VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

December 14, 1998

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Serial No. 98-724
SPS Lic/JSA R0
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

Gentlemen:

**VIRGINIA ELECTRIC AND POWER COMPANY**
**SURRY POWER STATION UNITS 1 AND 2**
**MONTHLY OPERATING REPORT**

The Monthly Operating Report for Surry Power Station Units 1 and 2 for the month of November 1998 is provided in the attachment.

If you have any questions or require additional information, please contact us.

Very truly yours,

E. S. Grecheck, Site Vice President
Surry Power Station

Attachment

Commitments made by this letter: None

cc: U. S. Nuclear Regulatory Commission
Region II
Atlanta Federal Center
61 Forsyth Street, S. W.
Suite 23T85
Atlanta, Georgia 30303

Mr. R. A. Musser
NRC Senior Resident Inspector
Surry Power Station

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# VIRGINIA ELECTRIC AND POWER COMPANY
# SURRY POWER STATION
# MONTHLY OPERATING REPORT
# REPORT NO. 98-11

Approved: _____ 12/14/98
Site Vice President Date

## TABLE OF CONTENTS

## OPERATING DATA REPORT

Docket No.: 50-280
Date: 12/03/98
Completed By: R. Stief
Telephone: (757) 365-2486

1. Unit Name: ..... Surry Unit 1
2. Reporting Period: ..... November 1998
3. Licensed Thermal Power (MWt): ..... 2546
4. Nameplate Rating (Gross MWe): ..... 847.5
5. Design Electrical Rating (Net MWe): ..... 788
6. Maximum Dependable Capacity (Gross MWe): ... 840
7. Maximum Dependable Capacity (Net MWe): ..... 801

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

| | | This Month | Year-To-Date | Cumulative |
|---|---|---|---|---|
| 11. | Hours in Reporting Period | 720.0 | 8016.0 | 227400.0 |
| 12. | Hours Reactor Was Critical | 232.1 | 6546.7 | 160573.1 |
| 13. | Reactor Reserve Shutdown Hours | 0.0 | 0.0 | 3774.5 |
| 14. | Hours Generator On-Line | 146.1 | 6427.0 | 158027.4 |
| 15. | Unit Reserve Shutdown Hours | 0.0 | 0.0 | 3736.2 |
| 16. | Gross Thermal Energy Generated (MWH) | 245387.7 | 16059355.7 | 372304371.3 |
| 17. | Gross Electrical Energy Generated (MWH) | 75706.0 | 5324391.0 | 122140145.0 |
| 18. | Net Electrical Energy Generated (MWH) | 71919.0 | 5143534.0 | 116377755.0 |
| 19. | Unit Service Factor | 20.2% | 80.1% | 69.4% |
| 20. | Unit Availability Factor | 20.2% | 80.1% | 71.1% |
| 21. | Unit Capacity Factor (Using MDC Net) | 12.5% | 80.1% | 65.7% |
| 22. | Unit Capacity Factor (Using DER Net) | 12.7% | 81.4% | 64.9% |
| 23. | Unit Forced Outage Rate | 37.8% | 7.6% | 14.5% |

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down at End of Report Period, Estimated Date of Start-up:

26. Unit In Test Status (Prior to Commercial Operation):

| | FORECAST | ACHIEVED |
|---|---|---|
| INITIAL CRITICALITY | | |
| INITIAL ELECTRICITY | | |
| COMMERCIAL OPERATION | | |

## OPERATING DATA REPORT

Docket No.: 50-281
Date: 12/03/98
Completed By: R. Stief
Telephone: (757) 365-2486

1. Unit Name: ..... Surry Unit 2
2. Reporting Period: ..... November 1998
3. Licensed Thermal Power (MWt): ..... 2546
4. Nameplate Rating (Gross MWe): ..... 847.5
5. Design Electrical Rating (Net MWe): ..... 788
6. Maximum Dependable Capacity (Gross MWe): ... 840
7. Maximum Dependable Capacity (Net MWe): ..... 801

