision/addendum/page numbers are identified correctly.	6.4.10
			CONTENTS	
X	74444		Previous calculation for the required analysis exists.	6.3
X			Calculation is appropriately titled for the intended scope.	6.4.1-2
X			Purpose and scope are clearly and adequately established.	6.4.1-7, 7.1
X			Safety classification is correct for the identified scope.	6.4.1-6
X			Topics/documents/equipment for cross-reference/retrieval are identified.	6.4.1-11
X			Calculation is clear and comprehensible.	6.1
		X	Applicable codes, standards, etc. are identified.	6.4.4-1
X			RBS references are identified.	6.4.4-2
X			Affected documents are identified	6.4.4-3
X			Inputs and sources are identified, appropriate, and correct.	7.2.2-1
X			Assumptions are identified and appropriate.	7.2.2-2
		X	Inputs derived from field walkdown have been witnessed/verified	7.2.2-5
		X	Engineering judgments are identified and appropriate.	7.2.2-6
X			Calculation methodology is identified and supported by technical bases.	7.2.3
X			Conclusion is appropriate and is justified by calculation.	7.3
		X	Confirmations are identified and indicated as required on Cover Page.	7.5.7
		X	Directions for Confirmations are included.	7.5.7-3
		X	Calculation data is appropriately included, attached, or referenced.	7.4
		×	Programs and software are identified and have been verified and validated.	8.0
		X	Methods/calculations use to check results are identified and included.	
X			Results are accurate and in accordance with the established methodology.	
			Certification by Professional Engineer is required.	11.7
			VENDOR CALCULATIONS	
			Calculation is performed in accordance with EDP-AA-25.	10.2
			Calculation content and format are acceptable.	10.2
			Vendor, preparer, reviewer, and approver are clearly identified.	10.3
X			Design verification review has been completed (as applicable).	

6-12-97

ENTERGY

ATTACHMENT I

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ES-P-002-00

Typical Design Verification Record

	,	Page 9 of 9
Document Number AX - 76A		Revision 3E
METHOD		
Verification methods to be used:		
Design Review Qualification Testing Alternate Calculations		
DOCUMENT(S) REVIEWED: (Attach	Additional S	heet(s), if needed)
Document Number	Revision	Document Title
Ax-76A	3 <i>E</i>	Pipe Stress Analysis for ICS in the Reactor Bidg- per MR 96-0069
	*	
SUMMARY OF REVIEW: (Attach Ad	lditional Sheet	(s), if needed)
. :		
Design Verification Completed B	Daju	Date: 6/16/97
Comment Resolutions Accepted By	U	Date:
Engineering Supervisor Royash	Gupl?	(1059) Date: 11-19-97



Record Revision Notice

ENGMSYS	AX-076	A-ADDE					
Department	Document No.		Page 1 of 2				
413.10.2.2 JBIX No.	3E/11/19/97 Revision/Date	11665084 FileNet Location	(ASG Use Only) 5 10056 ASG Use Only) 5 10056 Asach (Transmittal Lieux No.)				
Revision Description	Revision/Date	I Hervet Edeadon	Batch Halisinttal - Item 140.				
Change calculation / addendum status from pending to approved.							
,							
			,				
Reason For Revision							
MR 96-0069 has been ins	talled.						
			RECEIVED				
			JUN 2 5 1999				
			SA TIC				
A			- Alika ka				
Approvals:			1				
Milissa L. Le	therland 1	240	6/23/99				
Originator			Date				
^	1 11						
(a)	10.111	0658	1/2 /2-				
Section Supervisor	t (- Jelle	0030	6/2J/99				



CA	IC	NO.	_
CM	L.	NO.	-

AX-76A Rev. 3E

dot a

ATTACHMENT NO.:

JBI NO.: G13.10.2.2

PAGE 1 OF 9

TITLE:

Stress Analysis for ICS Piping in the Reactor Building per MR 96-0069

SUPERSEDES: N/A

SUPPLEMENTS: AX-76A Rev. 3

CALCULATION STATUS:	APPROVED	PENDING 199	□ CANCELED

SYSTEM NO.: 209 MARK NO.:

CLASSIFICATION: ■SAFETY RELATED NON-SAFETY RELATED: □ QAPA □ NON-QAPA

PURPOSE / SCOPE / OBJECTIVE:

To document the changes implemented by MR 96-0069. This MR re-routes the RCIC injection from the reactor vessel head to the feedwater nozzle. As a result of this changes, line ICS-006-6-1 is removed and ICS-006-57-1 is downgraded to ASME class 2 and abandoned in place.

