

ROCHESTER GAS & ELECTRIC CORPORATION

GINNA STATION

ANNUAL REPORT #11

JANUARY 1, 1975 TO DECEMBER 31, 1975 INCLUSIVE

DOCKET #50-244

TABLE OF CONTENTS

<u>TECHNICAL SPECIFICATION</u>	<u>TITLE</u>	<u>PAGE #</u>
6.9.1.b (1)	NARRATIVE SUMMARY	1 - 4
6.9.1.b (2) thru 6.9.1.b (2) (f)	FORCED REDUCTIONS, FORCED OUTAGES, AND SCHEDULED OUTAGES	5 - 41
6.9.1.b (3)	PERSONNEL EXPOSURES	42 - 43
6.9.1.b (4)	FAILED FUEL INDICATIONS	44
6.9.3.a	ENVIRONMENTAL MONITORING 1975	45
	ANNUAL REPORT OF ALL PROCEDURE CHANGES	46 - 214
	ANNUAL REPORT OF SYSTEM MODIFICATIONS AND SPECIAL TESTS	215 - 234

ANNUAL REPORT #11

JANUARY 1, 1975 TO DECEMBER 31, 1975 INCLUSIVE

6.9.1.b.1

NARRATIVE SUMMARY

The Station began the year at full power and continued at this level until 1/4, when there was a reduction to 71% level to isolate 1B Reheater for repair of the steam supply line. Reactor power returned to 100% level on 1/4. On 1/15 reactor power level was reduced to 83% and on 1/17 reactor power level was reduced to 60% due to turbine runbacks caused by a spurious downspike on a power range channel. Power level was also reduced to 95% on 1/26 and 2/24 to perform T14A (Monthly Intercept and Reheat Stop Valve Test). Reactor power returned to 100% level upon completion of the tests.

The Station was taken to hot shutdown on 3/5 to retrieve two dropped control rods (G5 and G9). The reactor returned critical the same day and returned to full power on 3/6. The Station was shut down on 3/10 due to a tube leak in the "A" Steam Generator and remained shutdown for refueling.

The refueling outage lasted from 3/10 until 5/10. During this time, thirty six new assemblies and two new secondary sources were loaded into the core. Four modifications were made to the secondary side of both Steam Generators. These modifications provided for reduced moisture carryover and flow redistribution at the tube plate. Routine inspection and overhaul of the high-pressure turbine were performed. The Steam Generator primary side tubes were partially tested and all tubes indicating a reduction of 40% of their original thickness were plugged. Under this criterion eleven tubes in the "B" Steam Generator and forty seven tubes in the "A" Steam Generator were plugged. The reactor vessel cold leg nozzle welds were ultrasonically inspected.

Annual Refueling Shutdown concluded with reactor criticality on 5/10. The power level was increased to 3% level on 5/11 for a flux map and then brought subcritical. The reactor returned critical on 5/12. Power level was increased to 3% level for a flux map, and then returned to a hot shutdown mode. The hot shutdown mode continued until 5/14, when the reactor was brought critical for operator training startups, upon completion of which, the reactor was brought subcritical. Plant startup was accomplished on 5/19, at 0145; and reactor power level was gradually increased to 25% level. The reactor tripped at 0615 due to low steam generator level; the turbine was tripped manually. Unit was started up at 1530, reactor power level was gradually increased to 25%.

Reactor power level was increased to 40% level on 5/20; a further increase to 55% level occurred on 5/21. At 2230 on 5/21, during performance of T-18B (Turbine Main Steam Stop Valves Test), the turbine tripped due to E. H. Control Failure. Reactor was brought critical at 1815 on 5/22. Unit was restored to service on 5/23, reactor power level was gradually increased to 25%. On 5/24 T-18C, (Turbine Overspeed Trip Test) was performed, reactor power level was increased to 45%.

Further Reactor power level increase occurred on 5/25, to 50% level. On 5/26 at 1645 the turbine was tripped manually due to E. H. Control Valve Position Limiter malfunction. Reactor trip followed. The unit was restored to service at 2230 on 5/26, reactor power level was increased to 25%. The reactor power level was increased to 50% on 5/27. The reactor power level remained at 50% level thru 5/30. On 5/31 the unit was removed from service at 0000 to perform repairs on the E-H Unit and 1-B Main Feedwater Pump and return to service at 1847 and the power level was increased to 50%.

Reactor power level was increased from 50% on 6/1 to 60% on 6/2; to 80% on 6/3; to 90% and to 100% on 6/5. At 0115 on 6/6, when the reactor power level was at 100%, a plant trip occurred due to the main steam isolation valves malfunctioning (valves closed). The plant was started up on 6/6 at 1315 and the reactor power level was increased to 50%. On 6/7 the power level was increased to 95%. Reactor power level remained at 90% from 6/8 through 6/11. On 6/12 the reactor power level was increased to 95%. Reactor power level remained at 95% level from 6/12 through 6/16. On 6/17 at 0415 a plant trip occurred due to loss of the "A" Inverter. The plant was started up at 0815 and a power level increase was taking place when, at 1000, the turbine was manually tripped due to excessive vibrations. The plant was started up on 6/21 at 1115 and the reactor power level was increased to 50%. On 6/23 the reactor power level was increased to 88%; at 0400, a plant trip occurred due to the main steam isolation valves malfunctioning (valves closed). The plant remained shut down through 6/25. On 6/26 the plant was started up at 1845, the reactor power level was increased to 25%. The power level increased to 50% level on 6/27 and to 90% level. The reactor power level remained at 90% through 6/29. Power level was increased to 95% and, further to 100% on 6/30.

On 7/8, reactor power was reduced to 88% level at 1030 due to main feedwater pump low suction alarms and swings in heater drain pump flow. Load increase to 100% level was initiated at 1215 after the generator hydrogen pressure was raised from 50-60 psi.

Reactor power level was decreased to 14% on 7/20 to repair a leak in a drain line on the high pressure turbine. Power level was returned to 100% after repairs were completed. Reactor power was maintained at 100% level until 7/24 when a turbine trip was followed by a reactor trip, at 1445, due to a lightning strike on Station 13A. Unit was returned to service at 0300 on 7/25 after a series of tests were conducted to verify the integrity of the Primary and Secondary Electrical Systems involved in the protective relays that operated.

Reactor power was maintained between 99% and 100% level from July 25 through October 10 except:

7/30 power was reduced to 48% to check for tube leaks in the 1-A-1 and 1-A-2 condenser water boxes. On 8/3 power was reduced to 56% per the load dispatcher's request to maintain transmission line capabilities. Severe thunderstorms were prevalent in the area on that date. On 8/24 power was reduced to 47% to perform T-18B (Turbine Main Steam Stop Valves Test). On 8/25 power was reduced to 96% to check HCV466 ("A" Feedwater Control Valve).

On 9/10 power was reduced to 97% to perform PT-6.4 (Excore-Incore Calibration). On 9/23 power was reduced to 96% level to perform PT-16 (Auxiliary Feedwater System Flow Check).

On 10/3 power was reduced to 47% level to perform T-18B (Turbine Main Steam Stop Valves Test). A Unit outage was required on 10/10 to replace cables for the lake intake heaters. The Unit returned to service on October 12 and returned to 100% level on 10/13.

The Unit remained at 100% level from 10/13 through 12/30 except:

On 10/26 power was reduced to 70% due to the loss of the #1 generator transformer cooling fans. Power was returned to 100% the same day. On 10/27 power was reduced to 97% to perform PT-16 (Auxiliary Feedwater System Flow Check).

On 11/18 power was reduced to 47% level to perform T-18B (Turbine Main Steam Stop Valves Test). On 11/23 power was reduced to 92% level to perform T-18A (Intercept and Reheat Stop Valves Test).

On 12/10 and 12/11 power was reduced to 95% level during Nuclear Instrumentation System adjustments. On 12/13 power was reduced to 47% level to perform T-18B (Turbine Main Steam Stop Valves Test). On 12/12 power was reduced to 47% to perform M-54.2 (1-B-1 Condenser Water Box Leak Check, with Freon). On 12/22 Power was reduced to 94% level to perform PT-16 (Auxiliary Feedwater System Flow Check). On 12/23 power was reduced to 46% level to perform M-54.2 (1-B-1 Condenser Water Box Leak Check, with Freon).

The Unit was taken to the cold shutdown on .12/30 due to a tube leak in the "B" Steam Generator. The Unit remained in cold shutdown through the end of the reporting period.

The safety-related maintenance not covered under Technical Specification Item 6.9.1.b (2) (e) is attached in tabulative form by month.

ANNUAL REPORT #11

JANUARY 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
1A Charging Pump	Corrective maintenance at steady load	Keyway wear on Varidisc Bushing	One Veridrive belt broken and two spacer bar studs failed	Replaced Varidisc bushing and installed new spacer bars and studs	Used maintenance procedure M-11.4.1
1A Boric Acid Transfer pump	None	Preventive Maintenance	Annual Inspection	Pump inspected; found in excellent condition	Used Maintenance procedures M-11 and M-11.11
Gas Analyzer Vacuum Pump	None	Failed Diaphragm	Loss of Capacity	Replaced diaphragm and reinstalled	N/A
Turbine Driven Auxiliary Feed Pump	None	Low Oil Pressure	Pump Failed to Start	Changed oil filter and set discharge pressure on D.C. oil pump to 78 psig.	N/A
1B Charging Pump Leak off Return Pump	Corrective	Failed Diaphragm	Loss of pump capacity	Replaced diaphragm	N/A
1A Auxiliary Building Supply Fan	None	Separation of seam in fan housing between the fan outlet ducts	Reduced supply air flow due to recirculation in duct	Repaired duct and reinforced seams	N/A

ANNUAL REPORT #11

JANUARY 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
Hydraulic Snubber Unit FW-82 (B Feedwater Line)	None	Accumulator piston seal not holding. Accumulator cylinder believed to be out of round	Leakage of hydraulic fluid from accumulator of snubber unit	Replaced accumulator with spare	Used Procedure M-40
TCV-130 Non-Regenerative Exchange Component Cooling Outlet Valve	None	Rubber seat in valve lifting and tearing due to high velocity	Excessive cooling water to the non-regenerative heat exchanger	Manual valve being used to control component cooling flow through the non-regenerative heat exchanger. Metal seated valve is on order	Used Procedure EM-136 for inspection
Boric Acid Filter Vent Valve 390	None	Failed diaphragm due to high temperature	Leakage thru valve	Replaced diaphragm	Used Maintenance procedures M-7.4 and M-37.16A

ANNUAL REPORT #11

FEBRUARY 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
1A Waste Gas Compressor	N/A	Preventive Maintenance	N/A	Annual Maintenance and Inspection	M-36.1
1B Waste Gas Compressor	N/A	Preventive Maintenance	N/A	Annual maintenance and inspection - replaced leaking mechanical seal.	M-36.2
1C Safety Injection Pump	Failure of pump to start on alternate bus 16 after proper operation on bus 14. Failures occurred during manual control board starts in the performance of PT-2.1	Possible breaker misalignment or mechanical binding of lockout solenoid plunger	Failure of pump to start	Temporary wedge installed to hold mechanical latch in position. During upcoming refueling outage, bus will be de-energized to allow breaker and lockout assembly operation without motor starting duty. Manufacturer notified and requested to investigate problem New breaker being ordered for replacement and eventual testing of present breaker increased to weekly surveillance.	Em-75 used to perform inspection and maintenance
1A and 1B Diesel Generator Oil Cooler	N/A	Service Water Sediment Buildup	High ΔP on coolers	Cleaned service water side of oil cooler	Used procedures M-15 and M-15.1
R11 and R12 Containment Particulate and Gas Radiation Monitor	N/A	N/A	N/A	Annual inspection and pump overhaul.	N/A

ANNUAL REPORT #11

FEBRUARY 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
R-13 and R-14 Particulate and Gas Radiation Monitor	N/A	N/A	N/A	Annual Inspection and pump overhaul.	N/A
R-10A Containment Iodine Monitor Pump	N/A	N/A	N/A	Annual inspection & pump overhaul.	N/A
Boric Acid Evaporator Distillate pumps 1 and 2	N/A	Bearing Failure	Loss of head; excessive noise	Inspection and pump overhaul	Used procedure M-11.11

JUNE 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
Steam Isolation Valve to turbine driven auxiliary feedwater pump valve 3504	N/A	Valve leaking thru	Turbine kept spinning when valve closed.	Lapped in seat and disc	Used Procedure M-37.35
"A" Travelling Screen	N/A	Broken Chain	Sheared key and broke drive sprocket	Replaced Chain, Key, Drive Sprocket	N/A
Boric Acid Evaporator feed tank pump 1B	N/A	Preventive Maintenance	---	Annual inspection and overhaul	N/A
"B" House Service Water Pump	N/A	Preventive Maintenance	---	Annual inspection and overhaul	N/A
South Turbine Stop Valve	N/A	Shaft Seal Leakage	Steam Leak	Replaced shaft and bushing	N/A

ANNUAL REPORT #11

JULY 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
1B Diesel Generator Pre-lube Pump	None	Phase compensation setting at 85%.	Pump tripping out	Adjusted setting to 100%.	N/A
1A Motor driven Auxiliary feed pump	None	Preventive Maintenance Program	-----	Major inspection and overhaul due to scheduled preventive maintenance.	Used M-11.5 series of procedures
No. 8 Environmental Monitor	None	-----	Pump failure	Replaced with rebuilt unit	N/A
1A Charging Pump	None	Failed upper and lower "O" Rings on plunger gland assembly	CVCS fluid leakage	Repacked charging pumps with rebuilt gland assemblies	Used procedure M-11.4.1
Blender flow control valve FCV-110C	None	Failed bonnet diaphragm	Valve bonnet leakage	Replaced diaphragm	Used procedure M-37.16
1A Motor driven Auxiliary feed pump recirc valve 4304	None	Cut valve plug and seat	Leakage of main feed-water back to condensate storage tank	Lapped out majority of cuts, new valve internals on order	Used maintenance procedure M-37.16H
1A motor driven auxiliary feed pump discharge check valve 4009	None	Undetermined parts worn	Leakage of main feedwater back into auxiliary feed pump casing	Rebuilt valve internals under direction of Crane-Chapman service representative, new internals on order	Used maintenance procedure M-37.16H

ANNUAL REPORT #11

AUGUST 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
Laundry and Hot Shower Pump	None	Preventive Maintenance	---	Major Inspection of Pump and Motor	Procedure M-11.13 & M-45.2 were used.
1B Diesel Generator Breaker to Bus 16	None	Undetermined	Breaker failed to close during PT-12.2	Inspected auxiliary contacts, wire connections, control switch and sync. switch. Tested successfully 8 times.	Used Procedure M-32.1 for breaker inspection and M-15 for diesel maintenance.
1A Waste Condensate Pump	None	Preventive Maintenance	---	Major Inspection of Pump	Used Procedure M-11.13
1B Waste Condensate Pump	None	Preventive Maintenance	---	Major inspection of pump	Used Procedure M-11.13
1B Sump Tank Pump	None	Preventive Maintenance	---	Major inspection of pump	Used Procedure M-11.13
Chemical Drain Tank Pump	None	Preventive Maintenance	---	Major inspection of pump and motor	Used procedures M-11.13 and M-45.2
Over Power Trip Bistable TC-405A. Reactor Protection Channel I	Setpoint shifted in the Non-Conservative Direction	Faulty Zener Diode	Setpoint Shift	Replaced Bistable with spare unit.	N/A

ANNUAL REPORT #11

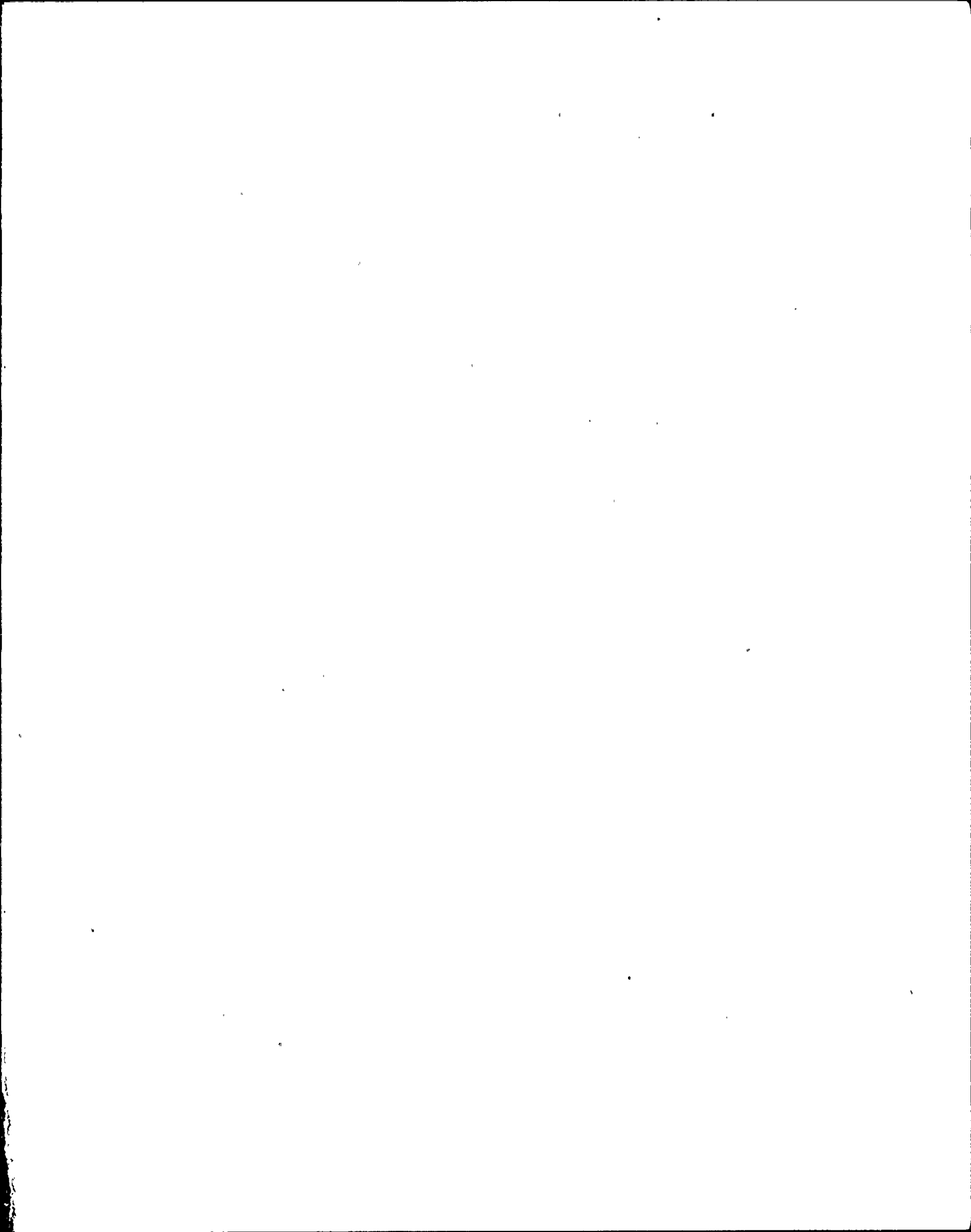
SEPTEMBER 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
#2 Boric Acid Evaporator Distillate Pump	None	-	Pump would not produce desired flow	Replaced valve diaphragms	N/A
1B Reactor Coolant Drain Tank Pump	None	Preventive Maintenance	-	Performed major inspection on pump	Used Procedure M-11.11
New Resin Sluice Pump	None	Muffler on Air Drive Exhaust Plugged	Pump would not produce desired flow	Cleaned Air Drive unit and removed muffler assembly	N/A
1C Service Water Pump	None	Preventive Maintenance	-	Performed minor inspection on pump	Used Procedure M-11.10
Blend Valves 110B and 110C	None	One or both valves leaking by	Borated water added to CVCS System while blending to the refueling water storage tank	Adjusted stroke on both valves	Used Procedures M-37.10 and M-37.16

ANNUAL REPORT #11

OCTOBER 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
Non-Regenerative Heat Exchanger Temperature Control Valve TCV-130	None	Rubber Valve Seat Lifting and Tearing	Excessive flow through valve. Loss of temperature control by valve	Replaced valve	Used Emergency Maintenance Procedure EM-136
Main Steam Safety Valve 3513	None	Gasket Leak	Steam leak at valve flange	Sealed leak using Furmainite process	Used Maintenance Procedure M-37.39
1B Containment Spray Pump	None	Preventive Maintenance Schedule	None	Performed minor inspection; change oil, aligned pump	Used Procedures M-11.2 and M-11.14
Plant Vent Monitor Pump R-13 & 14	None	Belt Failure	Loss of sample flow	Replaced belt	N/A
1A Boric Acid Evaporator feed-tank pump	None	Preventive Maintenance Program	---	Removed and performed major inspection	Used Procedure M-11.11
Diesel Fire Pump	None	Heat Exchanger tube leak	Diesel running hot due to loss of jacket water level	Plugged leaking tubes	Used Procedure M-38.1



ANNUAL REPORT #11

NOVEMBER 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
Waste Evaporator Hot Water Recirculation Pump	None	Preventive Maintenance Schedule	---	Performed inspection and overhaul. Replaced with spare.	Used Procedure M-11.13
1A Gas Stripper Feed Pump	None	Preventive Maintenance Schedule	---	Performed inspection and overhaul.	Used Procedure M-11.11.
1B Waste Gas Compressor	None	Preventive Maintenance Schedule	---	Performed annual inspection of compressor and controllers	Used Procedure M-36.2
Containment Purge Supply Fan Housing	None	Air turbulence	Tack welds on housing cracked	Repaired housing welds.	N/A
1A Service Water Pump Breaker	None	---	Pump failed to start automatically on diesel start	Replaced overcurrent trip devices, alarm switch and B phase arcing contact	Used Procedures M-32 and M-32.1

ANNUAL REPORT #11

NOVEMBER 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
Spent Fuel Pit Pump	None	Preventive Maintenance	---	Performed inspection and over-haul	Used Procedure M-11.14
1A Gas Stripper Bottoms Pump	None	Preventive Maintenance Scheduled	---	Performed inspection and overhaul	Used Procedure M-11.11
Waste Evaporator Distillate Pump	None	Preventive Maintenance Schedule	---	Performed inspection and overhaul	Used Procedure M-11.11
1A Concentrates Transfer Pump	None	Preventive Maintenance Schedule	---	Performed inspection and overhaul	Used Procedure M-11.11
1A Auxiliary Feedwater Pump Motor Operated Discharge Valve 4007	None	Valve plug and seat cut	Stop check valve leaking back through	Replaced valve plug assembly and lapped to seat	Used Procedure M-37.40
Blender Valve FCV-110B	None	Bonnet "O" Ring failed	Instrument air pressure on top of diaphragm caused failure	Replaced "O" Ring and valve diaphragm	Used Procedure M-37.10
1B Charging Discharge Drain Line	None	---	Pinhole leak in socket weld	Ground out and repaired weld	Used Procedure M-11.4.2 and WE-1A-73
Waste Evaporator Feed Tank Pump	None	Preventive Maintenance Schedule	---	Performed inspection and overhaul	Used Procedure M-11.11

DECEMBER 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
Boric Acid Filter Vent Valve 390	None	Diaphragm Failure	Valve Leaking Through	Replaced Bonnet Diaphragm	Used Procedure M-37.16A and M-7.4
Waste Evaporator Unit	None	Sludge Accumulation in Feed Tank	High Radiation readings at tank	Acid cleaned unit and vacuumed out feed tank	Used Procedure M-18
1B Charging Pump Discharge Drain Line	None	Poor initial weld	Leak on drain line at weld	Replaced section of affected pipe	Used Procedures M-11.4.2 and WE-1A-73
1A & 1B Diesel Generator Oil Coolers	None	Sediment in Service Water	High ΔP across cooler units	Cleaned Tubes	Used Procedures M-15 and M-15.1
1A Reheater Steam Inlet Control Valve 3425	None	Lower Plug Guide Worn Excessively	Valve binding and rattling during operation.	Installed new stem and plug	N/A
1A Waste Gas Compressor	None	Preventive Maintenance Schedule	---	Inspected unit replaced mechanical seal	Used Procedure M-36.1
Containment R-11, 12 Sample Pump	None	Bearing Failure	Pump failed	Replaced with spare unit	N/A
1B Diesel Generator Prelube Pump	N/A	---	Prelube pump not running	Changed holding coil on contactor	N/A

ANNUAL REPORT #11

6.9.1.b (2) Thru 6.9.1.b (2) (f)

January 4, 1975

Forced Reduction

- (a) The unit was reduced from 100% level to 71% level to permit the removal of the 1B Reheater Steam Line from service. The orifice flange gasket failed causing a steam leak. Repairs were made by removing the failed gasket and replacing with a new gasket.
- (b) No reportable occurrence pertained to this forced reduction.
- (c) No appropriate corrective action was required.
- (d) Operating time lost was 6 hours and 25 minutes.
- (e) No major safety-related corrective maintenance was performed during this forced reduction. The critical path activity dictating the length of the reduction was the replacement of the failed gasket.
- (f) No release of radioactivity or radiation exposure was associated with the forced reduction.

ANNUAL REPORT #11

March 5, 1975

FORCED OUTAGE

- (a) The unit was at 100% level when control rods G-5 and G-9 dropped into the core from the full out position. The drive control for both rods are located in the same Rod Drive Cabinet. A hole was drilled in the floor above the cabinet. Water used for lubricant leaked through the hole into the top of the cabinet and caused the holding coils to release the rods. The unit was taken to hot shutdown and the control circuits, for the rods, were checked. There being no damage, the rods were returned to their proper positions.
- (b) No reportable occurrence pertained to this forced outage.
- (c) Metal covers were installed above the Rod Control Cabinets to prevent a recurrence of this problem.
- (d) Operating time lost was 8 hours and 31 minutes.
- (e) No major safety-related corrective maintenance was performed during this forced outage. The critical path activity dictating the length of the outage was the determination of cause and verification that no damage to the rod control circuits occurred.
- (f) No release of radioactivity or radiation exposure was associated with the forced outage.

ANNUAL REPORT #11

March 10, 1975

SCHEDULED OUTAGE

- (a) The unit was scheduled to be shutdown for a refueling outage. A leak in the "A" Steam Generator was detected and the unit was removed from service four days early to minimize the effect of the tube leak.
- (b) Three reportable occurrences occurred during this scheduled outage. The reports were:
- Abnormal Occurrence 75-07, Thinning of Steam Generator Tube Walls
- Abnormal Occurrence 75-08, Post-Accident Charcoal Filters
- Abnormal Occurrence 75-09, Control Rod Drive Housing
- (c) Corrective Action consisted of eddy current test of tubes in both steam generators and explosive plugging of all tubes indicating a reduction to 40% of their original thickness.
- (d) Operating time lost was 1,658 hours, and 15 minutes.
- (e) Major safety-related corrective maintenance performed during the scheduled outage is listed below. The critical path activity dictating the length of the scheduled outage was repairs to the steam generator tubes and the refueling of the core.
- (f) No single release of radioactivity associated with the outage accounted for more than 10% of the allowable annual values. The radiation exposure of those individuals receiving greater than 0.5 rem from inspection of tubes is shown in the table.

<u>Dose Range</u> <u>(Rem)</u>	<u>Number of Individuals</u> <u>in each range</u>
0.50 - 0.75	17
0.75 - 1.00	22
1.0 - 1.5	58
1.5 - 2.0	18
2+	0

Total Exposure 132 Rem
Number of Individuals 115

ANNUAL REPORT #11

March 10, 1975

SCHEDULED OUTAGE

- (a) The unit was scheduled to be shutdown for a refueling outage. A leak in the "A" Steam Generator was detected and the unit was removed from service four days early to minimize the effect of the tube leak.
- (b) Three reportable occurrences occurred during this scheduled outage. The reports were:
 - Abnormal Occurrence 75-07, Thinning of Steam Generator Tube Walls
 - Abnormal Occurrence 75-08, Post-Accident Charcoal Filters
 - Abnormal Occurrence 75-09, Control Rod Drive Housing
- (c) Corrective Action consisted of ultrasonic test of tubes in both steam generators and explosive plugging of all tubes indicating a reduction to 40% of their original thickness.
- (d) Operating time lost was 1,658 hours, and 15 minutes.
- (e) Major safety-related corrective maintenance performed during the scheduled outage is listed below. The critical patch activity dictating the length of the scheduled outage was repairs to the steam generator tubes and the refueling of the core.

ANNUAL REPORT #11

March 10, 1975 to May 10, 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
Condensate Storage Tanks	None	Leakage at flanges and in bags on diafloat membranes	Air leakage into tanks	Repaired leaks and vacuum checked	N/A Refueling Shutdown
Pressurizer relief valves 434 and 435	None	Technical Specification Requirement	---	Bench checked safety valves	Used Procedure RSSP-12.
Main Feedwater Pump Suction Valves 3970 and 3971.	None	Bonnet Gasket leakage	Seal welded flange and stud holes to prevent steam leakage	Cut off studs and bonnets, dressed flanges and stud and nut areas, cleaned up gasket area, and reassembled valves	N/A
1A Main Steam Power Operated Relief Valve 3411	None	---	Valve disc and seat cut due to steam leakage	Installed new spindle and lapped in	Used Procedure M-37.34
1B Main Steam Admission Valve 3504 to turbine driven auxiliary feed pump	None	---	Valve disc and seat cut due to steam leakage	Replaced disc and lapped seat	Used Procedure M-37.35
Charging Pump Relief Valves 283, 284, 285	None	Annual Inspection	Valves found to be leaking	Cleaned and repaired valves, then retested	Used Procedure RSSP-12
1A and 1B Steam Generator Main Feedwater Control Valves 4269 and 4270	None	Original application of oversized valves	Excessive wear on valve cage and plug assemblies. Possible failure of cage guides	Installed new design valve internals of different material	N/A Refueling Shutdown

ANNUAL REPORT #11

March 12, 1975 to May 10, 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
Letdown Relief Valve 209	None	Cut disc and nozzle assembly	Valve leakage during test	Installed new disc, nozzle and bellows assembly then retested	Used Procedure M-34 and RSSP-12
Pressurizer Power Operated Relief Block Valve 515	None	---	Seat Leakage	Lapped out cuts on valve gate and body seats	Used Procedure M-37.2.
Pressurizer Steam Space Sample Valve 951	None	Cut seat and plug	Steam Leakage	Replaced Valve	Used Procedure M-37.36
1A and 1b Diesel Generators	None	Preventive Maintenance	---	Performed annual inspection	Used Procedure M-15 and M-15.1
1A and 1B Charging Pumps	None	Preventive Maintenance	---	Repacked pumps and replaced worn plungers	Used Procedures M-11.4.1 and M-11.4.2
1A Service Water Pump	None	Preventive Maintenance	---	Performed major inspection and overhaul of pump	Used Procedure M-11.10
Control Rod Drive K-7 Housing Cap	None	Possible Fabrication defect causing pin hole leak in canopy of cap above seal weld	Small borate deposit on cap indicating leakage	Replaced cap	Used Procedure Em-140
Main Feedwater Lines	None	Some initial construction welds poor	Welds MS-1000G MS-1000G2 MS-1002B (Continued)	Repaired welds using approved procedures and then re-examined	N/A Refueling Shutdown

ANNUAL REPORT #11

March 12, 1975 to May 10, 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
Main Stream Lines (Continued)			MS-1002B1 MS-1002D MS-1002D2 MS-1002E MS-1003B MS-1003B1 MS-1003C MS-1003D MS-1003E		
1C Safety Injection Pump Bus 16 breaker	None	Scheduled Inspection	Intermittent failure of breaker to close	Replaced old breaker with new spare unit	N/A
North Turbine Stop Valve	None	Galled leak-off seal seat at bushing and shaft	Shaft leaked steam	Installed new shaft and bushing	N/A
High Pressure Turbine	None	Scheduled Inspection	Excessive steam erosion in steam chest orifice and stationary seals and blades.	Repaired areas of erosion; remachined joints	N/A
Waste Condensate Demineralizer Resin Sluice Pump	None	Diaphragm Dried Out	Diaphragm Failure	Replaced with spare	N/A

ANNUAL REPORT #11

March 12, 1975 to May 10, 1975.

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
Main Feedwater and Steam Hydraulic Snubbers MS-22, FW-25, FW-82, FW-84, FW-85, MS-146 (lower), MS-160, MS-166, MS-167, MS-179, MS-185	None	Scheduled Inspection	---	Rebuilt hydraulic units with spare parts including ethylene-propylene seal material.	Used Procedures M-40, M-40.1, and M-40.2
Waste Disposal Return Valve 165A from Drumming Station	None	Failed Diaphragm	Bonnet Leak	Replaced diaphragm	Used Procedures M-37.16A and M-37.16G
1A Containment Post-Accident Charcoal Filter	None	Scheduled Test	Loss of Iodine Removal Efficiency	Replaced charcoal in filter units	Post Installation check okay. Refueling.

March 12, 1975 to May 10, 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
Boric Acid Evaporator Distillate System	None	Diaphragm on suction valve to 1A Distillate pump failed	Restricted pump flow	Replaced diaphragm	N/A
Main Steam Check Valves 3518 & 3519	None	Misalignment of keyways on shafts and disc arms	Sloppy key fits. Wear on shafts in area of disc arms.	Metalized and machined shafts; remachined keyways to align keyways	Used procedure M-37.21
Condenser Water Box 1B1	None	Fretting or Erosion	3 tubes with small leaks	Plugged tubes and also did eddy current examination of several tubes in waterboxes	N/A Refueling shutdown
Pressurizer Power Operated Relief Valve PCV-431C	None	Crud Buildup on valve body at lower cage gasket area	Bypass leakage around cage and plug	Lapped area for good gasket seating surface	Used procedure M-37.11 Refueling Shutdown
Safety Injection System Relief Valve 887	None	Preventive Maintenance	---	Removed, cleaned, and bench checked	REM-107 Used along with RSSP-12
1A Reheater Tube Bundle	None	4 Tubes leaking at tube to tube sheet joint. Cause believed to be thermal ratcheting	Increased steam flow to reheater, loss of unit efficiency	Plugged three tubes, repaired one tube end	N/A

ANNUAL REPORT #11

March 12, 1975 to May 10, 1975

SYSTEM OR COMPONENT INVOLVED	RESULTS & EFFECT ON SAFE OPERATION	MALFUNCTION		CORRECTIVE ACTION TO PREVENT REPETITION	PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY
		CAUSE	RESULT		
Letdown System Relief Valve 203	None	Slight cut on valve disc and nozzle	Valve leaked during bench check	Replaced with spare	Used procedure RSSP-12
1D Service Water Pump	None	Preventive Maintenance	---	Performed Inspection and Overhaul	Used Procedure M-11.10
1B Boric Acid Pump	None	Preventive Maintenance	---	Performed annual inspection and overhaul of 1B Boric Acid Transfer Pump	Used Procedure M-11 and M-11.11
Waste Evaporator Feed Tank Pump	None	Possible over concentration of waste bottoms	Pump bearings and motor stator failure	Replaced pump, reduced end point on batch concentration	Used Procedure M-11.3 and M-11.11
1A Steam Generator 1B Steam Generator	None	Slight tube leak in 1A S/G	Primary to Secondary Leakage	Plugged leaking tube; eddy current examination with all tubes with greater than 40% defects plugged (47 tubes plugged including one above in 1A S/G and 11 plugged in 1B S/G) Tube sheets lanced and secondary internals modified	Used M-43 Series Procedures and SM75-24 Series Procedures Refueling Shutdown
Seal Return Relief Valve 314	None	Preventive Maintenance	Valve Leakage on bench test	Replaced disc, spotted nozzle; retested valve	Used Procedure RSSP-12
Condenser Steam Dump Valves 3354, 3351, 3353 and 3349	None	Steam cuts on disc and cage	Steam leakage through valves	Replaced valve internals	N/A Refueling Shutdown

ANNUAL REPORT #11

May 19, 1975

FORCED OUTAGE

- (a) The unit was at 25% level when a trip occurred due to a lo lo level in the steam generators. The automatic hotwell level control failed causing the hotwell water to be rejected to the condensate storage tanks. This removed the source of water for the steam generators.
- (b) No reportable occurrence pertained to this forced outage.
- (c) The corrective action consisted of removing the electronic hotwell level controller that had been installed during the scheduled outage and reinstalling the pneumatic control system.
- (d) Operating time lost was 9 hours and 17 minutes.
- (e) No major safety-related corrective maintenance was performed during this forced outage. The critical path activity dictating the length of the outage was the time required to change hotwell level control systems.
- (f) No release of radioactivity or radiation exposure was associated with the forced outage.

ANNUAL REPORT #11

May 21, 1975

FORCED OUTAGE

- (a) The Unit was at 55% level when a Turbine trip occurred due to a failure of Electric-hydraulic controller for the turbine. Vibration of control wires at the turbine caused the insulation to wear and shorting occurred. Heat from the shorted wires caused additional damage.
- (b) No reportable occurrence pertained to this forced outage.
- (c) The corrective action taken to reduce the probability of recurrence was the installation of gasket material where the new wires enter or leave a conduit.
- (d) Operating time lost as a result of the forced outage was 43 hours and 40 minutes.
- (e) No major safety-related corrective maintenance was performed during the outage. The critical path activity dictating the length of the outage was replacement and testing of the damaged wires.
- (f) No release of radioactivity or radiation exposure was associated with the forced outage.

