

VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

May 15, 1989

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D. C. 20555

Serial No. 89-361
NO/RPC:vlh
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION UNITS 1 AND 2
MONTHLY OPERATING REPORT

Enclosed is the Monthly Operating Report for Surry Power Station Units 1 and 2 for the month of April 1989.

Very truly yours,



W. L. Stewart
Senior Vice President - Power

Enclosure

cc: U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, N. W.
Suite 2900
Atlanta, Georgia 30323

Mr. W. E. Holland
NRC Senior Resident Inspector
Surry Power Station

IE2A

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VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION

MONTHLY OPERATING REPORT

REPORT 89-04

APPROVED:


STATION MANAGER

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OPERATING DATA REPORT

DOCKET NO. 50-280
 DATE 05/02/89
 COMPLETED BY L. A. Warren
 TELEPHONE 804-357-3184 x355

OPERATING STATUS

1. Unit Name: Surry Unit 1
2. Reporting Period: April 01-30, 1989
3. Licensed Thermal Power (Mwt): 2441
4. Nameplate Rating (Gross MWe): 847.5
5. Design Electrical Rating (Net MWe): 788
6. Maximum Dependable Capacity (Gross MWe): 820
7. Maximum Dependable Capacity (Net MWe): 781
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: _____

Notes

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reasons For Restrictions, If Any: _____

| | This Month | Yr.-to-Date | Cumulative |
|---|------------|-------------|-------------|
| 11. Hours In Reporting Period | 719.0 | 2879.0 | 143375.0 |
| 12. Number of Hours Reactor Was Critical | 0 | 0 | 88478.6 |
| 13. Reactor Reserve Shutdown Hours | 0 | 0 | 3774.5 |
| 14. Hours Generator On-Line | 0 | 0 | 86605.4 |
| 15. Unit Reserve Shutdown Hours | 0 | 0 | 3736.2 |
| 16. Gross Thermal Energy Generated (MWH) | 0 | 0 | 201171267.0 |
| 17. Gross Electrical Energy Generated (MWH) | 0 | 0 | 65203673.0 |
| 18. Net Electrical Energy Generated (MWH) | 0 | 0 | 61840403.0 |
| 19. Unit Service Factor | 0 | 0 | 60.4% |
| 20. Unit Available Factor | 0 | 0 | 63% |
| 21. Unit Capacity Factor (Using MDC Net) | 0 | 0 | 56.0% |
| 22. Unit Capacity Factor (Using DER Net) | 0 | 0 | 54.7% |
| 23. Unit Forced Rate | 100% | 100% | 21.3% |

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
 Forced Maintenance Outage on 09/14/88, scheduled on line date of 06/01/89.

| 25. If Shut Down At End Of Report Period Estimated Date of Startup: | | |
|---|----------|----------|
| 26. Units In Test Status (Prior to Commercial Operation): | Forecast | Achieved |
| INITIAL CRITICALITY | _____ | _____ |
| INITIAL ELECTRICITY | _____ | _____ |
| COMMERCIAL OPERATION | _____ | _____ |

OPERATING DATA REPORT

DOCKET NO. 50-281
 DATE 05/02/89
 COMPLETED BY L. A. Warren
 TELEPHONE 804-357-3184 x355

OPERATING STATUS

1. Unit Name: Surry Unit 2
2. Reporting Period: April 01-30, 1989
3. Licensed Thermal Power (Mwt): 2441
4. Nameplate Rating (Gross MWe): 847.5
5. Design Electrical Rating (Net MWe): 788
6. Maximum Dependable Capacity (Gross MWe): 820
7. Maximum Dependable Capacity (Net MWe): 781
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: _____

| |
|-------|
| Notes |
|-------|

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reasons For Restrictions, If Any: _____

| | This Month | Yr.-to-Date | Cumulative |
|---|------------|-------------|-------------|
| 11. Hours In Reporting Period | 719.0 | 2879.0 | 140255.0 |
| 12. Number of Hours Reactor Was Critical | 0 | 0 | 89694.3 |
| 13. Reactor Reserve Shutdown Hours | 0 | 0 | 328.1 |
| 14. Hours Generator On-Line | 0 | 0 | 88293.0 |
| 15. Unit Reserve Shutdown Hours | 0 | 0 | 0 |
| 16. Gross Thermal Energy Generated (MWH) | 0 | 0 | 206740436.1 |
| 17. Gross Electrical Energy Generated (MWH) | 0 | 0 | 67136244.0 |
| 18. Net Electrical Energy Generated (MWH) | 0 | 0 | 63647378.0 |
| 19. Unit Service Factor | 0 | 0 | 63.2 % |
| 20. Unit Available Factor | 0 | 0 | 63% |
| 21. Unit Capacity Factor (Using MDC Net) | 0 | 0 | 58.2% |
| 22. Unit Capacity Factor (Using DER Net) | 0 | 0 | 57.6% |
| 23. Unit Forced Rate | 0 | 0 | 15% |

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Refueling Outage on 09/10/88, scheduled on line date of 06/22/89.