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

| | | This Month | Year-To-Date | Cumulative |
|---|---|---|---|---|
| 11. | Hours in Reporting Period | 720.0 | 8016.0 | 224281.0 |
| 12. | Hours Reactor Was Critical | 720.0 | 8016.0 | 159166.3 |
| 13. | Reactor Reserve Shutdown Hours | 0.0 | 0.0 | 328.1 |
| 14. | Hours Generator On-Line | 720.0 | 8016.0 | 157149.5 |
| 15. | Unit Reserve Shutdown Hours | 0.0 | 0.0 | 0.0 |
| 16. | Gross Thermal Energy Generated (MWH) | 1832936.7 | 20370118.5 | 371903685.8 |
| 17. | Gross Electrical Energy Generated (MWH) | 614975.0 | 6787245.0 | 121912853.0 |
| 18. | Net Electrical Energy Generated (MWH) | 595661.0 | 6564925.0 | 116208076.0 |
| 19. | Unit Service Factor | 100.0% | 100.0% | 70.1% |
| 20. | Unit Availability Factor | 100.0% | 100.0% | 70.1% |
| 21. | Unit Capacity Factor (Using MDC Net) | 103.3% | 102.2% | 66.2% |
| 22. | Unit Capacity Factor (Using DER Net) | 105.0% | 103.9% | 65.8% |
| 23. | Unit Forced Outage Rate | 0.0% | 0.0% | 11.4% |

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Refueling, April 19, 1999, 35 Days

25. If Shut Down at End of Report Period, Estimated Date of Start-up:

26. Unit In Test Status (Prior to Commercial Operation):

| | FORECAST | ACHIEVED |
|---|---|---|
| INITIAL CRITICALITY | | |
| INITIAL ELECTRICITY | | |
| COMMERCIAL OPERATION | | |

## UNIT SHUTDOWN AND POWER REDUCTION
### (EQUAL TO OR GREATER THAN 20%)

**REPORT MONTH:** November 1998

Docket No.: 50-280
Unit Name: Surry Unit 1
Date: 12/01/98
Completed by: J. R. Pincus
Telephone: (757) 365-2863

| Date | (1) Type | Duration Hours | (2) Reason | (3) Method of Shutting Down Rx | LER No. | (4) System Code | (5) Component Code | Cause & Corrective Action to Prevent Recurrence |
|---|---|---|---|---|---|---|---|---|
| 11/01/98 | S | 485H02M | C | 1 | N/A | N/A | N/A | Refueling Outage |
| 11/22/98 | F | 62H25M | A | 3 | S1-98-013-00 | SB | SG | "B" Steam Generator High Level |
| 11/26/98 | F | 26H27M | A | 2 | S1-98-014-00 | SJ | FCV | Flow Control Valve Failed Closed |

(1)
F: Forced
S: Scheduled

(2)
REASON:
A - Equipment Failure (Explain)
B - Maintenance or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training & Licensing Examination
F - Administrative
G - Operational Error (Explain)

(3)
METHOD:
1 - Manual
2 - Manual Scram
3 - Automatic Scram
4 - Other (Explain)

(4)
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

(5)
Exhibit 1 - Same Source

## UNIT SHUTDOWN AND POWER REDUCTION
### (EQUAL TO OR GREATER THAN 20%)

**REPORT MONTH:** November 1998

Docket No.: 50-281
Unit Name: Surry Unit 2
Date: 12/01/98
Completed by: J. R. Pincus
Telephone: (757) 365-2863

None during the Reporting Period

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(1)
F: Forced
S: Scheduled

(2)
REASON:
A - Equipment Failure (Explain)
B - Maintenance or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training & Licensing Examination
F - Administrative
G - Operational Error (Explain)

(3)
METHOD:
1 - Manual
2 - Manual Scram
3 - Automatic Scram
4 - Other (Explain)

(4)
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

(5)
Exhibit 1 - Same Source

## AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-280
Unit Name: Surry Unit 1
Date: 12/03/98
Completed by: J. S. Ashley
Telephone: (757) 365-2161