CONCLUSION:

The changes implemented by MR 96-0069 are acceptable. The removal of line ICS-006-6-1 reduces the loads on the RPV head nozzle and the Drywell Bellows Penetration DRB*Z130. Likewise, the abandonment of ICS-006-57-2 greatly reduces the stress acting on the piping and the support loads acting on penetration DRB*Z130, penetration KJB*Z19, and pipe supports. Therefore, the piping systems analyzed by this calculation remain within the allowables of the ASME Section III Code.

JUN 2 5 1999

SA TIC

					SH	1 1 6		
SOFTWARE USED FOR CALCULATION:	□ YES	□ NO	SDDF#:	N/A			**************************************	
Manufacturer:	N	lame	Ve	rsion/Rele	ase No.			
CONFIRMATIONS REQUIRED: □ YES ■	NO	CONFIRM	ATION COM	IPLETE:	☐ YES	ONO	O N/A	
KEYWORDS:								
TCS MP 06-0069 PCTC		*						

ICS, MR 96-0069, RCIC

REVIEW & APPROVAL

(Signature/Date)

Preparer Camilo Cuadra, KCN 0969 (Signature/Date)

☐ Reviewer (Non-Safety)

Design Verification Reviewer
(Printed Name, KCN, or SSN)

Kojish Gups 11-19-(Signature/Date)

Supervisor (Printed Name, KCN, or SSN)

RAJESH GUPTA, 1059



CALC. NO. - REV. ADDENDUM AX-76B, Rev. 1H.

ATTACHMENT NO.:

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TITLE: Pipe Stress Analysis for ICS-006-7-2 (abandoned in place) in the Auxiliary Building per MR 96-0069

SUPPLEMENTS: Colules 1273

		JOANSON	6/11/98 1663
CALCULATION STATUS: ,	PROVED	⊠ PENDING	CANCELED .
SYSTEM NO.: 204, 209		MARK NO.:	
CLASSIFICATION: SAFETY RELATED	NO	N-SAFETY RELATED: Q	APA ⊠ NON-QAPA
PURPOSE / SCOPE / OBJECTIVE:			
FWS system through a short portion revision is to evaluate the abandone This piping has no function and it is an elbow as shown on isometric. To cases only to meet Class 2 requirem components do not need evaluation all dynamic load cases in the previo	ed pipe IC is not remo he piping nents. The because	S-006-7-2 (downgraded from oved due to ALARA purpose is to be evaluated for the a se supports, penetration, valuated they are qualified with them	m Class 1 to 2) in place. The piping is cut near pplicable dynamic load ves, and other piping
		•	
	v.	RECEIVE	ED
		JUN 1 2 199	18
		SDC	
CONCLUSION:			

This analysis demonstrates that the piping system is within acceptable design limits of ASME Section III Code for changes implemented by MR 96-0069.

SOFTWARE USED FOR CALCULATION Manufacturer: Bechtel		ES 🗆 NO SDDF#: Name ME101	6229.400-100-001A Version/Releas	se No.	N4
CONFIRMATIONS REQUIRED: ☐ YES	⊠ NO	CONFIRMATION CO	OMPLETE: YES	□NO	⊠ N/A
KEYWORDS: pipe, stress calculation, MR	96-0069				
•	REVIE	W & APPROVAL			
CX 1 leliela			1/2:10	7 1	

Gignature/Date)

Preparer (Printed Name, KCN, or SSN) Rajni R. Patel (0861) Carle Cucolin 6/18/97 (Signature/Date)

☐ Reviewer (Non-Safety)

Design Verification Reviewer

Camilo Cuadra (0969)

of ish Gup to /11-19-97 —(Signature/Date) Supervisor

Rajesh Gupta (1059)



CALC. NO. REV. ADDENDUM AX-76B, Rev. 1H ATTACHMENT NO.:

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TOTAL PAGES:	340



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	AX-76B, Rev. 1H				
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		REVISION HISTORY				
Revision No.	Paragraph No.	Description of Change				
2	All	ICS function is added to FWS System through RHR per MR 96-0069. Rev 2 of this calculation qualifies the abandoned pipe ICS-006-7-2 from penetration Z-19 to NP 87-88 (downgraded from Class 1 to Class 2). All design input and results are documented in their appropriate section.				
,4						
	× ·					



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APPLICABLE DOCUMENTS

I. APPLICABLE CODES, STANDARDS, & REFERENCES (NON-RBS):

- ASME Boiler and Pressure Vessel Code, Section III, 1974 edition with addenda through Summer 1974.
- ME101 Version N4 Pipe Stress Analysis Computer Program.
- 3. Piping Design and Engineering by ITT Grinnel Industrial Piping, Inc. Fifth Edition.