25. If Shut Down At End Of Report Period Estimated Date of Startup: _____

| 26. Units In Test Status (Prior to Commercial Operation): | Forecast | Achieved |
|---|----------|----------|
| INITIAL CRITICALITY | _____ | _____ |
| INITIAL ELECTRICITY | _____ | _____ |
| COMMERCIAL OPERATION | _____ | _____ |

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-280
 UNIT NAME Surry Unit 1
 DATE 05/05/89
 COMPLETED BY L. A. Warren
 TELEPHONE 804-357-3184 x355

REPORT MONTH APRIL 1989

| NO. | DATE | Type ¹ | Duration (Hours) | Reason ² | Method of Shutting Down Reactor ³ | LICENSEE EVENT REPORT # | System Code ⁴ | Component Code ⁵ | CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENT |
|-------|----------|-------------------|---------------------|---------------------|--|-------------------------------|-----------------------------|--------------------------------|---|
| 89-04 | 04/01/89 | F | 720.0 | F | 1 | | | | Unit shutdown due to emergency diesel generator operability concerns. |

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & License Examination
 F - Administrative
 G - Operational Error (Explain)
 H - Other (Explain)

³
 Method:
 1 - Manual
 2 - Manual Scram.
 3 - Automatic Scram.
 4 - Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File
 (NUREG 0161)

⁵
 Exhibit 1 - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-281
 UNIT NAME Surv Unit 2
 DATE 05/05/89
 COMPLETED BY L. A. Warren
 TELEPHONE 804-357-3184 x355

REPORT MONTH APRIL 1989

| NO. | DATE | Type ¹ | Duration (Hours) | Reason ² | Method of Shutting Down Reactor ³ | LICENSEE EVENT REPORT # | System Code ⁴ | Component Code ⁵ | CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENT |
|-------|----------|-------------------|---------------------|---------------------|--|-------------------------------|-----------------------------|--------------------------------|---|
| 89-04 | 04/01/89 | S | 720 .0 | C | 1,3 | | | | Unit shutdown for refueling outage; automatic reactor trip. |

¹
 F: Forced
 S: Scheduled

² Reason:
 A - Equipment Failure (Explain)
 B - Maintenance or Test
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & License Examination
 F - Administrative
 G - Operational Error (Explain)
 H - Other (Explain)

³ Method:
 1 - Manual
 2 - Manual Scram.
 3 - Automatic Scram.
 4 - Other (Explain)

⁴ Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File
 (NUREG 0161)

⁵ Exhibit 1 - Same Source

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-280
 UNIT Surry Unit 1
 DATE 05/02/89
 COMPLETED BY L. A. Warren
 TELEPHONE 804-357-3184 x355

MONTH April 1989

| DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) | DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) |
|-----|--|-----|--|
| 1 | <u>0</u> | 17 | <u>0</u> |
| 2 | <u>0</u> | 18 | <u>0</u> |
| 3 | <u>0</u> | 19 | <u>0</u> |
| 4 | <u>0</u> | 20 | <u>0</u> |
| 5 | <u>0</u> | 21 | <u>0</u> |
| 6 | <u>0</u> | 22 | <u>0</u> |
| 7 | <u>0</u> | 23 | <u>0</u> |
| 8 | <u>0</u> | 24 | <u>0</u> |
| 9 | <u>0</u> | 25 | <u>0</u> |
| 10 | <u>0</u> | 26 | <u>0</u> |
| 11 | <u>0</u> | 27 | <u>0</u> |
| 12 | <u>0</u> | 28 | <u>0</u> |
| 13 | <u>0</u> | 29 | <u>0</u> |
| 14 | <u>0</u> | 30 | <u>0</u> |
| 15 | <u>0</u> | 31 | <u>0</u> |
| 16 | <u>0</u> | | |

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-281

UNIT Surry UnitDATE 05/02/89COMPLETED BY L. A. WarrenTELEPHONE 804-357-3184 x355MONTH April 1989

| DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) | DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) |
|-----|--|-----|--|
| 1 | 0 | 17 | 0 |
| 2 | 0 | 18 | 0 |
| 3 | 0 | 19 | 0 |
| 4 | 0 | 20 | 0 |
| 5 | 0 | 21 | 0 |
| 6 | 0 | 22 | 0 |
| 7 | 0 | 23 | 0 |
| 8 | 0 | 24 | 0 |
| 9 | 0 | 25 | 0 |
| 10 | 0 | 26 | 0 |
| 11 | 0 | 27 | 0 |
| 12 | 0 | 28 | 0 |
| 13 | 0 | 29 | 0 |
| 14 | 0 | 30 | 0 |
| 15 | 0 | 31 | |
| 16 | 0 | | |

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

SUMMARY OF OPERATING EXPERIENCEMONTH/YEAR APRIL 1989

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT ONE

04/01/89 0000 This reporting period begins with the Unit at CSD.