**MONTH:** November 1998

| Day | Average Daily Power Level (MWe - Net) | Day | Average Daily Power Level (Mwe - Net) |
|---|---|---|---|
| 1 | 0 | 17 | 0 |
| 2 | 0 | 18 | 0 |
| 3 | 0 | 19 | 0 |
| 4 | 0 | 20 | 0 |
| 5 | 0 | 21 | 132 |
| 6 | 0 | 22 | 30 |
| 7 | 0 | 23 | 0 |
| 8 | 0 | 24 | 32 |
| 9 | 0 | 25 | 322 |
| 10 | 0 | 26 | 487 |
| 11 | 0 | 27 | 0 |
| 12 | 0 | 28 | 431 |
| 13 | 0 | 29 | 754 |
| 14 | 0 | 30 | 806 |
| 15 | 0 | | |
| 16 | 0 | | |

**INSTRUCTIONS**

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

## AVERAGE DAILY UNIT POWER LEVEL

Docket No.: 50-281
Unit Name: Surry Unit 2
Date: 12/03/98
Completed by: J. S. Ashley
Telephone: (757) 365-2161

**MONTH:** November 1998

| Day | Average Daily Power Level (Mwe - Net) | Day | Average Daily Power Level (Mwe - Net) |
|---|---|---|---|
| 1 | 829 | 17 | 824 |
| 2 | 829 | 18 | 824 |
| 3 | 830 | 19 | 825 |
| 4 | 831 | 20 | 825 |
| 5 | 830 | 21 | 826 |
| 6 | 832 | 22 | 825 |
| 7 | 832 | 23 | 824 |
| 8 | 832 | 24 | 824 |
| 9 | 828 | 25 | 826 |
| 10 | 828 | 26 | 827 |
| 11 | 829 | 27 | 824 |
| 12 | 828 | 28 | 826 |
| 13 | 828 | 29 | 826 |
| 14 | 827 | 30 | 827 |
| 15 | 828 | | |
| 16 | 827 | | |

**INSTRUCTIONS**

On this format, list the average daily unit power level in MWe - Net for each day in the reporting month. Compute to the nearest whole megawatt.

## SUMMARY OF OPERATING EXPERIENCE

**MONTH/YEAR:** November 1998

The following chronological sequence by unit is a summary of operating experiences for this month that required load reductions or resulted in significant non-load related incidents.

**UNIT ONE:**

| | | |
|---|---|---|
| 11/01/98 | 0000 | Unit starts the month at 0% / 0 MWe due to Refueling Outage. |
| 11/19/98 | 0354 | Commenced reactor start-up. |
| 11/19/98 | 0527 | Reactor critical. |
| 11/21/98 | 0502 | Unit on-line, commenced ramp up. |
| 11/21/98 | 0550 | Stopped ramp, unit at 29% / 210 MWe. |
| 11/22/98 | 0031 | Clear for power increase > 30%. |
| 11/22/98 | 0430 | Reactor tripped. |
| 11/23/98 | 1218 | Commenced reactor start-up. |
| 11/23/98 | 1346 | Reactor critical. |
| 11/24/98 | 1855 | Unit on-line, commenced ramp up. |
| 11/24/98 | 1945 | Stopped ramp, unit at 30% / 217 MWe. |
| 11/25/98 | 0305 | Recommended ramp up, unit at 29% / 199 MWe. |
| 11/25/98 | 0505 | Stopped ramp, unit at 35% / 253 MWe. |
| 11/25/98 | 0610 | Recommenced ramp, unit at 35% / 253 MWe. |
| 11/25/98 | 1445 | Stopped ramp, unit at 54% / 415 MWe. |
| 11/25/98 | 1820 | Recommenced ramp, unit at 54% / 415 MWe. |
| 11/26/98 | 0200 | Stopped ramp, unit at 70% / 560 MWe. |
| 11/26/98 | 1511 | Recommenced ramp, unit at 68% / 571 MWe. |
| 11/26/98 | 2108 | Reactor tripped. |
| 11/27/98 | 1439 | Reactor critical. |
| 11/27/98 | 2335 | Unit on-line, commenced ramp up. |
| 11/28/98 | 0017 | Stopped ramp, unit at 30% / 210 MWe. |
| 11/28/98 | 0027 | Recommenced ramp, unit at 30% / 210 MWe. |
| 11/28/98 | 0740 | Stopped ramp, unit at 49.75% / 400 MWe. |
| 11/28/98 | 0758 | Unit at 50%, ramping up at 3% per hour. |
| 11/28/98 | 2325 | Stopped ramp, unit at 80% / 700 MWe. |
| 11/29/98 | 0051 | Recommenced ramp, unit at 80% / 700 MWe. |
| 11/29/98 | 0411 | Stopped ramp, unit at 90% / 770 MWe. |
| 11/29/98 | 0659 | Recommenced ramp, unit at 90% / 770 MWe. |