II. RBS REFERENCES:

- 1. EDS-ME-014, "Pipe Stress Analysis and Pipe Support Design Standard".
- Specification No. 228,000, Rev. 1H "Piping Engineering and Design".
- Design Criteria No. 228.010, "Pipe Stress and Pipe Support Design Criteria" dated 6-25-84.
- CHOC-EMTR 605-0, "Procedure for Qualification of Flanged Joints for ASME III, Class 1, 2, and 3 Piping Systems"
- Environmental Design Criteria S&W Document No. 215.150, Rev. 1H, Dated July 6, 1986.
- Calculation G13.18.1.5*08, Rev. 1, "Peak Spread ARS for Seismic Events Including Curves with N-411-1 Damping".
- CHOC-EMDM-81-08, "Valve Modeling Procedure" dated December 11, 9181
- LEM-34, "Frequencies and Acceptable Accelerations for Valve Qualification"
- Calculation RBI-EQ-416, "Valve Modeling and Acceptable Accelerations"
- Line Designation Table Change Notice LDTCN No.: 97-ICS-01
- PICL Change No.: AP-76-01
- CHOC-EMTR-601, "Design and Installation of Seismic Small Bore Piping, Instrumentation Tubing & Supports for River Bend", dated Sept. 1980
- Calculation 12210 NP(C)-PX-SB-316-0, "Pipe Stress Calc. for Vent, Drain & Instrument Connections", dated 1/7/83.
- 15. Calculation PX-334, Rev. 1H, "1/2 SSE Structural Seismic Displacements Data and Input Procedure for Piping Analysis" dated July 18, 1984,
- 16. Calculation PX-462, Rev. 0, "Structural Displacements Due to Thermal Expansion" dated April 9, 1976,
- DEM-1900, "Stress Analysis of Large Bore Piping and Pipe Supports for Jet Impingement Loads", dated 12-17-84
- 18. CHOC-EMDM-82-20, "Stress Intensification Factors for Reduced Outlet Branch Connections", dated 11-12-82.



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APPLICABLE DOCUMENTS

II. RBS REFERENCES:

- 19. DEM-P-4335, IOC from G. Martino/K. Basu to D. Christodoulou/D. Bhargava, "Model of Valve 1E51*AOVF065," dated 11-19-84.
- 20. AX-76B including all addendums

III. AFFECTED DOCUMENTS:

DOC. NO.	NO.	DOCUMENT TITLE	AFFECT (Superseded, Revised, Initiated, etc.)	DOC.
SR122	0A	Pipe Support Stress Report	Revised	MR 96-0069
SR126	0A	Pipe Support Stress Report	Revised	MR 96-0069
SR127	0A	Pipe Support Stress Report	Revised	MR 96-0069
SR128	0A	Pipe Support Stress Report	Revised	MR 96-0069
SR132	0A	Pipe Support Stress Report	Revised	MR 96-0069



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OBJECTIVE

MR 96-0069 reroutes the RCIC Injection Line from its original connection to the RPV Head to the FWS system through a short portion of the RHS piping RHS-010-65-2. Since some of the original ICS-006-7-1 piping beyond the diversion point is left in place and has been downgraded to ASME Class 2. The objective of Revision 2 of this calculation is to evaluate the abandoned piping ICS-006-7-2 from containment penetration Z-19 to the capped near elbow after the horizontal restraint PSST-2071A1 (BZ-76AN) at NP 90 as shown on isometric.

The ICS piping was originally analyzed for thermal, fluid transients, dynamic and hydrodynamic load cases because it was connected to the containment penetration Z-19. The abandoned pipe in place is not functional; therefore, there is no water weight, and fluid transients associated with this piping. Only dynamic and hydrodynamic load cases are applicable to demonstrate the structural integrity of piping is maintained for this piping.

There are no support configuration change. Supports are qualified while the piping is operational. Since the piping is not going to experience any thermal, fluid transient loads, the original support design is adequate and no further analysis is required.



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METHOD OF ANALYSIS

ME101

The abandoned piping in place is unsupported at free end; however, the support configuration is adequate to provide supports to maintain the structural integrity of piping during seismic and LOCA events. To demonstrate the piping system is within the ASME code requirements, the piping was analyzed by ME101, Version N4 using only the dynamic and hydrodynamic load cases.



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CONCLUSION

The completed analysis shows that the stresses and reactions on the subject piping are within acceptable limits in accordance with the requirements of the ASME Section III Code.

Functional capability evaluation is not performed because the piping is abandoned in place and is not functional.

Stresses on vents, drains, and test connections are within acceptable limits per calculation PX-SB-316.

Sleeve clearances are adequate.

Penetration Z-19 loads are reduced due to deletion of thermal and fluid transients load cases.

Therefore, the previous revision loads are still valid and are acceptable. New analysis is not required.