ANNUAL REPORT #11

May 26, 1975

FORCED OUTAGE

- (a) The unit was at 55% level when a manual trip was performed. The control valve position limiter malfunctioned due to a failed control circuit card. This card was damaged due to the shorted wires on. May 21, 1975.
- (b) No reportable occurrence pertained to this forced outage.
- (c) The valve position limiter circuit cards were tested to insure no outer damage was present.
- (d) Operating time lost was 5 hours and 37 minutes.
- (e) No major safety-related corrective maintenance was performed during this forced outage. The critical path activity dictating the length of the outage was the replacement and testing of the valve position limiter control card.
- (f) No release of radioactivity or radiation exposure was associated with the forced outage.

ANNUAL REPORT #11

May 31, 1975

FORCED OUTAGE

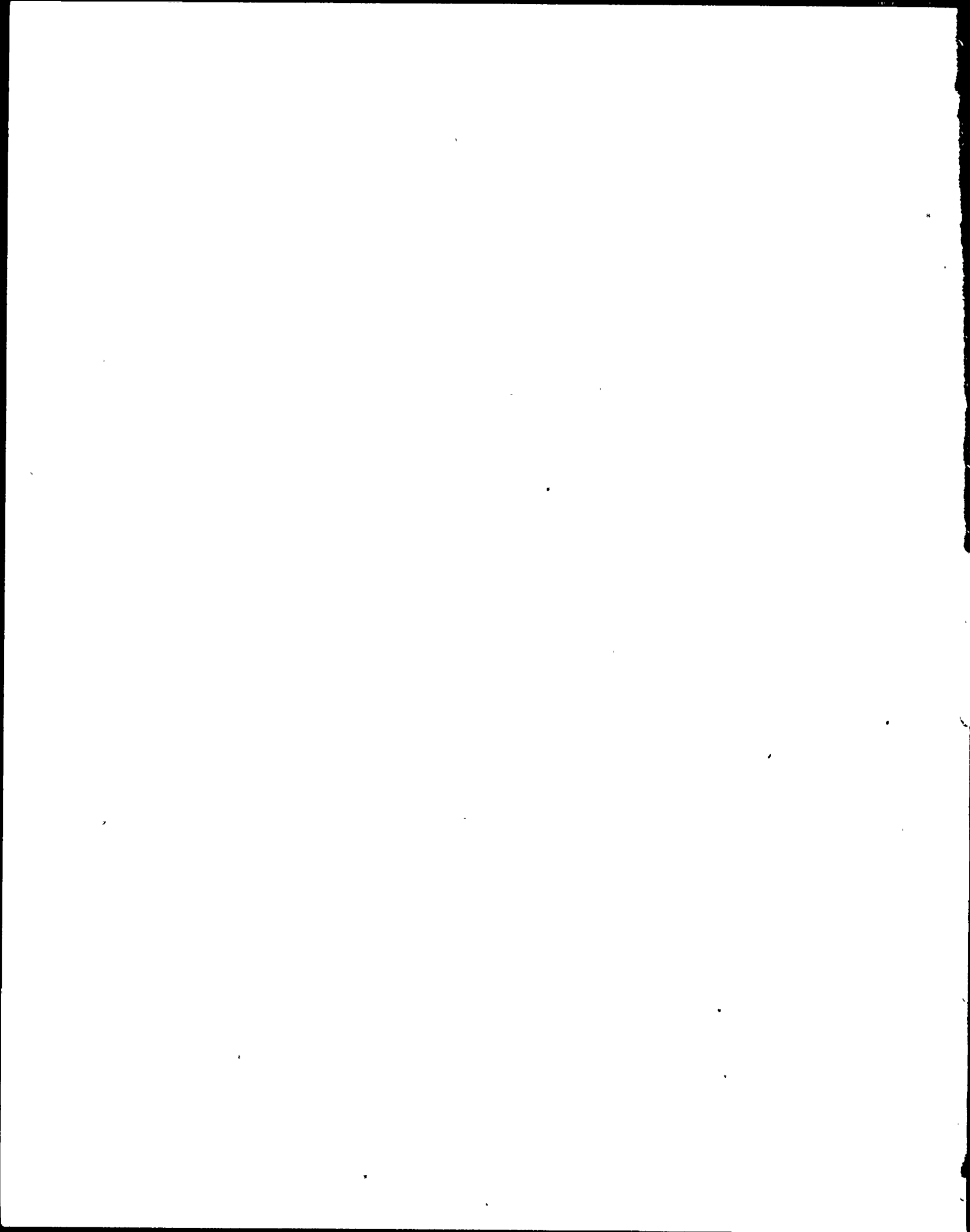
- (a) The unit was operating at 50% level when a mismatch was noted between the "A" and "B" Feedwater Pumps. The unit was manually shutdown. Investigation showed that the 1B Main Feed Pump Discharge valve had jammed, but freed itself during the shutdown.
- (b) No reportable occurrence pertained to this forced outage.
- (c) The valve was disassembled, inspected and the torque setting was readjusted.
- (d) Operating time lost was 23 hours and 30 minutes.
- (e) No major safety-related corrective maintenance was performed during this forced outage. The critical activity path activity dictating the length of the outage was the inspection of the valve.
- (f) No release of radioactivity or radiation exposure was associated with the outage.

ANNUAL REPORT #11

June 6, 1975

FORCED OUTAGE

- (a) The unit was operating at 100% level when a reactor trip occurred due to lo lo level in the "B" Steam Generator. The "A" and "B" Main Steam isolation valves malfunctioned and went closed.
- (b) No reportable occurrence pertained to this forced outage.
- (c) Valve actuators were cleaned and adjusted.
- (d) Operating time lost was 11 hours and 58 minutes.
- (e) No major safety-related corrective maintenance was performed during this forced outage. The critical path activity dictating the length of the outage was the cleaning and adjustment of the valve actuator.
- (f) No release of radioactivity or radiation exposure was associated with the forced outage.

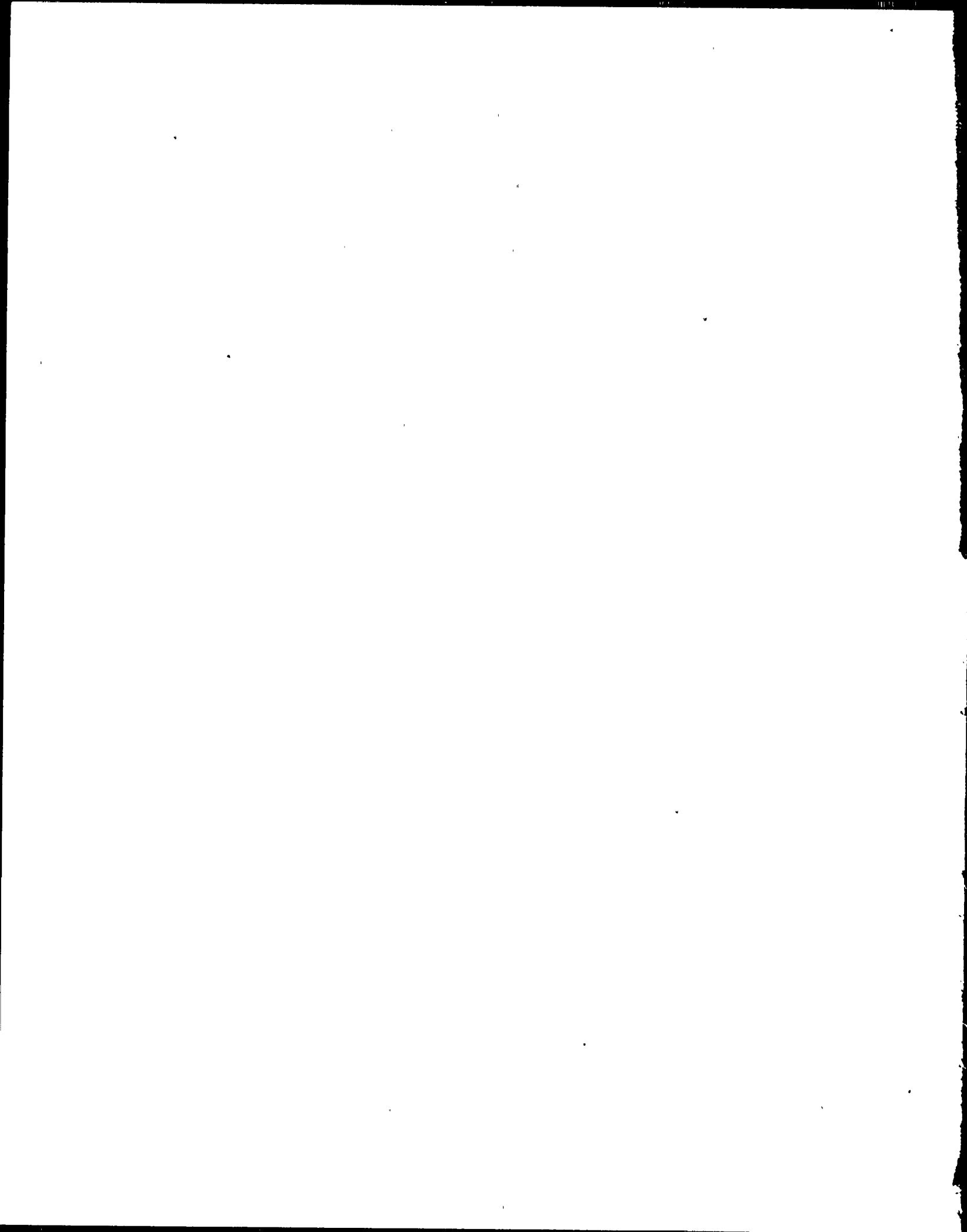


ANNUAL REPORT #11

June 17, 1975

FORCED OUTAGE

- (a) The unit was at 95% level when a reactor trip occurred due to the loss of the "1A" Inverter and corresponding loss of the "1A" Instrument Bus. The "1A" Inverter failed due to a failure of the pulse drive printed circuit card.
- (b) No reportable occurrence pertained to this forced outage.
- (c) Corrective action consisted of replacement of the failed pulse drive printed circuit card.
- (d) Operating time lost was 4 hours and 5 minutes.
- (e) Safety related corrective maintenance was performed on the 1A Inverter. This consisted of replacement of the pulse drive printed circuit card. The critical path activity dictating the length of the outage was the replacement of the pulse drive printed circuit card.
- (f) No release of radioactivity or radiation exposure was associated with the forced outage.

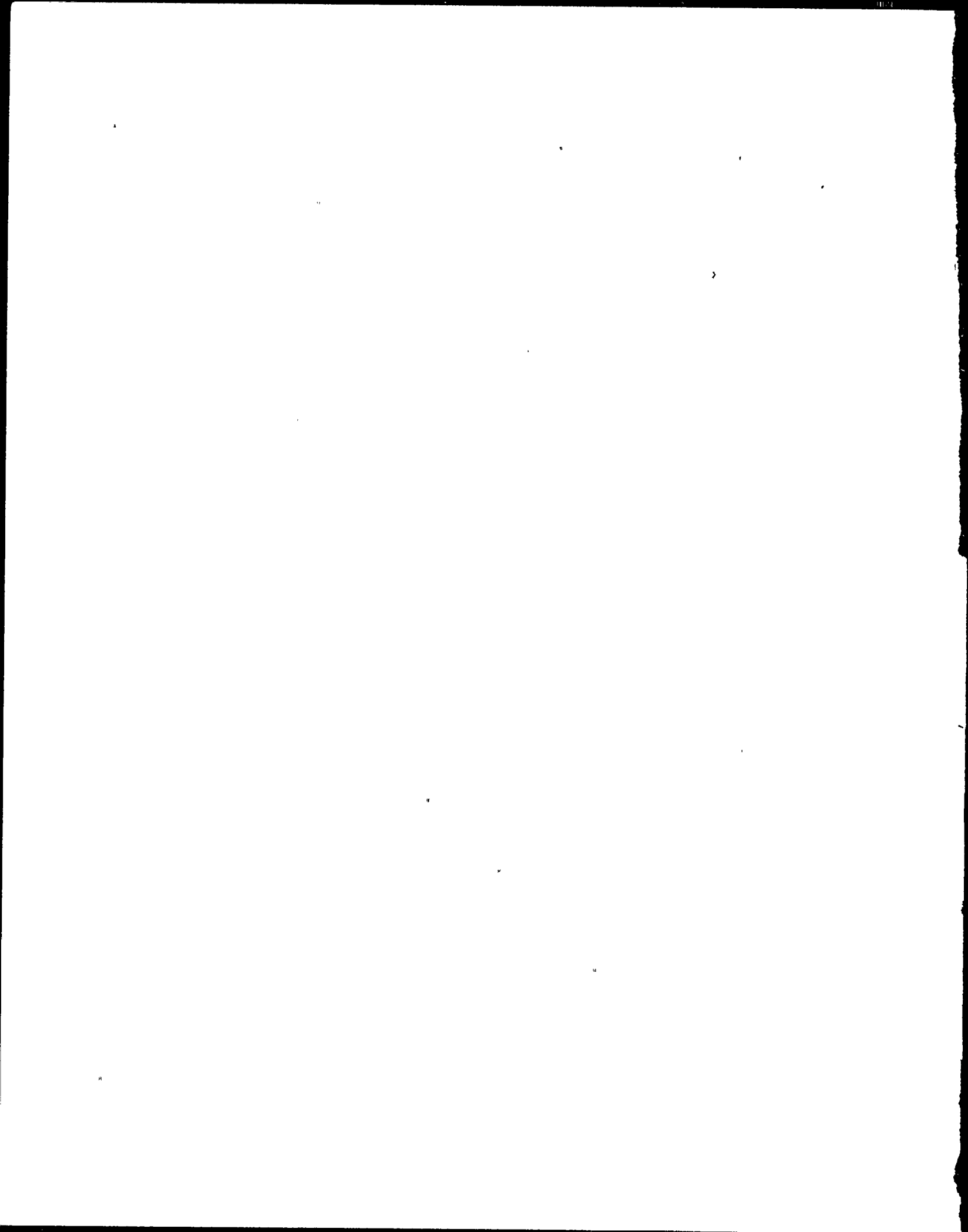


ANNUAL REPORT #11

June 17, 1975

FORCED OUTAGE

- (a) The unit was at 30% level when the turbine was manually tripped due to excessive vibration of the feedwater piping. The feedwater control valves were hunting causing pulsations in the feedwater piping.
- (b) No reportable occurrence pertained to this forced outage.
- (c) Corrective action consisted of modifying the feedwater control valves.
- (d) Operating time lost was 97 hours and 16 minutes.
- (e) No major safety-related corrective maintenance was performed during this forced outage. The critical path activity dictating the length of the outage was the modification of the feedwater control valves.
- (f) No release of radioactivity or radiation exposure was associated with the forced outage.

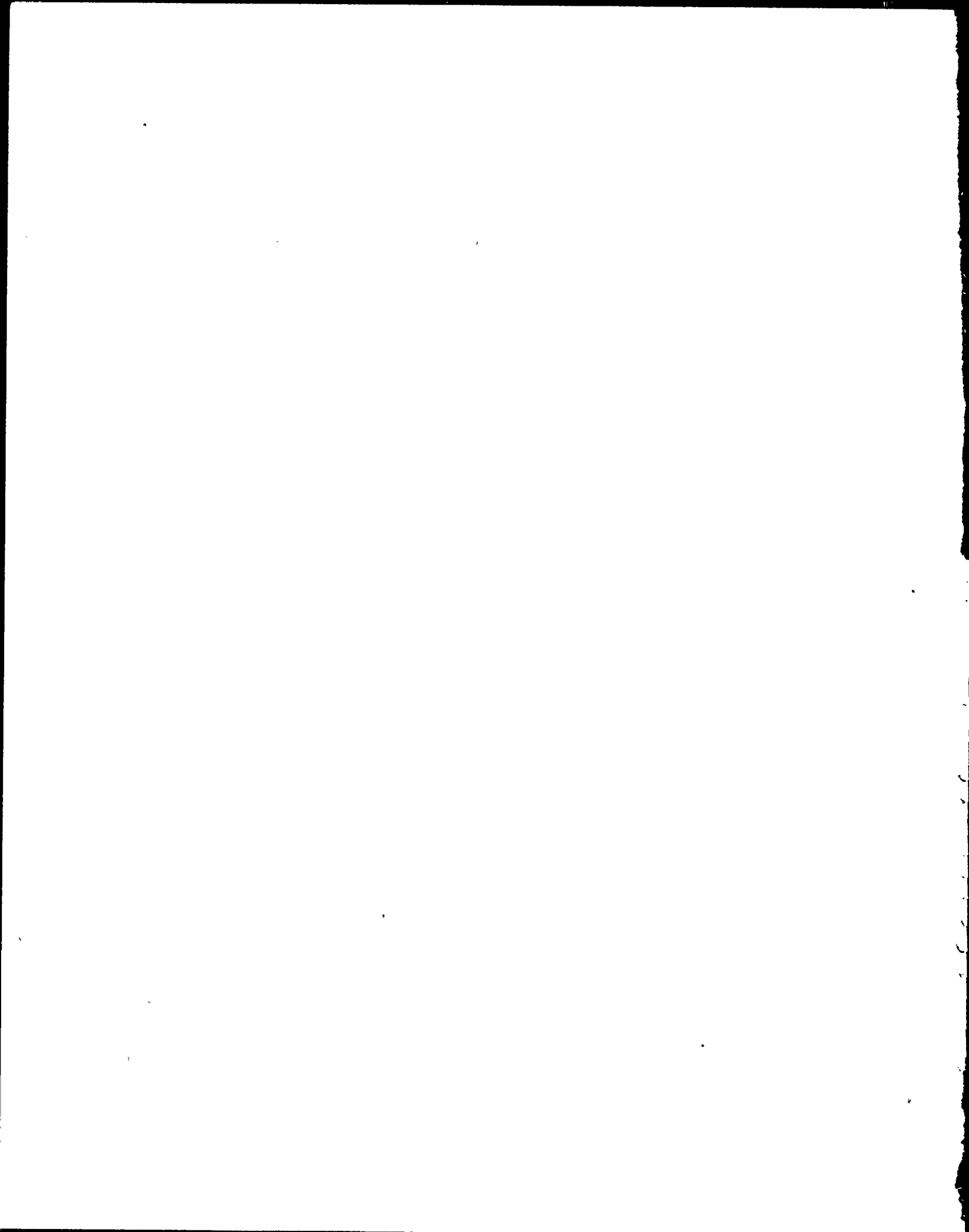


ANNUAL REPORT #11

June 23, 1975

FORCED OUTAGE

- (a) The unit was operating at 88% level when a reactor trip occurred due to lo lo level in the "B" Steam Generator. The "A" and "B" main steam isolation valves malfunctioned and went closed.
- (b) No reportable occurrence pertained to this forced outage.
- (c) Corrective action consisted of modifying the flapper stop to reposition the flapper when in the open position and modify the actuator to accept this new position.
- (d) Operating time lost was 86 hours and 45 minutes.
- (e) Safety-related corrective maintenance was the repositioning of the flapper and the modification of the actuator. These actions were the critical path activity dictating the length of the outage.
- (f) No release of radioactivity or radiation exposure was associated with the forced outage.



ANNUAL REPORT #11

July 20, 1975

FORCED REDUCTION

- (a) The unit was reduced from 100% level to 14% level to permit the repair of a steam leak on the high pressure turbine drain line.
- (b) No reportable occurrence pertained to this forced reduction.
- (c) No appropriate corrective action was required.
- (d) Operating time lost was 5 hours and 45 minutes.
- (e) No major safety-related corrective maintenance was performed during this forced reduction. The critical path activity dictating the length of the reduction was the repair of the drain line steam leak.
- (f) No release of radioactivity or radiation exposure was associated with the forced reduction.

ANNUAL REPORT #11

July 24, 1975

FORCED OUTAGE

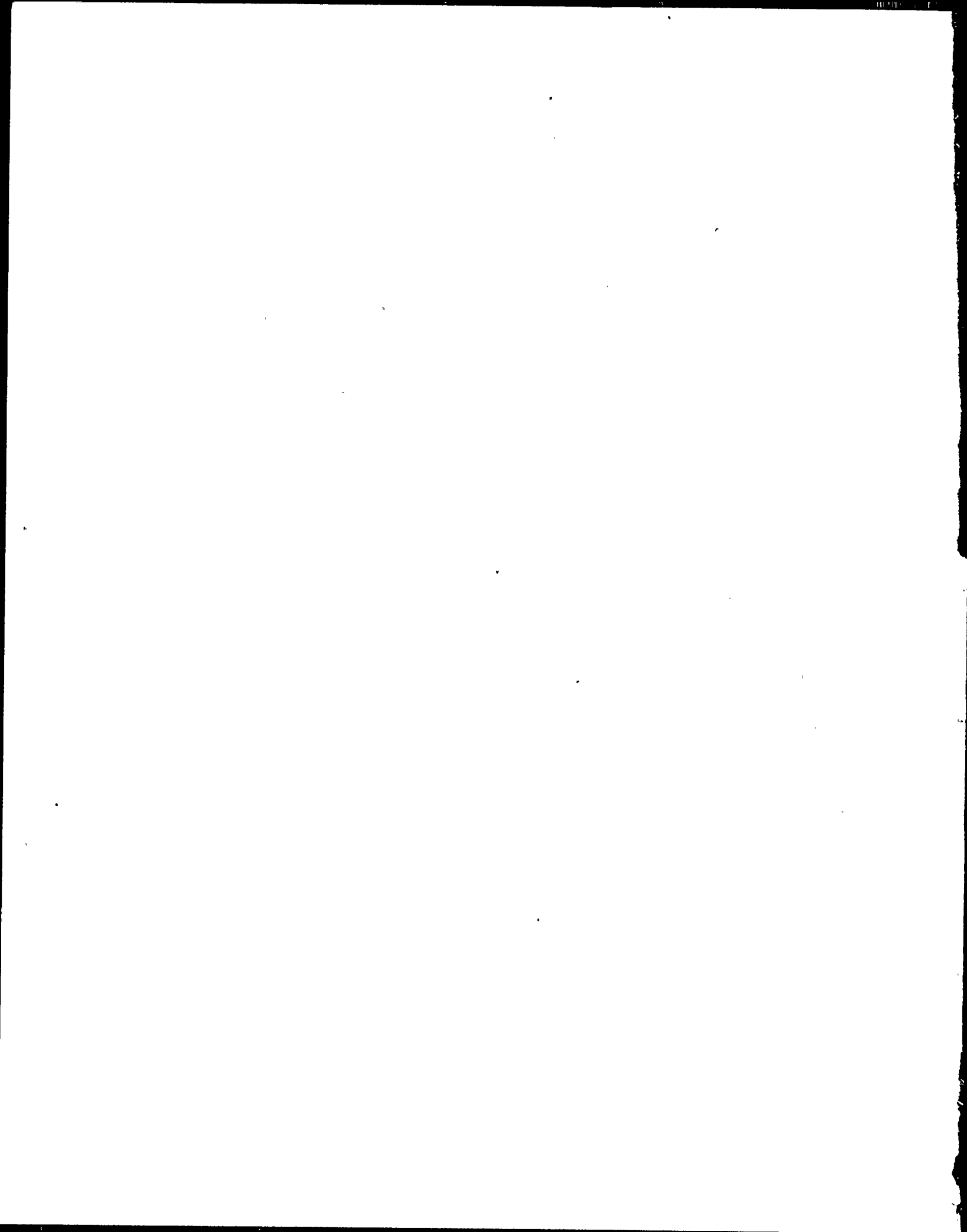
- (a) The unit was at 100% level when a lightning strike at station 13A caused a failure of the generator differential relay which in turn caused a turbine trip.
- (b) No reportable occurrence pertained to this forced outage.
- (c) No appropriate corrective action was required.
- (d) Operating time lost was 12 hours and 22 minutes.
- (e) No major safety-related corrective maintenance was performed during this forced outage. The critical path activity dictating the length of the forced outage was the repair of the generator differential relay and testing to verify the integrity of the Primary and Secondary Electrical Systems involved in the protective relays that operated.
- (f) No release of radioactivity or radiation exposure was associated with the forced reduction.

ANNUAL REPORT #11

July 30, 1975

FORCED REDUCTION

- (a) The unit was reduced from 99% level to 48% level to permit removal of the "1A" condenser from service. Chemistry samples indicated a leaking condenser tube. There were no leaking tubes. The source of sodium was traced to a leaking heating coil in the sodium hydroxide tank.
- (b) No reportable occurrence pertained to this forced reduction.
- (c) Corrective action consisted of diverting the condensate from the heating coil to a floor drain.
- (d) Operating time lost was 13 hours and 49 minutes.
- (e) No major safety-related corrective maintenance was performed during this forced reduction. The critical path activity dictating the length of the reduction was the inspection of the condenser tubes.



ANNUAL REPORT #11

October 10, 1975

SCHEDULED OUTAGE

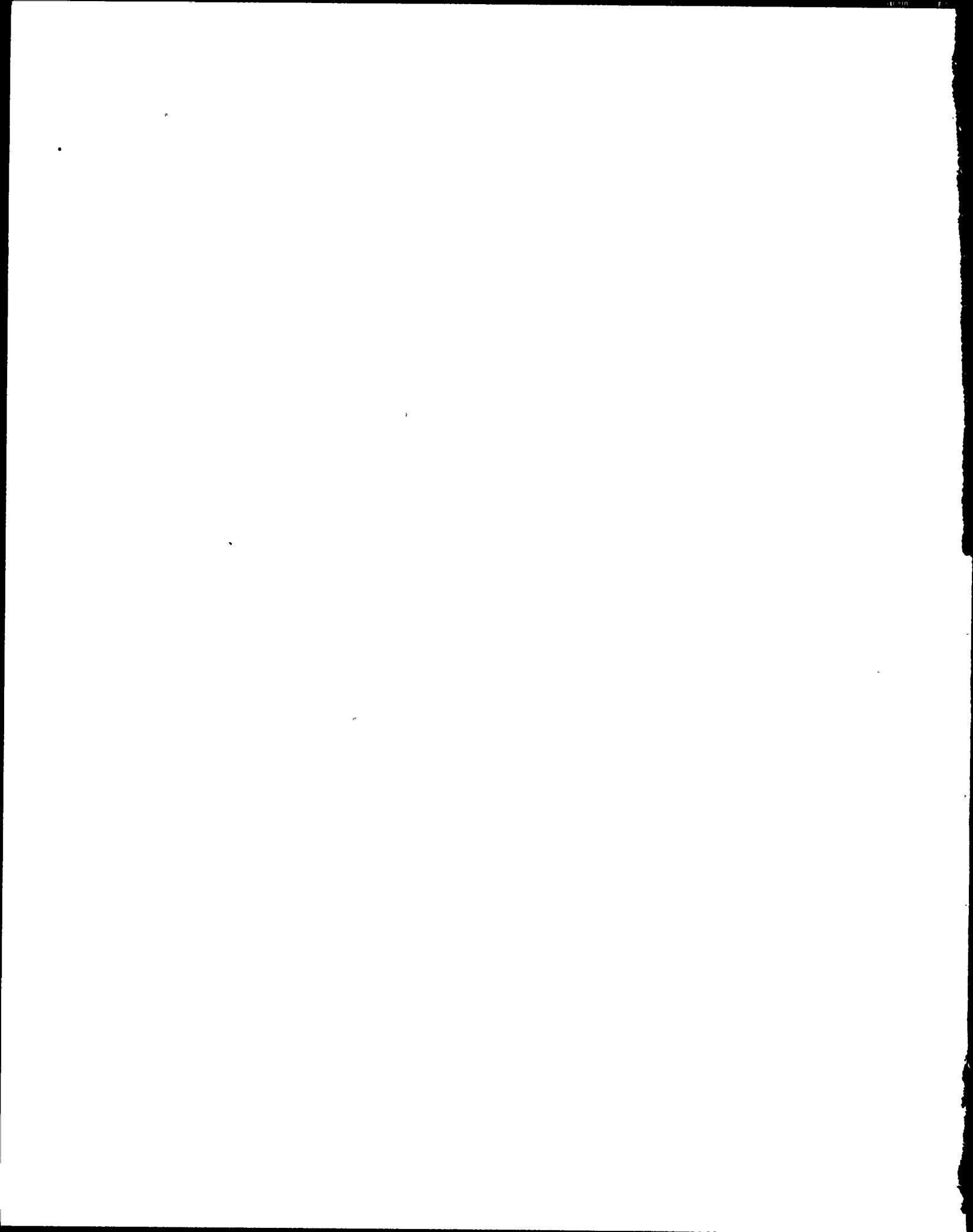
- (a) The unit was scheduled to be shutdown for replacement of the power cables to the lake intake heaters.
- (b) No reportable occurrence pertained to this scheduled outage.
- (c) No appropriate corrective action was required.
- (d) Operating time lost was 43 hours and 32 minutes.
- (e) Three safety-related corrective maintenance items were performed concurrent with the replacement of the heater power cables. However, the replacement of the heater power cables was the critical path activity dictating the lengths of the outage. The safety-related corrective maintenance items were:
 - 1. "B" Auxiliary Feedwater isolation valve was leaking at the seal ring gasket. The leak was sealed by using the Furmanite process.
 - 2. The Hydraulic Snubber on the "B" Feedwater line had a scored accumulator cylinder and was leaking fluid. The accumulator was replaced.
 - 3. Bank B Control Rod Drive Mechanism has a faulty connector at the Reactor head causing the Control Rod not to stop. The cable and connector were replaced at the reactor head.
- (f) No release of radioactivity or single radiation exposure specifically associated with the outage accounted for more than 10% of the allowable annual values.

ANNUAL REPORT #11

December 21, 1975

FORCED REDUCTION

- (a) The unit was reduced from 100% to 47% level to permit the removal of the "1B" condenser from service. The water boxes were opened, but no leak could be detected.
- (b) No reportable occurrence pertained to this forced reduction.
- (c) No appropriate corrective action was required.
- (d) Operating time lost was 15 hours and 15 minutes.
- (e) No major safety-related corrective maintenance was performed during this forced reduction. The critical path activity dictating the length of the reduction was inspection of the tube sheets.
- (f) No release of radioactivity or radiation exposure was associated with the forced reduction.



ANNUAL REPORT #11

December 23, 1975

FORCED REDUCTION

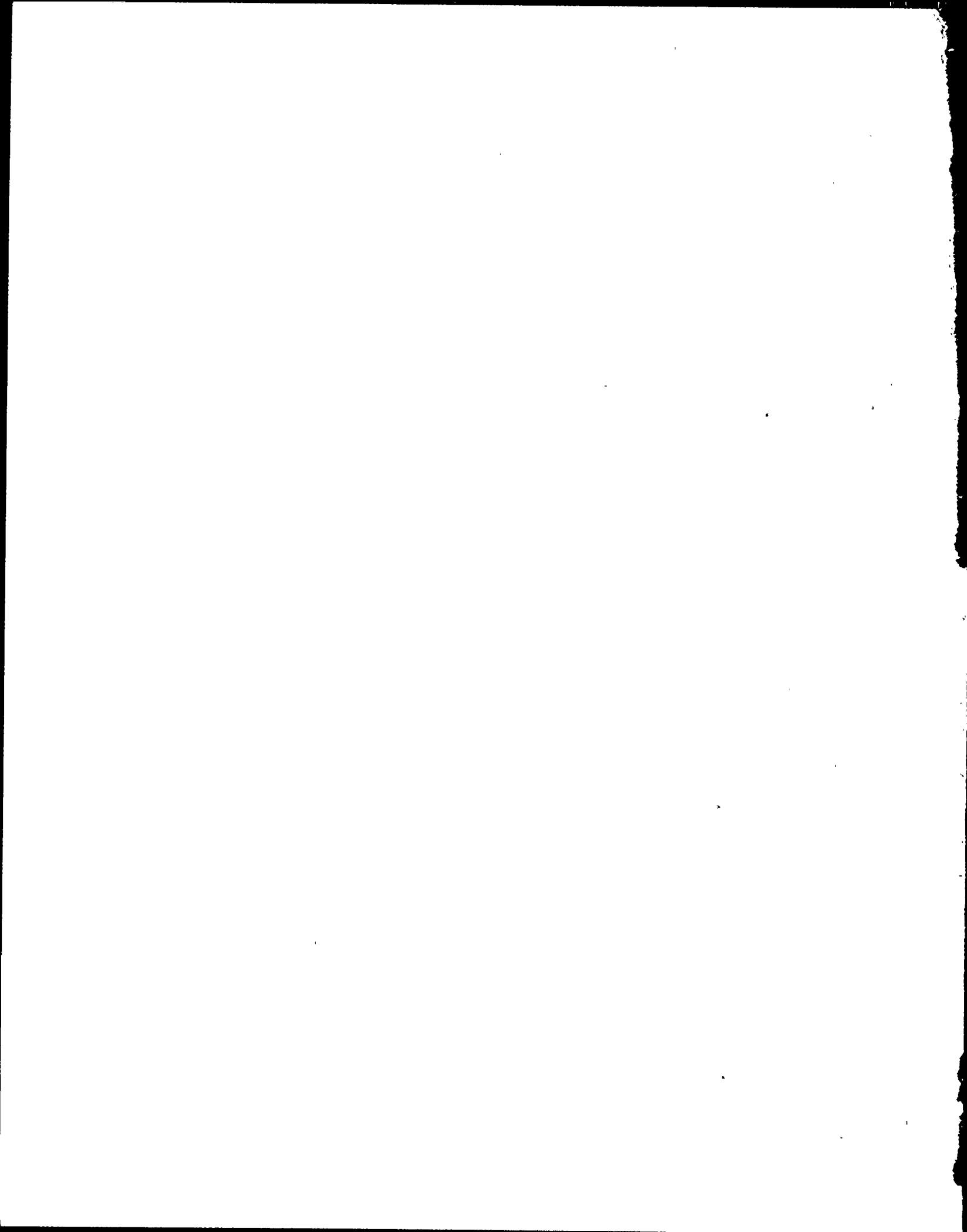
- (a) The unit was reduced from 100% to 46% level to permit the removal of the "1B" Condenser from service. Investigation showed that one tube was leaking.
- (b) No reportable occurrence pertained to this forced reduction.
- (c) Appropriate corrective action was to plug the leaking tube.
- (d) Operating time lost was 15 hours and 25 minutes.
- (e) No major safety-related corrective maintenance was performed during this forced reduction. The critical path activity dictating the length of the reduction was locating and plugging of the leaking tube.
- (f) No release of radioactivity or radiation exposure was associated with the forced reduction.

ANNUAL REPORT #11

December 30, 1975

FORCED OUTAGE

- (a) The unit was at 100% level when the increase in activity in the "B" Steam Generator started to approach the Technical Specification limit. To prevent exceeding this limit the unit was taken to cold shutdown.
- (b) No reportable occurrence pertaining to this forced outage had been written, as of the end of this reporting period.
- (c) No corrective action had taken place, as of the end of this reporting period.
- (d) Operating time lost was 43 hours and 45 minutes, as of the end of this reporting period.
- (e) No major safety-related corrective maintenance was started as of the end of the reporting period. The critical path activity dictating the length of the outage could not be determined, as of the end of this reporting period.
- (f) No single release of radioactivity or radiation exposure, specifically associated with the outage, accounted for more than 10% of the allowable annual values, as of the end of this reporting period.