04/06/89 0431 Loss of electrical transfer bus 'F' due to a failed lightning arrestor on #2 auto-tie transformer. During this evolution, the operating residual heat removal (RHR) pump was lost when the stub bus breaker tripped. The 'B' train RHR pump was started and RHR flow was restored within one minute. The 1H emergency bus de-energized.

0458 #1 EDG was started and loaded. 1H emergency bus re-energized.

0817 NRC notified of the above event.

04/13/89 1026 Loss of electrical transfer bus 'F' due to improper testing of relays in the distribution switchyard. 1H emergency bus de-energized.

1207 'F' transfer bus re-energized.

1343 1H emergency bus re-energized.

04/30/89 2400 This reporting period ends with the Unit at CSD.

UNIT TWO

04/01/89 0000 This reporting period starts with the Unit at CSD.

04/06/89 0431 Loss of electrical transfer bus 'F' due to failed lightning arrestor on #2 auto-tie transformer. #3 EDG automatically started and loaded on 2J emergency bus.

0715 'F' transformer bus re-energized.

0919 #3 EDG secured.

04/13/89 1026 Loss of electrical transfer bus 'F' due to improper testing of relays in the distribution switchyard. #3 EDG automatically started and loaded on 2J emergency bus.

1207 'F' transfer bus re-energized.

1335 #3 EDG secured.

04/30/89 2400 This reporting period ends with the Unit at CSD.

FACILITY CHANGES REQUIRING NRC APPROVAL

MONTH/YEAR APRIL 1989

NONE DURING THIS PERIOD

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVALMONTH/YEAR APRIL 1989**DC 83-03 LIFTING LUGS FOR MAIN STEAM SAFETY VALVES UNIT 1**

This design change installed lifting lugs in the ceiling of the upper level of the main steam valve house. The purpose of the lugs is to assist in the handling and rigging of the main steam safety valves during maintenance evolutions.

SUMMARY OF SAFETY ANALYSIS

The installation of the lugs did not affect the seismic or tornado resistance qualifications of the main steam valve house structure. The lugs do not affect any other safety related system and/or structure.

DC 84-67 CHARGING PUMP SERVICE WATER PIPING REPLACEMENT UNITS 1&2

This modification replaced the charging pump service water fiberglass and silver brazed Cu-Ni piping in the Turbine Building - Auxiliary Building pipe tunnel and in the Auxiliary Building with heavy wall socket welded copper - nickel piping. The valves in these pipe lines were replaced with bronze body flanged valves.

SUMMARY OF SAFETY ANALYSIS

The seismically qualified replacement piping will perform the same function as the fiberglass piping. The piping will not be as susceptible to fire damage. No changes other than material replacement were made to the charging pump service water system. The flow rate and heat removal capacity of the charging pump service water system which is used to ensure proper cooling of the charging pumps will not change from the existing requirements. The applicable technical specifications are not affected.

DC 87-33 WEED 9007 RTD REPLACEMENT UNIT 2

This modification replaced the eighteen (18) narrow range RTDs with RTDs which have adequately demonstrated environmental qualification as set forth by NUREG 0588 Category 1.

SUMMARY OF SAFETY ANALYSIS

The replacement RTDs are essentially one for one replacements of an enhanced design and will not affect any of the operations or ability of equipment important to safety to perform their safety functions.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVALMONTH/YEAR APRIL 1989SCAFFOLDING REQUEST

04/01/89

This request erected temporary scaffolding located in Unit 1 containment 'C' loop room to work 1-RC-FE-1482.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST

04/01/89

This request erected temporary scaffolding located in Unit 1 containment 'A' loop to work 1-RC-FE-1480.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST

04/01/89

This request erected temporary scaffolding located in Unit 1 containment A, B, and C loop room to work various valves and flanges.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST

04/02/89

This request erected temporary scaffolding located in Unit 1 containment 'A' loop room to work 01-RC-FE-1490.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST

04/02/89

This request erected temporary scaffolding located in Unit 2 containment seal table room to allow disconnect and layout of thermocouple cable.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVALMONTH/YEAR APRIL 1989**SCAFFOLDING REQUEST****04/06/89**

This request erected temporary scaffolding located in Unit 2 containment pressurizer cubicle to work NDE of 4" pressurizer spray line.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

TM-S1-89-69 TEMPORARY MODIFICATION**04/07/89**

Temporary modification to secure the exhaust damper for 1-VS-1B in the open position.