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NOTES

- 1. The Coordinate system for all data expressed in the global system is defined as follows:
 - (-X) Is plant "called north" per piping drawing
 - (Y) Is vertical "up"
 - (Z) According to right hand rule

The local system for straight piping members is defined as follows:

- (X) Is along the axis of the pipe, positive in the direction of coding.
- (Y) In the plane of the local X-Axis and the global +Y-Axis unless the local X-Axis is aligned with the global Y-Axis; then the local Y-Axis will be parallel to the global X-Axis.
- (Z) According to right hand rule.
- 2. Forces and moments on restraints, supports, and equipment are those imposed by the piping systems (i.e., actions, not reactions).
- 3. Unless otherwise noted, forces and moments reported at an inline anchor represent the total forces and moments acting upon that anchor considering contributions from piping on all sides of that anchor. Reactions reported on terminal anchors are due to piping shown in this report only. For total reactions, values must be combined with those from continuation summaries.
- Unless otherwise indicated, stresses at intersections have assumed an unreinforced branch Connection.
- 5. Intersection reinforcement requirements indicated in this AX are based on mechanical loads only. No pressure reinforcement has been calculated or considered. The piping fabricator shall perform these calculations, as well as area reinforcement requirements, and shall utilize the larger of the indicated pressure or mechanical reinforcement for fabrication.
- 6. Piping support loads may be found in Attachment B pipe support summary.
- 7. For lines requiring seismic analysis, the reactions reported are cyclic in character, and reactions must be considered as acting in either the positive or negative direction.
- The valve 1E51*A0VF065 operator weight reduction of 40 lb per Rev. 1B was neglected in this
 revision.
- Abbreviations

R.H	Rod Hanger
S.H	Spring Hanger
V. C	Vertical Restraint
RX	X-Restraint
RZ	Z-Restraint
RSKew	Skewed Restraint
SX	X-Snubber
SY	Y-Snubber
SZ	Z-Snubber
SSKew	Skewed Snubber
El	Elevation
Rdcr	Reducer
S.R. Elb	Short Radius Elbow



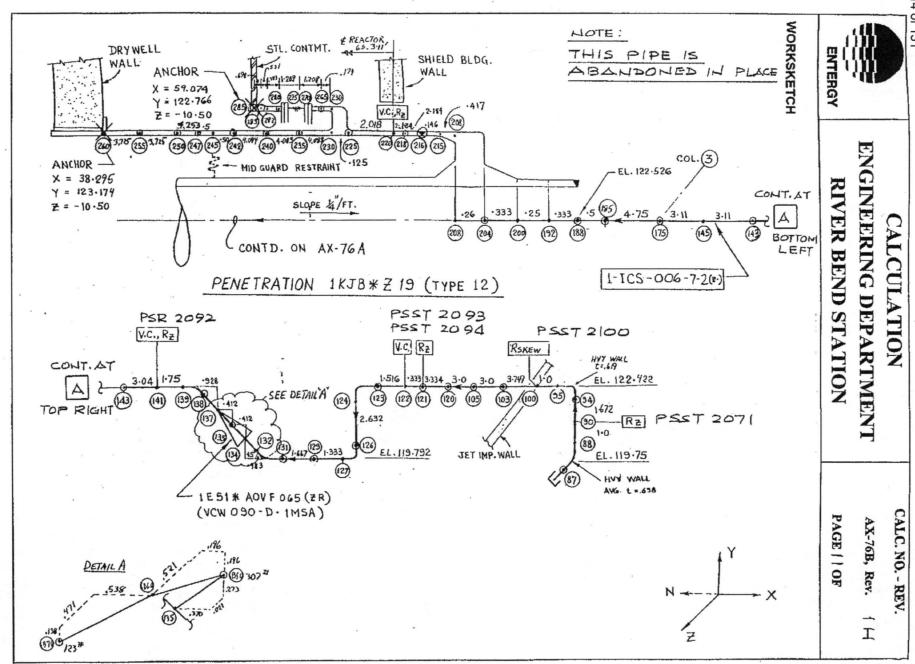
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ASSUMPTIONS