ANNUAL REPORT #11

JANUARY 1, 1975 THRU DECEMBER 31, 1975 INCLUSIVE

1975 GINNA STATION IN-PLANT EXPOSURE

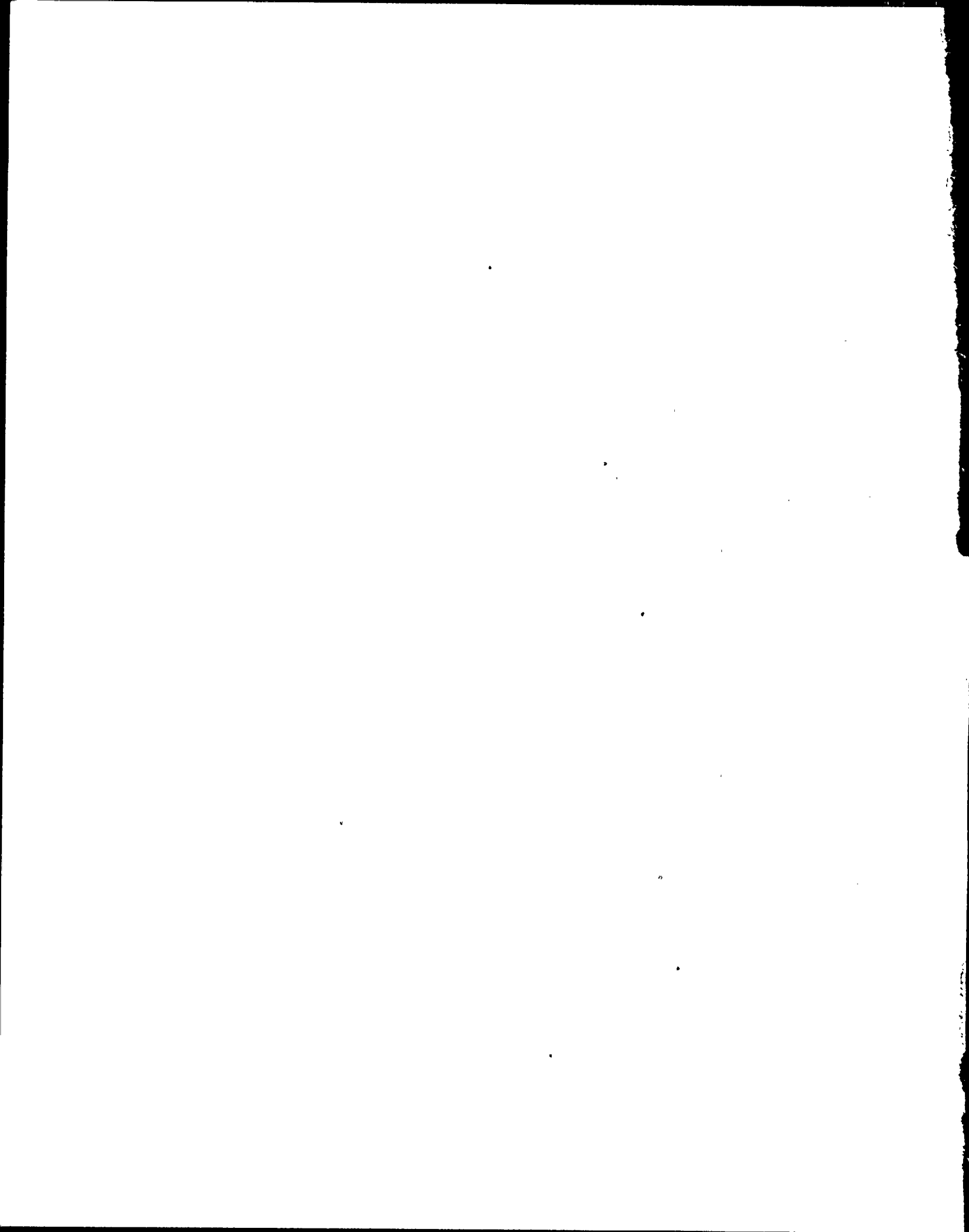
6.9.1.b.3

EXPOSURE IN REM (by Film Badge)

<u>Manpower Source</u>	<u>Number of Personnel >100 mrem</u>	<u>Reactor Operation and Surveillance</u>	<u>Routine Maintenance</u>	<u>Special Maintenance</u> a/b	<u>Refueling</u>	<u>Inservice Inspection</u>	<u>Waste Processing</u>	<u>Misc.</u>	<u>Total Rem</u>
Permanent Plant	87	27	54	6/0	11	2	13	0	113
Other RG&E	304	0	84	120/4	50	4	0	0	262
Contractors	167	0	24	30/20	0	24	6	17	121
Total	558	27	162	156/24	61	30	19	17	496

a. Steam Generator Tube Examination, Mechanical Modifications

b. Auxiliary Feedwater Piping Installation



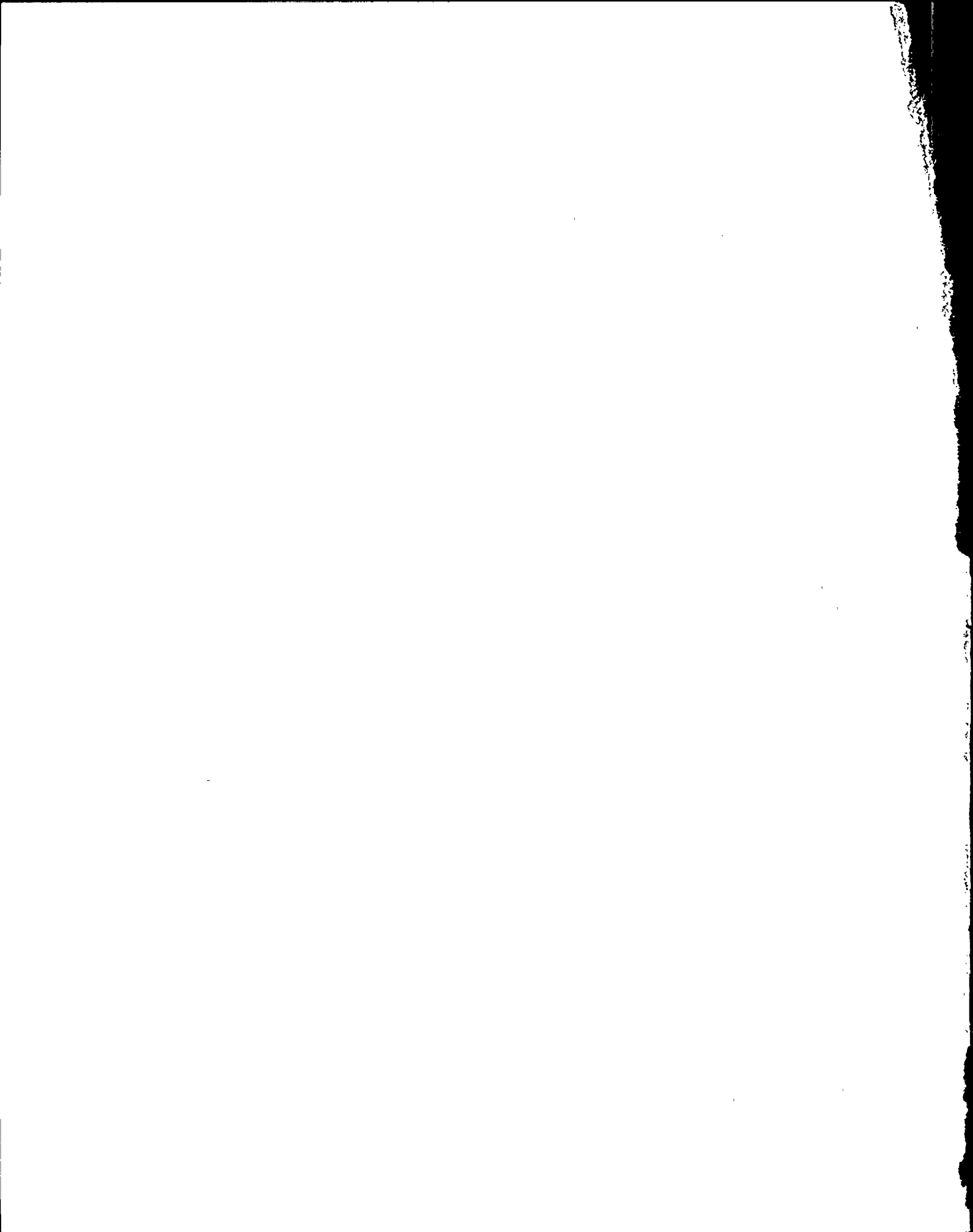
ANNUAL REPORT #11

JANUARY 1, 1975 THRU DECEMBER 31, 1975 INCLUSIVE

6.9.1.b.3

During 1975, Rochester Gas & Electric Corporation provided monitoring for 836 individuals. The table below summarizes the doses received by individuals during the year. The 4 individuals who exceeded 3.0 Rem were permanent Ginna Station employees and the maximum dose was 3.430 Rem.

Dose Ranges (Rem)	Number of Individuals in each range
No measurable Exposure	151
Measurable Exposure less than .100	127
.100 - .250	93
.250 - .500	93
.500 - .750	68
.750 - 1.000	74
1.000 - 1.500	149
1.500 - 2.000	57
2.000 - 3.000	20
3.000 - 4.000	4
4.000 +	0



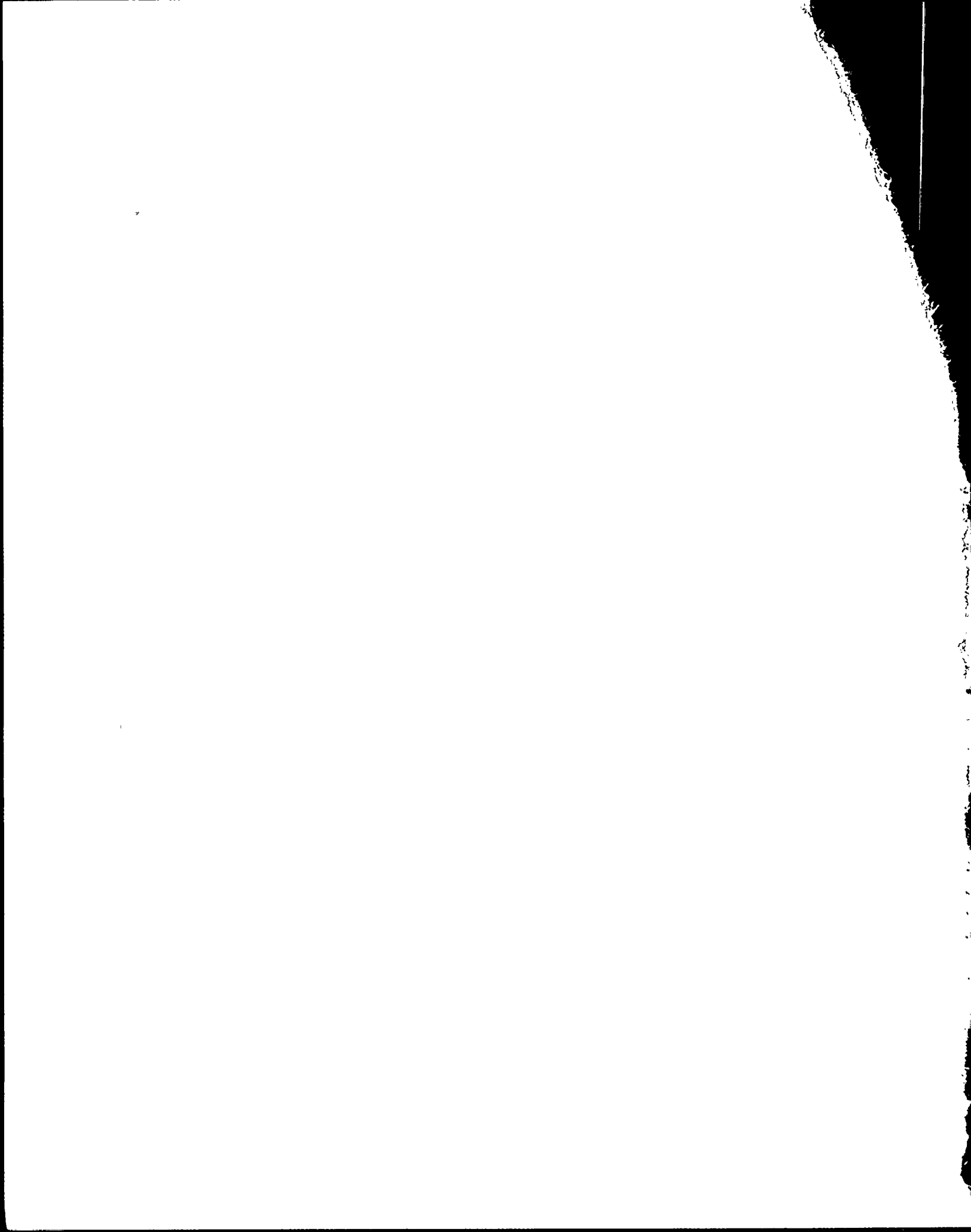
ANNUAL REPORT #11

January 1, 1975 to December 31, 1975 Inclusive

6.9.1.b (4) Failed Fuel Indications

There were no indications of failed fuel from visual examinations performed. Four fuel assemblies, X-01 thru X-04, were inspected by underwater television inspections. Other fuel assemblies were viewed by binoculars as they were transferred.

There were no examinations performed on irradiated fuel, nor were eddy current or ultrasonic testing performed.



ANNUAL REPORT #11

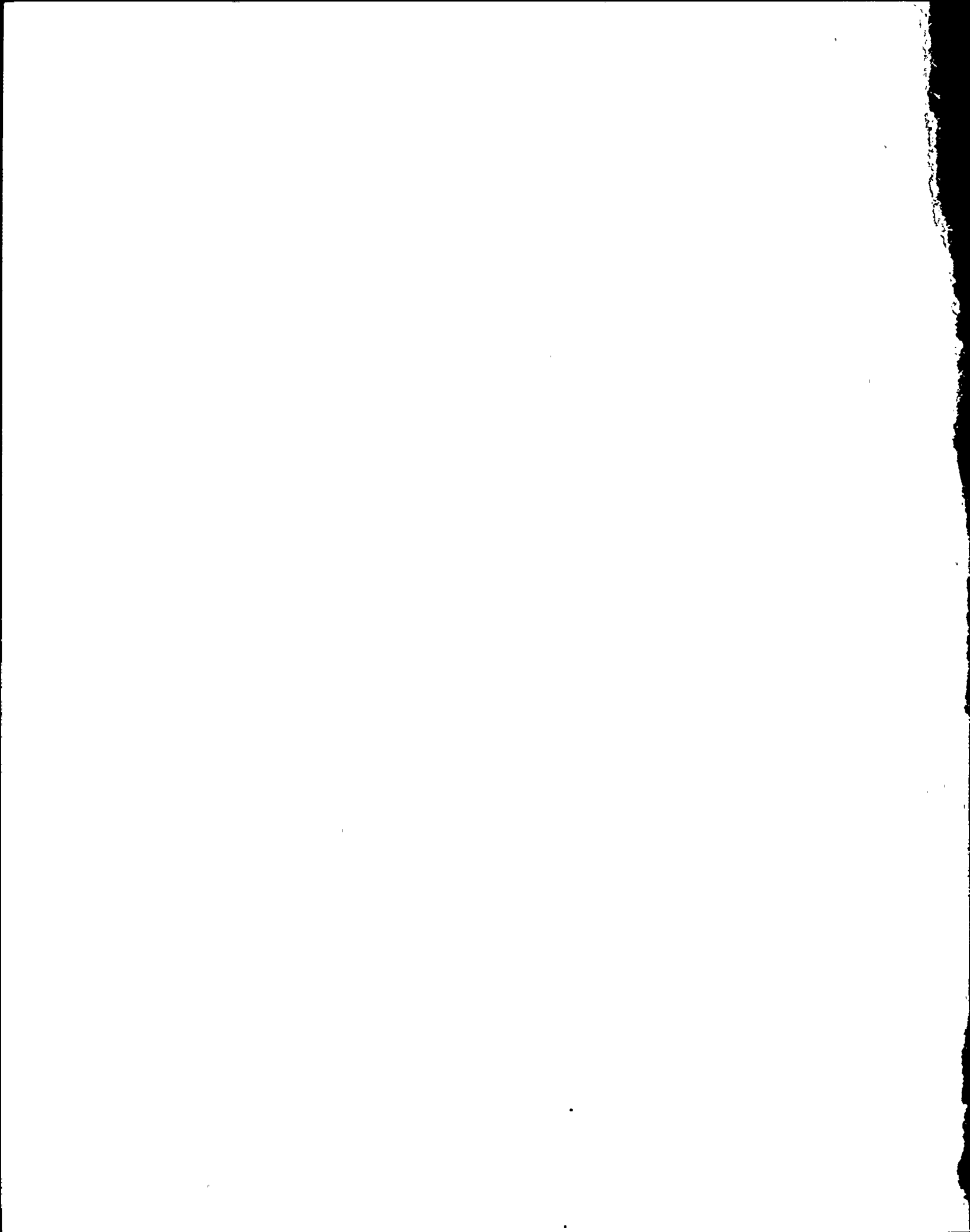
JANUARY 1, 1975 TO DECEMBER 31, 1975 INCLUSIVE

6.9.3.a Environmental Monitoring 1975

A summary of the survey results is presented below:

(1) <u>TYPE</u>	NO..OF LOCATION	(ab) NO. OF SAMPLES	(d) FREQUENCY	(c) <u>RESULTS</u>	
				AVG.	MAX.
Atmospheric Dust	11	541	Weekly	.077	.320 pCi/m ³
Fall Out	5	59	Monthly	42.5	86: pCi/m ²
Radiation - Film	12	144	Monthly	<10	<10 mRem
TLD	23	46	Quarterly	24.1	30.3 mRem
Lake Ontario Water	4	183	Weekly	5.5	13.5 pCi/l
Deer Creek Water	2	22	Monthly	7.6	14.4 pCi/l
Well Water	1	10	Monthly	12.1	16.7 pCi/l
Milk	2	10	Monthly	.38	1.21 pCi/l
Fruit	1	1	Annual	.028	<u>pCi Cs137</u>
Marine Organisms - Fish	1	4	Quarterly	2.84	^{gm} 3.45 pCi/gm
Algae	1	1		.9.7	pCi/gm
Lake Bottom	1	1	Annual	14.2	pCi/gm

6.9.3.a (2) No sample or measurement indicated statistically significant levels of radioactivity above the background.



ANNUAL REPORT #11

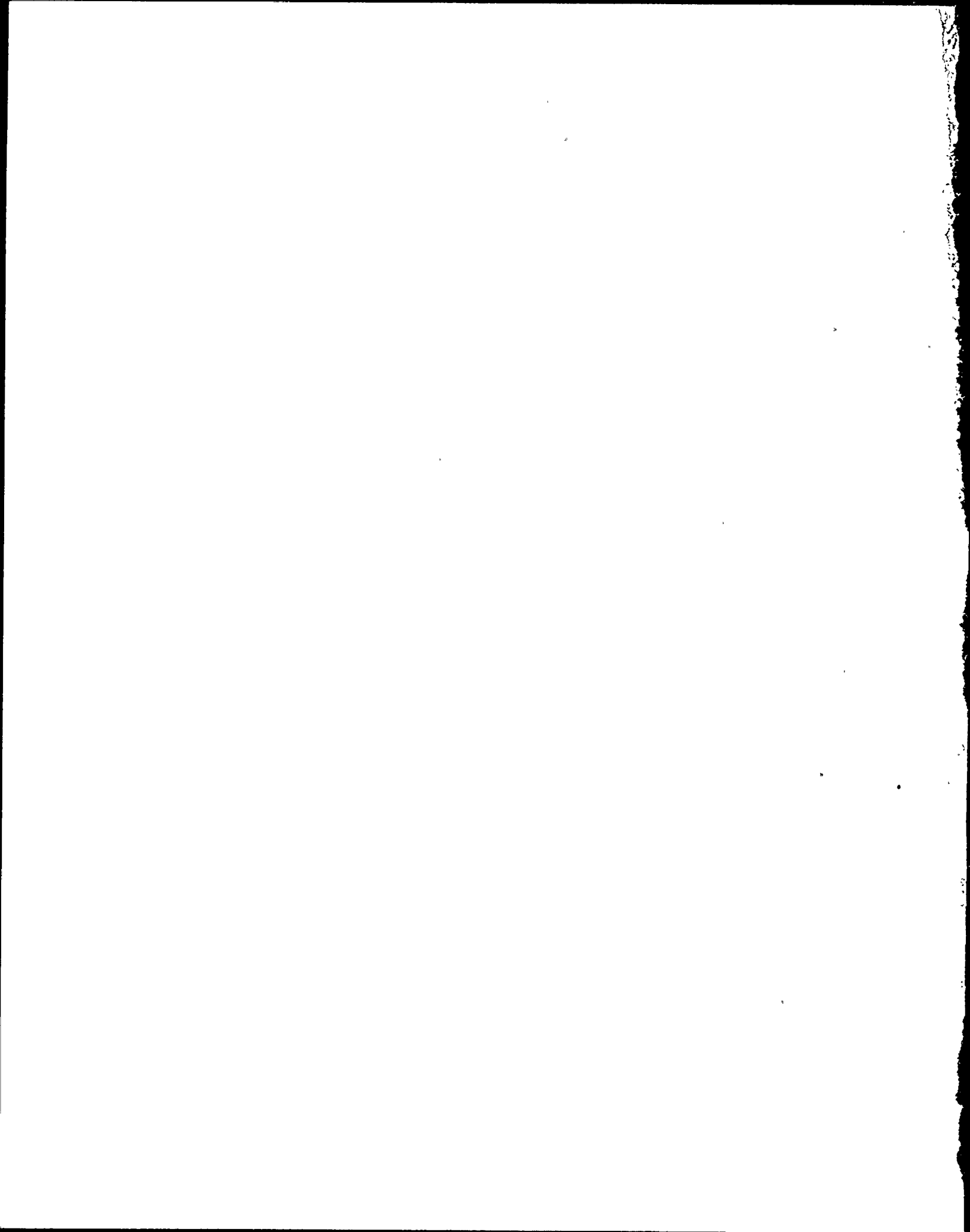
JANUARY 1, 1975 TO DECEMBER 31, 1975 INCLUSIVE

Annual Report of all procedure changes:

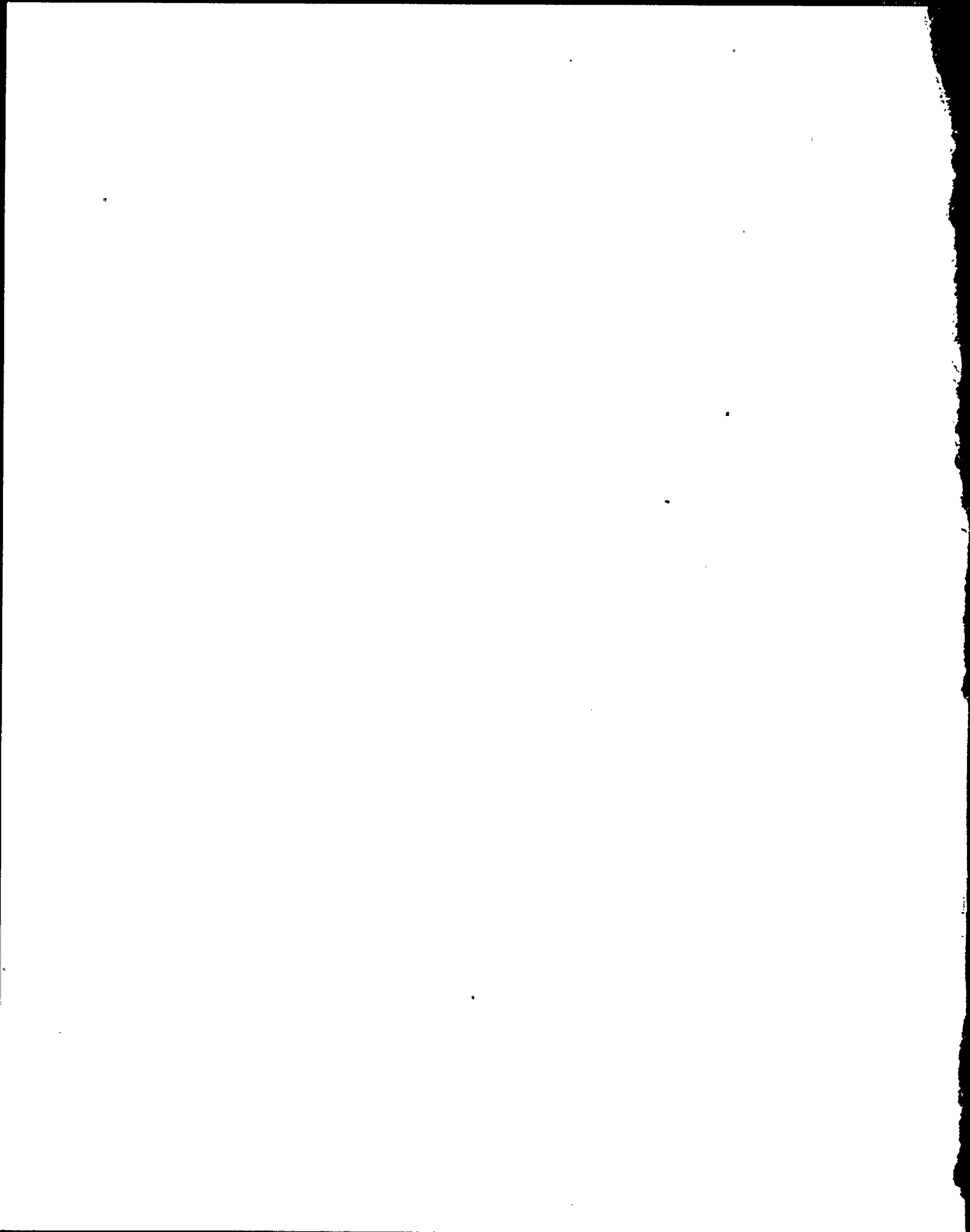
PORC ITEM
NUMBER:

DESCRIPTION

- 75-1 A-2.3, Maximum Unit Power, PCN74-1476. J. Straight presented this procedure change. It requested;
- Add reference 2.2;
- 2.2 - U.S. Atomic Energy Commission R.O. Inspection Report No. 50-244/74-09 - Item E.4.c
- Add to 3.1; If the calorimetric exceeds 102.0%, notify the plant supt. and/or duty engineer.
- Add note: notification of plant supt. and/or duty engineer ensures compliance with containment in Reference 2.2.
- The Committee recommended approval of this change.
-
- 75-3 PT-23.15, Containment Isolation Valve Leak Rate Testing, Containment Air Sample Out, PCN75-4. J. Straight presented this procedure change. These changes were to reflect valve numbers and other minor changes. The Committee recommended approval of this change.
-
- 75-4 RSSP-5, Immediate Boration System, PCN75-3. J. Straight presented this procedure change. It requested to incorporate test control tag steps along with the request to delete steps 4.4.1 through 4.4.3. The Committee recommended approval of this change.
-
- 75-5 RSSP-9, Rod Position Indicator Alignment, PCN75-7. J. Straight presented this procedure for deletion. This procedure is now covered by CP-2.0. The Committee recommended approval of this deletion.



<u>PORC ITEM NUMBER:</u>	<u>DESCRIPTION</u>
75-14	<u>ST-75-1, Rapid Fuel Assembly Power Escalation Test, PCN75-17.</u> C. Peck presented this new procedure. This procedure is to determine the effect of rapid power escalation upon fuel cladding that has a high burn-up. The Committee reviewed this procedure.
75-22	<u>O-6, Operation & Process Monitoring, PCN75-16.</u> J. Straight presented this procedure change. It requested step 5.10 be changed to read; "Log average delta-flux every half hour to insure adherence to Technical Specifications" The Committee recommended approval of this change.
75-23	<u>P-6, Precautions, Limitations & Setpoints Nuclear Instrumentation System, PCN75-2.</u> J. Straight presented this procedure change. It requested; B - <u>Set Points</u> Change NC-304 setpoint (P-8) to read 50% - Full Power. Also update to A-30 format. The Committee recommended approval of this change.
75-25	<u>PT-3, Containment Spray Pump & NaOH Additive System, PCN75-35.</u> J. Straight presented this procedure change. It requested step 6.3.5 be added; 6.3.5 - open test line valve 864A _____ The Committee recommended approval of this change.
75-26	<u>T-8F, Filling The Hypochlorite Tank, PCN75-8.</u> J. Straight presented this new procedure. This procedure is to describe the steps for the filling of the Sodium Hypochlorite Storage Tank. The Committee recommended approval of this procedure.



PORC ITEM
NUMBER:

DESCRIPTION

75-37
(74-331)

A-46, Bypass Of Safety Function Or Jumper Control, PCN75-117.
J. Straight presented this procedure change. These changes provide requirements that the jumper request has been verified by two qualified personnel, the defeat or bypass device installation locations are clearly described, and criteria for PORC review are identified. The Committee recommended approval of this change.

75-38

PT-17.2, Process Monitors R-11 - R-20, Iodine Monitors R-10A & R-10B, PCN75-46. J. Straight presented this procedure change. It requested several changes be made to further avoid repeated trips. The Committee recommended approval of this change.

75-39

E-16.1, High Activity, Radiation Monitoring System, PCN75-49. J. Straight presented this procedure change. This was a rewrite to conform to A-30 format. The Committee recommended approval of this change.

75-41

GS-21, Site Access, PCN75-37. J. Straight presented this procedure change. It requested step 3.1.2.1 be changed to read;
"Personnel issued a white photo-identification card, do not require permission to enter the site. During off hours the control room will be advised upon entry of these personnel. Personnel issued"

The Committee recommended approval of this change.

75-42

M-11, Replacement Or Maintenance Of A Boric Acid Transfer Pump, PCN75-56, 75-57, 75-121. J. Straight presented these temporary changes for review.

PCN75-56 - add to step 5.1.4; "to close valve 345 to provide .."

PCN75-57 - requested temporary approval to close valves 348-A, 341, 347 and 398 in order to check diaphragm in valve 442.

PCN75-121 - N.A. steps 5.10 and 5.11

The Committee reviewed these temporary change.

75-43

M-32, Use Of Circuit Breaker Multi-Amp Test Unit, PCN75-45.
J. Straight presented this procedure change. It requested minor changes be made to the Breaker Test Data sheet. The Committee recommended approval of this change.

PORC ITEM
NUMBER:DESCRIPTION

75-44

0-5.2, Load Increases, PCN75-116. J. Straight presented this procedure change. It requested a new precaution be added to read;

4.3 If valve position limit light is lit on the E.H. panel, reduce load until light clears, and before turbine load increase commences, raise valve position limit back to 100%

The Committee recommended approval of this change.

75-45

M-11.4.1, Isolation of 1A Charging Pump For Maintenance, And Restoring To Service After Maintenance. J. Straight presented this temporary change for review only.

It requested steps 4.4 through 4.9 and 5.1 through 5.8 be N.A.'d.

It requested also to change step 4.10 to read;

4.10.1 - Remove damaged shifting motor varidisc assembly

4.10.2 - Install new shifting motor varidisc assembly including studs and ball bearing

4.10.3 - Install new shift box upper cam followers

NOTE: See Ajax Iron Works Charging Pump with Motors and varidrive instruction manual for maintenance items.

The Committee reviewed this change.

75-46

E-12.1, Malfunctioning Rod Cluster Control Full Length Rod, PCN75-42,

&

E-12.2, Malfunctioning Part Length Rod, PCN75-43.

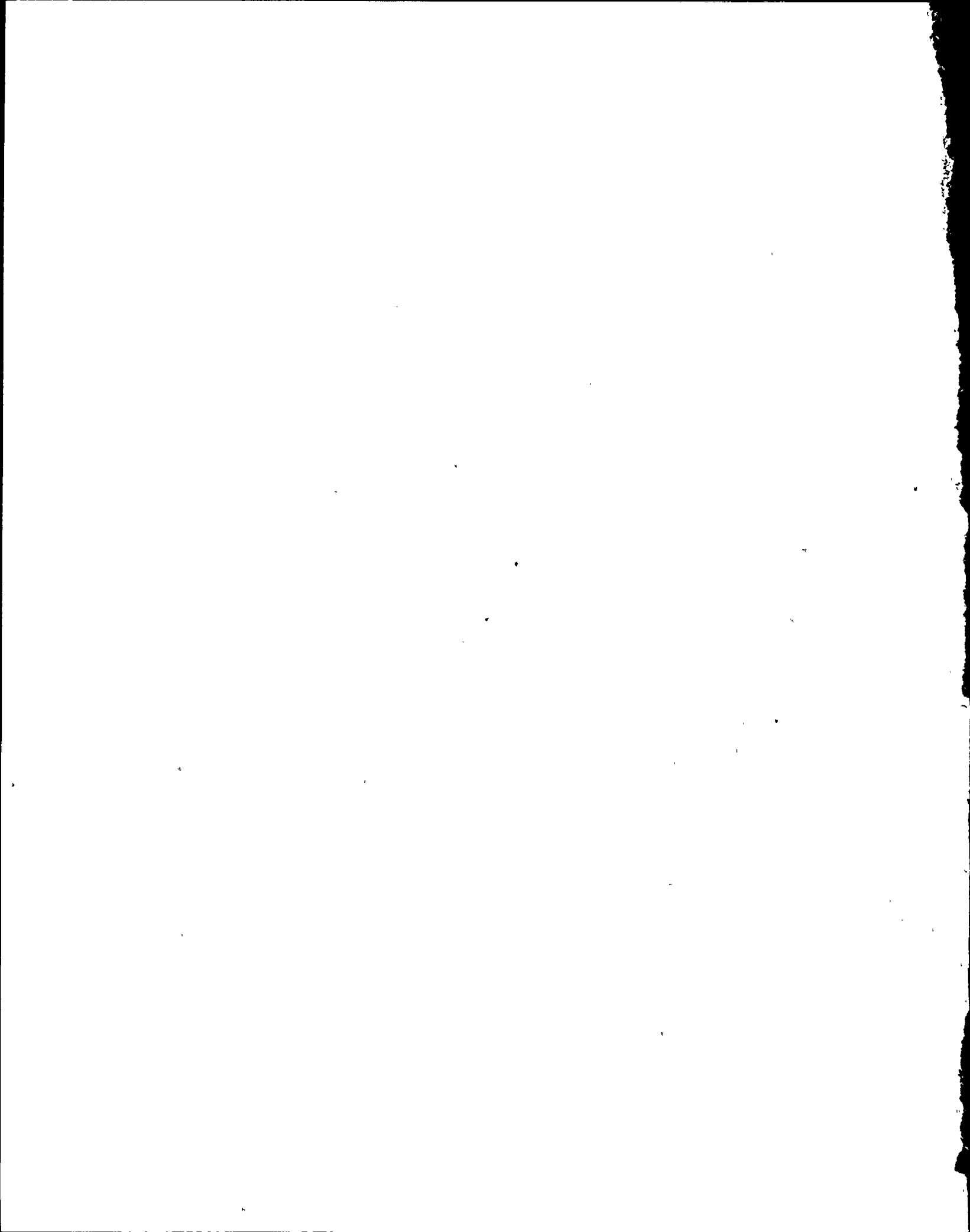
J. Straight presented these two new procedures. They were rewritten to conform to A-30 format, also to separate them from part & full length rods. The Committee recommended approval of this change. This item IS COMPLETE.

75-47

E-12, Malfunctioning Rod Cluster Control, PCN75-41. J. Straight presented this procedure change. It requested this procedure be deleted as it has been rewritten into 2 separate procedures. The Committee recommended approval of this deletion.

75-48

CP-128.0, Calibration and/or Maintenance Of Charging Line Flow Channel "128", PCN75-44. J. Straight presented this procedure change. It requested on the System Data Sheet, under Test Equipment to remove the S/N after Foxboro Current Simulator. The Committee recommended approval of this change.



PORC ITEM
NUMBER:DESCRIPTION

- 75-31 EM-136, Maintenance Of Non-Regenerative Heat Exchanger
Component Cooling Water Valve TCV 130, PCN75-54.
S. Spector presented this new procedure. The purpose of this procedure is to describe the steps necessary to isolate TCV-130, perform necessary maintenance and return system to service. The Committee recommended approval of this procedure.
- 75-32 S-3.2E, Establishing Normal Letdown, PCN75-55.
J. Straight presented this new procedure to go from excess letdown to normal letdown. The Committee recommended approval of this procedure.
- 75-35 TCV-130, Temperature Control Valve For The Component Cooling
From The Non-regenerative Heat Exchanger. J. Noon reported to the Committee that after determining the cause of failure of the rubber seat in the valve Mr. George Coons, of R.M.Newell Co. was contacted. R.M.Newell is the local representative for Continental (Fisher) Valves.

He has recently informed me that the seat is vulcanized to the valve body. This valve is not a shelf item and lead time is 30-36 weeks. He also checked alternative and stated the same style valve with a cast iron body and a metal seat is available. It has shutoff leakage of about 38 gpm and has a capacity well in excess of the 500 gpm in the original purchase specs. Since the valve is not required to provide shutoff isolation, this valve should be suitable. The method seat will provide a more maintenance-free valve. Mr. Coons is forwarding information on the metal seated valve to us.
- 75-36 GS-31.0, Door Alarm System, PCN75-38, 75-118. J. Straight presented these procedure changes.

PCN75-38 - requested a new step 3.16.1 be added to read;

"Doors 7 thru 14 and doors 19 thru 23 may be unlocked during normal working hours at the discretion of the shift foreman. Door 25 will be unlocked by operating personnel around 0700 hours Monday thru Friday, excluding holidays. Door 24 will be unlocked by general office personnel when the office is staffed, Monday thru Friday, excluding holidays unless advised otherwise by the shift foreman. The above doors will all be locked at 1700 hours unless advised otherwise by the shift foreman."