This change does not constitute an unreviewed safety question because the containment air recirculation system is not required to function during design basis events.

TM-S1-89-70 TEMPORARY MODIFICATION**04/07/89**

Temporary modification to secure the exhaust damper for 1-VS-F-1C in the open position.

This change does not constitute an unreviewed safety question in that the containment air recirculation system is not required to function during design basis events.

SCAFFOLDING REQUEST**04/12/89**

This request erected temporary scaffolding located in Unit 2 containment to allow disassembly and inspection of safety injection check valves.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST**04/12/89**

This request erected a temporary scaffolding located in Unit 1 containment 2' level to work I-SI-225.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVALMONTH/YEAR APRIL 1989**SCAFFOLDING REQUEST****04/12/89**

This request erected temporary scaffolding located in Unit 1 containment 27' level to work I-SI-148.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST**04/20/89**

This request erected temporary scaffolding located in mechanical equipment room # 1 to work piping insulation.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST**04/20/89**

This request erected temporary scaffolding located in Unit 1 emergency switchgear room to work on hanger installation.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

I&2-EWR-88-495 ENGINEERING WORK REQUEST**04/21/89**

Request to install instrumentation for obtaining an accurate measurement of each of the three emergency service water pumps discharge pressure and flow during pump testing and operation.

The addition of flow instrumentation is an enhancement to the service water pumps providing the ability to obtain more accurate flow rate data to help improve system operability. Therefore, an unreviewed safety question is not created.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVALMONTH/YEAR APRIL 1989**1&2-EWR-88-467 ENGINEERING WORK REQUEST****04/21/89**

This request was generated to ensure the capability of the emergency service water pumps to provide make up water to the Intake Canal for the thirty day Loss of Off-site Power Accident.

The changes reviewed enhance the plant capability to mitigate the effects of an accident. The procedural changes decrease the probability of equipment malfunction. Failure of the added equipment results in the same failure modes previously evaluated. Therefore, an unreviewed safety question is not created.

SCAFFOLDING REQUEST**04/23/89**

The request erected temporary scaffolding located in Unit 1 containment 27' level to work MOV-1720A repack.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST**04/23/89**

This request erected temporary scaffolding located in mechanical equipment room #3 to work 1-VS-P-23 and 1-SW-PCV-101B.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST**04/23/89**

This request erected temporary scaffolding located in Unit 1 containment elevation 13' to work 1-RH-E-1 B outlet.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST**04/27/89**

This request erected temporary scaffolding located in mechanical equipment room # 3, elevation 9'6 " to work mechanical equipment room #3 charging pump and service water pump room penetration.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVALMONTH/YEAR APRILSCAFFOLDING REQUEST

04/27/89

This request erected temporary scaffolding located in Unit 1 containment 'C' loop room to work 1-RC-102.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST

04/27/89

This request erected temporary scaffolding located in Unit 1 containment 'C' loop room to repack 1-RC-81.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST

04/27/89

This request erected temporary scaffolding located in Unit 1 containment 'C' loop room to work flange leak (1-RC-FNC).

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST

04/27/89

This request erected temporary scaffolding located in Unit 2 containment to work 2-RC-PCV-2455B.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST

04/27/89

This request erected temporary scaffolding located in Unit 1 containment to work 1-SI-228.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVALMONTH/YEAR APRIL 1989SCAFFOLDING REQUEST

04/27/89

This request erected temporary scaffolding located in Unit 1 containment basement to work 1-SI-MOV-1865B.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST

04/27/89

This request erected temporary scaffolding located in 'C' loop room Unit 1 containment to work electrical junction box (1-EPL-MISC-Box)

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST

04/28/89

This request erected temporary scaffolding located in Unit 1 containment 'B' loop room to work 1-SI-238.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST

04/28/89

This request erected temporary scaffolding located in Unit 1 containment 'B' loop room to work 1-RC-50.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

SCAFFOLDING REQUEST

04/28/89

This request erected temporary scaffolding located in mechanical equipment room #3 to work 1-VS-287/291; 1-VS-P-2A1B outlet isolations.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVALMONTH/YEAR APRIL 1989SCAFFOLDING REQUEST

04/30/89

This request erected temporary scaffolding located in Unit 2 containment 3' elevation to work 2-SI-224, Tc discharge check valve.