None



BASIC ANALYTICAL DATA

LINE NUMBER	CLASS	MATERIAL	E COLD	ALLOWABLE STRESS)Sh(PSI)	OUTSIDE DIA (IN.)	WALL THKNS (IN.)	INSUL THKNS (IN.)	E HOT 106PSI)	COEFF OF EXPN. IN./FT.	WT. TOTAL LB./FT
006-7-21	///	SANOG GR.B	27.9	15000	6.625	.432	0	27.4	.0/82	28.57
PENETRATION	~	_	_	_			_	_	-	
188-500	-	SA508 CL1	29,9	17500	6.625	,432	.0	29,0	.0182	28.57
200-204	-	н	11	11	7.625	,932	1+	"	"	66.62
204-208	-	11	μ	U	24.0	9.1195	4	"	"	1460.62
208-215	-	h	н	11	η	1.843	И	27.82	.0038	436.13
215-225	_	SA333 GR.6	27.9	15000	11	1.218	14	t ^f	н	296.36
225-230	-	SA508 CLI	29.9	17500	30.0	4.218	JI.	"	. #	1161.46
290-282	-	11.	п	#	ч.	,375	"	l'	11	118.65
282-283	-	SASO8 CLI	29,9	17500	32.125	1.438	0	27,82	.0038	471.29
283-285	-	11	11	11	34.25	2,5	0	"	"	847.74
3/4" LINE	911	SA106, GA.B	27,9	15000	1.05	,154	0	27,4	.0076	1.66
12 LINE	201	4	11	ii	.840	.147	0	27,4	.0182	1.19
ELBOWS	911		-11-	u	6.625	.808	0			58.74
43-45		- 11		11		,633	0	-11		5029
49-50	"	- 11	-11		11-11-	:635	0	- n	-	50.39
-59-60	11	ļi.	-1/			.620	0		- "	49.64
73-75		11	-4	- 11	11	.598	0			48.53
87-88	' '	11	1/	11	11	.638	0	"	11	39.24
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ENGINEERING DEPARTMENT RIVER BEND STATION CALCULATION

CALC. NO. - REV.

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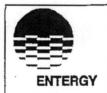
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CALCULATION	LAGINET
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PORATION

					VALVE	DATA					د
	NODE NO.	NOMINAL SIZE(IN)	VALVE NO.	S&W DESCRIPTION NO.	TYPE	S&W TYPE FILE NO.		VALVE LENGTH (FT.)	VALVE WEIGHT (LB.)	OPER. WT. (LB.)	0. OR 1 12210
	20/22	6	VGW090-A-/Y6C	1651*MOVF0/3	GATE	228.212	47-0039	1.666	750	360	0.0
	1350/850 1370	6	VCW090-D-1M54	1E51#AOVF065	CHECK	228,218-0	062-003F	1,166	307	123	, o
	350/ 352	4	VOWO90-A-1M5Q	IEIL*MOVFO23	GLOBE	228.212-0	47-084	1.166	325	10	DIVISION EM/N
	320	4	VCW090-D-IM	1E12*VF019	CHECK	228.211-0	49-00RF	1.166	145	_	P & C
N/	520	4	VGW030-C-2K5	IEIR# VFORG	GATE	228.211-04	19-007K	1.0	-	350	GROUP C)
14	725	3/4"	VOS150-G-Z	1IC5*VF084	GLOBE	188,213-00	58-011L	.334	10		ATION C
	745	3/4	Vow150-G-2	1ICS#4F085	GLOBE	228.213-0	58-011L	334	10	-,*	
	2010	3/4	VOW150-G-2M	1E51*VF034		228.213-0	258-049	.292	9	_	TIFICA.
:	2010	3/4	VOW150-G-2M	/E51#VF035				.292	10		IDENTIFICATION NUMB
	1220	3/4	VOW150-G-2M	/IC5*V302				.3125	10	_	OPTI
	1820	3/4	VOW 150-G-ZM	1IC5#V100		-		. <i>3</i> 33	10	-	WBER OPTIONAL REY. 1H
	3325	3/4	VOW150-G-2M	1E12*VF061				.3125			TASK
N/	3345	3/4	VOW/50-9-2M	IETE# VFO62				.3/25	10	_	CODE
A	4020	1/2	V05060-B-2	IRHS*+74		228,213-0	58-008T	,25	4	_	PAG
	4120	42	V05060-B-2	1RH5#V73		008,213-0	58-00BI	.266	6		E 25
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CONCENTRATED WEIGHTS:

Component Description	Weight in Ib	Node No
Valve Body 1E51*AOVF065	307.	B50
Valve Operator 1E51*AOVFO65	123.	B70
Small Bore Support Weight	40.	188

CALCULATION SHEET

	CALCULATION IDEN	NTIFICATION NUMBER		15
JO. OR WO. NO. 12210	DIVISION & GROUP	CALCULATION NO. AX-76B REV. 1H	OPTIONAL TASK CODE	PAGE

REF.

DESIGN INPUT (CONT'D)

RELATIVE SEISMIC ANCHOR AND/OR SUPPORT DISPLACEMENTS

(* SEE CALCULATION ON THE FOLLOWING PAGE(S) 55-6/ .)