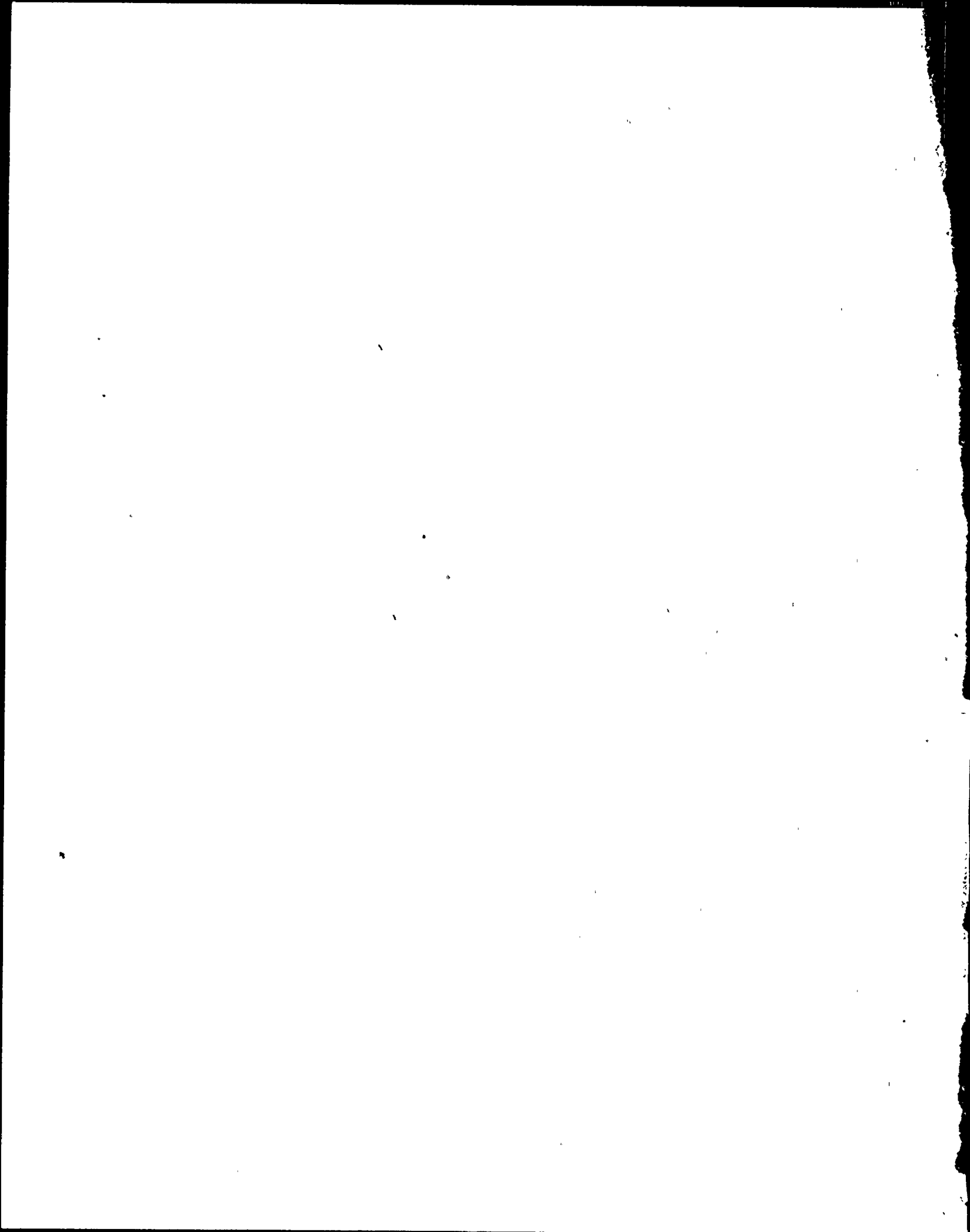
PCN75-118 - temporary for review only. Requested to remove section 3.15 for period of printer maintenance.

The Committee recommended approval of PCN75-38.

PORC ITEM
NUMBER:

DESCRIPTION

- 75-54 PT-11, 60 Cell Battery Banks "A" & "B", PCN75-125. J. Straight presented this procedure change. It requested to;
Add under step 6.1.2; If battery is placed on equalizing charge use M-38.
The Committee recommended approval of this change.
- 75-63 T-27.1, 1A Emergency Diesel Generator Pre-Start Alignment, PCN75-133.
T-27.2, 1B Emergency Diesel Generator Pre-Start Alignment, PCN75-132.
J. Straight presented this procedure change. Changes were made to alignment check-off to provide better definition of switches which are to be checked. The Committee recommended approval of these changes.
- 75-64 PT6.3A, Calibration Procedure, Power Range Channels, PCN75-134.
J. Straight presented this procedure change. It was requested this procedure be deleted as CP-41 replaces it. The Committee recommended approval of this deletion.
- 75-65 PT6.4, Excure/Incore Recalibration, PCN75-138. J. Straight presented this procedure change. It requested;
Add new step 1 to read "The following steps may be marked N.A. if Flux Maps are generated by other procedure, test of operation, shift foreman signatures are not required. Nuclear Engineer of alternate must note flux map number used".
The Committee recommended approval of this change.
- 75-70. GS-31.0, Door Alarm System, PCN75-143. J. Straight presented this temporary change request. It was requested to remove section 3.15 for period of printer maintenance. The Committee reviewed this temporary change.
- 75-71 J. Straight presented the following new procedures;
RF-8.4, Fuel & Core Component Movement In The Spent Fuel Pit, PCN75-131. This procedure describes the steps necessary to move fuel and/or core components in the spent fuel pit.
RF-14.1.1, Thimble Retraction Procedure For Plant Refueling, PCN75-72. This new procedure describes the steps to be taken in Flux Thimble Retraction for refueling.
RF-14.1.2, Thimble Insertion Procedure For Plant Refueling, PCN75-71. This new procedure describes the steps necessary to insert the flux thimbles into the core.
The above new procedures were recommended for approval by the Committee.



PORC ITEM
NUMBER:

DESCRIPTION:

75-50

CP-41, 42, 43 & 44, Calibration &/or Maintenance Of Power Range N41, 42, 43 & 44, PCN75-50, 51, 52 & 53.
J. Straight presented these new procedures to replace PT-6.3A also to conform to A-30 format. The Committee recommended approval of these new procedures.

75-51

J. Straight presented the following CP procedures;

CP-405, Calibration &/Or Maintenance of Delta T (Channel 1), PCN75-12

CP-406, Calibration &/Or Maintenance of Delta T (Channel 2), PCN75-13

CP-407, Calibration &/Or Maintenance of Delta T (Channel 3), PCN75-14

CP-408, Calibration &/Or Maintenance of Delta T (Channel 4), PCN75-15

CP-405.10, Calibration &/Or Maintenance of Δ T SP-1, Module In Channel I, PCN75-30.

CP-406.10, Calibration &/Or Maintenance of Δ T SP-1, Channel-II, PCN75-32.

CP-407.10, Calibration &/Or Maintenance of Δ T SP-1, Channel III, PCN75-31.

CP-408.10, Calibration &/Or Maintenance of Δ T SP-1, Channel IV, PCN75-33.

The Committee recommended approval of these procedures.

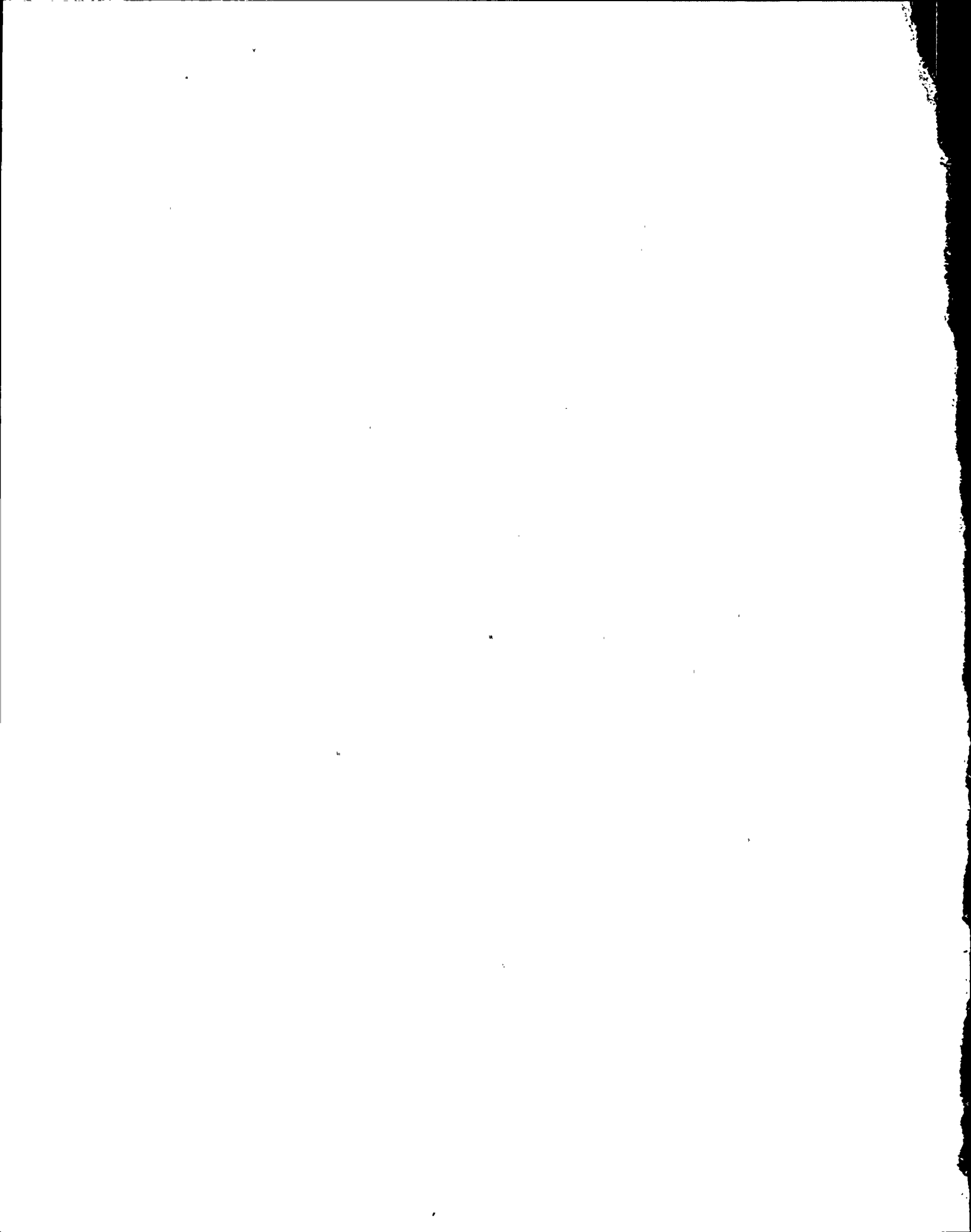
75-52

PT-6.3A, Calibration Procedure Power Range Channels, PCN75-123.
J. Straight presented this temporary change for review only. It requested steps 14 through 17.6 to be marked N.A. The Committee reviewed this change.

75-53

M-38, Equalizing Charge Station Battery Systems, PCN75-124.
J. Straight presented this procedure change. It requested a new step 4.10 be added to read;

4.10- Take complete set of voltage and specific gravity readings of all cells and temperature readings of designated pilot cells. One week after the battery has received an equalizing charge Date _____



PORC ITEM
NUMBER:

DESCRIPTION

75-72

J. Straight presented the following procedures which were rewrites to conform to A-30 format;

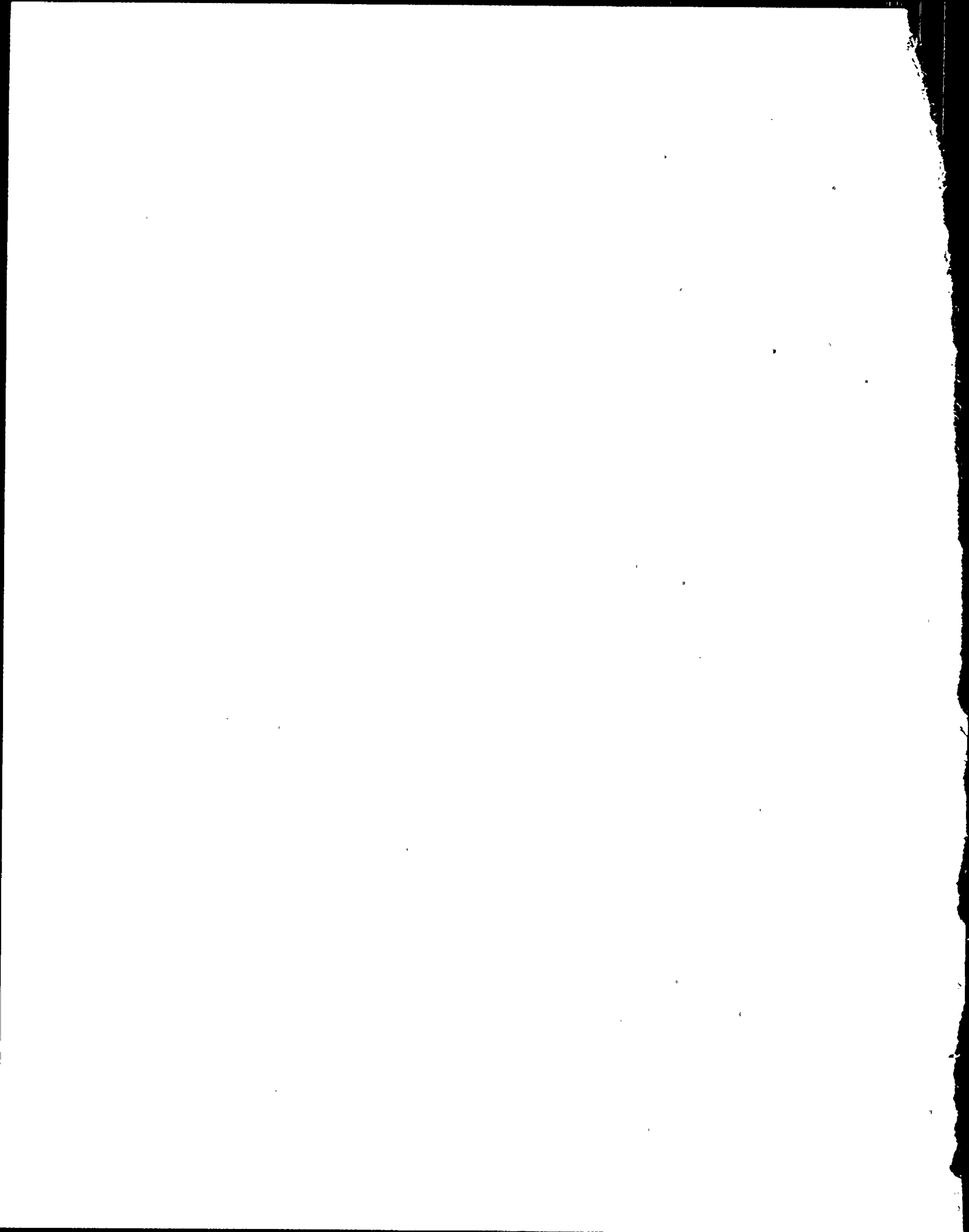
- RF-17, Drive Rod Bow Measurement Procedure, PCN75-60.
- RF-25, Movement of Irradiated Flow Mixers With Rack Inserts, PCN75-58.
- RF-14.8, Procedure For Handling Cast DOT 55-14300, PCN75-64.
- RF-14.5, Fuel Handling During Inspection In The Spent Fuel Pit, PCN75-67.
- RF-9.16, Lower Core Support Structure Removal & Replacement, PCN75-79.
- RF-9.14, Irradiation Sample Handling Tool Operating Procedure, PCN75-81.
- RF-9.8.2, Part Length (P/L) Control Rod Drive Shaft Relatching, PCN75-90.
- RF-9.8.1, Part Length (P/L) Control Rod Drive Shaft Unlatching, PCN75-91.
- RF-4.0, Site Removal Of New Fuel Assemblies From Shipping Containers And Handling Of Shipping Containers, PCN75-107.
- RF-9.15, New Fuel Handling Tool Operating Procedure, PCN75-80.
- RF-2, Reactor Refueling Outage Operations and Activities, PCN75-114.
- RF-9.2, Fuel Transfer System, PCN75-97.
- RF-9.4, Thimble Plug Handling Tool, PCN75-95.
- RF-9.7, Burnable Poison Rod Assembly Handling Tool Operating Instruction, PCN75-92.
- RF-9.13, Core Map After Fuel Loading, PCN75-82.
- RF-8.3, Core Loading Periodic Equipment Status Check, PCN75-100.
- RF-8.1, Step by Step Fuel Loading Sequence and Maps, PCN75-102.
- RF-9.3, Spent Fuel Assembly Handling Tool, PCN75-90.
- RF-9.5, RCC Change Fixture, PCN75-92.
- RF-6.0, Handling Of Burnable Poison Rod Assemblies, PCN75-105.
- RF-8.0, Fuel Assembly & Core Component Movement Prerequisites & Precautions, PCN75-103.
- RF-10.0, Fuel Transfer System Checkout & Demonstration Procedure, PCN75-78.
- RF-9.12.2, Assembly Of Instrument Port Conoseals, PCN75-85.
- RF-9.6, Guide Tube Cover Handling Tool Operating Instructions, PCN75-93.
- RF-3.0, Fuel Assembly & RCC Replacement, PCN75-108.

The Committee recommended approval of these rewrites.

75-73

- RF-11.1, Dropping Of A New VO₂ Fuel Assembly Or Collision Of A New VO₂ Fuel Assembly With Another Object, PCN75-76.
- RF-11.2, Dropping Of A New PO₂ Fuel Assembly Or Collision Of A New PO₂ VO₂ Fuel Assembly With Another Object, PCN75-75.
- RF-11.3, Dropping Of A Spent Fuel Assembly Or Rod Control Cluster Or Collision Of A Spent Fuel Assembly Or Control Cluster With Another Object, PCN75-74.
- RF-11.4, Sticking Of A Fuel Assembly Inside The Reactor Vessel Or Inside The Transfer Basket, PCN75-73.

The Committee disapproved these change requests as it was the recommendation of the PORC that these procedures be written a E's.



PORC ITEM
NUMBER:

DESCRIPTION

75-74

RF-15, Step By Step Loading Used Burnable Poison Rods Into Used Fuel, PCN75-62.

RF-16, Step By Step Change In Location Of Radioactive Material In Spent Fuel Pit, PCN75-61.

J. Straight presented these procedure changes. It was the recommendation of the Committee that these two procedures be deleted as this information is incorporated in A-4.

The Committee recommended approval of these deletions.

75-77

T-25, Instrument And Control Bus, PCN75-135. J. Straight presented this procedure change. This change request was to incorporate steps from S-20. This item IS COMPLETE.

75-78

S-20, Instrument Busses, PCN75-136. J. Straight presented this procedure to the Committee for deletion. It was requested to delete this procedure as it is being incorporated in T-25.

75-80

M-37.16A, Inspection & Maintenance Of Manual Operated Grinnel Diaphragm Valves, PCN75-158, & PCN75-161.

J. Straight presented this procedure change.

PCN75-161 - requested to change second line of step 5.8 to read;

"tighten bonnet nuts (with torque wrench if possible)"

PCN75-158 - requested to change in 5.1,

"M-37.16 series to M-7.4"

The Committee recommended approval of this change.

75-82

PT-22.1, Equipment Hatch Door Seal Leakrate Test, PCN75-147.

J. Straight presented this procedure change. It was requested the following be added;

Equipment lock door seal leak rate test add;

6.3.1 - Turn outer hatch door hand wheel 1-3/4 turns open.

remove: note page PT-22.1:2

add: 6.26.1 - Resecure hatch door by turning handwheel 1-3/4 turns closed.

add: 6.35.1 - Turn outer hatch door handwheel 1-3/4 turns open.

add: 6.40.1 - Resecure hatch door by turning handwheel 1-3/4 turns closed

The Committee recommended approval of this change.

PORC ITEM
NUMBER:

DESCRIPTION

75-83

PT-22.2, Personnel Hatch, Door Seal Leakrate Test, PCN75-148.
J. Straight presented this procedure change. It was requested the following steps be added:

add: 6.3.1 - Turn outer hatch door handwheel 1-3/4 turns open.

drop: Note on page PT-22.2:2

add: 6.25.1 - Resecure hatch by turning handwheel 1-3/4 turns closed.

add: 6.39.1 - Resecure hatch by turning handwheel 1-3/4 turns closed.

The Committee recommended approval of this change.

75-84

M-45.1A, Inspection of Safe Guard Motor For _____, PCN75-172 & M-45.3, Inspection Of Non Safeguard Motor For _____, PCN75-171.

J. Straight presented this procedure change. It requested to move step 5.13 up to follow step 5.9.3. The Committee recommended approval of this change.

75-88

RF-8.4, Fuel And Core Component Movement In The Spent Fuel Pit, PCN75-174. J. Straight presented this temporary change to be made permanent. It was requested to change step 4.1 in precautions to step 3.10 in initial conditions. The Committee recommended approval of this change.

75-93

E-37.1, Dropping Of A New UO₂ Fuel Assembly Or Collision Of A New UO₂ Fuel Assembly With Another Object, PCN75-153.

E-37.2, Dropping Of A New PuO₂-UO₂ Fuel Assembly Or Collision Of A New PuO₂-UO₂ Fuel Assembly With Another Object, PCN75-154.

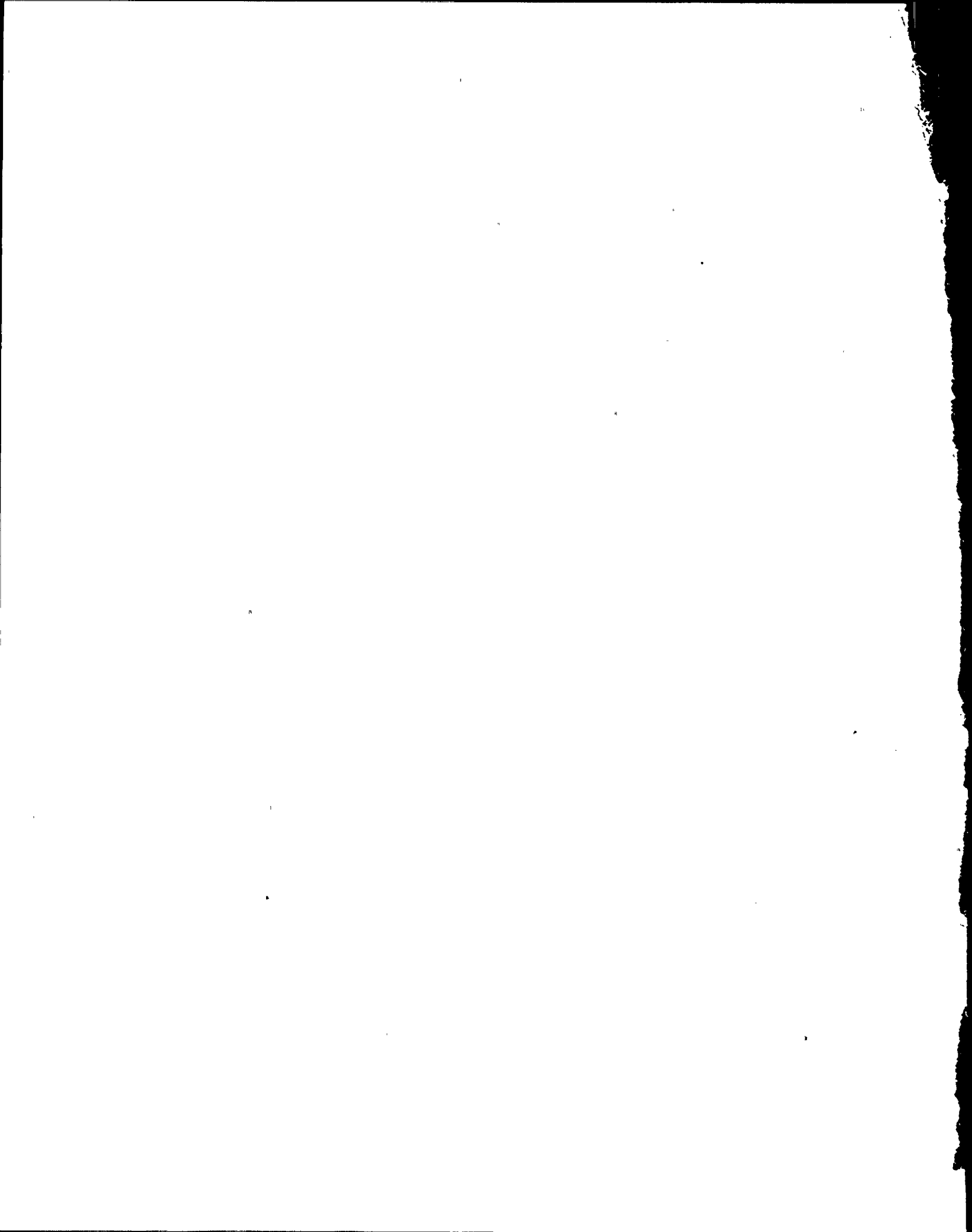
The Committee recommended disapproval of these procedures as they are not ones we need.

E-37.3, Dropping Of A Spent Fuel Assembly Or Rod Control Cluster Or Collision Of A Spent Fuel Assembly Or Rod Control Cluster With Another Object, PCN75-155.

E-37.4, Sticking Of A Fuel Assembly Inside The Reactor Vessel Or Inside The Transfer Basket, PCN75-156.

The Committee recommended approval of these new procedures.

PORC ITEM NUMBER:	DESCRIPTION
75-94	<p>J. Straight presented these procedures, which are rewrites to conform to A-30 format.</p> <p><u>RF-9.9, Control Rod Drive Shaft Unlatching Tool Operating Instruction, PCN75-89.</u></p> <p><u>RF-9.10, Head Lifting Rig Operating Instruction, PCN75-88.</u></p> <p><u>RF-9.11, Internals Lifting Rig Operating Instruction, PCN75-87.</u></p> <p><u>RF-9.12.1, Disassembly Of Instrument Port Conoseals, PCN75-86.</u></p> <p><u>RF-1, Refueling Organization, PCN75-115.</u></p> <p><u>RF-2A, Requirement For Reactor Head Removal, PCN75-113.</u></p> <p><u>RF-2B, Required Valve Lineup For Reactor Head Removal, PCN75-112.</u></p> <p><u>RF-2D, Filling Of The Refueling Canal, PCN75-111.</u></p> <p><u>RF-2E, Draining Of Refueling Canal, PCN75-110.</u></p> <p><u>RF-2F, Reactor Cavity Filtration System Operation, PCN75-109.</u></p> <p><u>RF-8.2, Fuel Handling Instruction Pre-Loading & Periodic Valve Test, PCN75-101.</u></p> <p><u>RF-14.6, Air Eductor Operating Procedure, PCN75-66.</u></p> <p><u>RF-5.0, Fuel Handling Instruction Limitations And Precautions For Handling New & Partially Spent Fuel Assemblies, PCN75-106.</u></p> <p><u>RF-9.0, Refueling Equipment Operating Instructions List, PCN75-99.</u></p> <p><u>RF-9.1, Manipulator Crane, PCN75-98.</u></p> <p><u>RF-9.12.3, Disassembly Of The Part Length Control Rod Conoseals, PCN75-84.</u></p> <p><u>RF-9.12.4, Assembly Of The Part Length Control Rod Conoseals, PCN75-83.</u></p> <p><u>RF-10.1, Auxiliary Building Crane Interlock Checkout, PCN75-77.</u></p> <p>The Committee recommended approval of these rewrites.</p>
75-98	<p><u>PT-22.10, Mechanical Manifold "C" Leakrate Test, PCN75-182.</u></p> <p>J. Straight presented this procedure change. It was requested step 4.3 be deleted, step 6.4 read, "approx. 60 minutes", and step 6.11 read;</p> <p style="padding-left: 40px;">"record ambient temperature at penetration and time"</p>
75-100	<p><u>S-2.3, Pressurizer Relief Tank Control, PCN75-184.</u> J. Straight presented this procedure change. This was a rewrite to conform to A-30 format. The Committee recommended approval of this change.</p>



PORC ITEM
NUMBER:

- 75-102. GS-31, Door Alarm System, PCN75-157, PCN75-195.
J. Straight presented these procedure changes.
PCN75-157 - 3.16.1 - Door 46 may be deactivated from 0200 to 2400 hours Monday through Friday and at any other time when the area is considered manned. Under these conditions the requirements of 3.10 and 3.10.1 do not apply.
- PCN75-195 - change 3.14 - the shift foreman or head control operator shall initial all printer entries for identified door alarms and defeated doors indicating he is aware of the reason for the activity. Unidentified door alarms shall be handled as outlined in 3.11 thru 3.13.
- change 3.14.1 - eliminate words "alarm logs"
eliminate door alarm system - daily log sample

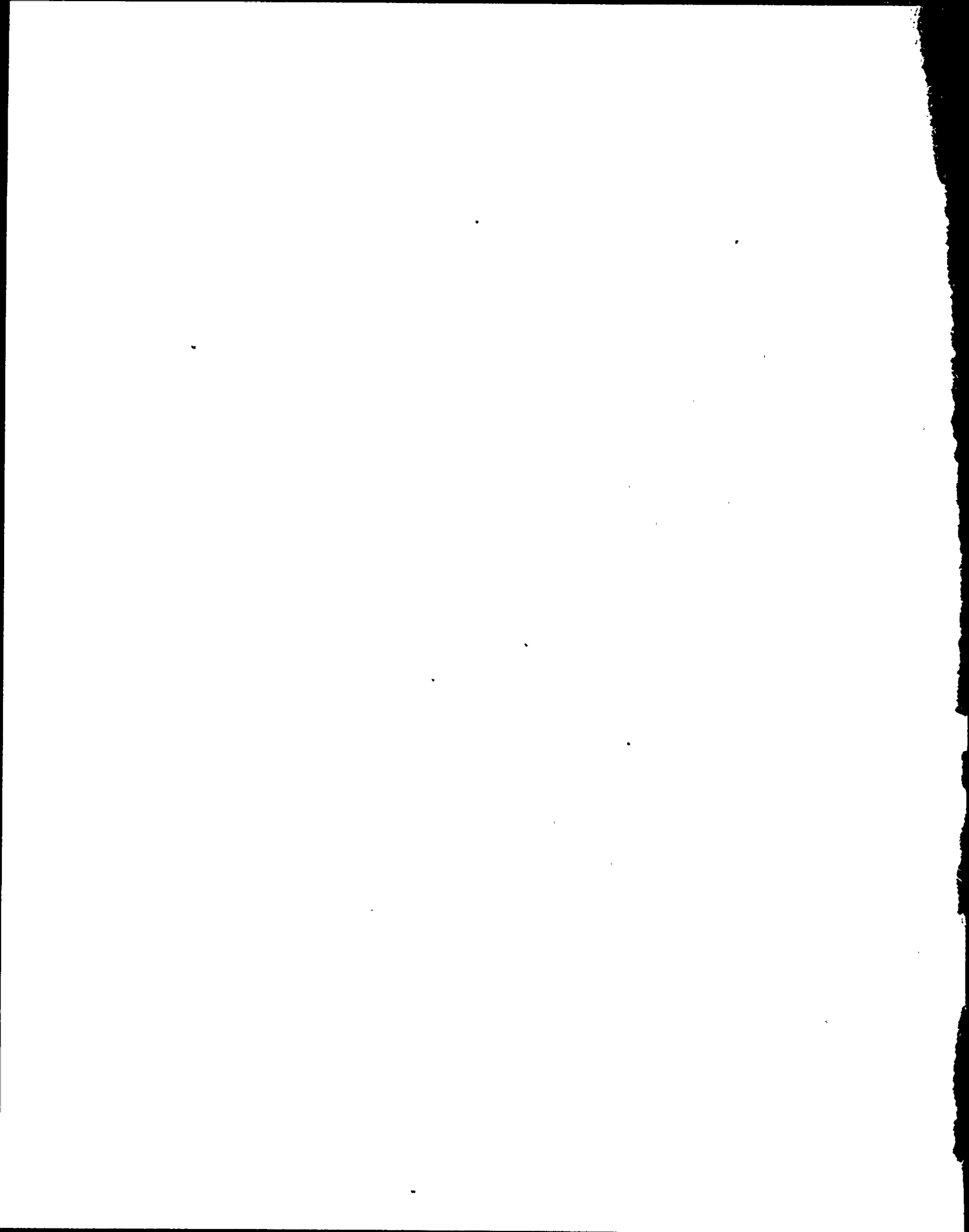
The Committee recommended approval of this change.

- 75-107 M-40, Surveillance & Maintenance Of Hydraulic Snubbers, PCN75-196.
J. Straight presented this procedure change. It was requested snubbers #MS-179 and MS-180 be added to the list. The Committee recommended approval of this change. This item IS COMPLETE.

- 75-109 RSSP-12, Testing Of Primary & Secondary Relief Valves On Test Stand, PCN75-179. J. Straight presented this procedure change. It was requested;
Add to precautions 5.2 - test medium should be air for a valve which is used on gas or vapor service and air or water for one used on liquid service. Also on page five after R.V. 1818-RG&E should be 1100 psi.

The Committee recommended approval of this change.

- 75-111 J. Straight presented these two new procedures;
M-60.1, Reactor Trip Undervoltage Protection Bus 11A, PCN75-193.
M-60.2, Reactor Trip Undervoltage Protection Bus 11B, PCN75-194.
- These procedures outline the steps for removing and replacing protective relays 273/11B and 274/11B for calibration or corrective maintenance. The Committee recommended approval of these procedures.



PORC ITEM
NUMBER:DESCRIPTION

75-115 1C Safety Injection Pump Circuit Breaker Maintenance & Inspection, PCN75-230. B. Snow presented this procedure change. It was requested to add bus 14 where bus 16 appears. Also requested a note be added to read;

"This procedure is written for both breakers. If only one breaker is being inspected, other steps for other breaker maybe marked N.A."

The Committee recommended approval of this change.

75-119 O-6, Operation & Process Monitoring, PCN 75-218. J. Straight presented this procedure change. It was requested to change step 5.10 to read;

The computer is programmed to monitor conformance of flux difference band operating limits in the Technical Specifications. One program alarms to indicate non conformance with the +5% flux difference band around the target valve for operation at power levels greater than 90% of rated power. The other program alarms to indicate non conformance with the limit on time (1 hr. in 24) that the 5% flux difference band may be exceeded for operation at or below 90% of rated power. If these programs are inoperable, the flux difference shall be logged hourly for the first 24 hrs. and half-hourly thereafter.

The Committee recommended approval of this change.

75-122 RSSP-10.1, Conditions For Main Steam Safety Valve Test, PCN 75-209. J. Straight presented this procedure change. It was requested to delete steps 4.5 & 4.7, also to add at end of step 6.1 "or by condition steam damp". The Committee recommended approval of this change.

PORC ITEM
NUMBER:

DESCRIPTION

75-123

John Straight presented the following new RF procedures:

RF-42, Television Visual Fuel Assembly Inspection PCN 75-211. This procedure is to outline the scope and the examination instructions for fuel inspection program. This will consist of television visual examination of selected fuel assemblies and will be performed in the spent fuel pit.

RF-43, Fuel Rod Length Measurements, PCN 75-213. This procedure describes the steps necessary to perform fuel rod length measurements of irradiated fuel.

RF-44, Fuel Rod Diameter Measurement, PCN 75-214. This procedure describes the steps necessary to perform fuel rod diameter measurements of irradiated fuel.

RF-45, Fuel Assembly Bow Measurement, PCN 75-215. This procedure describes the steps necessary to perform fuel assembly bow measurements of irradiated fuel.

The Committee recommended approval of these procedures.

75-124

SC-1, Radiation Emergency Plan, PCN 75-139. J. Straight presented this procedure change. It was requested that on appendix A, E, and K to omit note at bottom of page. "The general office will update this call list on a quarterly basis." The Committee recommended approval of this change.

75-125

PT-17.2, Process Radiation Monitors R-11 - R-20, Iodine Monitors R10A and R10B, PCN 75-226. J. Straight presented this procedure change. It requested step 6.5.13 should read as follows:

- operating vent valve 1581.

Step 6.5.19

change 4.5" of water to 5.0" of water.

Step 6.6.14

Change 4.5 " of water to 5.0" of water.

The Committee recommended approval of this change.

75-132

T-36.1, Station Service Water Header Valve Alignment for Two Loop Operation, PCN 75-206. J. Straight presented this new procedure. This procedure is to provide procedural coverage, to describe the valve alignment necessary for two loop operation. The Committee recommended approval of this procedure.

PORC ITEM
NUMBER:

DESCRIPTION

75-138

M-43.6, Installation of Cover On Permanent Ring On Steam Generator Loop Nozzle, PCN 75-241, PCN 75-233. J. Straight presented this procedure change.

PCN 75-241 - was to facilitate step 5.10.5 marked N.A.

PCN 75-233 - new step 3.9 to read: In the event that refueling will not be conducted concurrent with S/G work, the following steps may be marked NA - 3.10, 3.11, 5.1, 5.4, 5.6, 5.8, 5.9.

Renumber present steps 3.9 and 3.10 to 3.10 and 3.11.

Step 5.7 - delete words "all hold down"

- delete 20

- add sentence: All studs must be used if refueling is to be conducted concurrent with S/G work, otherwise a minimum number of (4) will be sufficient.

The Committee recommended approval of this change.

75-139

M-43.13, Removal of Steam Generator Permanent Ring Loop Nozzle Cover, PCN 75-242. J. Straight presented this procedure change. It was requested;

Step 5.2 - delete number 20

Step 5.4 - insert after bars, if used, and pass outside.

Remove steps 5.7 and 5.8

Step 5.10 - add "if used"

Step 5.11 - add "if used"

The Committee recommended approval of this change.

75-140

M-43.25, Installation of Steam Generator Handhold Cover, PCN 75-244.

M-43.26, Installation of Steam Generator Secondary Manway Cover, PCN 75-243.

J. Straight presented this procedure change for both above procedures. A figure sheet has been added to these two procedures. The Committee recommended approval of this change.

75-141

EM-75, 1C Safety Injection Pump Circuit Breaker Maintenance & Inspection, PCN 75-240. J. Straight presented this temporary change for review. This PCN was to facilitate the N.A. of step 5.3. The Committee reviewed this temporary change.

