ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649

TELEPHONE
AREA CODE 716 546-2700

Ginna Station
February 28, 1976

Reg. Files 50-244

Mr. James P. O'Reilly
U. S. Nuclear Regulatory Commission
Directorate of Regulatory Operations
Region 1
631 Park Avenue
King of Prussia, Pennsylvania 19406

Subject: Reportable Occurrence #76-08: Abnormal Degradation of Steam Generator Tubes

Dear Mr. O'Reilly:

This is to confirm my phone call to Mr. John Hannon, on February 27, 1976, to report Abnormal Occurrence #76-08 at Ginna Station.

A program of planned eddy current examination was conducted on both loop "A" and "B" steam generators at the Ginna Station from February 15th to February 27, 1976, during the present refueling turbine maintenance outage. This program specified examinations at 400 KHZ to detect and measure potential tube defects and at 25 KHZ to provide a profile of sludge deposits.

A 100 percent inspection of both steam generators (hot and cold legs) was performed. As a result of this effort, 39 tubes were explosively plugged in the "A" steam generator and 2 in the "B" steam generator. These tubes had exhibited tube wall deterioration in excess of 40 percent.

Very truly yours,

Charles E. Platt

Charles E. Platt
Superintendent

CEP:dg

cc: Mr. G. J. Swarthout, RG&E Corp.
Mr. L. D. White, Jr., RG&E Corp.

50-244

ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14604

TELEPHONE
AREA CODE 716 546 2700

Ginna Station
March 9, 1976

Mr. James P. O'Reilly
U. S. Nuclear Regulatory Commission
Directorate of Regulatory Operations
Region 1
631 Park Ave.
King of Prussia, Pennsylvania 19406

SUBJECT: Reportable Occurrence #76-11: Control Room Charcoal Filter Damper Improperly Positioned

Dear Mr. O'Reilly:

This is to confirm my phone call to Mr. Bert Davis on March 8, 1976, to report the improper positioning of one of the two dampers for the control room charcoal filter system.

While conducting a flow check on the control room charcoal filter system on 3-8-76, it was discovered that one of the two dampers that controls 25% of the control room air flow through the charcoal filters during a radiation emergency was in the wrong position. This condition allowed one of the two dampers to always be closed so that the recirculation of 25% of the control room air through the charcoal filter was not possible.

A trouble card was issued and work is being done today to correct the problem.

Very truly yours,

Charles E. Platt

Charles E. Platt
Superintendent

EP:dl

cc: Mr. G. J. Swarthout, RG&E Corp.
Mr. L. D. White, Jr., RG&E Corp.

NRC FORM 195 (2-76)

U.S. NUCLEAR REGULATORY COMMISSION

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

DOCKET NUMBER	50-244
FILE NUMBER	INCIDENT REPORT
DATE OF DOCUMENT	3-1-76
DATE RECEIVED	3-5-76
NUMBER OF COPIES RECEIVED	1

TO: J.P.O'Reilly

FROM: Rochester Gas & Elec. Corp.
Rochester, N.Y.
L.D. White, Jr,

☐ LETTER ☐ NOTORIZED
☐ ORIGINAL ☒ UNCLASSIFIED
☒ COPY

PROP

INPUT FORM

DESCRIPTION
Ltr. trans the following.....

ENCLOSURE
Reportable Occurrence # 76-07, on 2-7-76, Concerning Relays for Train "A" Containment isolation ans containment ventilation isolation failure to latch...

(1 Cy. Recieved)

PLANT NAME: RE Ginna # 1

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED SEND DIRECTLY TO KREGER/J. COLLINS

SAFETY — FOR ACTION/INFORMATION — ENVIRO — RKB 3-11-76

BRANCH CHIEF:	Purple		
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LIC. ASST:	Sheppard		
W/ CYS			
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CONTROL NUMBER

2239

ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649

LEON D. WHITE, JR.
VICE PRESIDENT

TELEPHONE
AREA CODE 716 546-2700

Regulatory Docket File

March 1, 1976

Mr. James P. O'Reilly, Director
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

Subject: Reportable Occurrence 76-07 (30-day report), Relays for Train "A" containment isolation and containment ventilation isolation failure to latch.
R. E. Ginna Nuclear Power Plant, Unit No. 1
Docket No. 50-244

Dear Mr. O'Reilly:

In accordance with Technical Specifications, Article 6.9.2b, the attached report of Reportable Occurrence 76-07, 30-day, is hereby submitted. Two additional copies of this letter and the attachment are enclosed.