NODE	SUPT.			TACHMENT (IN)				
NO.	TYPE	(STRUCTURE)	ELEVATION (FT)	DX	DY	DZ		
T	ANCHOR	AUX. BLOG.	91.719	0.2550	0.1800	0.2370		
9	RX		97.25	0.2750				
47	RSKEW		104.75		0.1810	0.2680		
48	SSKEW		104.75	0.3070		0.2680	N/A	
52	5₹		104.75			0.2680		
61	RZ		106.0			0.2720		
71	SSKEW		118.167	0.3620		0.3030		
76	Vc		119.75		0.1810			
90	RZ		120.75			0.3100		
100	RSKEW		122,526	0.3800	0.1810			
121	RZ		122.526	400 400 400 400 400		0.3130		
122	Ve	1, 1	122,526		0.1810			
141	VE, RZ	1	122.526		0.1810	0.3130		
220	VE, RZ	SHEED BLDG	122.60		0.4110	0.8530		
285	ANKHOR	STEEL CONT.	122.76	0.9870	0.4390	0.8690		

STONE & WEBSTER ENGINEERING CORPORATION

CALCULATION SHEET

	CALCULATION IDEN	TIFICATION NUMBER	12.20	16
JO. OR WO. NO. 12210	DIVISION & GROUP NP(C)	CALCULATION NO.	OPTIONAL TASK CODE	PAGE

REF.

DESIGN INPUT (CONT'D)

RELATIVE SEISMIC ANCHOR AND/OR SUPPORT DISPLACEMENTS

(* SEE GALCULATION ON THE FOLLOWING PAGE(5) 55-6/ .)

NODE	SUPT.	BUILI	DING	SUPPORT ATTACHMENT	DISPLACEMENTS * (IN)			REMARKS
NO.	TYPE	(STRUC	TURE)	ELEVATION (FT)	DX	DY	DZ	
260	ANCHOR	DRYV	VELL	123.17	1.0460	0.4780	0.9310	
338	RVE	AUX.	BLDG.	98.75	0.2800	0.1800		
375	RZ			100:385			0.2570	
380	Rx			103.677	0.3020			
382	SH			104.78	/	0.1810		N/A
410	Ve			10075		0.1810		
445	ANCHER			106.75	0.3150	0.1810	0.2740	
537	VesRe			98.39		0.1800	0.2520	
	ANCHOR	•	•	103.016	0.3000	0.1800	0.2640	



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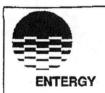
Dynamic Loads File Names

TABLE 1

Structural Attachment Elevation (FT)		Curve I	Number
		OBE	SSE
Auxiliary Building	114	auxbon05.ace	auxbsn06.ace
Auxiliary Building	141	auxbon07.ace	auxbsn08.ace
Drywell	114	rbdwon43.ace	rbdwsn44.ace
Drywell	133	rbdwon45.ace	rbdwsn46.ace
Shield Building	112.67	rbsbon03.ace	rbsbsn04.ace
Shield Building	. 133	rbsbon05.ace	rbsbsn06.ace

TABLE 2

Structural Attachment	Elevation (ft)		1		
		SRV1V	SRV16V	снсо	PS
Drywell	121	rbdwtn43.ace	rbdwan43.ace	rbdwcn43.ace	rbdwpn43.ace
Drywell	128	rbdwtn45.ace	rbdwan45.ace	rbdwcn45.ace	rbdwpn45.ace
Shield Building	121	rbsbtn03.ace	rbsban03.ace	rbsbcn03.ace	rbsbpn03.ace
Shield Building	128	rbsbtn05.ace	rbsban05.ace	rbsbcn05.ace	rbsbpn05.ace



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MAXIMUM STRESS LEVELS

PIPING STRESS SUMMARY CHECK AND COVER SHEET

RB/562367 (BG4043) 06/11/97 BG4043

ASME SECT. III, CLASSES 2 & 3

PROJECT					1 .
JOB NO.	22497	PLANT DESIGN GROU	lş	; c	
SYSTEM	RIVER BEND RO	IC SYSTEM PART N	OT IN OPERATION		
CALC NO	AX-76B	ISO NO		REV N	0

DESIGN CONDITION	LEVEL		CATION MAXIMUM ELEMENT	MAXIMUM COMPUTED STRESS (PS:	ALLOWABLE STRESS I) (PSI)	COMPUTED ALLOWABLE
SUSTAINED LOADS EQN. 8		134	132 134	776	SH 15000	.052
CCCASIONAL LOADS EQN. 9	В	134	132 134	3511	1.2 SH 18000	.195
OCCASIONAL LOADS EQN. 9	С	134	132	3793	1.8 SH 27000	.140
OCCASIONAL LOADS EQN. 9	ď	. 13.4	132 134	3830	.2.4 SH 36000	.106
THERMAL EXPANSION EQN. 10	77.00	200	192 200	10114	SA · 22500	.450



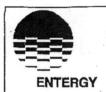
CALC. NO. - REV.

AX-76B, Rev. 1H

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FUNCTIONAL CAPABILITY EVALUATION:

This abandoned pipe ICS-006-7-2 is not performing any function per MR 96-0069; therefore, functional capability evaluation is not required.