PORC ITEM NUMBER:	DESCRIPTION
75-142	<p><u>P-9, Precautions, Limitations & Setpoints P-9 Radiation Monitoring System, PCN 75-762, PCN 75-245.</u> J. Straight presented these procedure changes.</p> <p>PCN 75-762 requested; step 2.3.2.2 regarding R-11 setting with Containment Isolated; change to read: "7X10⁵ CPM is Sufficient For Operations,"</p> <p>Also change R-11 setpoint on page P-9:6 for temporary review only.</p> <p>PCN 75-245 requested: step 2.3.2.2 regarding R-11 setting with Containment Isolated; change to read: "5X10⁵ CPM is sufficient For Operations,"</p> <p>Also change R-11 setpoint on page P-9:6 to be made permanent.</p> <p>The Committee recommended approval of PCN 75-245 and reviewed PCN 75-762.</p>
75-143	<p><u>S-4.1A, Liquid Waste Process Startup, PCN 75-231.</u> J. Straight presented this procedure change. It was requested to delete step 7.8 also to rewrite into A-30 format. The Committee recommended approval of this change.</p>
75-144	<p><u>S-4.1C, Liquid Waste Process Operation and Shutdown, PCN 75-232.</u> J. Straight presented this procedure change. It was requested to delete last sentence in step 4.2, add a new step 4.10 to read: "When boron approaches 10%, verify drums ready per S-4.1D thru steps 4.8 _____."</p> <p>Also a note to be added to read:</p> <p>NOTE: If evaporator is to be stopped due to lack of chemistry coverage or full waste condensate tanks, put on reject.</p> <p>The Committee recommended approval of this change.</p>
75-145	<p><u>GS-31, Door Alarm System, PCN 75-228, PCN 75-246.</u> J. Straight presented these procedure changes.</p> <p>PCN 75-228 - request to change data sheet to remove door alarm number 49.</p> <p>PCN 75-246 requested; 3.16 alarms on doors 7 through 14, 19, 20, 22, 24, and 25 . . .</p>

PORC ITEM
NUMBER:DESCRIPTION

75-148

J. Straight presented the following CP's for changes:

CP-41.0	PCN 75-248
CP-41.1	PCN 75-249
CP-41.2	PCN 75-250
CP-41.3	PCN 75-251
CP-41.4	PCN 75-252

The procedures were put in check-off form, also consolidated the components into one larger procedure. The Committee recommended approval of these changes.

75-150

RF-8.4, Fuel & Core Component Movement In The Spent Fuel Pit, PCN75-261. J Straight presented this procedure change. It requested a new shuffle sheet be added. The Committee recommended approval of this change.

75-151

SM75-3.2, Steam Relief Vent Pipe Support Steel & Concrete, PCN75-229. J. Straight presented this temporary procedure change for review.

Pg. 2 - Delete Jan. 18, replaced with Jan. 22
Pg. 7 - Delete E-9, except for final backup ---
Pg. 8 - Delete E-13, --- the final boltup of steps
has been completed ---

E-13 --- add removed and the three ---

The Committee reviewed this change.

75-152

T-18B, Turbine Main Steam Stop Valves Test, PCN75-257. J. Straight presented this procedure change. This PCN was to facilitate steps that were marked N.A. The Committee reviewed this change.

75-153

E-38, Loss Of Service Water, PCN75-205. J. Straight presented this new procedure. This procedure describe operator actions to be taken in the event of loss of service water. The Committee recommended approval of this procedure.

75-154

PT-9, Undervoltage & Underfrequency Protection 11A & 11B, 4160 Volt Busses, PCN75-258. J. Straight presented this procedure change. This request was to add to existing steps of procedure as follows:

6.1.1 and 6.2.1 - (knife switch #2)
6.1.2, 6.2.2, 6.3.1, 6.3.7, 6.4.1, and 6.4.7
(knife switch #8)

The Committee recommended approval of this change.

PORC ITEM
NUMBER:DESCRIPTION

75-155

PT-12.1, & PT-12.2, Emergency Diesel Generator on 1A and 1B, PCN75-259, PCN75-260. J. Straight presented these two procedures for the same change. It was requested;

Add note after step 6.2 as follows:

NOTE: If this procedure is being used to perform post-maintenance reliability check, mark steps 6.2.1 through 6.2.7 N.A. and continue with step 6.3.

The Committee recommended approval of this change.

75-156

RSSP-11, Pressurizer Safety Valve Test, PCN75-265. J. Straight presented this procedure change. It requested a new step 3.4 be added to read;

3.4 - Crosby Nozzle Relief Valves, Inst. Book No. 20
Installation, Maintenance and Adjustment

Also several minor changes were requested. The Committee recommended approval of this change.

75-157

RSSP-12, Testing of Primary & Secondary Relief Valves On Test Stand, PCN75-264. J. Straight presented this procedure change. It requested the following be added at the end of step 6.2;

Flush water through stand until water at drain valve is clear

75-164

Technical Specification Change #20. Heatup and Cooldown limits specifications 3.1.2, 3.1.3, 4.2, plus table changes 3.1-1, 3.1-2.

Bob Eliaz presented this proposed Technical Specification change to the Committee for review. There were no unreviewed safety questions and the Committee recommended approval of these Technical Specification changes.

75-172

M-60.3, Reactor Trip Underfrequency Protection Bus 11A, PCN - 75-280. J. Straight presented this new procedure pertaining to calibration or corrective maintenance on Bus 11A. The Committee recommended approval of this procedure.

PORC ITEM
NUMBER:DESCRIPTION

75-173

M-60.4, Reactor Trip Underfrequency Protection Bus 11B, PCN-75-281.
J. Straight presented this new procedure pertaining to calibration or corrective maintenance on Bus 11B. The Committee recommended approval of the procedure.

This item IS COMPLETE.

75-174

E-4, Station Blackout Operation, PCN-75-321. J. Straight presented this procedure change as follows:

Step 4.1.2.2 - Insert the word "starts" at end of sentence.
Step 4.1.2.4 - Insert the word "starts" at end of sentence.

ADD: New step 4.2.1 and change numerical sequence thereafter.

Step 5.8 changed to read: Maintain reactor in a safe shutdown state by referring to operating instruction, 0-3, "Plant At Hot Shutdown with Xenon Present".

Also add new Step 5.10.

The Committee recommended approval of this procedure change.

75-175

T-35C, Auxiliary and Intermediate Building Ventilation System Operation with the 1A Auxiliary Building Supply Air Handling Unit, PCN-75-270. J. Straight presented this procedure change.

Step 5.5.3 - change monometer reading to .36 from .2.

The Committee recommended approval of this change.

75-176

EM-75, 1C Safety Injection Pump Circuit Breaker Maintenance and Inspection, PCN-75-322. J. Straight presented this procedure change. It was requested that Step 5.1 to 5.1.4 be changed to provide procedural coverage for further investigation into breaker problem.

The Committee recommended approval of this procedure change.

PORC ITEM
NUMBER: _____

DESCRIPTION

75-177

J. Straight presented the following CP procedures:

CP-42.0, Calibration and/or Maintenance of Power Range N-42, PCN-75-298.

CP-42.1, General Defeat Procedure for N-42, PCN-75-299.

CP-42.2, General Reinstate Procedure for N-42, PCN-75-300.

CP-42.3, Calibration of Power Range N-42, PCN-75-301.

CP-42.4, Calibration of Power Range N-42 Bistable Relay
PCN-75-302.

CP-43.0, Calibration and/or Maintenance of Power Range N-43, PCN
75-303.

CP-43.1, General Defeat Procedure for N-43, PCN-75-304.

CP-43.2, General Reinstate Procedure for N-43, PCN-75-305.

CP-43.3, Calibration of Power Range N-43, PCN-75-306.

CP-43.4, Calibration of Power Range N-43 Bistable Relay Drivers,
PCN-75-307.

CP-44.0, Calibration and/or Maintenance of Power Range N-44,
PCN-75-308.

CP-44.1, General Defeat Procedure for N-44, PCN-75-309.

CP-44.2, General Reinstate Procedure for N-44, PCN-75-310.

75-181

GS-31.0; Door Alarm System, PCN-75-330 (also 228, 246, 330)

J. Straight presented further changes to this procedure (in the interim Rev. #6 never issued due to additional PCN's) (75-320 and 75-330).

The Committee recommended approval of this change.

75-182

A-52.2, Control of Locked Valve Operation, PCN-75-263.

J. Straight presented this new procedure. The purpose of this procedure is to describe the requirements for a locked valve.

The Committee recommended approval of this procedure.

PORC ITEM
NUMBER:DESCRIPTION

75-207

S-23.2.2, Containment Purge Procedures, PCN-75-284.

J. Straight presented this procedure change. It requested in step 5.2 to change the following;

1. Open PEVO purge exhaust valve outlet to open PEVO purge exhaust valve outside V-5879.
2. Open PEVI purge exhaust valve inlet to open PEVI purge exhaust valve inside V-5878.
3. Open PSVO purge supply valve outlet to open PSVO purge supply valve outside V-5869.
4. Open PSVI purge supply valve inlet to open PSVI purge supply.

The Committee recommended approval of these changes.

75-208

T-27.1, 1A Emergency Diesel Generator Pre-Start Alignment, PCN-75-272.

J. Straight presented this procedure change. It requested a valve number change. The Committee recommended approval of this change.

75-209

A-2.8, Special Nuclear Material Inventory & Record Requirements, PCN-75-273. J. Straight presented this procedure change. This requested;

Attachment A Item 1 changed to read: "Filled out and signed by Quality Control Inspector Level II."

The Committee recommended approval of this change.

75-213

PT-17.2, Process Radiation Monitors R-11 - R-20, Iodine Monitors R-10A and R-10B, PCN-75-331. J. Straight presented this procedure change. This change requested steps 6.3.12, 6.4.12, 6.5.11, 6.6.9 be added.

The Committee recommended approval of this change.

PORC ITEM
NUMBER:DESCRIPTION

- 75-215 PT-23.37, Reactor Compartment Cooling Unit "A", PCN-75-278.
J. Straight presented this new procedure. The purpose of this procedure is to test leak tightness of Reactor Compartment Cooling "Unit A".
- The Committee recommended approval of this procedure.
- 75-216 RSSP-2.2, Diesel Generator Load and Safeguard Sequence Test, PCN-75-359. J. Straight presented this procedure change. It requested new steps 2.4 thru 2.7 be added, also to delete the first line in the NOTE.
- The Committee recommended approval of this change.
- 75-217 M-17, Isolation and Relocation of Reactor Makeup Water Piping, PCN-75-328. J. Straight presented this new procedure. The purpose of this procedure is to describe the steps necessary to isolate reactor makeup water piping for maintenance.
- The Committee recommended approval of this change.
- 75-218 M-48.5, Isolation of Bus 17, PCN75-429. J. Straight presented this new procedure. The purpose of this procedure is to describe steps necessary to isolate Bus 17 for cleaning and breaker maintenance including a list of work performed. The Committee recommended approval of this procedure.
- 75-219 S-3.3D, CVCS Cation Demineralizer Bed Operation (Using The A Deborating Unit), PCN75-403. J. Straight presented this procedure change. This change requested that valve numbers be changed, as valves associated with 1A deborating demineralizer were incorrectly assigned to be 1B demineralizer. The Committee recommended approval of this change.
- 75-220 S-25.4, Steam Generator: Draining The A and/or B Steam Generator To Blowdown Tank, PCN75-411. B. Snow presented this new procedure. The purpose of this procedure is to describe operations necessary to drain 1A or 1B steam generator to the steam generator blowdown tank and transfer to the blowdown tank water to the monitor tank via waste condensate polishing D. I.'s. The Committee recommended approval of this procedure.
- 75-221 S-12.1, Leakage Detection Systems Pre-Startup, PCN75-316.
J. Straight presented this new procedure. This procedure provides the necessary conditions to ensure proper leakage detection systems necessary for operation are operable. The Committee recommended approval of this procedure.

PORC ITEM
NUMBER:

DESCRIPTION

- 75-222 RSSP-2.1, Safety Injection Functional Test, PCN75-400. J. Straight presented this procedure change. This was a rewrite to A-30 format with a new step 4.8 added to read:

"all prealignment sheets have been completed"

The Committee recommended approval of this change.
- 75-223 PT-23.38, Reactor Compartment Cooling Unit "B", PCN75-279. J. Straight presented this new procedure. The purpose of this procedure is to test the leak tightness of reactor compartment heat exchanger and manual isolation valves 4635 & 4636. The Committee recommended approval of this procedure.
- 75-224 RF-8.4, Fuel & Core Component Movement In The Spent Fuel Pit, PCN75-405. J. Straight presented this procedure change. It requested the Fuel Handling Data Sheet have some figures added. The Committee recommended approval of this change.
- 75-225 RF-8.1, Step By Step Fuel Loading Sequence And Maps, PCN75-406. J. Straight presented this procedure change. This requested a new schedule be added using step by step fuel load sequence and maps. The Committee recommended approval of this change.
- 75-227 M-43.8, Eddy Current Inspection Of Steam Generator Tubes, PCN75-441. J. Straight presented this procedure change. It requested removing present attachment A and replacing it with a new attachment A (W MRS 2.4.2 GEN-9, Eddy Current Inspection Of Steam Generator Tubes). A new attachment A will be added to this procedure. (W MRS 2.4.2 GEN-5, Positive Identification of Steam Generator Tubes By Eddy Current Inspection).
- 75-230 M-37.34, Maintenance Of Main Steam Power Operated Relief Valves, 3410 or 3411, PCN75-388. J. Straight presented this new procedure. This procedure describes the steps necessary to isolate, inspect and repair power operated relief valve 3410 or 3411. The Committee recommended approval of this procedure.
- 75-233 SM 75-1.1, Instrument Relocation-Intermediate Bldg, PCN 75-336. J. Straight presented this temporary procedure change. It requested:

Step E-6 and E-7 to be done after E-1 and before E-2.

The Committee reviewed this temporary change.

PORC ITEM
NUMBER:

DESCRIPTION

75-234

M-45.4, Inspection of Safeguard Valve Motors, PCN 75-367.
J. Straight presented this new procedure. The purpose of this procedure is to describe the instructions necessary for removal of a safeguard valve motor from service and for performing inspections and repairs.

The Committee recommended approval of this new procedure.

75-235

A-46, Bypass of Safety Function On Jumper Control, PCN 75-408.
J. Straight presented this procedure change. It requested to;

Add to step 3 2.3: At cold shutdown, the PORC review requirement will not be necessary for those engineered safety features at reactor control and protection systems whose functions are not required.

The Committee recommended approval of this change.

75-236

S-25.4, Steam Generators: Draining A and /or B Steam Generator to Blowdown Tank, PCN 75-437. J. Straight presented this procedure change. This change was to correct mislabeled value numbers.

The Committee recommended approval of this change.

75-237

SM 75-3.3, Revisions To Reheater Relief Vent Pipe, PCN 75-402.
J. Straight presented this temporary change to be made permanent. It requested the following changes:

Change step E-3 to read as follows: The plant is in cold shutdown condition and EM valve 6963, main steam STOP valves bypass valves 3545A, 3544A have been tagged as "marked up" by cognizant RG&E engineer.

Change step E-4 to read as follows: The undersigned has verified that valves 6963, 3545A, 3544A have been tagged as "marked up" by the RG&E Engineer.

75-238

M-11.10, Major Inspection of Service Water Pumps, PCN 75-428.
J. Straight presented this procedure change. It requested;

Add step after 5.3: Measure distance from top of shaft nut to shaft, loosen nut and drop shaft until in bottom, then take measurement. Record this total shaft drop.

Add step after 5.32: Raise shaft .125" off bottom.

The Committee recommended approval of this change.

PORC ITEM
NUMBER:

DESCRIPTION

75-240

O-2.2, Plant Shutdown From Hot Shutdown, PCN 75-401.

J. Straight presented this procedure change. It requested the following changes:

Hydrogen Peroxide should be added to the primary system while there is still a RCP in service and after the system has cooled to 200°F. Enough H₂O₂ should be added to obtain a positive residual as determined by the chem. lab.

also a note to be added;

NOTE: (to be inserted after 5.25)

If this cooldown is for a refueling outage, add Hydrogen Peroxide and maintain at least one Reactor coolant pump running until proper residual is determined by the Chemical Lab.

5.25.1 Secured the running reactor coolant pump at approximately 150°F.

The Committee recommended approval of these changes.

75-241

PT-23.18A, Containment Isolation Valve Leak Rate Testing "A" Containment Spray Header, PCN 75-424, and PT 23.18B, Containment Isolation Valve Leak Rate Testing "B" Containment Spray Header, PCN 75-430. J. Straight presented this procedure change. Both PCN's requested;

75-430

Step 6.8 Delete words "close valve 869B" insert the following words "Close test valve 2859 at conn. B1

75-424

Step 6.8 Delete words "Close valve 869A insert in its place the following "Close test valve at valve #2857 conn. B1 etc.

The Committee recommended approval of this change.

75-244

PT-27, Containment Tendon Stressing, PCN 75-407. J. Straight presented this new procedure. This procedure is to provide steps necessary for performing the Containment Vessel (CV) Tendon Lift Off Test.

The Committee recommended approval of this new procedure.

75-245

P-1, Precautions, Limitations, and Setpoints P-1, Reactor Control and Protection System, PCN 75-542.

J. Straight presented this procedure change. There were several changes to conform to a change in the Reactor Coolant System pressure setpoints. The Committee recommended approval of this change.

PORC ITEM
NUMBER:

DESCRIPTION

75-246

J. Straight presented the following CP's for the following change;

It was requested to increase primary loop pressure from 1985 psi to 2235 psig per Westinghouse functional analysis II.

CP-10.0, Calibration and/or Maintenance of Delta Flux Controllers, PCN 75-578.

CP-20.0, Calibration and/or Maintenance of Delta Flux Controllers, PCN 75-579.

CP-30.0, Calibration and/or Maintenance of Delta Flux Controllers, Channel 3, PCN 75-580.

CP-40.0, Calibration and/or Maintenance of Delta Flux Controllers, PCN 75-581.

CP-405.10, Calibration and/or Maintenance of TSPI Channel 1, PCN 75-544. The Committee recommended approval of these changes. This item IS COMPLETE.

75-247

The following CP procedure changes were presented by J. Straight.

CP-429.0, Calibration and/or Maintenance of Pressurizer Pressure Channel "429"; PCN 75-545, requested an update on the latest P1 change.

CP-429.6, Pressurizer Pressure Bistable PC-429A, PCN 75-546,

&

CP-429.7, Pressurizer Pressure Bistable PC-429B, PCN 75-547.

Both PCN's requested to change setpoint 2185 psig = 44.25 MA.

75-248

J. Straight presented all the following CP's for a change in the setpoints, per P I, Rev. XII.

CP-429.8, Pressurizer Pressure Bistable PC-429 C/D, PCN 75-548.

CP-429.9, Pressurizer Pressurizer Bistable PC-429E, PCN 75-549.

CP-430, Calibration and/or Maintenance 430 Pressurizer Pressure Channel, PCN 75-550.

PORC ITEM
NUMBER:DESCRIPTION75-248
(Continued)

- CP-430.6, Pressurizer Pressure Bistable, PC-430A, PCN-75-551
- CP-430.7, Pressurizer Pressure Bistable PC-430 E/F, PCN 75-552.
- CP-430.8, Pressurizer Pressure Bistable PC-430H, PCN 75-553
- CP-431.0, Calibration and/or Maintenance of Pressurizer Pressure Channel "431", PCN 75-554.
- CP-431.6, Pressurizer Pressure Bistable PC-431A, PCN 75-555
- CP-431.7, Pressurizer Pressure Bistable PC-431, PCN 75-556
- CP-431.8, Pressurizer Pressure Bistable PC-431J, PCN 75-557
- CP-449.0, Calibration and/or Maintenance of Pressurizer Pressure Channel "449", PCN 75-558
- CP-449.6, Pressurizer Pressure Bistable PC-499A, PCN 75-559
- CP-499.7, Pressurizer Pressure Bistable PC-449B, PCN 75-560
- CP-407.10, Calibration and/or Maintenance of Δ TSP1 Channel 3, PCN 75-561
- CP-408.10, Calibration and/or Maintenance of Δ TSP1 Channel 4, PCN 75-562
- CP-408.20.0, Calibration and/or Maintenance of Δ TSP2 Channel 4, PCN 75-563
- CP-406.20.0, Calibration and/or Maintenance of Δ TSP2 Channel 2, PCN 75-565
- CP-406.10, Calibration and/or Maintenance of Δ TSP1 Channel 2, PCN 75-566
- CP-405.20.0, Calibration and/or Maintenance of Δ TSP2 Channel 1, PCN 75-567
- CP-404.6, T Avg Bistable TC - 404 A, PCN 75-568
- CP-403.0, Calibration and/or Maintenance of T Avg. Channel "403" PCN 75-569
- CP-402.6, T Avg. Bistable TC - 402A, PCN 75-570
- CP-402.0, Calibration and/or Maintenance of T Avg Channel "402", PCN 75-571
- CP-404.0, Calibration and/or Maintenance of T Avg. Channel "404", PCN 75-572.

The Committee recommended approval of these changes.

PORC ITEM
NUMBER:

DESCRIPTION

75-249

CP-401.6, T Avg. Bistable TC- 401 A, PCN 75-573

J. Straight presented the following procedure change. It requested to change data sheet to read;

High Tavg 578°F = 30.26 MA

The Committee recommended approval of this change.

J. Straight presented the following changes;

75-250

CP-401.0, Calibration and/or Maintenance of T avg Channel "401", PCN 75-574, requested the following changes;

High Tavg Bistable Charge as shown on attached data sheets circled in red.

Od Setpoint 571°F = 26.52 MA

New Setpoint 578°F = 30.26 MA

Change per Revision P1 dated March 17, 1975. Rev. 3, Change 10. (Westinghouse Functional Analysis II) also,

CP-403.6, T Avg Bistable TC-403A, PCN 75-575

Change procedure per attached sheet

High Tavg 578°F = 30.26 MA

The Committee recommended approval of this change.

75-253

O-2.4, Plant Shutdown From Hot Shutdown to Cold Shutdown During Blackout, PCN 75-377. J. Straight presented this procedure change. It requested a new precaution be added, step 4.14 to read as follows;

When the RHR system is in operation and the RCS temperature is greater than 200°F a maximum of 2 charging pumps are to be operated to prevent over pressurizing the RHB system. Below 200°F the operation of 3 charging pumps is all operable, but should be limited if possible.

Also, add new reference, step 2.3 as follows letter dated 2-25-75, procedure change to limit charging pump use at cold shutdown.

The Committee recommended approval of this change.

75-254

PT-23.24, Reactor Support Coolers (Inlet and Outlet), PCN 75-480. J. Straight presented this procedure change.

It was requested step 6.5 through 6.17 be modified for draining water from containment side of MOV 813 and 814 and to utilize high point vents at valve 2733 and 2734 to check PIV separately.

The Committee recommended approval of this change.

PORC ITEM
NUMBER:

DESCRIPTION

75-255

PT-7, Hydro Test of Reactor Coolant System, PCN 75-383.

J. Straight presented this procedure change. It requested under references a new step 3.3 be added to read;

3.3 Letter dated 2-25-75, procedure change to limit charging pump use at Cold Shutdown.

Also to add a new step 5.7 under precautions to read;

5.7 When the RHR system is in operation, and the RCS temperature is greater than 200°F, a maximum of two charging pumps are to be operated to prevent over-pressurizing the RHR system. Below 200°F, the operation of 3 charging pumps is allowable but should be avoided if possible.

PCN 75-415 requested the following additions;

Add to Primary Hydro Checklist:

- | | | |
|---------------------------------------|--------|-------------|
| 1) Pressurizer: | (psig) | 2335 (only) |
| e) Heater Connections | | _____ |
| 2) Reactor (delete Head) | | |
| f) Incore Penetrations (lower vessel) | | _____ |

Add new step 7;

M-40.3 completed for Spring Hangers and Supports in containment.

The Committee recommended approval of these changes.

75-256

S-23.2.1, Containment Purge System Pre-Startup Procedure, PCN 75-315. J. Straight presented this new procedure. This procedure describes the steps necessary to ensure the containment purge system is ready for operation.

The Committee recommended approval of this procedure.

75-257

PT-13, Fire Pump Operation and System Alignment, PCN 75-340, PCN 75-341. J. Straight presented this procedure change.

PCN 75-341 was to facilitate steps N.A.'d.

PCN 75-340 requested a step 4.3.5 be added to read; Verify proper water level in Die Fire pump Heat Exchanger (Radiator)

The Committee recommended approval of this change.

PORC ITEM
NUMBER:DESCRIPTION

75-258

EM-117, Removing Faulty 4KV Underfrequency Relay For Inspection And/Or Repair, PCN 75-314 and EM-72, Reactor Trip Undervoltage Protection, PCN 75-313. J. Straight presented these procedures for deletion as they were replaced by M-60.1, M-60.2, M-60.3 and M-60.4.

The Committee recommended approval of these deletions.

75-259

M-43.20B, "B" Steam Generator Secondary Side Pressure Test, PCN 75-426. J. Straight presented this procedure change: It requested;

Step 5.1.10 should read for AOV-4297 to be operable and closed.
Step 5.1.11 should read for AOV-4298 to be operable and closed.
Step 5.1.57 valve 4348A to read open.

The Committee recommended approval of this change.

75-260

M-48.6, Installation of Bus 18, PCN 75-463. J. Straight presented this new procedure. This procedure is to describe the steps necessary to isolate Bus 18 for cleaning and breaker maintenance including a list of work performed.

The Committee recommended approval of this procedure.

75-261
SM-74-30

Steam Generator Blowdown Jumper. B. Vanderweel presented this design criteria and safety analysis for this SM. The Committee performed the safety evaluation and found that the modification does not involve any unreviewed safety question and that Technical Specifications are not effected.

75-262

SM-74-41, Monitor Circuits for UV and Trip Scheme Test. Mr. Scheetz presented the design criteria and safety analysis for this SM. The Committee performed the safety evaluation and found that the modification does not involve any unreviewed safety question and that Technical Specifications are not effected.

75-265

Tech. Spec. Chg., Cycle 5. Dr. McCreedy presented the proposed Tech. Spec. chg., attachment A and B, Cycle 5 to the Committee for comments.

The Committee recommended approval of this change.

PORC ITEM
NUMBER:DESCRIPTION

- 75-267 M-43.16, Steam Generator Explosive Tube Plugging, PCN 75-409.
J. Straight presented this procedure change. It requested;

Change step 3.9 to read as follows: 3.9 at least one door in each personnel access hatch shall be closed and all automatic containment isolation valve operable on locked closed.
- 75-268 PT-14, Circulating Pumps - High Water Trip Logic, PCN 75-457.
J. Straight presented this procedure change. It requested several changes to include computer print out of relay operation.

The Committee recommended approval of this change.
- 75-270 CP-111.0, Calibration and/or Maintenance of Reactor Make-Up Water Flow Channel "III", PCN 75-576. J. Straight presented this procedure change. This requested a change to the data sheet also to add step 5.1.12 to read;

5.1.12 (CP 111.11) power supply FQ111 to outline a procedure for calibration and/or maintenance on reactor make-up water flow power supply.
- 75-271 PT-23.16A, Containment Isolation Valve Leak Rate Testing "A" Steam Generator Blowdown, PCN 75-433. J. Straight presented this procedure change. It was requested;

Steps 6.1 and 6.12.2 Delete "Manual Valve RV-3" and replace with "A" Steam Generator Blowdown Regulating Valve."

Step 6.3 Delete "Manual Valve RV-1."

Delete step 6.7 and replace with "6.7 Place test tag on "A" Steam Generator Blowdown (5709) Regulating Valve and open to provide an atmospheric Vent for PIV AOV 5737".

The Committee recommended approval of this change.
- 75-272 S-4.1A, Liquid Waste Process Startup, PCN 75-584. J. Straight presented this procedure change. It requested step 5.3.10 to read;

Leave inlet valve from RO unit open due to hot laundry water.

The Committee recommended approval of this change.
- 75-273 PT-23.7, Containment Isolation Valve Leak Rate Testing Letdown From Reactor Coolant System, PCN 75-_____. J. Straight presented this procedure change. It requested step 6.5 be changed to read;

Request operations to open or ensure open AOV 371.

The Committee recommended approval of this change.

PORC ITEM NUMBER:	DESCRIPTION
75-274	<p><u>PT-23.20, Containment Isolation Valve Leak Rate Testing, R.C.D.T Gas Header, PCN 75-391.</u> J. Straight presented this procedure change. It was requested step 6.3 read;</p> <p>Ensure that AOV 1786 is in the closed position and ensure that AOV 1787 is in the open position.</p> <p>The Committee recommended approval of this change.</p>
75-275	<p><u>PT-23.21, Containment Isolation Valve Leak Rate Testing, R.C.O.T. Gas Analyzer, PCN 75-390.</u> J. Straight presented this procedure change. It requested step 6.3 to read;</p> <p>Close or ensure closed AOV 1789 and place test tag at selector switch on panel.</p> <p>The Committee recommended approval of this change.</p>
75-276	<p><u>PT-23.1, Containment Isolation Valve Leak Rate Testing Pressurizer Relief Tank Gas Analyzer Line, PCN 75-389.</u> J. Straight presented this procedure change. It was requested step 6.4 read;</p> <p>Close or ensure closed AOV 539 by use of the gas analyzer panel selector switch and place test tag at selector switch.</p> <p>The Committee recommended approval of this change.</p>
75-277	<p><u>PT-23.28; Containment Isolation Valve Leak Rate Testing Auxiliary Coolant System From "A" Reactor Coolant Pump, PCN 75-586.</u> J. Straight presented this procedure change. It requested several valve number designations changed.</p> <p>The Committee recommended approval of this change.</p>
75-278	<p><u>PT-23.23, Containment Isolation Valve Leak Rate Testing Sump "A" Discharge, PCN 75-587.</u> J. Straight presented this procedure change. It was requested under step 6.1 to change MCB to <u>WDP</u>, to correct the switch location.</p> <p>The Committee recommended approval of this change.</p>
75-279	<p>J. Straight presented the following CP's, it was requested under Instructions step 5.5 to change \pm 1% of Span to read \pm 2% Span.</p> <p><u>CP-950.3, Containment Pressure Transmitter "PT-950," PCN 75-488.</u></p> <p><u>CP-949.3, Containment Pressure Transmitter, "PT-949," PCN 75-489.</u></p> <p><u>CP-948.3; Containment Pressure Transmitter, "PT-948," PCN-75-490.</u></p>

PORC ITEM
NUMBER:DESCRIPTION75-279
(Continued)

CP-947.3, Containment Pressure Transmitter, "PT-947," PCN 75-491.

CP-946.3, Containment Pressure Transmitter "PT-946," PCN 75-492.

CP-945.3, Containment Pressure Transmitter "PT-945," PCN 75-493.

CP-941.1, Accumulator Pressure Transmitter "PT-941," PCN 75-494.

CP-940.1, Accumulator Pressure Transmitter "PT-940," PCN 75-495.

CP-937.1, Accumulator Pressure Transmitter, "PT-937," PCN 75-496.

CP-936.1, Accumulator Pressure Transmitter, "PT-936," PCN 75-497.

CP-920.3, Refueling Water Storage Tank Level Transmitter, LT-920, PCN 75-498.

CP-626.3, Residual Heat Removal Flow Transmitter FT-626, PCN 75-499.

CP-486.3, Turbine 1st Stage Pressure Transmitter PT-486, PCN 75-500.

CP-485.3, Turbine 1st Stage Pressure Transmitter, PT-485, PCN 75-501.

CP-483.3, Steam Generator Pressure Transmitter, PT-483, PCN 75-502.

CP-482.3, Steam Generator Pressure Transmitter PT-482, PCN 75-503.

CP-479.3, Steam Generator Pressure Transmitter PT-479, PCN 75-504.

CP-478.3, Steam Generator Pressure Transmitter PT-478, PCN 75-505.

CP-106.3, Boric Acid Tank Level Transmitter LT-106, PCN 75-538.

CP-128.3, Charging Line Flow Transmitter FT-128 and Local Indicator FI 128A, PCN 75-537.

CP-171.3, Boric Acid Tank Level Transmitter LT-171, PCN 75-536.

CP-172.3, Boric Acid Tank Level Transmitter LT-172, PCN 75-535.

CP-411.3, Reactor Coolant Flow Transmitter FT-411, PCN 75-534.

CP-412.3, Reactor Coolant Flow Transmitter FT-142, PCN 75-533.

CP-413.3, Reactor Coolant Flow Transmitter FT-413, PCN 75-532.

CP-414.3, Reactor Coolant Flow Transmitter FT-414, PCN 75-531.

CP-415.3, Reactor Coolant Flow Transmitter FT-415, PCN 75-530.

CP-416.3, Reactor Coolant Flow Transmitter FT-416, PCN 75-529.

CP-426.3, Pressurizer Level Transmitter LT-426, PCN 75-528.

CP-427.3, Pressurizer Level Transmitter LT-427, PCN 75-527.

FORC ITEM
NUMBER:

DESCRIPTION

75-279
(Continued)

- CP-428.3, Pressurizer Level Transmitter LT-428, PCN 75-526.
- CP-429.3, Pressurizer Pressure Transmitter PT-429, PCN 75-525.
- CP-430.3, Pressurizer Pressure Transmitter PT-430, PCN 75-524.
- CP-431.3, Pressurizer Pressure Transmitter PT-431, PCN 75-523.
- CP-449.3, Pressurizer Pressure Transmitter PT-449, PCN 75-522.
- CP-461.3, Steam Generator Level Transmitter LT-461, PCN 75-521.
- CP-462.2, Steam Generator Level Transmitter, LT-462, PCN 75-520.
- CP-463.3, Steam Generator Level Transmitter, PCN 75-519.
- CP-464.3, Steam Flow Transmitter FT-464, PCN 75-518.
- CP-465.3, Steam Flow Transmitter FT-465, PCN 75-517.
- CP-466.3, Feedwater Flow Transmitter FT-466, PCN 75-516.
- CP-467.3, Feedwater Flow Transmitter FT-467, PCN 75-515.
- CP-468.3, Steam Generator Pressure Transmitter, PT-468, PCN 514.
- CP-469.3, Steam Generator Pressure Transmitter PT 469, PCN 75-513.
- CP-471.3, Steam Generator Level Transmitter LT-471, PCN 75-512.
- CP-472.3, Steam Generator Level Transmitter LT-472, PCN 75-511.
- CP-473.3, Steam Generator Level Transmitter LT-473, PCN 75-510.
- CP-474.3, Steam Flow Transmitter FT-474, PCN 75-509.
- CP-475.3, Steam Flow Transmitter FT-475, PCN 75-508.
- CP-476.3, "B" Feedwater Flow Transmitter FT-476, PCN 75-507.
- CP-477.3, Feedwater Flow Transmitter FT-477, PCN 75-506.
- CP-102.3, Boric Acid Tank Level Transmitter LT-102, PCN 75-539.

The Committee recommended approval of these changes.

75-282

M-48.2, Isolation of Bus 14, PCN 75-596, M-48.5, Isolation of Bus 17, PCN 75-598. J. Straight presented these procedure changes. They requested to add steps for notification of Relay Dept. for the purpose of performing calibration and trip.

PORC ITEM
NUMBER:DESCRIPTION

75-283

CP-624.1, Residual Heat Removal Heat Exchanger Outlet Valve
HCV-624, Isolation, PCN 75-369, PCN 75-361. J. Straight
presented this procedure change.

PCN 75-361 - was a temporary change requesting step 3.1
be N.A.'d.

PCN 75-369 - requested on data sheet valve calculation %
be changed as they were reversed. This is to be made per-
manent.

The Committee reviewed the temporary change and recommended
approval of PCN 75-369

75-284

CP-625.1, Residual Heat Removal Heat Exchanger Outlet Valve,
HVC-625, Isolation, PCN 75-362 and PCN 75-370. J. Straight
presented this procedure change.

PCN 75-362 - was temporary requesting step 3.1 be N.A.'d.

PCN 75-370 - requested to change valve position calculation
of % be changed on the data sheet as they were reversed.
This is to be made permanent.

The Committee recommended approval of PCN 75-370 and reviewed
PCN 75-362.

75-289

M-37.35, Maintenance of Valves 3504 and 3505, Main Steam to
Turbine Driven Auxiliary Feedwater Pump, PCN 75-443. J. Straight
presented this new procedure. The purpose of this procedure
is to describe the steps necessary to inspect and repair valves
3504 and 3505.

The Committee recommended approval of this procedure.

75-291

O-2.1, Normal Shutdown to Hot Shutdown, PCN 75-339 and PCN 75-349.
J. Straight presented these procedure changes.

PCN 75-339 - requested mark step 5.35 and make this possible
in future procedures when the plant is to remain in hot shut-
down or started back up and stopping 1 RCP is not necessary.

PCN 75-342 requested - add to step 4.6.2.4 - if computer is
not operating.

The Committee recommended approval of this change.

75-292

RF-8.4, Fuel and Core Component Movement In the Spent Fuel
Pit, PCN 75-368. J. Straight presented this procedure change.
It requested to delete steps 12 and 14 in Appendix "A".

The Committee recommended approval of this change.

PORC ITEM
NUMBER:DESCRIPTION

75-293

PT-23.16B, Containment Isolation Valve Leak Rate Testing "B" Steam Generator Blowdown, PCN 75-434. J. Straight presented this procedure change. It requested;

Step 6.1 delete "Manual Valve RV-3" and replace with B Steam Generator Blowdown Regulator Valve 5710 and step 6.12.2.

Step 6.3 delete "Manual Valve RV-3".

Step 6.7 should read: Place test tag on B Steam Generator Blowdown Regulator Valve 5710 and open to provide an atmospheric vent for primary isolation valve AOV 5738.

The Committee recommended approval of this change.

75-294

M-43.20A, "A" Steam Generator Secondary Side Pressure Test, PCN 75-439. J. Straight presented this procedure change. It requested;

Step 5.1.63 - B Chem pump discharge change V-5038 from open to closed.

Step 5.1.63.4 - common suction valve for chem pump from open to closed. V-5036.

Step 5.1.63.7 - Chem. discharge to A S/G change from open to closed. V-4281.

The Committee recommended approval of this change.