## SUMMARY OF OPERATING EXPERIENCE (CONTINUED)

**MONTH/YEAR:** November 1998

**UNIT ONE (CONTINUED):**

| | | |
|---|---|---|
| 11/29/98 | 0824 | Stopped ramp, unit at 94.5% / 810 MWe. |
| 11/29/98 | 1255 | Recommenced ramp, unit at 94.5% / 810 MWe. |
| 11/29/98 | 1505 | Stopped ramp, unit at 96% / 820 MWe. |
| 11/29/98 | 1720 | Commenced ramp down, unit at 96% / 820 MWe. |
| 11/29/98 | 1940 | Stopped ramp, unit at 90.2% / 785 MWe. |
| 11/29/98 | 1945 | Commenced ramp up, unit at 90.2% / 785 MWe. |
| 11/29/98 | 2305 | Stopped ramp, unit at 98.5% / 835 MWe. |
| 11/30/98 | 2400 | Unit finishes the month at 98.5% / 837 MWe. |

**UNIT TWO:**

| | | |
|---|---|---|
| 11/01/98 | 0000 | Unit starts the month at 100% / 855 MWe. |
| 11/30/98 | 2400 | Unit finishes the month at 100% / 855 MWe. |

**FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR:** November 1998

DCP 94-071 — **Design Change Package** — 11/03/98
(Safety Evaluation 94-051)

Design Change Package 94-071, "Condensate Polishing Setpoint Changes for Resin Regeneration/Surry/1 &2", added additional cation resin to the polishers for longer run time.

DCP 97-059 — **Design Change Package** — 11/11/98
(Safety Evaluation 98-051 Rev. 1)

Design Change Package 97-059, "CS CAT Supply Valves Replacement 01-CS-MOV-102A/B", eliminates the leakage of the Chemical Addition Tank (CAT) contents past the CAT isolation valves into the Containment Spray piping by replacing the valves with new ones.

DCP 98-007 — **Design Change Package** — 11/03/98
(Safety Evaluation 98-058, Rev. 1)

Design Change Package 98-007, "Containment Radiant Energy Shields", installs Marinite board and stainless steel in containment that either replaces or sheaths existing Thermo-Lag material to perform as radiant energy shields. The installation of the Marinite board and stainless steel will provide a noncombustible radiant energy shield in accordance with NRC requirements.

SE 98-076, Rev. 1 — **Safety Evaluation** — 11/14/98

JCO 97-006 set Compensatory Actions to provide additional monitoring of containment temperatures for indication of a possible fire due to a regulatory compliance issue regarding Thermo-lag radiant energy shields inside Unit 1 and 2 containments. Revision 1 of Safety Evaluation 98-076 addresses Revision 2 of JCO 97-006 which: (1) updates the status of the JCO in accordance with administrative procedures; (2) evaluates the discontinuance of the Unit 1 Compensatory Actions following implementation of Unit 1, DCP 98-007, which resolved the Thermo-lag regulatory compliance issue inside Unit 1 containment; and (3) evaluates continuance of the Unit 2 Compensatory Actions.

SE 98-103, Rev. 1 — **Safety Evaluation** — 11/05/98

A safety evaluation has been performed to determine whether an unreviewed safety question will result from the refueling and operation of Surry Unit 1 Cycle 16. Revision 1 of Safety Evaluation 98-103 accommodates the reevaluation performed for introducing an additional reconstituted assembly in the core – the present core loading already contains a reconstituted assembly at the core's center. Fuel inspection at the end of Cycle 15 identified a leaker among the once burned assemblies intended for re-use in Cycle 16. The failed assembly was reconstituted by replacing the failed fuel rod with a stainless steel filler rod. The reevaluation shows the effect of this reconstitution is negligible.

**FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR:** November 1998

TM S1-98-020
TM S1-98-021

**Temporary Modifications** 11/03/98
(Safety Evaluation 98-114)

Temporary Modifications S1-98-020 and S1-98-021 were used to remove the containment sump pump discharge containment isolation trip valve actuators, one at a time, for maintenance and install electrical jumpers to allow operation of the containment sump pumps with the trip valve actuator removed.

DCP 98-069

**Design Change Package** 11/03/98
(Safety Evaluation 98-115)

Design Change Package 98-069, "Control Rod Drive Mechanism (CRDM) Housing Leak Repair/Surry/Unit 1", describes the repairing of two reactor vessel head nozzle canopy seal weld leaks using Canopy Seal Clamp Assemblies (CSCA). CSCA is a permanent non-welded mechanical method of leak repair developed by ABB Combustion Engineering and has been fully tested at operating temperature, pressure and seismic conditions for nuclear power plant use.

TM S1-98-022

**Temporary Modification** 11/06/98
(Safety Evaluation 98-116)

The Primary Drain Transfer Tank (PDTT) Pressure Control Valve, 1-DG-PCV-100, has been noted to be difficult to open once closed due to plant conditions. Temporary Modification S1-98-022 was used to fail 1-DG-PCV-100 in the open position during Cold Shutdown or Refueling Shutdown in order to provide a flow path for Reactor Cooling System (RCS) loop drains to the PDTT. The TM was removed before unit start-up.

DCP 98-044

**Design Change Package** 11/13/98
(Safety Evaluation 98-118)

Design Change Package 98-044, "Turbine Driven Auxiliary Feedwater Pump (TDAFWP) Steam Supply Circuit Upgrade", modifies the TDAFWP circuit logic such that, when the pump is started by an automatic signal and the initiating signal is cleared, steam flow to the TDAFWP can only be stopped after the circuit is manually reset.

TM S1-98-023

**Temporary Modification** 11/17/98
(Safety Evaluation 98-121)

Temporary Modification S1-98-023 will be used during plant shutdown to bypass the bearing lift oil pressure anti-start interlock in the Main Turbine Turning Gear motor circuitry to allow periodic rotation of the turbine rotor using the turning gear.

TM S2-98-007

**Temporary Modification** 11/20/98
(Safety Evaluation 98-123)

Temporary Modification S2-98-007 is to disable a leaking thermal relief valve (RV) and install another RV at the vent valve on the discharge side of the Unit 2 Feedwater Heater. ASME Code section VIII allows this change until the next available outage in which the defective RV shall be replaced.

## FACILITY CHANGES THAT DID NOT REQUIRE NRC APPROVAL

**MONTH/YEAR:** November 1998

SE 98-124 **Safety Evaluation** 11/23/98

Safety Evaluation 98-124 evaluates the reduction in Containment wall thickness from that specified in UFSAR Section 15 while work is in process to remove and repair spalled/loose concrete from the Unit 2 Containment exterior wall.

TM S1-98-024
TM S1-98-025 **Temporary Modifications** 11/27/98
(Safety Evaluation 98-125)

Temporary Modifications S1-98-024 and S1-98-025 were used to bypass the bearing lift oil pressure anti-start interlock in the Main Turbine Turning Gear motor circuitry to allow periodic rotation of the turbine rotor using the turning gear.

**PROCEDURE OR METHOD OF OPERATION CHANGES THAT DID NOT REQUIRE NRC APPROVAL**

**MONTH/YEAR:** November 1998

1/2-OPT-FW-008
1/2-AP-21.01

**Operations Periodic Test Procedures** 11/03/98
**Abnormal Procedures**
(Safety Evaluation 96-074, Rev. 1)

UFSAR Chapter 10.3.5, Condensate and Feedwater System specifies that the Auxiliary Feedwater (AFW) motor operated throttle valves are in full open position during normal operations. Operations Periodic Test Procedures 1/2-OPT-FW-008, "AFW Check Valve Test", were evaluated for unit operation with all six MOV's closed during the testing of AFW cross-connect capability to the other unit which was in cold shutdown or refueling mode. Abnormal Procedures 1/2-AP-21.01 were evaluated for check valve back leakage concerns.