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AX-76B, Rev. 1H

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VALVE ACCELERATIONS

Valve 1E51*AOV065 is abandoned in place and is not performing any function; therefore, it is not active. Valve accelerations from this analysis are reduced due to the use of N411 damping values for all dynamic and hydrodynamic load cases. Furthermore, the fluid transients are not applicable to this piping anymore. The valve accelerations from the previous revision are provided for information only.

..

\$ 5 T ō

30 29 28

Loading Case	Coord 'C.G.	ration in inate Syst ' of Opera	em at tor (g)	Coordina of Valve	ration in te System Body/at G Valve & On VODE /35	at 'C.G.'	Acc	Acceptable eleration Gee Ref. 8	(g)	12210
	ax	ay	az	a _x	ay	az	a _x	ay	az	EM/NP (C)
OBEI	.412	.610	.452	./87	.323	.241	2.0	2.0	2,0	OAIN/
SSEI	.795	1.175	.87/	.360	.620	.463	3,0	3,0	3,0	CROUP
SRV _{max} (SYMMETRIC-16V)	3.777	6.638	4.013	2,455	3,570	2,613	3.5	3,5	3,5	Ū
Chugg /co	2,441	4.572	2,476	1.974	2.351	1.732	3,0	310	3,0	122
pool swell	.315	.532	.347	.306	.292	.206	4.0	4.0	4.0	CALCULATION 12210-AX-76E
AP NA		,		*				,		AT10
Fluid Transient	1.846	4.715	2.984	4.342	2.674	2.326	4.0	4.0	-4.0-	Q N
								•		REV. II
TOTAL COMBINED -UPSET	4.224	8.165	5,021	4.991	4,472	3,507	5.0	5,0	5,0	TASK C
TOTAL COMBINED - FAULTED	4.926	9.412	5.648	5.376	5.080	3.931	6.1	6.1	6.1	ODE



CALC. NO. - REV.

AX-76B, Rev. 1H

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VENT & DRAIN QUALIFICATION:

The following Vent & Drain is qualified with generic calculation PX-SB-316.

Valve No:

ICS*VF302 ICS*VF100



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SLEEVE CLEARANCES

Piping displacements due to reanalysis of Revision 2 have not changed significantly, most of them have decreased due to the deletion of thermal and fluid transients load cases, which are not applicable anymore. The Seismic Anchor Movement load case movements are not altered, and this analysis has used N411 damping values, which reduces the dynamic displacements.

CALCULATION SHEET

	O/A	TOOLX HOLD SHEE	·	884
	CALCULATION IDEN	TIFICATION NUMBER		P.S. 6724824
JO. OR WO. NO. 12210	DIVISION & GROUP NP(C)	CALCULATION NO.	OPTIONAL TASK CODE	PAGE 205

REF.

SLEEVE CLEARANCE EVALUATION

NODE 100		NU	PIPE Output	NUPIPE Input	
Load Case	Case	-	e Deflection	Penetration	Sum of Relative
Description	No.	Dir.	Movement - 0/4	Movement	Movements
Deadweight	31	- Z	002		014
2004,023.10	-	Y	.054	0	.054
Thermal (N+U)	20	Z	,∞3	1 0	.003
		Y	.118	0	.118
Thermal (Max.)	26	===	.003	0	.003
		Y	.015	0	.015
Thermal (Min.)	23	군	.002	0	.002
Building	19-CLASS	Y	.004	0	.004
Settlement 1	2 RUN	圣	.0	.0	0
Building	20-045	Y	272	270	-,002
Settlement 2	2 RUN	圣	0	0	0
	2	Y	.195	-181	-014
OBEA	37	군	.314	.3/3	,00/
722		_ Y			.028
SSEA (2*OBEA)		근			-002
	1-	Y	.004		.004
OBEI	40	곧	.007	_	.007
		Y	.008	-	.008
SSEI	41	군	.013		.013
	10	Y	.050		-050
Dynamic OCCU	45	곤	. 075		. 075
	5=	_Y_	.050		.050
Dynamic OCCE	35	곡	.075		.075
	21	4	.05/		.05/
Dynamic OCCF	36	근	.076	-	.076

- Notes: 1. The sign of the total relative movements must be the sign of the maximum combination of Deadweight, Thermal and Building Settlement.
 - The maximum of either equation shown above is recorded on the next page to be compared with the sleeve clearance.