75-295

M-48.5, Isolation of Bus 17, PCN 75-440 and PCN 75-436. J. Straight presented this procedure change.

PCN 75-440 requested; add to step 5.6 "open and hold main incoming breaker Bus 17 on MCC 1G."

PCN 75-436 requested; change procedure so that step 5.12 and 5.13 come after 5.14.

The Committee recommended approval of these changes.

75-297

M-43.1, Steam Generator Manway Cover and Insert Removal, PCN 75-482, and M-43.15, Reinstallation of Steam Generator Manway Insert and Cover Closure, PCN 75-483. J. Straight presented these two procedures for the same procedure change, which was;

Add step 3.4.1 - verify that supplied air is being passed thru the del-monox purifier and the filter cartridge is acceptable for use.

The Committee recommended approval of this change.

PORC ITEM
NUMBER:DESCRIPTION

75-298

J. Straight presented the following procedures for temporary procedure changes, for review.

PT-5.10, Process Instrumentation Reactor Protection Channel Trip Test, PCN 75-442. Requested steps 6.15, 6.62, 6.1, 6.100 and 6.101 be marked N. A.

PT-5.20, Process Instrumentation Reactor Protection Channel Trip Test, PCN 75-444, requested steps 6.15, 6.67 and 6.68 be marked N.A.

PT-5.30, Process Instrumentation Reactor Protection Channel Trip Test, PCN 75-446, PCN 75-447. Requested steps 6.106, 6.107, 6.16 and 6.50 be N.A.'d.

PT-5.40, Process Instrumentation Reactor Protection Channel Trip Test, PCN 75-449. Requested steps 6.14, 6.42 and 6.65 be marked N.A.

The Committee reviewed the above changes. This item

75-299

PT-23.12A, Containment Isolation Valve Leak Rate Testing Pressurizer Steam Space Sample, PCN 75-396, PCN 75-453.
J. Straight presented this procedure change.

PCN 75-396 - requested to change step 6.4 to read: Close or ensure closed AOV 966 and place test control tag at control switch location.

PCN 75-453 - requested the same change as 75-396.

The Committee recommended approval of this change.

75-300

PT-23.12B, Containment Isolation Valve Leak Rate Testing Pressurizer Liquid Space Sample, PCN 75-395, PCN 75-452.
J. Straight presented this procedure change.

PCN 75-395 - requested step 6.4 to read, Close or ensure closed AOV 966B and place test control tag at control switch location.

PCN 75-395 - requested the same change as 75-395.

The Committee recommended approval of this change.

75-301

PT-23.12C, Containment Isolation Valve Leak Rate Testing, PCN 75-394, PCN 75-454. J. Straight presented this procedure change. PCN 75-394 and PCN 75-454 requested step 6.4 be changed to read:

Close or ensure closed AOV 966C and place test control tag at control switch location.

The Committee recommended approval of this change.

PORC ITEM
NUMBER:DESCRIPTION

75-302

PT-23.13A, Containment Isolation Valve Leak Rate Testing, "A" Steam Generator Sample, PCN 75-393 and 75-455. J. Straight presented this procedure change. Both PCN's requested to change 6.4 to read;

Ensure that control switch (located at sample panel) for AOV 5735 is in the "AUTO" position. Request operation to close or ensure closed AOV-5735 by use of CB master switch.

The Committee recommended approval of this change.

75-303

PT-23.13B, Containment Isolation Valve Leak Rate Testing "B" Steam Generator Sample, PCN 75-392 and PCN 75-456. J. Straight presented this procedure change. Both PCN's requested step 6.4 to read;

Ensure that control switch (located at sample panel) for AOV-5736 is in the "AUTO" position. Request operation to close or ensure closed AOV-5736 by use of CB master switch.

The Committee recommended approval of this change.

75-304

J. Straight presented the following procedures with temporary changes for review only.

PT-12.1, Emergency Diesel Generator 1A, PCN 75-438. Requested steps 6.2.1 through 6.2.7 be N.A.'d.

PT-6.3.3, Power Range Nuclear Instrumentation System, PCN 75-472, Channel 43, requested step 6.54 be marked N.A.

PT-6.3.4, Power Range Nuclear Instrumentation System, Channel 44 PCN 75-473, requested step 6.54 be marked N.A.

PT-6.3.1, Power Range Nuclear Instrumentation System, Channel 41, PCN 75-474, requested step 6.54 be marked N.A.

PT-6.3.2, Power Range Nuclear Instrumentation, Channel 42, PCN 75-475, requested step 6.5.4 be marked N.A.

75-305

RF-2A, Requirements For Reactor Head Removal, PCN 75-470. J. Straight presented this procedure change for review only. It was requested;

Step 5.10.5 - Water level at steam generator repair level.

Step 5.11 - Mark N.A. since not making head lift.

The Committee reviewed this change.

PORC ITEM
NUMBER:DESCRIPTION

- 75-307 M-43.20A, "A" Steam Generator Secondary Side Pressure Test, PCN 75-448 and PCN 75-427. J. Straight presented this temporary procedure change. It requested some steps to be marked N.A.
- The Committee reviewed this change. PCN 75-427 requested to add "valve 436 open."
- The Committee recommended approval of this change.
- 75-308 M-48.5, Isolation of Bus 17, PCN 75-445. J. Straight presented this procedure change. It requested to;
- Add new step between step 5.2.6 and 5.27 to read; Remove hold and close main incoming breaker Bus 17 at MCC-1G position 4M clear.
- The Committee recommended approval of this change.
- 75-309 CP-43.01, Calibration and/or Maintenance 430 Pressurizer Pressure Channel, PCN 75-4581A and CP-485, Calibration and/or Maintenance of Turbine 1st Stage Pressure Channel "485", PCN 75-464 and PCN 75-466.
- PCN 75-458A requested under instructions step 5.12 change Channel 1 Rack 1 to read Channel 2, Rack 1.
- PCN 75-464 - requested the same change on steps 5.6 and 5.7.
- PCN 75-466 - requested the same change on steps 5.1 and 5.2.
- The Committee recommended approval of these changes.
- 75-310 RF-2B, Required Valve Lineup for Reactor Head Removal, PCN 75-469. J. Straight presented this procedure change. It was requested;
- Change step 5.9.1 to read: Isolation valves locked closed.
- The Committee recommended approval of this change.
- J. Straight presented the following temporary changes for review only;
- 75-311 CP-626.1, Defeat of Residual Heat Removal Flow Channel "626" PCN 75-365, N. A. step 5.3.2.
- CP-626.2, Reinstate of Residual Heat Removal Flow Channel "626" PCN 75-335, N.A. steps 5.3.2, 5.3.3 and 5.3.4.
- CP-464.0, Calibration and/or Maintenance of Steam Flow Channel "464", PCN 75-476, steps 5.7 and 5.9 to be marked N.A.

PORC ITEM
NUMBER:

DESCRIPTION

75-311
(Continued)

S-4.1H, "B" Laundry and Hot Shower Tank Release To Discharge Canal, PCN 75-477, N.A. step 5.6.

M-43.5, Installation of Steam Generator Temporary Loop Nozzle Covers, PCN 75-481, N.A. step 5.9 temporary to be made permanent.

The Committee reviewed these procedure changes.

75-312

J. Straight presented the following two procedures for deletion;

RF-14, Special Instructions and Tech. Manual Procedures, PCN 75-297.

O-4, Load Follow, PCN 75-366.

The Committee recommended approval of these deletions.

75-313

SM 74-231, Install Boric Acid Supply to the Safety Injection System, PCN 75-471. J. Straight presented this new procedure. This procedure is to describe the steps necessary to change the logic and timer circuits for the refueling water storage tank to safety injection system valves 825A and 825B.

The Committee recommended approval of this procedure.

75-316

SM 75-1.2, Instrument Cable Rerouting, PCN 75-458B.

J. Larizza presented this procedure change. It requested;

1. Change reference B.11.5 to read 4155 D-215-081.
2. Change reference B.11.10 to read 4666 B-281-041.

The Committee recommended approval of this change.

75-317

SM 75-2.1, "TV System Insert In Containment, PCN 75-478.

J. Larizza presented the following procedure change;

Change step E.3 to read as follows: Nitrogen supply valve has been closed and flange on penetration sleeve AE-12 has been removed in accordance with B.11, B.12, B.13.

Also add signoffs.

Add step E.4 to read as follows: The outboard cap on penetration sleeve AE-17 has been removed in accordance with B.11 B.12, B.13.

Also add signoffs.

Change numbering sequence from present E-9 to E-10.

The Committee recommended approval of this change.

PROC ITEM
NUMBER :

DESCRIPTION

- 75-321 EM-138, New Fuel Element Index Hole Reaming, PCN 75-625.
C. Peck presented this new procedure. The purpose of this procedure is to provide instruction for reaming out the indexing hole of a new Westinghouse fuel assembly in the new fuel elevator. The Committee recommended approval of this procedure.
- 75-322 PT-25, Containment Air Filtration Post Accident Charcoal Filter By-Pass Flow & Pressure Drop Test, PCN 75-626.
C. Hartlieb presented this new procedure. The purpose of this procedure is to permit by-pass flow determination of the containment post accident charcoal filters (A & B Units). The committee recommended approval of this new procedure.
- 75-323 M-61.1, Mechanized Ultrasonic Examination of Reactor Vessel Nozzle Welds, PCN75-595. J. Straight presented this new procedure. The purpose of this procedure is to provide instructions for the Mechanized Ultrasonic Examination of Reactor Vessel Nozzle Welds. The Committee recommended approval of this procedure.
- 75-325 O-2.3.1, Draining The Reactor Coolant System, PCN75-637. J. Straight presented this temporary procedure change. This requested steps 1.0 through 5.16 and steps 5.18 be marked N.A. The Committee reviewed this change.
- 75-327 RF-8.2, Fuel Handling Instruction Pre-Loading & Periodic Valve Test, PCN75-594. J. Straight presented this procedure change. It was requested under instructions, step 5.10 be changed to read;
"H₂ recumbiner - Isolation Valves locked closed"
The Committee recommended approval of this change.
- 75-328 RF-9.9, Control Rod Drive Shaft Unlatching Tool Operating Instruction, PCN75-654, and PCN75-652.
PCN75-652 was to correct a typing error in step 5.2.5.
PCN75-654 requested two data sheets be added.
The Committee recommended approval of these changes.
J. Straight presented these temporary procedure changes;
- 75-329 M-37.11, Repair & Inspection of Pressurizer Power Operated Relief Valves 430 and 431C, PCN75-655, requested step 3.2 N.A. and steps 5.3 through 5.15 N.A.
RF-9.2, Refueling Procedure RF-9.2, Fuel Transfer System Operation, PCN75-589, requested under Instruction, step 5.6.12 be N.A.'d.
O-2.3.1, Draining The Reactor Coolant System, PCN75-599; requested step 3.4 be marked N.A.
The Committee reviewed these procedures.

PROC ITEM NUMBER:	DESCRIPTION
75-331	<p><u>M-7.4, Boric Acid Storage Tank Filter, PCN75-653.</u> J. Straight presented this procedure change. It was requested step 4.1 be deleted. The Committee recommended approval of this change. This item <u>IS COMPLETE.</u></p> <p><u>P-9, Precautions, Limitations & Setpoints P-9 Radiation Monitoring System, PCN75-590.</u> J. Straight presented this procedure change; it requested in step 2.3.2.11, the setpoint shall be set at about 15,000 cpm to keep the alarm above normal levels.</p> <p>The Committee recommended approval of this procedure.</p>
75-332	<p><u>RF-8.1, Step By Step Fuel Loading Sequence and Maps, PCN75-613, PCN75-614, PCN75-615, PCN75-616, PCN75-617, PCN75-622, PCN75-623, PCN75-628, PCN75-629, PCN75-630, PCN75-631, PCN75-632, PCN75-633, PCN75-634, PCN75-635, PCN75-639, PCN75-643, PCN75-644, PCN75-645, PCN75-646, PCN75-647, PCN75-648.</u> J. Straight presented all these procedure changes. The Committee recommended approval of these changes which were made in regards to fuel shuffle.</p>
75-333	<p><u>O-2.3.1, Draining The Reactor Coolant System, PCN75-599.</u> J. Straight presented this temporary procedure change. It requested step 3.4 be N.A.'d. The Committee reviewed this change.</p>
75-334	<p><u>RF-8.1, Step By Step Fuel Loading Sequence & Maps, PCN75-669.</u> J. Straight presented this procedure change. It was requested after step 117a to read;</p> <p style="text-align: center;">"conduct test assembly inspection in accordance with RF-8.4, Rev. 5"</p> <p>The Committee recommended approval of this change.</p>
75-335	<p><u>RF-8.4, Fuel and Core Component Movement In The Spent Fuel Pit, PCN75-662.</u> J. Straight presented this procedure change. It requested to remove Revision 3, Fuel Handling Data Sheets and replace them with new updated ones. The Committee recommended approval of this change.</p>
75-336	<p><u>SM 75-20.1, Control Rod Drive Shaft Chamfering Operating, PCN 75-667.</u> J. Straight presented this new procedure. This procedure defines the steps to accomplish the machining of a chamfer on a radioactivity contaminated control rod assembly.</p> <p>The Committee recommended approval of this procedure.</p>

PORC ITEM
NUMBER:DESCRIPTION

75-337

SM 75-11.1, Switch Control Board Recorders & Controllers Instrument Bus 1C, PCN 75-668. J. Straight presented this new procedure. This procedure describes the steps necessary to provide uninterruptable power supply to control board recorders and controllers during blackout.

The Committee recommended approval of the procedure.

75-338

PT-10, Station Battery Load Test, PCN 75-642, PCN 75-663. J. Straight presented this procedure change.

PCN 75-642 requested - Period II - decrease battery load to 484 amps for 12 min. add 10 min. reading for "A" battery on page PT-10:9.

Period II - decrease battery load to 229 amps for 12 min. add 10 min. reading for "B" battery on page PT-10:13.

PCN 75-663 requested to - add to PT-10 station battery load test where indicated on marked-up copy.

Page 3,4,5,6 add lines to page 8 and 12 also heading on battery charger data.

Page 9 and 13 (add data to be taken) and changes where indicated. Page 10 (add data to be taken) Page 11 (add lines) Page 15 (add lines) Page 14 (add data to be taken).

The Committee recommended approval of this change.

75-340

PT-23, Containment Isolation Valve Leak Rate Testing, PCN 75-274. J. Straight presented this procedure change. It was requested:

1. PT-23.37 Change "Reactor Compartment Cooling In" to read "Reactor Compartment Cooling Unit A".
2. PT-23.38 Change "Reactor Compartment Cooling Out" to read "Reactor Compartment Cooling Unit B".

The Committee recommended approval of this change.

75-342

S-23.3A, Penetration Cooling System, Control Rod Drive Cooling System, Reactor Compartment Cooling System and Auxiliary Charcoal Filter System Pre-start-up, PCN 75-468. J. Straight presented this new procedure. This procedure describes the pre-startup of the penetration cooling system, control rod drive cooling system, reactor compartment cooling system, and the auxiliary charcoal filter system.

The Committee recommended approval of this procedure.

PORC ITEM
NUMBER:

DESCRIPTION

75-344

RF-8, Fuel Assembly and Core Component Movement Prerequisites, PCN 75-601, PCN 75-604. J. Straight presented this procedure change. PCN 75-601 requested step 4.3 to read;

A daily sample shall be taken from the reactor cavity and the spent fuel pit and analysed for boron concentration during refueling.

75-345

PCN 75-604 requested to delete precaution 4.13, this was disapproved as PCN 75-601 took place of this.

J. Straight presented the following PCN's to facilitate steps N/A'd.

RF-9.2, Fuel Transfer System Operation, PCN 75-657, step 5.6.12.

RF-2E, Draining of Refueling Canal, PCN 75-602, steps 5.8.1 thru 5.13 also step 3.6 N/A'd.

S-16.4, Venting N² From the "B" Accumulator, PCN 75-611, step 3.3 N/A'd.

S-16.3, Venting N² From the "A" Accumulator, PCN 75-612, step 3.3 N/A'd.

M-37.30, Repair and Inspection of "A" Accumulator Fill Valve 835A, PCN 75-658, mark steps 5.4 thru 5.14 N/A'd.

RF-8.3, Core Loading Periodic Equipment Status Check, PCN 75-664, requested steps 5.8.1 and 5.13 be N/A'd.

S-3.4B, Gas Stripper Startup, PCN 75-609, mark steps 5.3.20, 5.3.21 and 5.3.22 N/A. Also requested to add;

NOTE: If recirculating H.U.T. to reduce activity steps 5.3.20 5.3.21 and 5.3.22 may be marked N/A.

The Committee reviewed the above items.

75-346

M-11.11, Inspection and Maintenance of Crane Chem pumps, Series G, PCN 75-585. Requested;

Step 5.5 should read serial no. of replacement pump.

Step 5.6.3 add - measure oriface bore I.D..

Step 5.6.7 add - measure oriface bore I.D.

Step 5.6.11 add - line to sign on.

The Committee recommended approval of this change.

PORC ITEM
NUMBER:-DESCRIPTION

- 75-347 PT-23.23, Containment Isolation Valve Leak Rate Testing Sump "A" Discharge, PCN 75-587. J. Straight presented this procedure change. It was requested under instructions step 6.1 the MCB be changed to;
- WDP control _ _ _ _
- WDP control _ _ _ _
- Also Step 6.5 change MCB to WDP.
- The Committee recommended approval of this change.
- 75-348 RF-9.5, RCC Change Fixture, PCN 75-606. J. Straight presented this procedure change for review only. It was requested;
- add to step 5.1.11 and turn on air
- step 5.1.14 and turn off air,
- step 5.1.24 and turn on air
- step 5.1.26 and turn off air
- The Committee reviewed this change.
- 75-349 PT-23.20, Containment Isolation Valve Leak Rate Testing, R.C.D.T Gas Header, PCN 75-608, and PT-23.21, Containment Isolation Valve Leak Rate Testing, R.C.D.T. Gas Analyzer, PCN 75-607. Both these procedures requested a minor temporary change for review only. The Committee reviewed these changes.
- 75-350 SM 75-4.1, Reroute of Steam Generator Blowdown Line, PCN 75-610, PCN 75-618. J. Straight presented this procedure change.
- PCN 75-610 requested to add step E 6.1 - Insure blowdown tank is drained, "Remove manway cover, H.P. to survey tank to see if protective clothing is required, and inspect tank from the inside for wall thinning". Also sign off.
- PCN 75-618 requested to change step E.6 - Change valve # 3630 to 3530.
- Add these two valves #5763, #4694.
- The Committee recommended approval of this change.

PORC ITEM
NUMBER:

DESCRIPTION

75-352

M-37.11, Repair and Inspection of Pressurizer Power Operated Relief Valves 430 and 431C, PCN 75-641. J. Straight presented this procedure change. It was requested;

Step 5.4 eliminate (slings are used)

Add step 5.2.1 - Locate air set at job area

Step 5.7 - Eliminate the words "by the eye bolts" as they are not used.

Step 5.13 - Eliminate words "Torque to 340 ft. lbs." as this is not presently possible.

Step 5.14 - Eliminate words "Torque to 271 ft. lbs." as this is not presently possible.

The Committee recommended approval of this change. This item IS COMPLETE.

75-353

PT-23.7, Containment Isolation Valve Leak Rate Testing Letdown From Reactor Coolant System, PCN 75-649, 75-650. J. Straight presented this procedure change.

PCN 75-540 requested steps 5.4.5 through 5.8 be N/A, also after step 5.4.4 after the words "torquing to" add the words, "at least".

This was temporary for review only.

PCN 75-541 requested to; add (if used) after each of the following steps: 5.2, 5.3, 5.4, 5.8, 5.9, 5.10, 5.11 to allow N/A when covers are used only for loop protection and reactor water level is not to be raised.

Add checkoff line for step 5.15

Add (as applicable) to sign off for Foreman.

The Committee recommended approval of this change.

75-356

M-48.6, Isolation of Bus 18, PCN 75-660. J. Straight presented this procedure change. It was requested to change the following steps to read;

5.18 Remove hold on Switch #6, Bus 18 emergency DC Supply, in screenhouse DC distribution panel 1B and close. clear _____

5.19 Remove hold on Switch #2, Bus 18 normal DC Supply, in screenhouse DC distribution panel 1A and close. clear _____

The Committee recommended approval of this change.

PORC ITEM

NUMBER:DESCRIPTION

- 75-357 CP-428.3, Pressurizer Level Transmitter LT 428, PCN 75-651.
J. Straight presented this procedure change. It was requested to provide sign-off lines per each Instruction for Transmitter Calibration. The Committee recommended approval of this change.
- 75-358 M-43.11, Steam Generator Tube Removal, PCN 75-627. J. Straight presented this procedure change. It requested;

5.4.10 "with the 13/20" - 20 tap" should be changed to: "with the 13/16 - 20 tap".

The Committee recommended approval of this change.
- 75-359 CP-139, Calibration and/or Maintenance of Volume Control Tank Level Channel "139", PCN 75-582. J. Straight presented this procedure change. It was requested % on the Data Sheet Column LI-139A.

The Committee recommended approval of this change.
- 75-360 S-21.2, 1B Hydrogen Recombiner Purging and Operation, PCN 75-592.
J. Straight presented this new procedure. This procedure is to describe the steps necessary to purge air from the recombiner system hydrogen piping.

The Committee recommended approval of this procedure.
- 75-361 S-21.2, 1A Hydrogen Recombiner Purging and Operation, PCN 75-593.
J. Straight presented this new procedure. This procedure is to describe the steps necessary to purge air from the recombiner system hydrogen piping.
- 75-363 PT-2.2, Residual Heat Removal System, PCN 75-780.
J. Straight presented this procedure change. It was requested to add new precaution, step 5.4 to read;

When RHR system is in operation and the RCS temperature is greater than 200°F, a maximum of 2 charging pumps are to be operated to prevent overpressurizing the RHR system below 200°F, the operation of 3 charging pumps is allowable but should be limited if possible.

Also a new reference step 3.3 to read; Letter dated 2-25-75 from R. Mecredy to Bruce Snow concerning procedure change to limit charging pump use at cold shutdown.

The Committee recommended approval of this change.

PORC ITEM
NUMBER:DESCRIPTION

75-364

PT-4, Residual Heat Removal Loop, Annual Hydro Test of Low Pressure Piping, PCN 75-381. and PT-4.1, Residual Heat Removal Loop Hydro Test of High Pressure Piping, PCN 75-382.

J. Straight presented these procedure changes. It requested both procedures be changed as follows;

3.7 Letter dated 2-25-75 from R. Mecredy to Bruce Snow concerning procedure change to limit charging pump use at cold shutdown.

5.4 When RHR system is in operation and the RCS temperature is greater than 200°F, a maximum of 2 charging pumps are to be operated to prevent overpressurizing the RHR system below 200°F, the operation of 3 charging pumps is allowable but should be limited if possible.

The Committee recommended approval of this change.

75-365

S-13A, RHR System Line-up for Safety Injection, PCN 75-379.

J. Straight presented this procedure changes. It requested the following steps read;

2.2 Letter dated 2-25-75 from R. Mecredy to Bruce Snow concerning procedure change to limit charging pump use at cold shutdown.

4.3 When RHR system is in operation and RCS temperature is greater than 200°F, a maximum of 2 charging pumps are to be operated to prevent overpressurizing the RHR system below 200°F, the operation of 3 charging pumps is allowable but should be limited if possible.

The Committee recommended approval of these changes.

75-366

J. Straight presented the following procedures for changes to step 2.2 and 4.14;

Coolant System, PCN 75-376.

O-1.1, Plant Heat up from cold shutdown to hot shutdown, PCN 75-371.

O-1.1D, Pre-Heatup Plant Requirement Check List, PCN 75-372.

O-2.3, Plant at Cold Shutdown, PCN 75-374.

O-2.3.1, Draining the Reactor Coolant System, PCN 75-375.

O-2.2, Plant Shutdown from Hot Shutdown to Cold Condition, PCN 75-373. These changes were to read;

2.2 Letter dated 2-25-75 from R. Mecredy to Bruce Snow concerning procedure change to limit charging pump use at cold shutdown.

<u>FORC ITEM NUMBER:</u>	<u>DESCRIPTION:</u>
75-366 (Continued)	4.14 When RHR system is in operation and the RCS temperature is greater than 200°F, a maximum of 2 charging pumps are to be operated to prevent overpressurizing the RHR system below 200°F, the operation of 3 charging pumps is allowable but should be limited if possible.
75-367	<u>SM 74-2.1, RPI Pulse to Analog Converter Change, PCN 75-211.</u> J. Straight presented this new procedure. The purpose of this procedure is to provide instructions for the removal of the two existing pulse -to- analog converters, and the installation of the new solid state pulse to analog converter. This item is <u>NOT COMPLETE</u> pending completion of the job.
75-368	<u>S-4.2.10, Burping the Pressurizer Relief Tank and RCDF and Isolating from the Vent Header, PCN 75-450.</u> J. Straight presented this procedure change. It was requested step 5.3.2 read V-554. The Committee recommended approval of this change.
75-369	<u>PT-17.2, Process Radiation Monitors R-11 - R-20, Iodine Monitors R-10A and R-10B, PCN 75-485.</u> J. Straight presented this procedure change. It was requested step 6.11.9 read; Raise setpoint to 10K The Committee recommended disapproval of this change.
75-375	<u>RSSP-2.6, Emergency Diesel Start and Breaker Closure Time, PCN 75-670.</u> J. Straight presented this procedure change. It was requested; Initial Condition 4.2 should read - Service water cooling requirements are such that one service water pump is adequate for heat load removal. The Committee recommended approval of this change.
75-376	<u>M-50.1, Isolation of #12 Transformer, PCN 75-672.</u> J. Straight presented this procedure change. It requested to add steps for notification of Relay Dept. for the purpose of performing calibration and trip test on #12 transformer. The Committee recommended approval of this change.
75-377	<u>M-11.10, Major Inspection of Service Water Pump, PCN 75-673.</u> J. Straight presented this procedure change. It requested to; Add to instructions - 5.8.1 inspect rubber expansion joint and replace if necessary. The Committee recommended approval of this change.

PORC ITEM
NUMBER:

DESCRIPTION:

75-379

O-1.2, Plant from Hot Shutdown to Steady Load, PCN 75-675.

O-5.2, Load Increases, PCN 75-676.

J. Straight presented these procedure changes;

Add reference 2.4 to read as follows - Letter from P.L. Walker (Westinghouse Nuclear Fuel Division) to A.A. Fuierer regarding start-up requirements for Westinghouse Zircaloy Clad Fuel, NFD Newsletter, December, 1974.

Also to add;

PRECAUTION: During the initial return to power following a refueling shutdown or following a cold shutdown where fuel assemblies have been handled (e.g., inspection), the rate of reactor power increase shall be limited to 3% of full power in an hour between 20% and 100% of full power. This ramp rate requirement applies during the initial start-up and may apply during subsequent power increases depending on the maximum power level achieved and length of operation at that power level. Specifically, this requirement can be removed for reactor power levels at or below a given level P (20% \leq P \leq 100%) provided the plant has operated at or above level P for at least 72 cumulative hours out of any 7-day operating period.

PRECAUTION: The rate of reactor power increase above the highest power level sustained for at least 72 cumulative hours during the preceding 30 days of operation at power shall be limited to 3% of full power in an hour. Alternatively, reactor power increase can be accomplished by a single step increase less than or equal to 10% of full power followed by a maximum ramp rate of 3% of full power in an hour beginning 3 hours after the step increase.

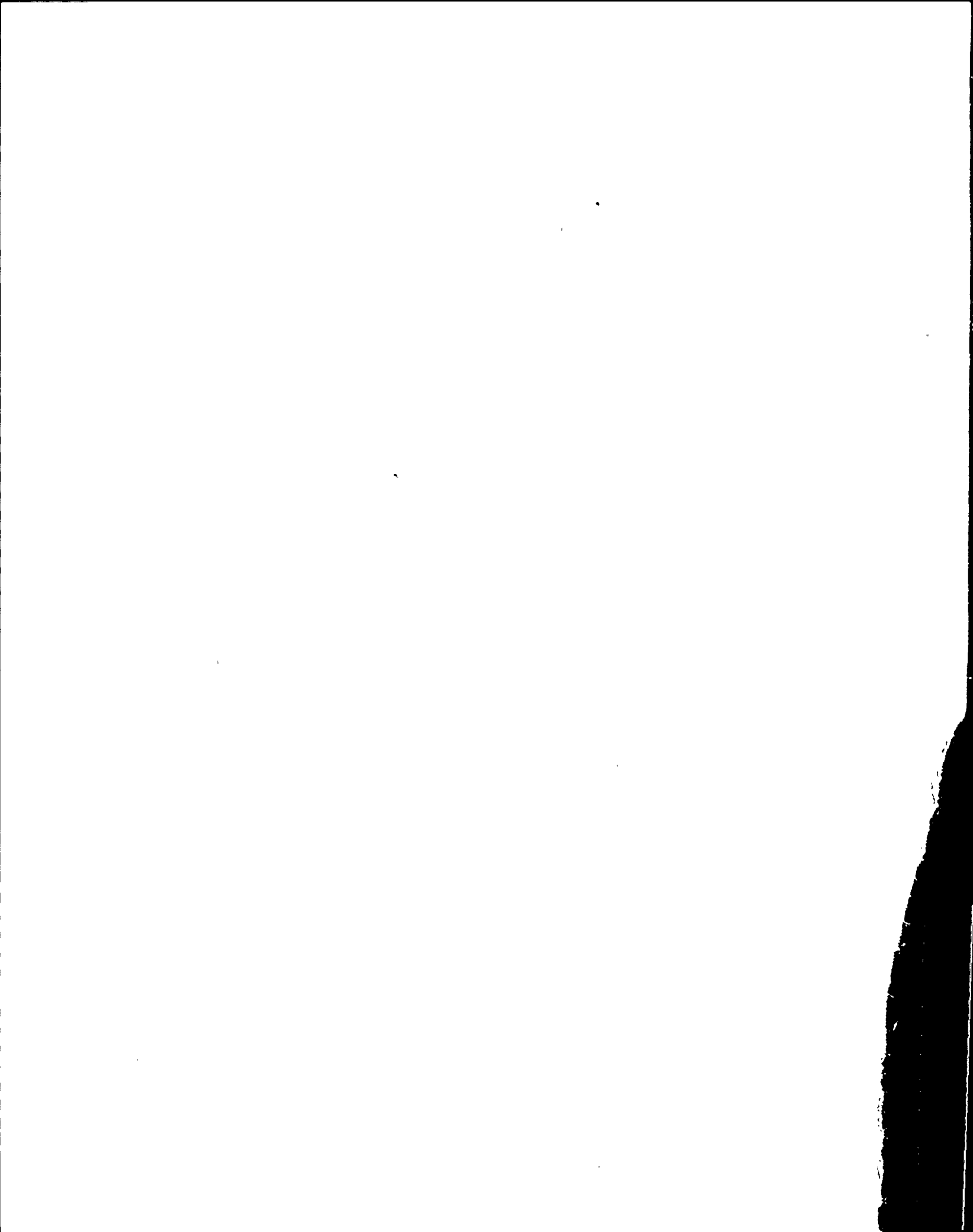
The Committee recommended approval of these changes.

75-381

RF-9.8.1, Part Length (P/L) Control Rod Drive Shaft Unlatching, PCN 75-465. J. Straight presented this procedure change. It requested a new step 3.6 be added to read;

Refueling integrity established as outlined in Tech. Specs. 3.8.1.

The Committee recommended approval of this change.



PORC ITEM NUMBER:	DESCRIPTION:
75-382	<p><u>M-37.16A, Inspection and Maintenance of Manual Operated Grinnell Diaphragm Valves, PCN 75-680.</u> J. Straight presented this procedure change. It requested;</p> <p>3.3 Valve to be inspected. Valve No. _____</p> <p>Add step 5.9.4 Note any irregularities found during the inspection. and attach or note a list of parts used. _____</p> <p>Step 4.1 "diaphragm"</p> <p>The Committee recommended approval of this change.</p>
75-387	<p><u>SM 74-41.1, Reactor Trip Breaker, Train A & B Undervoltage, 20ET and 20AST Monitor Circuit Installation, PCN 75-683.</u> J. Straight presented this new procedure. The purpose of this procedure is to describe the steps necessary for installing monitor lights to verify proper operation and circuit make-up of the trip breakers and undervoltage trip scheme.</p> <p>The Committee recommended approval of this procedure.</p>
75-389	<p><u>M-61.1, Mechanized Ultrasonic Examination of Reactor Vessel Nozzle Welds.</u> S. Spector presented his past maintenance review on this procedure. This job was completed on 4-1-75.</p>
75-390	<p><u>RSSP-2.7, Safety Injection Sequence Timers Trains A and B, PCN 75-682.</u> J. Straight presented this new procedure. The purpose of this procedure is to set timers for proper loading time sequence.</p> <p>The Committee recommended approval of this change.</p>
75-394	<p><u>M-43.11, Steam Generator Tube Removal, PCN 75-716.</u> J. Straight presented this procedure change. It was requested to delete step 4.0, step 5.5 and attachment B and attachment C.</p> <p>Step 3.11 should read; "required plant conditions shall be established as per paragraph 5.1 of attachment "A", except that 5.1.2 shall require that the water level in the primary system be drained to the proper level and maintained there as per operating procedure O-2.3.1 _____".</p> <p>Step 4.1 should have added, "where tube plugging is in progress".</p> <p>Step 5.3 should read; Specific instructions for the tube removal and tube hole closure procedure were followed per MRS-2.3.2 generator 1 series 44 S/G in _____ leg of _____ Steam Generator.</p> <p>The Committee recommended approval of these changes.</p>

PORC ITEM
NUMBER:

DESCRIPTION:

- 75-396 EM-139, Isolation And Maintenance Of Immediate Borate Valve MOV 350, PCN75-714. S. Spector presented this new procedure. The Committee recommended approval of this procedure. This item is NOT COMPLETE pending completion of the job, plus a post-maintenance review.
- 75-397 M-15.1, A or B Diesel Generator Removal From Service, PCN75-692. Jack Noon presented the change to this procedure. The PORC recommended approval of the changes.
- 75-398 M-37.36, Repair of Pressurizer Steam Space Sample, Valve 951, PCN 75-678. J. Straight presented this new procedure. The purpose of this procedure describes the steps necessary for maintenance to the pressurizer steam space air operated valve 951. If this procedure is to be used in other than a cold shutdown condition it must be modified. Maintenance not required, may be marked N/A.
- The Committee recommended approval of this procedure.
- 75-399 M-38, Equalizing Charge Station Battery Systems, PCN 75-707. J. Straight presented this procedure change. It requested a change in the data sheets to include serial numbers of instruments used. Also to change step 4.7 to read;
- Continue equalizing charge according to manufacturer's instructions. Stabilization of current reading can be calculated by type of equalizing charge required. When normal main and small total charger current level is reached record voltage and specific gravity readings on all cells, temperature readings on designated pilot cells and amperage readings on main and small battery chargers for 3 consecutive hours. Then set timers for completion time designated by manufacturer's table. (NOTE: Manufacturer's instructions and data sheets attached).
- The Committee reviewed and approved this change.
- 75-400 P-9, Precautions, Limitations and Setpoints P-9 Radiation Monitoring System, PCN 75-694. J. Straight presented this procedure change to be made permanent. It was requested;
- Step 2.3.2.10 should read: R-19, (Steam Generator Blowdown), Monitors for primary to secondary leakage. It is located in an area of variable background and therefore, cannot alarm at too low a level. 6000 cpm is sufficient to not allow above impc releases and yet not give spurious alarm.
- The Committee recommended approval of this change.

PORC ITEM
NUMBER:

DESCRIPTION:

75-401

J. Straight presented the following temporary procedure changes for review only;

S-4.1C, Liquid Waste Process Operation and Shutdown, PCN 75-711.
Change steps 5.6.1 and 5.6.2 from 12.5% to 9%.

SM 74-23.1, Installation of the Modification for Control of the Boric Acid Supply to the Safety Injection System, PCN 75-684.
This request was to change terminal black numbers.

RF-8.3, Core Loading Periodic Equipment Status Check, PCN 75-704, PCN 75-705, and PCN 75-712. All PCN's were to facilitate step N/A'd.

S-3.4F, Boron Recycle Process Shutdown, PCN 75-698.
Requested step 5.8.6 be N/A'd.

The Committee reviewed these changes.

75-402

M-605.5, 1A Emergency Diesel Protective Relay Calibration and Trip Test, PCN 75-708.

M-60.6, 1B Emergency Diesel Protective Relay Calibration and Trip Test, PCN 75-709.

J. Straight presented these two new procedures. The purpose of these procedures are to provide instructions for removing and replacing protective relays associated with the 1A or 1B Emergency Diesel for calibration or corrective maintenance.

The Committee recommended approval of these two procedures.

75-403

CP-464.0, Calibration and/or Maintenance of Steam Flow, Channel "464", PCN 75-710. J. Straight presented this procedure change. It requested some figures be changed on the system data sheet.

The Committee recommended approval of this change.

75-405

RSSP-2.6, Emergency Diesel Start and Breaker Closure Time, PCN 75-693. J. Straight presented this procedure change. It was requested to add to step 6.1.2 and 6.2.2;

"Connect trace recorder to Diesel Generators voltmeter and diesel bus tie breaker".

The Committee recommended approval of this change.