Installation of this temporary scaffolding was reviewed for effect on accident analyses and equipment operability/function. Conclusion is that assumptions, bases and probabilities of accident analyses and equipment malfunctions are not affected.

PROCEDURE OR METHOD OF OPERATION CHANGES
REQUIRING NRC APPROVAL

MONTH/YEAR APRIL 1989

NONE DURING THIS PERIOD

**PROCEDURE OR METHOD OF OPERATION CHANGES
THAT DID NOT REQUIRE NRC APPROVAL**

MONTH/YEAR APRIL 1989

1 & 2-TOP-2096 TEMPORARY OPERATION G PROCEDURE

04/10/89

This procedure allows filling a reactor coolant system loop and opening its loop stop valves following cold shutdown maintenance.

The procedure requires greater than 5% shutdown margin, the loop be filled with water from the reactor coolant system, the containment ambient temperature be within 20°F of reactor coolant temperature and sufficient initial reactor coolant system inventory. These actions ensure an unreviewed safety question is not generated.

1-TOP-2097 TEMPORARY OPERATING PROCEDURE

04/15/89

During ST-244 testing, VB 1-III, circuit 15, will be opened rendering a residual heat removal (RHR) system valve controlling heat exchangers bypass inoperable (1-RH-HCV-1758). In order not to place a bypass flow or temperature transient on the system, HCV-1758 will be placed in the full open position and flow will be controlled by 1-RH-19 manually.

In accordance with ST-244, this procedure ensures that during the testing of VB-1-III, circuit 15, that the residual heat removal (RHR) system does not receive a temperature or flow transient. RHR flow will be maintained during testing. Therefore, this temporary procedure does not create an unreviewed safety question.

TESTS AND EXPERIMENTS REQUIRING NRC APPROVAL

MONTH/YEAR APRIL 1989

NONE DURING THIS PERIOD

TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

MONTH/YEAR APRIL 1989

NONE DURING THIS PERIOD

VIRGINIA POWER
SURRY POWER STATION
CHEMISTRY REPORT

APRIL 19 89

| PRIMARY COOLANT ANALYSIS | UNIT NO. 1 | | | UNIT NO. 2 | | |
|---|------------|---------|---------|------------|---------|---------|
| | MAX. | MIN. | AVG. | MAX. | MIN. | AVG. |
| Gross Radioact., $\mu\text{Ci/ml}$ | 7.80E-4 | 2.58E-4 | 5.15E-4 | 3.28E-3 | 9.70E-4 | 1.94E-3 |
| Suspended Solids, ppm | 0.0 | 0.0 | 0.0 | 0.0 | 0.0 | 0.0 |
| Gross Tritium, $\mu\text{Ci/ml}$ | - | - | - | - | - | - |
| Iodine ^{131}I , $\mu\text{Ci/ml}$ | - | - | - | - | - | - |
| $^{131}\text{I} / ^{133}\text{I}$ | - | - | - | - | - | - |
| Hydrogen, cc/kg | - | - | - | - | - | - |
| Lithium, ppm | 0.55 | 0.49 | 0.52 | 0.21 | 0.14 | 0.17 |
| Boron-10, ppm* | 422 | 395 | 404 | 434 | 422 | 427 |
| Oxygen, (DO), ppm | 1.000 | 0.500 | 0.800 | 0.050 | 0.005 | 0.007 |
| Chloride, ppm | 0.010 | 0.004 | 0.006 | 0.013 | 0.006 | 0.009 |
| pH @ 25 degree Celsius | 5.24 | 5.02 | 5.13 | 5.01 | 4.75 | 4.90 |

* Boron-10 = Total Boron X 0.196

UNIT ONE:

The unit remained at cold shutdown with RHR and B-MB in service for the entire month. 'C' loop was isolated on 04/03/89 and drained on 04/10/89 for maintenance on valves. 'C' loop has remained drained and isolated through 04/30/89.

UNIT TWO:

The unit remained at cold shutdown with RHR and A-MB in service for the entire month. 'C' loop was isolated on 04/01/89 for normal valve maintenance, and filled and returned to normal line-up on 04/11/89 at 0945 hrs. Letdown was secured from 0935 to 1725 hrs. on 04/11/89 for maintenance to CVCS and charging MOV s.

DESCRIPTION OF PERIODIC TEST WHICH WERE NOT COMPLETED
WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS

MONTH/YEAR APRIL 1989

NONE DURING THIS PERIOD