CALCULATION SHEET

	CALCULATION IDEN	ITIFICATION NUMBER		PS 1834 25
JO. OR WO. NO. 12210	DIVISION & GROUP NP(C)	CALCULATION NO.	OPTIONAL TASK CODE	PAGE 206
REF.				h
	CIPPUP	TEADAMCE ENALMANT	OM.	
	SEEEVE C	CLEARANCE EVALUATION	ON	
Slee	ve Clearance Looki	ing North Nobe	= 100	
	Y		Total	Relative
1	•	M	w. Mc	vements
	. 1	-z = /	5/8 = 1.625-	.079"
		z = /5	18 -1.625-	.079"
z	-2	1/5	3/" =/8/2"-	187" OK
		x = //	5/8 = 1.625"- 3/6" = 1.812"- 3/6" = 1.812"-	2.4
		-у - //	116 -1.812 -	. 187"
	-y			
5100	ve Clearance Looki	ng Fast		
5100		ing East		
	Y			
	-, ' -	-x =	= -	
		x =		
		у =	-	
		-y =		
			1 1	, i
	-у	* , * , * *		
Slee	ve Clearance Looki	ing Up Facing Nort	h	
~	×	*	* *	
	1	-z = 1	i = - 1	1
	* -			
2	-2	z =		
,		x =	z -	
	,	-x =	= -	,
	1 .		1 1	1
	-x			
Conclusion :	OK	,		
				and the same area

====
ENTERGY

AX-76B, Rev. 1H ATTACHMENT NO.:
ATTACIDATENT NO.
ATTACHMENT NO.:

ENTERGY		RIVER BEND STATION				PAGE 26 OF		
			REVIE	WER'S CH	ECKLIST			
tem	Calc Page		Comments			Respo	nse/Resolution	
		None	,					
						~~~		
-						The state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the state of the s		
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CALC. NO RI	EV. ADDENDUM
AX-76B	, Rev. 1H
ATTACHMENT I	NO.:

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### CALCULATION CHECKLIST

YES	NO	N/A	FORMAT	EDP-AA-20 SECTION
x			Cover Page completed.	6.4.1
		x	Table of Contents completed (as required).	6.4.2
x			Revision History Sheet completed (as required).	6.4.3
		х.	Revisions are identified with revision lines in right margin.	6.4.3
x	1		Applicable Documents Page completed.	6.4.4
		x	Definitions established (as required).	6.4.5
x			Calculation/revision/addendum/page numbers are identified correctly.	6.4.10
			CONTENTS	
х.			Previous calculation for the required analysis exists.	6.3
x			Calculation is appropriately titled for the intended scope.	6.4.1-2
x			Purpose and scope are clearly and adequately established.	6.4.1-7, 7.1
x			Safety classification is correct for the identified scope.	6.4.1-6
x			Topics/documents/equipment for cross-reference/retrieval are identified.	6.4.1-11
x			Calculation is clear and comprehensible.	6.1
x			Applicable codes, standards, etc. are identified.	6.4.4-1
x			RBS references are identified.	6.4.4-2
x			Affected documents are identified	6.4.4-3
x			Inputs and sources are identified, appropriate, and correct.	7.2.2-1
x			Assumptions are identified and appropriate.	7.2.2-2
		x	Inputs derived from field walkdown have been witnessed/verified	7.2.2-5
x			Engineering judgments are identified and appropriate.	7.2.2-6
x			Calculation methodology is identified and supported by technical bases.	7.2.3
x	* K		Conclusion is appropriate and is justified by calculation.	7.3
		x	Confirmations are identified and indicated as required on Cover Page.	7.5.7
		X	Directions for Confirmations are included.	7.5.7-3
x			Calculation data is appropriately included, attached, or referenced.	7.4
		x	Programs and software are identified and have been verified and validated.	8.0
x			Methods/calculations use to check results are identified and included.	
x			Results are accurate and in accordance with the established methodology.	
		X	Certification by Professional Engineer is required.	11.7
			VENDOR CALCULATIONS	
		x	Calculation is performed in accordance with EDP-AA-25.	10.2
		x	Calculation content and format are acceptable.	10.2
		x	Vendor, preparer, reviewer, and approver are clearly identified.	10.3
x			Design verification review has been completed (as applicable).  For SSN/Date:	

Reviewer (Signature/KCN or SSN/Date):

(0969) 6/18/97



### **CALCULATION** ENGINEERING DEPARTMENT ATTACHMENT NO.: RIVER BEND STATION

CALC.	NO.	- RE	v.	Al	DDENDUM
	AV	700	D		****

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		TAGE 280F					
	Design Verifi	cation Record					
Document No. AX-76B, Rev. 1H							
METHOD							
Verification methods to be used:							
X Desig	n Review						
Qualification Testing							
Alter	nate Calculations						
DOCUMENT(S) RE	VIEWED: (Attach Additional S	Sheet(s), if needed)					
Document No.	Revision	Document Title					
AX-76B	1H	Pipe Stress Analysis for ICS-006-7-2					
-	· · · · · · · · · · · · · · · · · · ·						
SUMMARY OF RE	VIEW: (Attach Additional Shee	t(s), if needed)					
Acceptable as is.							
Design Verification Completed By Garile Curole. Date 6-18-97							
		*					
Comments Resolution Acc	cepted By	Date					
Engineering Supervisor	Koyish Gupla	Date 11-19-97					