PORC ITEM
NUMBER:

DESCRIPTION:

75-406

SM 75-23, Relocation of Accumulator Level Transmitter.
T. Schuler presented the modification design criteria report and system description, plus safety analysis for relocation of level transmitters on both A and B loop accumulator. The level transmitter will be cut from their present position and raised 17 inches.

The Committee reviewed the modification concept and performed the safety evaluation.

75-407

SM74-30.1, Steam Generator Blowdown Jumper Switch, PCN75-732.
J. Straight presented this new procedure. This procedure is to provide instructions for modifying the steam generator blowdown isolation valves (CV-70 & CV-71) control circuitry to permit opening of the valves when main feedwater pumps are not out of service. The Committee recommended approval of this procedure.

75-408

SM-74-30.2, Post System Modification Operational Test Of The Steam Generator Jumper Switch, PCN75-746. J. Straight presented this new procedure. This procedure is to outline steps for testing system modification 74-30 for operability. The Committee recommended approval of this procedure.
pending completion of the job.

75-409

M-42.2, Removal And Installation Of Charcoal Filters In The Post Accident Or Containment Purge Air Filter Systems, PCN75-759. E. DeMeritt presented this new procedure. This procedure is to describe the steps necessary to remove or install charcoal filter trays. The Committee recommended approval of this procedure.

75-411

RSSP-3.0, Verification Of Emergency Start Logic Auxiliary Feedwater Pumps, PCN75-751. J. Straight presented this procedure change. The procedure was rewritten according to the A-30 format. The rewrite also included the Steam Generator Blowdown mode switch which has been added as a system modification. The Committee recommended approval of this change.

75-415

M-55.1, Repair or Replacement of Type W-2 Breaker Control Switches, PCN75-715. J. Straight presented this procedure. This procedure was SM74-3 and rewritten to M-55.1. The procedure describes the steps necessary for the repair or replacement of W-2 switches. The Committee recommended approval of this procedure.

75-416

E-6, Loss Of Component Cooling, PNC75-165. J. Straight presented this procedure change. It was requested;

On page E-6:5, Case III under "NOTE"

Remove reference of component cooling going to containment spray pumps, RHR pumps, and safety injection pumps.

The Committee disapproved this PCN because the reference should not be removed.

PORC ITEM
NUMBER:

DESCRIPTION

75-418

RSSP-5.0, Immediate Boration System, PCN75-733. J. Straight presented this procedure change. This change was to add valve number for system vent valve and to add system flow diagram. The Committee recommended approval of this change.

75-420

M-43.25, Installation Of Steam Generator Handhole Cover, PCN75-720. J. Straight presented for review the following changes;

Delete steps 5.5 to 5.8
Change step 5.2 to read;
"use old gasket"

&

RF-8.3, Core Loading Periodic Equipment Status Check, PCN75-717, requested to N.A. the second RHR sample in step 5.1.3.

The Committee reviewed these changes.

75-423

J. Straight presented the following procedure changes. They all requested;

Step 6.54 to read "return rod mode selector switch on MCB to auto position (if desired)"

PT-6.3.1, Power Range Nuclear Instrumentation System, Channel 41, PCN75-739.

PT-6.3.2, Power Range Nuclear Instrumentation System, Channel 42, PCN75-740.

PT-6.3.3, Power Range Nuclear Instrumentation System, Channel 43, PCN75-741.

PT-6.3.4, Power Range Nuclear Instrumentation System, Channel 44, PCN75-742.

The Committee recommended approval of this change. This item IS COMPLETE.

75-424

S-4.1G, "A" Laundry & Hot Shower Tank Release To Discharge Canal, PCN75-737.

&

S-4.1H, "B" Laundry & Hot Shower Tank Release To Discharge Canal, PCN75-738.

J. Straight presented both procedure for the following change;

Change step 5.9 to add: or mark N.A. if followed immediately by a low activity waste condensate tank release.

The Committee recommended approval of this change. This item

PORC ITEM
NUMBER:

DESCRIPTION:

75-428

SM74-40.2, Installation of Voltmeter for indication of backup supply for Instrument Busses, A-B-C-D, PCN75-755. J. Straight presented this new procedure. This procedure is to describe the steps necessary to install a voltmeter for the backup power supply to instrument busses A-B-C-D. The Committee recommended approval of this procedure. This item is NOT COMPLETE pending completion of the job.

75-429

EM-139, Isolation And Maintenance Of Immediate Borate Valve MOV 350, PCN75-719 and PCN75-721. J. Straight presented these procedure changes.

PCN75-719, requested to add to step 5.1; 398A Closed _____

PCN75-721, requested to change step 5.2 to read;

Hold valve MOV 350. Remove motor leads and control leads from MOV350 (Be sure to maintain proper wire identification)

Step 5.2.1 Remove heat tracing from area of MOV350, if necessary (otherwise mark N.A.)

Step 5.6 Replace wiring into MOV 350 and reconnect heat tracing if removed

Step 5.6.1 Stroke valve to verify proper operation after removing holds from step 5.2

The Committee recommended approval of these changes.

75-430

SM75-23.1, Relocation of "A" & "B" Accumulator Level Transmitters, PCN75-767. J. Straight presented this new procedure. This procedure describe the steps to be followed for removal and reinstallation of the A and B Loop Accumulator Level Transmitters LT-934, LT-935, LT-938 and LT-939. It is necessary to relocate the level indicators to a higher elevation in the existing location to be able to meet the new Technical Specification requirements of at least 1100 cu. ft. of water in each accumulator. To register the proposed levels, the level transmitters must be raised 16.778 inches. The Committee recommended approval of this procedure.

<u>PORC ITEM NUMBER:</u>	<u>DESCRIPTION:</u>
75-432	<p><u>PT-14, Circulating Water Pumps - High Water Trip Logic, PCN75-758, and PCN75-761.</u> J. Straight presented this procedure change.</p> <p><u>PCN75-758</u> - was temporary for review only</p> <p><u>PCN75-761</u> - requested, to add to step 6.5.8 as follows:</p> <p style="padding-left: 40px;">"close states block #6 in THE terminal box"</p> <p style="padding-left: 40px;">also make additional changes and correction as per attached marked up copy (marked up copy incorporates PCN's 75-758 and 75-761)</p> <p>This was to be made permanent. The Committee reviewed PCN75-758 and recommended PCN75-761 be approved.</p>
75-438	<p><u>M-34, Removal & Reinstallation of CVCS Relief Valve 209, PCN75-774.</u> J. Straight presented the following procedure change. It requested;</p> <p>Step 5.2.17 to read; "Seal water heat exchanger inlet, valve 265"</p> <p>Add step 5.3.1; "Open seal water heat exchanger outlet, valve 321"</p> <p>Add step 5.3.2; "Open seal water heat exchanger drain, valve 282C"</p> <p>The Committee recommended approval of this change.</p>
75-439	<p><u>CP-465.0, Calibration and/or Maintenance Of Steam Flow Channel "465", PCN75-735.</u> J. Straight presented this procedure change. It was requested some figures be changed on the System Data Sheet. The Committee recommended approval of this change.</p>
75-440	<p><u>M-45.3, Inspection Of Non-Safeguard Motor For _____, PCN75-785.</u> J. Straight presented this procedure change. It requested the title of procedure be removed from first page, also step 5.2 should read;</p> <p style="padding-left: 40px;">"Uncouple or remove belts, from driven unit"</p> <p>along with several minor changes. The Committee recommended approval of this change.</p>
75-449	<p>J. Straight presented the following new procedures.</p> <p><u>E-6.1, Loss of Component Cooling During Power Operation, PCN 75-701.</u></p> <p><u>E-6.2, Loss of Component Cooling While Plant is Shutdown, PCN 75-702.</u></p> <p><u>E-6.3, Loss of Component Cooling During Recirculation Phase of Accident, PCN 75-703.</u></p>

PORC ITEM
NUMBER:

DESCRIPTION:

75-449
(Continued)

E-18.1, Loss of Spent Fuel Pit Cooling Due to the Failure of the Spent Fuel Pit Pump, PCN 75-725.

E-18.2, Loss of Spent Fuel Pit Cooling Due to the Failure of the Spent Fuel Pit Heat Exchanger, PCN 75-724.

E-39.0, Fire At Facility, PCN 75-825.

E-39.1, Containment Vessel Fire, PCN 75-826.

E-39.2, Relay/Computer Room Fire, PCN 75-827.

E-39.3, Grinnell Deluge System Fire, PCN 75-828.

E-39.4, Smoke Detector System Fire, PCN 75-829.

E-39.5, Sprinkle System Fire, PCN 75-830.

E-39.6, Auxiliary Building/Controlled Area Fire, PCN 75-831.

75-450

0-2.5; Plant Shutdown From Hot Shutdown to Cold Shutdown When Condenser Steam Dump Is Unavailable, PCN 75-876.

The purpose of this procedure is to describe alternate methods of cooldown in the event condenser steam dump is unavailable. The Committee recommended approval of this procedure.

75-451

S-3.2F, Draining the A, B, or C, CVCS Holdup Tank to the Waste Holdup Tank via the Sump Tank Pumps, PCN 75-954.

The purpose of this procedure is to outline the steps necessary to drain one of the CVCS Holdup Tanks to the waste holdup tank via the sump tank pump. The Committee recommended approval of this procedure.

0-1.17, Communications, PCN 75-953.

This procedure is to describe the communications available during a radiation emergency for use by the Emergency Coordinator, on-site and off-site emergency survey teams. The Committee recommended approval of these procedures.

75-453

J. Straight presented the following procedures for changes to comply with Item 4 of I.E. Bulletin 75-04A.

E-20, Immediate Boration, PCN 75-836.

E-7, Drop of a Full Length Rod Cluster Control Assembly, PCN 75-939.

E-19, Continuous Insertion of RCCA Control Bank, PCN 75-937.

E-22.2, Complete Loss of Changing Flow Due to Rupture of System, PCN 75-935.

E-26.1, Emergency Shutdown Resulting From A Reactor Trip From 8.5% Power or Loss, PCN 75-934.

Annual Report #11 of all procedure changes: (Continued) 1/1/75 to 12/31/75
Inclusive

<u>FORC ITEM NUMBER:</u>	<u>DESCRIPTION:</u>
75-453 (Continued)	<u>E-26.2, Emergency Shutdown Resulting from A Reactor Trip When Between 50% and 8.5% Power, PCN 75-933.</u> <u>E-26.3, Emergency Shutdown Resulting From A Reactor Trip When Operating at Greater Than 50% Power, PCN 75-932.</u> <u>A-17, Plant Operation Review Committee Operating Procedure, PCN 75-931.</u> <u>A-51, Plant System Modifications Activities, PCN 75-930.</u> <u>O-2.1, Normal Shutdown to Hot Shutdown, PCN 75-919.</u> <u>A-30, Plant Procedure Requirements, PCN 75-929.</u> <u>O-1.1, Plant Heat Up From Cold Shutdown to Hot Shutdown, PCN 75-909.</u> <u>O-2.2, Plant Shutdown From Hot Shutdown to Cold Condition, PCN 75-910.</u> The Committee recommended approval of these changes. This item <u>IS COMPLETE.</u>
75-454	<u>M-45.1A, Inspection of Safe Guard Motor For PCN 75-835.</u> J. Straight presented this temporary change for review. It requested step 5.15 be N/A'd. <u>ST-73-4, DB-75, DB-50, & DB-25, Circuit Breaker Overcurrent Trip Device Test and/or Replacement Procedure, PCN 75-801.</u> J. Straight presented this temporary procedure change. It was requested steps 4.1, 4.2, and 4.6 be N/A'd. <u>T-14B, Isolation and Restoration of 5-A Feedwater Heater, PCN 75-811.</u> J. Straight presented this temporary change. It requested steps 5.16 thru 5.31 be N/A'd. <u>T-14C, Isolation and Restoration of 5-B Feedwater Heater, PCN 75-812.</u> J. Straight presented this temporary change. It was requested steps 5.16 thru 5.31 be N/A'd. The Committee reviewed these changes.
75-456	<u>O-2.3, Plant at Cold Shutdown, PCN 75-911.</u> J. Straight presented this procedure change. It requested Step 5.3 change procedure M-1 to O-2.3.1. Step 5.4 change procedure M-4 to O-2.3.2 The Committee recommended approval of this change.

PORC ITEM
NUMBER:

DESCRIPTION:

75-457

E-5, Control Room Inaccessibility, PCN 75-918.

J. Straight presented this procedure change. It was requested in step 4.9 1985 psig be changed to 2235 psig, also in step 4.12, the following be added;

Refer to procedure, E-20, "Immediate Boration" if necessary.

The Committee recommended approval of this change.

75-458

E-17.1, Reactor Coolant Drain Tank Pump Operation For Core Cooling, PCN 75-917.

J. Straight presented this procedure change. It was requested this procedure be eliminated. It was the recommendation of PORC that this be disapproved and the procedure remain as is.

This item IS COMPLETE.

75-459

O-1.2, Plant From Hot Shutdown to Steady Load, PCN 75-920.

J. Straight presented this procedure change. It was requested a new step 5.5.16 be added and some minor changes made for axial flux difference.

The Committee recommended approval of this change. This item IS COMPLETE.

75-460

E-4, Station Blackout Operation, PCN 75-921.

J. Straight presented this procedure change. Changes were made to comply with I.E. Bulletin 75-04, Item 4, plus rewritten to conform to A-30 format.

The Committee recommended of these changes. This item IS COMPLETE.

75-461

T-32, Fire Service Water System, PCN 75-916.

J. Straight presented this procedure change. It was requested changes be made to incorporate additions to the system.

The Committee recommended approval of this change. This item IS COMPLETE.

75-462

E-28, Reactor Coolant Leak, PCN 75-757.

J. Straight presented this procedure which was revised and rewritten to A-30 format.

The Committee recommended approval of this change. This item IS COMPLETE.

75-494

E-17, Loss of Residual Heat Removal, PCN 75-936.

J. Straight presented this procedure change. It requested in step 1.1.1, FA-626 be changed to A-20, also in step 1.1.4 after trip alarm "J-9" be added.

The Committee recommended approval of this change.

PORC ITEM
NUMBER:DESCRIPTION

75-495

E-15.1, Malfunction of Pressurizer Power Relief or Safety Valves, PCN 75-938.

J. Straight presented this procedure change. This change was to include new pressure setpoints useful annunciators and improved idea for determining PRV leak.

The Committee recommended approval of this change.

75-496

EM-108, Repair of Leak on Letdown to Reactor Coolant Filter, PCN 75-871.

J. Straight presented this procedure change. It was requested;

Add New Steps:

5.5.1 Open Drain Valve 2237

5.5.2 Remove flange and open vent valve 2257

5.8.1 Close vent valve 2257 and drain valve 2237

The Committee recommended approval of this change.

75-497

M-28, Removal of Pressurizer Safety Valves, PCN 75-729, PCN 75-771.

J. Straight presented this procedure change. This was changed from M-28 to M-37.

PCN 75-729 - add NOTE to allow steps to be marked N/A if pressurizer has been previously purged.

PCN 75-771 - Add References;

2.1 Reactor Coolant System Engineering Flow Diagram

2.2 RSSP-11, Refueling Shutdown Surveillance Procedure, Pressurizer Safety Valve Test.

Change step no. D.3 to read; Pin spring restraint at discharge elbow.

Change step no. D.7 to read; Remove pin in discharge elbow spring restrain.

The Committee recommended approval of this change.

75-498

J. Straight presented the two following alarm cards for additions;

K-15, Fire System Tank Lo Pressure, PCN 75-915.

Requested a step 4 be added to read; 4. Verify motordriven and diesel driven fire pumps have started (85 psig for diesel)

K-31, Fire System Alarm Panel, PCN 75-914.

Requested to add to 4; 4. Refer to SC-3, Fire Emergency Plan, E-39, Fire Series and T-32, Fire Service Water System.

The Committee recommended approval of these changes.

PORC ITEM
NUMBER:DESCRIPTION

- 75-499 A-54.2, Ginna Station Staff Responsibilities for Fire Protection, PCN 75-928.
J. Straight presented this new procedure. The PORC discussed this procedure but recommended it be reviewed further and presented at a later date.
- 75-463 EM-140, Replacement Of Rod Drive Housing Cap, PCN75-802.
J. Noon presented this new procedure. This procedure is to describe the steps necessary for replacement of a rod drive housing cap. The Committee recommended approval of this procedure.
- 75-464 PT-32, Reactor Trip Logic Test on "A" or "B" Trains, PCN 75-874.
C. Hartlieb presented this procedure change. Changes were requested to provide steps to verify monitoring lights for 20ET and 20AST trip scheme, also Train A & B reactor trip breaker undervoltage scheme.

The Committee recommended approval of these changes.
- 75-465 M-11.12, Isolation of Safety Injection Pump for Maintenance or Inspection, PCN 75-873.
J. Noon presented this new procedure. The purpose is to outline a procedure for isolating a safety injection pump for maintenance or inspection.

The Committee recommended approval of this procedure.
- 75-468 J. Straight presented the following procedures for review:

RSSP2.6, Emergency Diesel Start & Breaker Closure Time, PCN75-793.
Requested to omit step 6.2.8.

PT-6.1A, Calibration Procedure - Source Range Channels, PCN75-745,
requested steps 12 through 12.17 be N.A.'d.

PT-32.1, Plant Safeguard Logic Test A or B Train, PCN75-872,
regarding steps 6.8 through 6.11, "all test switches concerned with logic development for the main S.I. logics are to be placed in test position".

RF-10.0, Fuel Transfer System Checkout & Demonstration Procedure, PCN75-605 - requested steps 5.13.1, 5.13.15 and 5.14.5 be N.A.'d.

PCN75-558 - step 5.13 to be performed with a visual check before flooding and the first element with an RCC will be used to verify the check. This is to be made permanent. The Committee reviewed these items. This item IS COMPLETE.

PORC ITEM
NUMBER:DESCRIPTION75-373

M-43.15, Reinstallation of Steam Generator Manway Insert and Cover Closure, PCN 75-697. J. Straight presented this procedure change. Several changes were requested to provide more information and better guidelines for torquing.

The Committee recommended approval of these changes.
IS COMPLETE.

75-469

E-22.1, Malfunction Of The Charging System, PCN75-764. J. Straight presented this new procedure. The Committee recommended this procedure for approval.

E-22.3, Malfunction Of The Letdown System, J. Straight presented this new procedure. The Committee recommended approval of this procedure also the deletion of;

E-22, Malfunction Of Charging Or Letdown, as it has been rewritten into two new procedures.

75-470

PT-23.5A, Containment Isolation Valve Leak Rate Testing Containment Sump Recirculation To "A" RHR Pump, PCN75-743. J. Straight presented this procedure change. It requested step 6.4 to read;

NOTE: Piping flanges must be tight to insure zero leakage between flange and gasket

The Committee recommended approval of this change.

75-473

EM-107, Inspection & Repair Of Safety Injection System Relief Valve 887, PCN75-640. J. Straight presented this procedure change. It was requested step 5.2; change valve number to 2918, step 5.3, change valve number to 2918 also in step 5.6.1. The Committee recommended approval of these changes.

75-474

P-2, Reactor Coolant System Precautions & Limitations, PCN75-805. J. Straight presented this procedure change. This change requested new setpoints be incorporated. The Committee recommended approval of this change.

75-475

RSSP-11, Pressurizer Safety Valve Test, PCN75-773. J. Straight presented this procedure change. It was requested to remove check-off lines and sign-off lines on the procedure, except the data sheets, also change Item 3 to read;

"Valve Set Press, 2485 psig"

The Committee recommended disapproval of the change to removed check-off lines and recommended approval of item 3.

PORC ITEM
NUMBER:DESCRIPTION

- 75-476 GS-5, Security Guard General Instructions, PCN75-794. J. Straight presented this procedure change. It requested to;
- Add; 3.7.1, Maintain appropriate log sheets to ensure accountability of all personnel within the protected area
- The Committee recommended approval of this addition.
- 75-477 GS-10, Security Guards Actions During A Radiation Emergency, PCN75-795. J. Straight presented this procedure change. It requested to;
- Add: 3.6.1 If an emergency occurs between the hours of 0001 and 0800, include log sheets and sign-in sheets for previous day.
- 3.7 If required, verify attendance reports submitted to the Emergency Coordinator to determine the necessity of search and rescue operations.
- 3.7.1 Verification shall be determined through a check of appropriate logs and sign-in sheets.
- The Committee recommended approval of these additions.
- 75-478 O-1.1, Plant Heat-Up From Cold Shutdown, PCN75-806. J. Straight presented this procedure change. It requested in step 5.7 to change it to read;
- "RCS Filled and Vented (O-2.3.2)"
- The Committee recommended approval of this change.
- 75-480 SM-75-1.2, Instrument Cable Rerouting, PCN 75-722. J. Straight presented this procedure change. It was requested in step E.6 to delete the word "pulled" also to change step E.5 to read; "All conduit, junction boxes, and supports outside of containment have been installed and cable has been pulled, all in accordance with B.9, B.11 and B.12.
- The Committee recommended approval of these changes.

PORC ITEM
NUMBER:

DESCRIPTION

- 75-481 PT-5.30, PT-5.40, PT-5.20, PT-5.10, Process Instrumentation Reactor Protection Channel Trip Test (Channel 1), PCN75-807, PCN75-808, PCN75-809, and PCN75-810. J. Straight presented these procedure changes. These changes requested changes in setpoints. The Committee recommended approval of these changes.
- 75-483 O-2.1, Normal Shutdown To Hot Shutdown, PCN75-834. J. Straight presented this procedure change. It requested;
add note between steps 5.16 and 5.17 as follows;

NOTE: Immediately upon opening 1G1372, verify that generator voltage is less than 19 KV. If not adjust with base adjuster.

The Committee recommended approval of this change.
- 75-484 RSSP-1.4, Valve Interlock Verification - Reactor Coolant System, PCN75-824. J. Straight presented the following procedure changes. It requested in step 4.1 a sign-off line be added, also several setpoints be changed in the procedure along with several new steps added. The Committee recommended approval of these changes.
- 75-491 The following SC series were presented;

SC-1.1, Radiation Emergency Procedures General Information, PCN75-904. This procedure is to provide information of a general nature which pertains to Local, Site and General Emergency procedures.

SC-1.2, Local Radiation Emergency Procedure, PCN75-899. This procedure is to outline action required in the event that a Local Radiation Emergency is declared.

SC-1.3A, Site Radiation Emergency (Shift Foreman & Control Room), PCN75-901. This procedure is to establish the duties performed and the actions required by the on-duty shift-foreman and operations for a Site Radiation Emergency.

SC-1.3B, Site Radiation Emergency, (Emergency Coordinator & Survey Center Assignees) SC-1.3B, PCN75-883. This procedure is to establish the duties performed and the actions required by the Emergency Coordinator and the Survey Center Assignees.

SC-1.3C, Site Radiation Emergency, (Non-Operating Personnel) PCN75-912. This procedure is to outline the duties and responsibilities of non-operating personnel during a Site Radiation Emergency.

PORC ITEM

NUMBER:DESCRIPTION

75-491 (Cont'd) SC-1.4, General Radiation Emergency, PCN75-903. This procedure is to provide instructions for actions to be taken in the event of a General Radiation Emergency.

SC-1.5, Evacuation Procedure, PCN75-898. This procedure is to outline the evacuation procedures to be followed for local or site radiation emergencies.

SC-1.6, Major Release To The Lake, PCN75-897. This procedure is to provide instruction for steps to be taken if a release exceeds the maximum permissible concentration as specified in 10 CFR 20.

The Committee recommended approval of the above procedures with modifications.

75-492

SC-1.7A, Emergency Radiation Monitoring Procedure, PCN75-913.

This procedure is for the prime objective of Radiation on-site and off-site Monitoring Teams is to rapidly survey areas downwind of the plant site in order to determine the extent and magnitude of any uncontrolled release of radioactive material following an incident. It should be stressed that the initial off-site survey is of great importance, decisions regarding extent and types of protective actions required will be based upon data reported by survey teams.

SC-1.7B, Field Determination of Iodine or Particulate, PCN75-903.

This procedure is for the determination of the iodine concentration so as to relate to inhalation and resultant thyroid dose is extremely important during the first 2 hours after a release. The information is required with as little delay as possible. Therefore, the procedure consists of simple methods with approximate results and expected loss of sensitivity, subsequent surveys can be made to obtain more accurate information.

SC-1.7C, Monitoring Site Radiation Level By TLD, PCN75-900. This procedure describes the use of thermo-luminescent dosimeters, TLD, to determine the radiation level at the site boundary and at the site environmental stations.

SC-1.8, Post Accident Environmental Sampling, PCN75-884. This procedure is to be used in the event of an emergency, the normal environmental sampling program is suspended by a more intensive and specific program of sample collection aimed at determining as soon as possible, the extent of the emergency and its effect on the environment. This emergency program utilizes the on site and off site continuous monitoring stations. In addition grab samples will be collected from the down wind direction as deemed necessary, such as soil, crops, milk or water. The nature of the emergency and environmental conditions will indicate the extent of sampling and the locations from which samples will be collected.

SC-1.9, Post Accident In-Plant Radiation Monitoring, PCN75-896. This procedure is to establish the guidelines for in-plant radiation monitoring during a radiation emergency.

SC-1.10, Planning Conduct, and Evacuation of A Radiation Emergency Drill, PCN75-895. This procedure is to provide a means of testing evaluating and documenting the response of all participating off-site agencies and/or plant personnel during conduct of a "Radiation Emergency Plan" drill.

PORC ITEM
NUMBER:DESCRIPTION

- 75-492 (Cont'd) SC-1.11A, Immediate Re-Entry, PCN75-893. This procedure is to provide instructions for immediate entry for search and rescue operations or other needs as determined necessary by the Emergency Coordinator.
- SC-1.12A, Immediate Call List, PCN75-892. This is to provide an immediate call list and associated instructions.
- SC-1.12B, Station Call List, PCN75-891. This procedure is to provide the Emergency Coordinator with a list of phone numbers for station personnel.
- SC-1.12C, Specialized Call List, PCN75-890. This procedure is to provide the Emergency Coordinator with a listing of names and phone numbers of off-site personnel and facilities that may be required for special assignments during a Radiation Emergency.
- SC-1.12D, Management Call List, PCN75-889. This procedure is to provide the Vice President of Electric and Steam Production and Division Superintendent of Electric and Steam Operations a readily available listing of phone numbers to be used during a Radiation Emergency.
- SC-1.13, Estimating Off-Site Doses, PCN75-905. This procedure is to provide estimates of the dose in the areas around the plant and guidance for the selection of sampling locations. Information is needed early to decide what action must be taken to limit the exposure of the general public. In the period before state and federal agencies arrive at the site to provide assistance, steps must be taken to define the affected areas, assess the extent and significance of the release and provide data on which appropriate protective actions can be based.
- SC-1.14A, Accountability Of Personnel, PCN75-887. This procedure is to provide instruction for accountability of personnel within the plant protected area in the event of plant evacuation.
- SC-1.14B, Search/Rescue Operation, PCN75-888. This procedure is to provide instructions to institute a search and rescue operation.
- SC-1.15, Inspection Of Emergency Equipment, PCN75-885. This procedure outlines the equipment required by the emergency plan and the means of assuring it is available.
- SC-1.16, Transfer of Emergency Survey Center To Station 13A, PCN75-886. This procedure is to describe the reasons for and the steps required to transfer equipment and personnel from the Emergency Survey Center to the Auxiliary Emergency Survey Center, (Station 13A).

The Committee recommended approval of these procedures with some modifications.

PORC ITEM

NUMBER:DESCRIPTION

75-502 O-2.4, Plant Shutdown From Hot Shutdown To Cold Shutdown, PCN75-875.
J. Straight presented this procedure change. It requested the following changes;

Step 4.7 - eliminate reference to A-15 add to steps 4.5 and 4.7 reference to TS 3.1.2 Heatup and Cooldown

Step 4.6 note: change 1800 to 2000 psig
Step 4.12, 5.8, 5.9 note only change 1800 to 2000 psig
Step 5.11, change pressurizer no load level to 19.5 % and lab seal A P to 35"
Step 5.5 add, Refer to O-2.5 if atmospheric steam dump is unavailable.

The Committee recommended approval of these changes.

75-503 S-3.1B, Pre-Operational Line-Up Of Boric Acid System, PCN75-752 & PCN75-870. J. Straight presented this procedure change. It requested;

PCN75-752 - step 6 should leave valves 827A and 827B closed

also to update to A-30 format.

PCN75-870 - requested to omit step 19

The Committee recommended approval of these changes.

75-504 RSSP-2.3, Emergency Diesel Generator Annual Trip Testing, PCN75-941, PCN75-942. J. Straight presented this procedure change.

PCN75-941 - requested under Initial Condition 4.4 to delete (PT-12A or PT-12B) change to (T-27.1 or T-27.2)

PCN75-942 - requested changes be made to the procedure to reflect addition of new governor booster.

The Committee recommended approval of the change.

75-507 SC-1.9, In-Plant Radiation Monitoring, PCN75-963. J. Straight presented this procedure change. It requested a new step 3.1.2.4 be added to read;

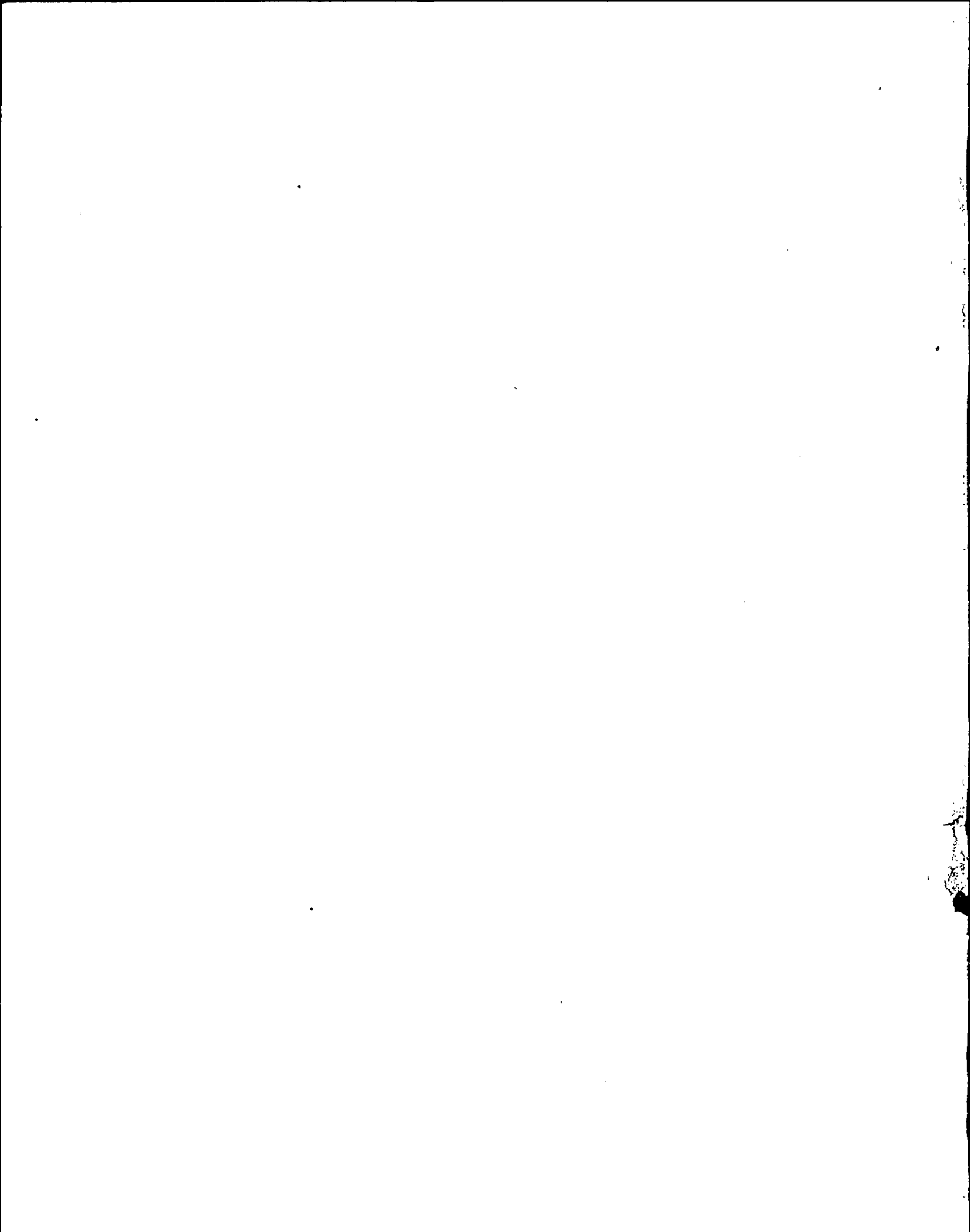
"Survey Meter"

The Committee recommended approval of this change.

75-508 SC-1.10, Planning Conduct and Evaluation of A Radiation Emergency Drill, PCN75-964. J. Straight presented this procedure change. It requested to change existing step 3.1 to 3.2 and add step 3.1 to read;

"A planned site radiation emergency drill involving on-site personnel shall be conducted annually"

The Committee recommended approval of this change.



PORC ITEM
NUMBER:

DESCRIPTION

- 75-509 SC-6, Action For Hazardous Chemical Spill, PCN75-927. J. Straight presented this new procedure. The purpose of this procedure is to outline actions to be taken in case of a major spill of hazardous chemicals. The Committee recommended approval of this procedure. item IS COMPLETE.
- 75-510 M-32.1, DB-25, DB-50, and DB-75 Circuit Breaker Overcurrent Trip Device Test and/or Replacement Procedures, PCN75-677. J. Straight presented this procedure change. This procedure was ST73-4 changed to M-32.1. This procedure is to perform maintenance and overcurrent trip device test and/or replacement on DB-25, DB-50 and DB-75 air circuit breakers.
- S-2.1, Reactor Coolant Pump Operation, PCN75-965.
- &
- S-2.2, Pressurizer Pressure and Spray Control, PCN75-969.
J. Straight presented these procedures for the same change. It requested a new heat-up and cooldown curve page be added. Also to add a note to the procedures.
- 75-511 J. Straight presented these procedures for changes:
- O-1.1, Plant Heatup From Cold Shutdown To Hot Shutdown, PCN75-968.
- O-2.2, Plant Shutdown From Hot Shutdown To Cold Condition, PCN75-967.
- O-2.4, Plant Shutdown From Hot Shutdown To Cold Shutdown, PCN75-966.
- It was requested to add a new heatup and cooldown curve page to these procedures. The Committee recommended approval of this change.
- 75-512 H-6, Component Cool Service Service Water Lo Flow, PCN75-837.
J. Straight presented this procedure change. It was requested the alarm setpoint be changed to 1000 gpm. The Committee recommended approval of this change.
- 75-513 SC-1.16, Transfer Of Emergency Survey Center To Station 13A, PCN75-973.
J. Straight presented this procedure change. It was requested;
- Steps 3.1.1.2 and 3.1.1.4 change "cars" to "vehicles" also to add new step;
- 3.1.1.3 - Vehicles greater than 2000 cpm shall be decontaminated prior to exiting plant site. The Emergency Coordinator may contact the Ontario Fire Company, (Refer to SC-1.12C Specialized Call List) to assist working down of the vehicles. A hydrant is located at the corner of the plant driveway and Lake Road.
- The Committee recommended approval of these changes.

PROC ITEM
NUMBER:

DESCRIPTION75-514

SC-1.6, Major Release To The Lake, PCN75-974. J. Straight presented this procedure change. It was requested to insert the following;

- 3.3 Obtain a sample from the sytem or source of the effluent that was being discharged to be analyzed by the chemist.
- 3.4 Record the estimated dilution flow (for example; one or two circulating pump flow) and estimated quantity of effluent.

Renumber steps accordingly.

The Committee recommended approval of these changes.

75-515

SC-1.7B, Field Determination Of Iodine On Particulate, PCN75-975. J. Straight presented this procedure change. It requested;

add the following note after step 3.3.4 and 3.4.3;

NOTE: The possibility exists that the background may be too high to determine a reading on the filter. If this condition exists, an alternate location should be determined to obtain filter and background readings.

The Committee recommended approval of these changes.

75-516

SC-1.14A, Accountability Of Personnel, PCN75-976. J. Straight presented this procedure change. It was requested;

Add to step 3.3: In addition, the Emergency Coordinator will:

- 3.3.1 - Notify the Control Room to have the unaccounted personnel paged on the plant P.A. System
- 3.3.2 - Consider the possibility that the unaccounted personnel may have left the site. The guard stationed at the plant driveway may be interviewed to determine if he had witnessed any personnel leaving, or possibly a telephone call to the individuals home may be considered.

The Committee recommended approval of these changes.

75-517

SC-1.14B, Search And Rescue Operation, PCN75-977. J. Straight presented this procedure change. It requested;

Add new step 3.1.4

"assume an Emergency Medical Treatment situation may exist and alert the plant doctors. Refer to SC-1.12C, Specialized Call List"

PROC ITEM
NUMBER:

DESCRIPTION75-518

SC-1.3B, Site Radiation Emergency, (Emergency Coordinator & Survey Center Assignees), PCN75-979. J. Straight requested this procedure change. It requested to add to step 6;

"To assure personnel accountability, enter names in the emergency log of individuals assigned and the teams to which they are assigned"

Also to add the following after step 17;

The following routes shall be used as determined by Emergency Coordinator:

1. Lake Road East: When wind is from the east
2. Lake Road West: When the wind is from the west
3. Route 350 South: When the wind is from the south
4. Lake Road East and West: When the wind is from the north

The Committee recommended approval of these changes.