Very truly yours,

L. D. White, Jr.

Attachment

cc: Mr. Ernst Volgenau (30)
Mr. William G. McDonald (3)

2239

LICENSEE EVENT REPORT

CONTROL BLOCK: [1 — 6] [PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME	LICENSE NUMBER	LICENSE TYPE	EVENT TYPE
01 NYREG1 (7–14)	00–00000–00 (15–25)	4111 1 (26–30)	03 (31–32)

CONT	CATEGORY	REPORT TYPE	REPORT SOURCE	DOCKET NUMBER	EVENT DATE	REPORT DATE
01 (7–8)	(57–58)	L (59)	L (60)	050–0244 (61–68)	020776 (69–74)	030176 (75–80)

EVENT DESCRIPTION

02 During valve alignment verification following manual containment isolation (CI) and con-
03 tainment ventilation isolation (CVI) in preparation for ILRT, sump A discharge AOV had
04 not closed. Investigation revealed that MG-6 relays in "A" safeguards train for CI and
05 CVI would not latch and would drop back on release of pushbutton. Train "B" functioned
06 properly. There has been no previous failure of this (cont'd. under Additional Factors)

SYSTEM CODE	CAUSE CODE	COMPONENT CODE	PRIME COMPONENT SUPPLIER	COMPONENT MANUFACTURER	VIOLATION
07 IB (10)	B (11)	RELAYX (12–17)	N (43)	W120 (44–47)	N (48)

CAUSE DESCRIPTION

08 Westinghouse Electric Corp. type MG-6 relays, style 289B363A11, 125 VDC, were found
09 with improper latch screw factory adjustment. Removed relays were adjusted and stored
10 as spares. Remaining MG-6 relays in safeguard racks will be checked during safe-
guard functional test during this refueling outage.

FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
11 H (9)	000 (10–12)	NA (13–44)	(45)	NA (46–80)

FORM OF ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
12 Z (9)	Z (10)	NA (11–44)	NA (45–80)

PERSONNEL EXPOSURES

NUMBER	TYPE	DESCRIPTION
13 000 (9–11)	Z (12)	NA (13–80)

PERSONNEL INJURIES

NUMBER	DESCRIPTION
14 000 (9–11)	NA (12–80)

OFFSITE CONSEQUENCES

15 NA

LOSS OR DAMAGE TO FACILITY

TYPE	DESCRIPTION
16 Z (9)	NA (10–80)

PUBLICITY

17 NA

ADDITIONAL FACTORS

18 (cont'd. from Event Description) type. Identical replacement relays from original spare
parts inventory were adjusted, tested, installed and retested satisfactorily by qualified
19 test personnel. (Reportable Occurrence 76-07, 30-day)

NAME: E. Clair Edgar PHONE: 716/546-2700, ext. 291-223

GPO 881-667

ROCHESTER GAS AND ELECTRIC CORPORATION · [illegible] EAST AVENUE · ROCHESTER, N.Y. [illegible]

[illegible], 1976

Mr. James P. O'Reilly, Director
United States Nuclear Regulatory Commission
Region 1
631 Park Avenue
King of Prussia, Pennsylvania 19406

SUBJECT: Reportable Occurrence #76-05: Abnormal degradation of primary containment components discovered while doing the containment integrated leakage rate test.

Dear Mr. O'Reilly:

This is to confirm my phone call to Mr. John Hannon this morning to give him information on Reportable Occurrence #76-05.

Pressurization of the containment for the containment integrated leakage rate test began at 1600 hours on Saturday, February 7. The 35 psig pressure was established at 0800 hours on Sunday, February 8, 1976. After maintaining the 35 psig pressure for approximately five hours, it became apparent that there was excessive leakage through several containment penetrations. Further investigation found the components that were responsible for the excessive leakage to be:

(1) Containment Purge Supply Valve - Inside Containment

(2) Containment Purge Exhaust Valves

(3) Motor Operated Valves #813 & #814 - Reactor Support Cooling

(4) Check Valve #1713 - Nitrogen supply to the Reactor Coolant Drain Tank

Repairs are underway to the above listed valves. The containment integrated leakage rate test will be resumed as soon as repairs are completed.

Very truly yours,

Charles E. Platt

Charles E. Platt
Superintendent

CEP/dg

cc: G. J. Swarthout, RG&E Corp.
J. F. Sweet, RG&E Corp.
L. D. White, RG&E Corp.
C. V. Hartlieb, RG&E Corp.