0-ECM-0103-02

**Electrical Corrective Maintenance Procedure** 11/10/98
(Safety Evaluation 98-117)

Electrical Corrective Maintenance Procedure, 0-ECM-0103-02, "Station Battery UPS System Maintenance", was revised to connect a dummy load to the UPS for troubleshooting, periodic maintenance and testing. This is allowed during plant shutdown conditions or by entering the appropriate limiting condition for operation (LCO) time limit.

1-OP-RC-001A

**Operations Procedure** 11/14/98
(Safety Evaluation 98-119)

Loop "A" drain valve 1-RC-HCV-1557A is leaking by at a rate of 1.6 gpm to the Primary Drain Transfer Tank (PDTT) which is not acceptable for long term unit operation. Operations Procedure 1-OP-RC-001A, "Reactor Coolant System Valve Alignment", was changed to continue Unit 1 operation with Loop 'A' drain valve manually isolated.

0-ECM-1401-02

**Electrical Corrective Maintenance Procedure** 11/19/98
(Safety Evaluation 98-122)

Electrical Corrective Maintenance Procedure, 0-ECM-1401-02, "Emergency Operation of Charging Pump Component Cooling Water Motors", installs the TM to allow Operations to operate the Charging Pump CC Pumps by energizing the Motor Control Center breaker. This provides additional detail for post Appendix R fire scenarios.

## TESTS AND EXPERIMENTS THAT DID NOT REQUIRE NRC APPROVAL

**MONTH/YEAR:** November 1998

None during the Reporting Period

## CHEMISTRY REPORT

**MONTH/YEAR:** November 1998

| Primary Coolant Analysis | Unit No. 1 | | | Unit No. 2 | | |
|---|---|---|---|---|---|---|
| | Max. | Min. | Avg. | Max. | Min. | Avg. |
| Gross Radioactivity, μCi/ml | 3.13E-1 | 7.73E-4 | 3.32E-2 | 1.82E-1 | 9.21E-2 | 1.37E-1 |
| Suspended Solids, ppm | 0.250 | ≤ 0.010 | 0.064 | - | - | - |
| Gross Tritium, μCi/ml | 4.04E-2 | 3.07E-2 | 3.35E-2 | 5.41E-1 | 4.57E-1 | 5.05E-1 |
| $I^{131}$, μCi/ml | 8.21E-3 | 2.68E-5 | 4.79E-4 | 1.37E-4 | 3.27E-5 | 7.45E-5 |
| $I^{131}/I^{133}$ | - | - | - | 0.09 | 0.03 | 0.05 |
| Hydrogen, cc/kg | 33.1 | 23.7 | 28.9 | 40.3 | 38.3 | 39.1 |
| Lithium, ppm | 3.59 | ≤ 0.10 | 2.77 | 2.25 | 1.96 | 2.12 |
| Boron - 10, ppm* | 485.9 | 269.5 | 392.4 | 77.6 | 58.6 | 67.8 |
| Oxygen, (DO), ppm | 9.0 | ≤ 0.005 | 3.57 | ≤ 0.005 | ≤ 0.005 | ≤ 0.005 |
| Chloride, ppm | 0.038 | ≤ 0.001 | 0.008 | 0.004 | ≤ 0.001 | 0.002 |
| pH @ 25 degree Celsius | 6.30 | 4.35 | 4.98 | 7.27 | 6.59 | 6.96 |

* Boron - 10 = Total Boron x 0.196

Comments:

None

## FUEL HANDLING
## UNITS 1 & 2

**MONTH/YEAR:** November 1998

| New Fuel Shipment or Cask No. | Date Stored or Received | Number of Assemblies per Shipment | Assembly Number | ANSI Number | Initial Enrichment | New or Spent Fuel Shipping Cask Activity |
|---|---|---|---|---|---|---|

None during the Reporting Period

## DESCRIPTION OF PERIODIC TEST(S) WHICH WERE NOT COMPLETED WITHIN THE TIME LIMITS SPECIFIED IN TECHNICAL SPECIFICATIONS

**MONTH/YEAR:** November 1998

None during the Reporting Period