75-519

SC-1.12C, Specialized Call List, PCN75-980. J. Straight presented this procedure change. It requested title or names be added to some of the steps. It also requested the following new steps be added:

- 3.1.8 Nuclear Regulatory Commission
 - 3.1.8.1 Nuclear Regulatory Commission 7-9-215-337-1150
Region I
King of Prussia, Pa.
 - 3.1.8.2 Radiation Assistance Program 7-9-516-345-2200
Nuclear Regulatory Commission
Brookhaven Area

The Committee recommended approval of these changes.

75-520

S-7J, Transferring & Purification Of R.W.S.T. Water From R. W. S. T. To Refueling Cavity, PCN75-906. J. Straight presented this procedure change. It was requested in title-after last two words (refueling cavity), add "or Primary Loop". The Committee recommended this be reviewed only and disapproved.

75-534

EM-141, Excess Letdown Line Relocation, PCN 75-1025. J. Straight presented this new procedure. This procedure is to describe the steps necessary to relocate the excess letdown line to allow sufficient rattle space at the "A" Steam Generator platform. The Committee recommended approval of this procedure.

PROC ITEM
NUMBER:

DESCRIPTION

- 75-536 SM-75-4.1, Reroute of Steam Generator Blowdown Line, PCN 75-987.
J. Straight presented this temporary change for review:
- Step E.13, Change words to read as follows:
-Work Procedure (WP004), with the exception of the tie-ins of blowdown and drain lines to two 8" header to blowdown tank which were completed and accepted by RG&E, has been inspected -----
- The Committee reviewed this change.
- 75-538 S-4.1E, Waste Condensate Release, PCN 75-972. J. Straight presented this procedure change. It requested the following be added to step 5.13:
- "If DI's are not available, mark steps 5.13 through 5.28 N/A and transfer to WHT _____"
- The Committee recommended approval of this change.
- 75-539 T-27.2, 1B Emergency Diesel Generator Pre-Start Alignment, PCN 75-986.
J. Straight presented this procedure change. It requested:
- Step 6.3.8.1 should read15 on scales _____ .8 on indicator
- Step 6.3.8.2 should read - Speed droop setting 5.2
- The Committee recommended approval of this change.
- 75-541 PT-7, Hydro Test of Reactor Coolant System, PCN 75-990.
J. Straight presented this procedure change. It requested to delete Step 7 from Primary System Hydro-Checklist. The Committee recommended approval of this change. This item is complete.
- 75-542 GS-10.1, Security Guard Action During A Local Radiation Emergency, PCN 75-981. J. Straight presented this new procedure. This procedure is to outline action required by the Security Force in the event of a local radiation emergency. The Committee recommended approval of this new procedure.
- 75-544 E-29.1, Loss of Normal Feedwater (Not Recoverable), PCN 75-952.
J. Straight presented this procedure change. It requested a note be added after 3.2 to read:
- NOTE: If all rods are not known to be fully inserted after a reactor trip refer to E-20, "Immediate Boration."
- The Committee recommended approval of this change.

PROC ITEM
NUMBER:DESCRIPTION

- 75-546 E-11, Continuous Rod Withdrawl, PCN 75-951. J. Straight presented this procedure change. It was requested a new step 3.2.1 be added to read:
- "If all control rods are not known to be fully inserted following a manual reactor trip refer to procedure E-20 "Immediate Boration."
- Several other changes were requested to comply with I.E. Bulletin 75-04A, Item 4. The Committee recommended approval of these changes.
- 75-547 M-56.1, Removal & Replacement of Penetration Seals, PCN 75-1056
T. Meyer presented this new procedure. This procedure is to describe removal and placement of penetration seals using ceramic fiber material. The Committee recommended approval of this procedure with modifications.
- 75-548 SM 75-32, Installation of Seals With Bisco Silicone Foam, PCN-75-1055. J. Scheetz presented the design criteria to the Committee for review on this SM.
- Paul Wilkins presented the safety analysis. The Committee performed the safety evaluation and found that the modification does not involve an unreviewed safety question and that Technical Specifications are not effected.
- J. Larizza presented the SM procedure. The Committee recommended approval of this SM with modifications.
- 75-555 SM75-23.1, Relocation Of "A" And "B" Accumulator Level Transmitters, PCN75-880. J. Straight presented this procedure change for review. It was requested;
- For SM75-23.1 usage on LT-935 and LT-938 change 5.4 to read;
- "and at bottom"pipe" of level transmitter , just above floor sleeve".
- Also modify weld job traveler to include the above.
- The Committee reviewed this change.
- 75-558 S-4.1C, Liquid Waste Process Operation And Shutdown, PCN75-800.
J. Straight presented this procedure change. It requested to add to Initial Conditions the following;
- "If Evaporation is shutdown for reason other than drumming N.A. steps 5.2 and 5.9 and state reason for shutdown on data log sheet"
- The Committee recommended approval of this change.

PROC ITEM
NUMBER:

DESCRIPTION

75-559 S-16A,
Safety Injection System Alignment, PCN75-957, 956, 961, 985,
and 983. J. Straight presented these procedure changes.

PCN75-956 - requested to rearrange present steps to reflect more accurate locations;

(1st) Auxiliary Building

(2nd) Intermediate Bldg. by Sample Rm.

5.2.1.30

5.2.1.28

5.2.1.29

(3rd) Intermediate Bldg. by Aux. F.W. Pump

5.2.1.25

5.2.1.26

5.2.1.27

(4th) Auxiliary Bldg. Gas Storage Room

5.2.1.21 thru 5.2.1.24

PCN75-957 - requested

step 5.2.1.16 make V-1816 to read V-1816A

step 5.2.1.26 make V-1819 to read V-1819G

PCN75-961 - requested

step 5.7.13 should read:

5.7.13 | 878D | 1D | 8C _____

step 5.7.14 should read:

5.7.14 | 878D | 1D | 8F _____

PCN75-985 - requested to change;

Steps 5.2.1.16 - should be V1816A

5.2.1.26 - should be V1819G

5.2.1.27 - should be V1819F

PCN75-983 - was an explanation of steps marked with astrisk and crossed out (different) valve position.

The Committee reviewed and approved these changes.

75-560

0-2.3.2, Filling And Venting The Reactor Coolant System, PCN75-1000.

J. Straight presented this procedure change. It was requested to;

Add to step 5.5.10; Root valve outlet R.C.D.T. to vent header outside CV

Valve 1716A Open _____

The Committee recommended approval of this change.

PROC ITEM
NUMBER:

DESCRIPTION

75-561

S-8A, Component Cooling Water System Start-Up And Normal Operation Valve Alignment, PCN75-962. J. Straight presented this procedure change. It requested;

Change 5.10.2 to read; CCW Supply to waste evaporator air ejector ---2617 open _____

Change existing step 5.10.2 to 5.10.3, 5.10.3 to 5.10.4, 5.10.4 to 5.10.5 and 5.10.5 to 5.10.6.

The Committee recommended approval of this change.

75-562

RSSP2.2, Diesel Generator Load & Safeguard Sequence Test, PCN75-999, PCN75-1011, PCN75-1020. J. Straight presented this procedure change.

PCN75-999 - requested step 6.5.5.2 to read;

"1B Pump recirculation valve AOV 4310 Open"

PCN75-1020 - was to facilitate steps 6.6 through 6.23.10 N.A.'d.

PCN75-1011 - requested;

- 1) Test Requirement 2.9.1, Delete existing requirement and insert the following: MOV 871A starts to close in 3.5 seconds (+ 1 second) upon failure of 1B SIP to start, after loss of bus voltage and receipt of SI signal.
- 2) Test Requirement 2.9.2, Delete existing requirement and insert the following: MOV 871B starts to close in 3.5 seconds (+ 1 second) upon failure of 1A SIP to start, after loss of bus voltage and receipt of SI signal.

The Committee recommended approval of these changes.

75-563

S-4.2.1, Waste Gas System Valve Alignment For Automatic Operation, PCN75-1001. J. Straight presented this procedure change. It requested a new step be added;

5.33.1 - RCDT outlet isolation Valve, outside C.V. Valve 1716A
Open _____

The Committee recommended approval of this change. This item IS COMPLETE.

75-564

SM75-1.2, Instrument Cable Rerouting, PCN75-1004. J. Straight presented this procedure change. It was requested;

Step E.9, Delete sign-off of GAI Q.C. Supervisor and add sign-off for RC&E Q.C. Engineer

Step E.14, Delete word "contractor"
Delete sign-off of GAI, Q.C. Supervisor and add sign-off for RC&E Q.C. Engineer

PROC ITEM
NUMBER:

DESCRIPTION

75-565

M-37.20, Inspection & Repair of Main Steam Isolation Valves, PCN75-1005. J. Straight presented this procedure change. It requested;

Step 5.1.9 should read: Replace or repair the parts found defective. Note all replacement or repaired parts on attached report.

The Committee recommended approval of this change.

75-568

M-37.21, Inspection & Repair Of Main Steam Check Valves, PCN75-1006. J. Straight presented this procedure change. It requested step 5.1.8 to read;

"Replace or repair any of the parts found defective, note all replacement or repaired parts on the attached report"

The Committee recommended approval of this change.

75-572

J. Straight presented the following two procedures for temporary changes to be made permanent.

PT-25, Containment Air Filter Post Accident Charcoal Filter By-Pass Flow, PCN75-1009. This requested after step 6.1 a note be added to read;

NOTE: IF only one unit is to be tested, mark other one N.A.

M-2A, Resin Replacement & Conditioning A and B Mixed Bed Demineralizers, PCN75-1010. It requested steps 5.9 through 5.9.9 be omitted, also a note be added to read;

NOTE: If it is desired to use excess lithium to raise the lithium concentration in the RCS the following steps to be marked N.A.

The Committee recommended that these temporary changes be made permanent.

75-573

PT-7, Hydro Test Of Reactor Coolant System, PCN75-1021. J. Straight presented this procedure change. It requested to;

Change step 4.2 to read "operating procedure b-1.1 is done up to step 5.36"

The Committee recommended approval of this change.

FORC ITEM
NUMBER:

DESCRIPTION75-574

J. Straight presented the following new procedures;

E-1.1, Safety Injection System Actuation, PCN75-865.

E-1.2, Loss of Coolant Accident, PCN75-866.

E-1.3, Steam Line Break Accident, PCN75-867.

E-1.4, Steam Generator Tube Rupture Accident, PCN75-868.

The Committee recommended approval of these new procedures.

75-575

E-1, Accident Symptoms & Detailed Recovery For Loss Of Coolant, Steam Line Break & Steam Generator Tube Rupture, PCN75-207, PCN75-173, PCN74-720, PCN74-721, PCN74-722, PCN74-723. J. Straight presented these PCN's for several changes which was incorporated into the new E-1 series. The Committee recommended approval of these changes.

PCN75-869 - requested the E-1 procedure be deleted and incorporated into E-1.1, E-1.2, E-1.3 and E-1.4.

The Committee recommended approval of the deletion.

75-577

O-1.1, Plant Heat-Up From Cold Shutdown To Hot Shutdown, PCN75-1008. J. Straight presented this procedure change. It was requested;

Add to step 5.6, "The steam generator blowdown mode selector switch is in the AUX position"

The Committee recommended approval of this change.

75-581

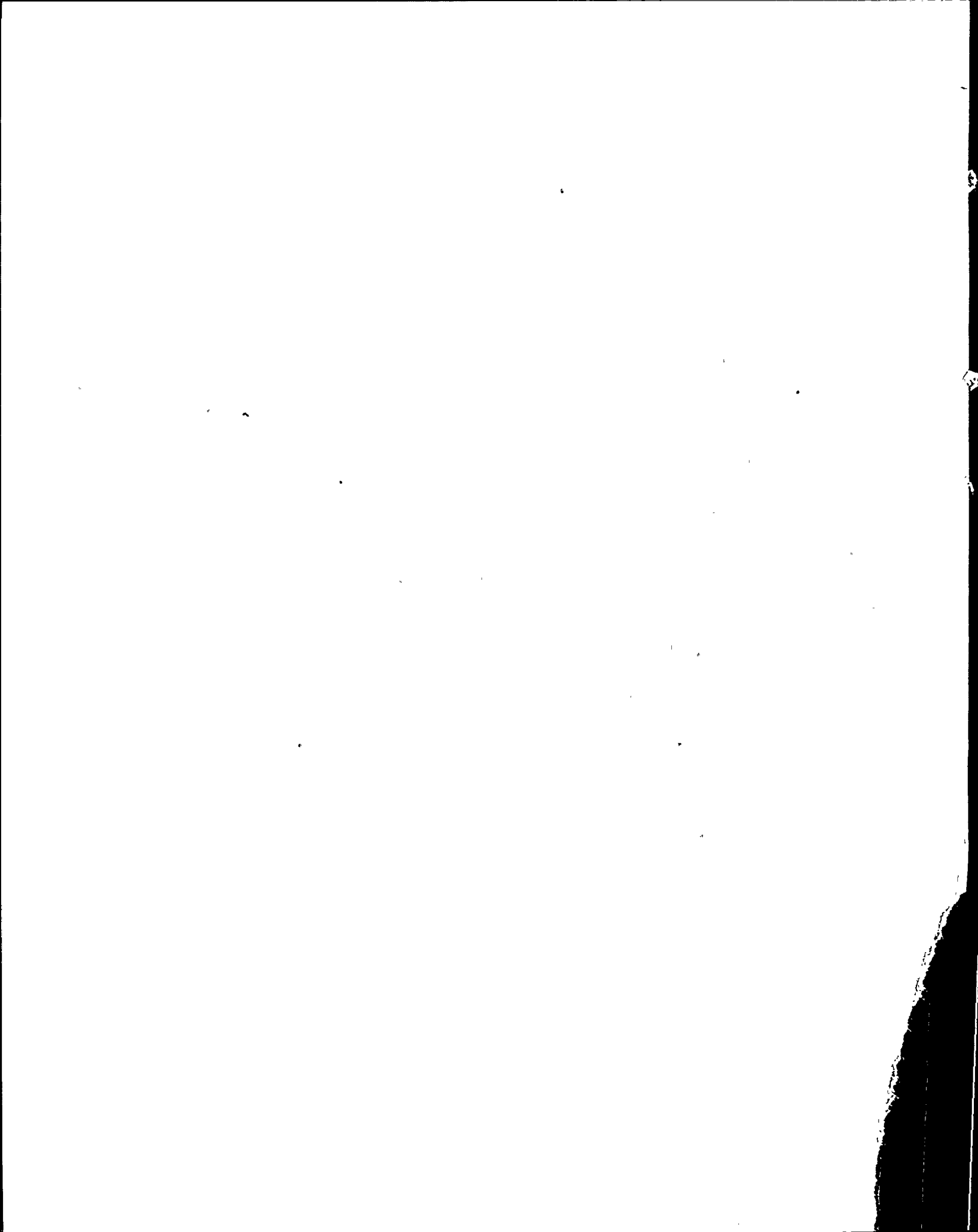
S-4.2.1, Waste Gas System Valve Alignment For Automatic Operation, PCN75-1029, PCN75-1030.

PCN75-1029 - requested steps 5.45 through 5.50 be N.A.'d. This was a temporary request which the Committee reviewed.

PCN75-1030 - requested the following changes;

Eliminate steps 5.45, 5.45, 5.46
Change step 5.47 valve number from 1674B to 1875
Change step 5.49 valve number from 1674D to 1830
Eliminate step 5.50

The Committee recommended approval of this change.



PROC ITEM
NUMBER:

DESCRIPTION75-582

RSSP-2.7, Safety Injection Sequence Timers, Train A and B, PCN75-996. J. Straight presented this procedure change. It requested all $\pm .5$ sec. tolerances in steps 6.14 and 6.33 be changed as follows:

The $+ .5$ second tolerance should be changed to $+ 0$ second and the $-.5$ second tolerance should remain as is.

The Committee recommended approval of this change.

75-583

O-1.2, Plant From Hot Shutdown To Steady Load, PCN75-1018. J. Straight presented this procedure change. It requested to; add step 5.5.6 as follows:

step 5.5.6 at 100 MW close the vents from the Main Steam to reheaters piping to the condenser.

1A Reheater Main Steam Vent	Closed	_____
1B Reheater Main Steam Vent	Closed	_____
2A Reheater Main Steam Vent	Closed	_____
2B Reheater Main Steam Vent	Closed	_____
Vent piping condenser isolation valve	closed	_____

Renumber existing steps accordingly.

The Committee recommended approval of this change.

75-586

A-52.2, Control of Locked Valve Operation, PCN 75-1102. J. Straight presented this procedure change. It requested to add the following valves:

4070A Condensate Storage Tank B - Makeup to Hotwell - Open
4071A Condensate Storage Tank A - Makeup to Hotwell - Open
4071B "Makeup Line to Hotwell Drain - Closed

The Committee recommended approval of this change.

PROC ITEM
NUMBER:DESCRIPTION75-588

T-41A, Alignment of Auxiliary Feedwater System Prior To Power Operation, PCN 75-1104. J. Straight presented this procedure change. It requested to add the following steps:

- | | | | |
|---------|---|-------------|-------|
| 5.4.11A | Condensate Storage Tank Drain V-4073 | Lock Closed | _____ |
| 5.5.11B | Condensate Storage Tank Drain V-4072 | Lock Closed | _____ |
| 5.8.1 | Condensate Makeup Line to Hotwell Drain
V-4071B
(Located in Sub-basement) | Lock Closed | _____ |
| 5.5.2 | Condensate Storage Tank B Makeup to Hotwell
V-4070A | Lock Open | _____ |
| 5.4.2 | Condensate Storage Tank A Makeup to Hotwell
V-4071A | Lock Open | _____ |

The Committee recommended approval of these changes.

75-593

0-1.1, Plant Heatup From Cold Shutdown to Hot Shutdown, PCN 75-1032. J. Straight presented this procedure change for review only. It requested the following changes:

Delete Step 5.3.1 and move that statement to next page under new Step 5.37.2.

5.30 Set non-regenerative heat exchanger temperature controller TCV-130 to maintain 127°F on manual.

The Committee reviewed these changes.

75-594

0-1.2, Plant From Hot Shutdown to Steady Load, PCN 75-1072. J. Straight presented this procedure change. It requested:

Insert additional step after Steam Generator Feedwater pump is started to return jumper switch to normal and return key to Shift Foreman.

The request was modified to insert this step just prior to the pump start, since that position is necessary to start the pump.

The Committee recommended approval of this change.

75-595

0-2.1, Normal Shutdown to Hot Shutdown, PCN 75-1071. J. Straight presented this procedure change. It requested:

Insert provision to place blowdown jumper switch to the Auxiliary position.

The Committee recommended approval of this change.

PROC ITEM
NUMBER:

DESCRIPTION

- 75-596 O-2.2, Plant Shutdown From Hot Shutdown to Cold Shutdown, PCN 75-1070. J. Straight presented this procedure change. It requested:
- Include provision to place blowdown jumper switch in defeat position.
- Make some minor typing changes.
- The Committee recommended approval of these changes.
- 75-598 PT-23.1, Containment Isolation Valve Leak Rate Testing Pressurizer Relief Tank Gas Analyzer Line, PCN 75-1043. J. Straight presented this procedure change. It requested change of formula on Page 3. The Committee recommended approval of this change.
- 75-599 PT-23.2, Containment Isolation Valve Leak Rate Testing Nitrogen Supply To Pressurizer Relief Tank Line, PCN 75-1044. J. Straight presented this procedure change. It requested change of formula on Page 4. The Committee recommended approval of this change.
- 75-600 PT-23.7, Containment Isolation Valve Leak Rate Testing Letdown From Reactor Coolant System, PCN 75-1042. J. Straight presented this procedure change. It requested a change of formula on Page 5. The Committee recommended approval of this change.
- 75-601 PT-23.8, Containment Isolation Valve Leak Rate Testing RCS Charging Line, PCN 75-1049. J. Straight presented this procedure change. It requested a change of formula on Page 4. The Committee recommended approval of this change.
- 75-602 PT-23.10, Containment Isolation Valve Leak Rate Testing Alternate Charging Line, PCN 75-1046. J. Straight presented this procedure change. It requested a change of formula on Page 4. The Committee recommended approval of this change.
- 75-603 PT-23.11, Containment Isolation Valve Leak Rate Testing Alternate Charging Line, PCN 75-1045. J. Straight presented this procedure change. It requested a change of formula on Page 5. The Committee recommended approval of this change.
- 75-604 PT-23.12A, Containment Isolation Valve Leak Rate Testing Pressurizer Steam Space Sample, PCN 75-1041. J. Straight presented this change. It requested a change of formula on Page 4. The Committee recommended approval of this procedure.

PROC ITEM NUMBER:	DESCRIPTION
<u>75-605</u>	<u>PT-23.9A, Containment Isolation Valve Leak Rate Testing "A" R.C.P. Seal Water Line, PCN 75-1048.</u> J. Straight presented this procedure change. It requested a change of formula on Page 5. The Committee recommended approval of this change.
<u>75-606</u>	<u>PT-23.9B, Containment Isolation Valve Leak Rate Testing "B" R.C.P. Seal Water Line, PCN 75-1047.</u> J. Straight presented this procedure change. It requested a change of formula on Page 5. The Committee recommended approval of this change.
<u>75-607</u>	<u>PT-23.5A, Containment Isolation Valve Leak Rate Testing Containment Sump Recirculation to "A" RHR Pump, PCN 75-1051.</u> J. Straight presented this procedure change. It requested change of formula on Page 4. The Committee recommended approval of this change.
<u>75-608</u>	<u>PT-23.5B, Containment Isolation Valve Leak Rate Testing Containment Sump Recirculation to "B" RHR Pump, PCN 75-1050.</u> J. Straight presented this procedure change. It requested a change in formula on Page 4. The Committee recommended approval of this change.
<u>75-609</u>	<u>PT-23.3, Containment Isolation Valve Leak Rate Testing Makeup Water to Pressurizer Relief Tank, PCN 75-1052.</u> J. Straight presented this procedure change. It requested a change in formula on Page 4. The Committee recommended approval of this change.
<u>75-611</u>	<u>PT-7, Hydro Test of Reactor Coolant System, PCN 75-1034.</u> J. Straight presented this procedure change. It requested: Change 6.4 to read: Close RHRs Letdown valve HCV-133 and Normal Letdown orifices 200A, 200B and 202, open AOV 427 The Committee recommended approval of this change.
<u>75-619</u>	<u>CP-209, Calibration of Area Radiation Monitors (TA-62 Detectors) PCN 75-959.</u> J. Straight presented this new procedure. This procedure describes the steps necessary to calibrate the area radiation monitors with TA-62 detectors. The Committee recommended approval of this procedure.
<u>75-620</u>	<u>PT-14, Circulating Water Pumps-High Water Trip Logic, PCN 75-1040.</u> J. Straight presented this procedure change to incorporate computer print out verification. The Committee recommended disapproval of this request, as it was previously done.

PROC ITEM
NUMBER:DESCRIPTION

75-621 M-55.2, Maintenance Removal or Repair of Reactor Head Control Rod Drive Cables and Assemblies, PCN 75-1152. J. Straight presented this new procedure. This procedure is to provide the guidelines for the maintenance of Reactor Head Control Rod drive power feed cables and assemblies. The Committee recommended approval of this procedure.

75-625 RF-8.3, Core Loading Periodic Equipment Status Check, PCN 75-815. J. Straight presented this procedure change. It requested:

Steps 5.11 and 5.8.3 to read:

5.11 Refueling Communications:
Voice communications have been established between the Control Room and manipulator crane in containment whenever core geometry is being changed."

5.8.3 Refueling surface falls running (as needed)

The Committee recommended approval of this change.

75-626 RF-9.9, Control Rod Drive Shaft Unlatching Tool Operating Instruction, PCN 75-817, 75-846. J. Straight presented the following procedure changes. They requested:

PCN 75-817 Add a sign off block at the end of the procedure as follows:

 Date Completed:

 Shift Foreman :

 Date: _____ Nuclear Engineer or Alt. _____

PCN 75-846 Add sign off lines for initial conditions
Add signature block to and for procedure

The Committee recommended approval of these changes.

75-629. Fuel Assembly and Core Component Movement Prerequisites and Precautions, PCN 75-816. J. Straight presented this procedure change. It requested :

(1) Step 5.1.16 to read:

Refueling Communications: Voice communications have been established between the Control Room and the manipulator crane in containment whenever core geometry is being changed.

(2) Eliminate Step 4.3

The Committee recommended approval of these changes.

PROC ITEM
NUMBER:

DESCRIPTION

75-631

J. Straight presented the following procedures for review of the temporary changes:

M-11.12, Isolation of Safety Injection Pump for Maintenance or Inspection, PCN 75-958.

S-3.4F, Boron Recycle Process Shutdown, PCN 75-1154.

S-3.4P, Transferring Concentrates From Boric Acid Evaporator Feed Tank to Waste Evaporator Feed Tank for Drumming, PCN -75-1068.

S-3.4B, Gas Stripper Startup, PCN 75-1063.

T-4B, Main Feedwater System Lineup, PCN 75-1125.

75-632

PT-33.0, Startup Test Program, PCN 75-1128, 75-1129, 75-1131, 75-1133. J. Straight presented these procedure changes. They requested:

PCN 75-1128 Change step 6.6 to read "Mark step 6.2 through 6.5.4 "N/A".

PCN 75-1129 After Step 6.7, Bank D was borated out to 160 steps and reactor was then placed in Hot shutdown mode.

PCN 75-1131 Add new step 6.7.1 to read:

Return reactor initial. Increase to 3% power and take flux map. Return reactor to Hot Shutdown.

The Committee reviewed the above temporary PCN's.

PCN 75-1133 Add to Step 4.2:

Ensure that power range channel that is supplying input to the reactivity computer is placed in the defeat mode per appropriate General Defeat procedure of CP-41, CP-42, CP-43 or CP-44 Series.

The Committee reviewed the above temporary PCN 75-1133 and recommended that it be made a permanent revision.

75-633

J. Straight presented the following temporary change to be made permanent.

SM 75-32, Installation of Penetration, Seals With Bisco Silicone Foam, PCN 75-1120.

The Committee reviewed the above PCN and recommended that the temporary change be made permanent.

PORC ITEM
NUMBER:

DESCRIPTION

75-634

S-2.1A, Pre-Startup Line-up of Reactor Coolant Pumps, PCN 75-984.

- (1) Change Step 5.3.14 to read:
Parts 5.1.18 through 5.1.26 must be completed.
- (2) Change numbers at checkoff space also.
- (3) Change Checkoff number at top of page to S-2.1A:5

The Committee recommended approval of these changes.

75-635

Immediate Boration, PCN 75-1037. J. Straight presented this procedure change. It requested that the following NOTE should be added after Step 3.7.4.4;

NOTE: Valves 878-A and 878-C are normally closed at power per Tech. Specs. 3.3.1.1 - g. and the breakers are off with control power fuses removed.

The Committee recommended approval of this change.

75-638

PT-33.2, Design Check Tests, PCN 75-1127. J. Straight presented this procedure change. It requested to:

Change Step 4.3 to read "at or about 200 steps"

The Committee recommended approval of this change.

75-642

PT-22.15, Mechanical Manifold "I" Leakrate Test, PCN 75-1053. J. Straight presented this procedure change. It requested to:

Change Step 6.9 to read:

Allow pressure to decay for 15 minutes

Also add - psig on sign off step 6.10

The Committee recommended approval of this change.

75-643

PT-2.3, Safeguard Valve Operation, PCN 75-1061. J. Straight presented the following procedure change. It requested to add the following to Step 6.5.3

MOV 878A Safety Injection Discharge to Loop B Hotleg MOV 878 A.
Status Light

MOV 878A	Open	<u> B </u>	_____
MOV 878A	Closed	<u> D </u>	_____
MOV 878C	Open	<u> B </u>	_____
MOV 878C	Closed	<u> D </u>	_____

The Committee recommended approval of this change.

PROC ITEM
NUMBER:

DESCRIPTION

75-644 PT-2.2, Residual Heat Removal, PCN 75-1064. J. Straight presented this procedure change. It requested to change:

Step 6.3, Change MOV 825A to MOV 852A

The Committee recommended approval of this change.

75-647 O-1.1C, Pre-critical Technical Specification Check Sheet, PCN 75-1159, 75-1160, 75-1118, 75-1119. J. Straight presented these procedure changes. They requested:

PCN 75-1160 Add the following valves to Step 3.3.3 of PCN 75-1119

MOV 700	Valve Closed _____	MOV 720	Valve Closed _____
	Breaker Open _____		Breaker Open _____
	Fuses In _____		Fuses In _____
MOV 701	Valve Closed _____	MOV 721	Valve Closed _____
	Breaker Open _____		Breaker Open _____
	Fuses In _____		Fuses In _____

PCN 75-1159 Add the following valves to Step 3.3.2 of PCN 75-1118

MOV 896A	Valve Open _____
	Breaker Open _____
	Fuses In _____
MOV 896B	Valve Open _____
	Breaker Open _____
	Fuses In _____
	ECCS Tech Specs _____

PCN 75-1118 Add to Initial Conditions:

The following valves are in the Open Position, Breakers Open, Control Fuses in Place.

MOV 856 Open _____	MOV 878B Valve Open _____	MOV 841 Open _____
Fuses In _____	Breaker Open _____	Breaker Open _____
Breaker Open _____	Fuses In _____	Fuses In _____
	MOV 878D Valve Open _____	MOV 865 Open _____
	Breaker Open _____	Breaker Open _____
	Fuses In _____	Fuses In _____

PCN 75-1119 Add to Initial Conditions:

The following valves are in the closed position. Breakers open, fuses in place.

878A Closed _____	878C Closed _____
Breaker Open _____	Breaker Open _____
Fuses In _____	Fuses In _____

The Committee recommended approval of these changes.

FORM ITEM
NUMBER:

DESCRIPTION

75-649

S-3.2D:1, Transferring Water From CVCS Huts to RWST or SFP, PCN 75-1075. J. Straight presented this procedure change. It requested:

After Step 5.2.2, add a step to direct operator to close the CVCS tank discharge valve opened in Step 5.1.4.

The Committee recommended approval of this change.

75-650

SC-1, Radiation Emergency Plan, PCN 75-1097. J. Straight presented this change. The plan now conforms more closely with the US NRC guidelines. The Committee recommended approval of this change.

75-651

SC-3, Fire Emergency Plan, PCN 75-1239. B. Snow presented this procedure change. It requested changes to comply with IE 75-04 and 75-04A reports. The Committee recommended approval of these changes.

75-652

E-1.1, Safety Injection System Actuation, PCN 75-1161. J. Straight presented this procedure change. It requested to add the following step 3.6:

Upon actuation of SIS and as directed by Control Room personnel, an operator stationed in the Auxiliary Building shall open the breakers for Valves 852A and 852B. Control Room personnel shall advise the operator to do this as soon as 852A and 852B have opened. If communications cannot be established with the Control Room, the operator in the Auxiliary Building will perform this action - 5 minutes after SIS activation.

NOTE: ~ 5 minutes will allow sufficient time for 852A and 852B valves to attain proper position.

The Committee recommended approval of this change.

75-653

E-1.2, Loss of Coolant Accident, PCN 75-1162, 75-1163. J. Straight presented this procedure change. It requested to add the following step 4.3.1a :

When 10½ RWST level is approached (NOTE: Indication is available in Auxiliary Building Basement) and during transfer to recirculation phase, an operator stationed in the Auxiliary Building will put breakers for 8964 & 896B back in, ensure the valves are closed and pull the breakers.

If communications with Control Room is not available, this action will be done when the SIS pumps and the RDR pumps have stopped (NOTE: This is an indication that Control Room personnel are ready to commence recirculation phase.

The Committee recommended approval of this change with modifications.

PROC ITEM
NUMBER:

DESCRIPTION

- 75-658 A-2.7, Calculating Core Quadrant Power Tilt, PCN 75-1195.
J. Straight presented this procedure change to comply with Tech. Spec. Definition 1.9. The Committee recommended approval of this change.
- 75-659 PT-2.3, Safeguard Valve Operation, PCN 75-1196. J. Straight presented this procedure change to reflect NRC's attitude regarding critical Emergency Core cooling valves. The Committee recommended approval of this change.
- 75-661 PT-6.2A, N.I.S. - Intermediate Range Channels Alignment and/or Bistable Adjustment Procedure, PCN's #75-1170, #75-1171. J. Straight presented this procedure for review of the following temporary changes:
- PCN 75-1170 Steps 5.2 through 5.21.10 to be marked N/A.
- PCN 75-1171 Steps 5.2 through 5.21.10 to be marked N/A.
- The Committee reviewed these temporary changes.
- 75-662 S-17B, Addition of 50% NaOH to the Spray Additive Tank, PCN 75-1074, 75-1099. J. Straight presented these procedure changes. They request:
- PCN 75-1074 N/A Steps D. 1-2-4
- Change Step D-3 to Local Level Indicator (delete the word "Manway")
- Change Step D-13 to Local Level Indicator (delete the word "Manway")
- Change Step D-14 and D-15 to read "Truck Fill Vent Valve"
- The Committee reviewed this temporary change.
- PCN 75-1099 This procedure change requests to make this procedure more flexible for increasing NaOH concentration as to raise level in spray addition tank.
- The Committee recommended approval of these changes.
- 75-664 RSSP-1.1, Valve Interlock Verification Residual Heat Removal System, PCN 75-1035. J. Straight presented this procedure change. It requested:
- Initial Condition 4.3
- Add the following: ...or plant in process of heat up with RHR System available for service.

PROC ITEM
NUMBER:

DESCRIPTION

75-666 0-1.2, Plant From Hot Shutdown to Steady Load, PCN'S # 75-1164, 75-1172, 75-1167, 75-1186, 75-1188, 75-1190. J. Straight presented the following procedure change. It requested to reword Step 5.4.1 to read:

PCN 75-1164 Select the highest reading power range channel to the NIS recorder and insure the power range over power recorders are in operation.

The Committee recommended approval of this change.

The following temporary changes were presented for review. They requested:

PCN 75-1172 N/A the rest of procedure starting with Step 5.3.3

PCN 75-1167 Mark N/A the following steps:
5.3.6, 5.4.6.1, 5.4.6.7, 5.4.6.8, 5.4.6.22, 5.4.6.12,
5.4.33, 5.5.22

PCN 75-1186 N/A the following steps:
5.59 through 5.511
5.5.12.2 through 5.5.22

PCN 75-1188 Step 5.5.7.6
Valves 4147A and 4147C are tagged and held closed for ST-75-3 MSR Vent Condenser test. Request that these valves not be opened per 0-1.2 so that ST-75-3 may proceed

PCN 75-1190 Add to step 5.5.12.6
CV-24B _____
CV-25 _____
CV-25B _____

Step 5.5.17 correct "Use 7106" by changing to U-7016.

The Committee reviewed these temporary changes.

75-669 PT-32, Reactor Trip Logic Test "A" or "B" Trains, PCN 75-1078. J. Straight presented this procedure change. It requested:

Steps 6.10 and 6.11 Undervoltage and Underfrequency trip logic cannot be generated according to existing steps.

The Committee recommended approval of this change.

75-670 P-9, Precautions, Limitations and Setpoints P-9 Radiation Monitoring System, PCN 75-1193. J. Straight presented this procedure change. It requested:

Change setpoint of "R-20 SFP IIX Service Water" from 15,000 cpm to 20,000 cpm

The Committee recommended approval of this change.

<u>FORC ITEM NUMBER:</u>	<u>DESCRIPTION</u>
<u>75-675</u>	<u>RF-2, Reactor Refueling Outage Operations and Activities,</u> <u>PCN 75-943 and 75-820.</u> J. Straight presented these procedure changes. They requested:
PCN 75-820	Change the procedure to provide a more positive method of head alignment with guide studs.
PCN 75-943	Add a new Step 5.13.1 to read: I & C disconnect reactor head Accoustical Emmission detector. Change Step 5.98 to read 5.98.1 Add a new step 5.98 to read I & C reconnect reactor head Accoust- ical Emmission detector. The Committee recommended approval of these changes.
<u>75-689</u>	<u>S-4.4, Spent Resin Removal To Shipping Casks, PCN75-1240.</u> J. Straight presented this procedure change. It requested to; change step 3.4 -----"150 MR/hr to a setting to permit clearing the alarm" The Committee recommended approval of this change.
<u>75-690</u>	<u>PT-12.2, Emergency Diesel Generator IB, PCN75-1252.</u> J. Straight presented this procedure change. It was requested to add to the NOTE after step 6.2.4, "and 6.11 N.A ". The Committee recommended approval of this change. This item <u>IS COMPLETE.</u>
<u>75-691</u>	<u>PT-33.0, Startup Test Program, PCN75-1267.</u> J. Straight presented this temporary change for review. It requested; "delete step 6.11, mark N.A. on procedure" change step 6.12 to read "at approximately 760 MW" The Committee reviewed this temporary change.
<u>75-693</u>	<u>PT-6.2, N.1.5. Intermediate Range Channels; PCN75-1260.</u> J. Straight presented this procedure change. It requested to add to step 6.7; NOTE: If gamma level has not reduced to permit testing, mark steps 6.7 thru 6.11 N.A. The Committee recommended approval of this change.
<u>75-694</u>	<u>SC-3, Fire Emergency Plan, PCN75-832.</u> J. Straight presented these procedure changes. The Committee recommended disapproval of this request as these changes have already been incorporated.

PROC ITEM NUMBER:	DESCRIPTION
<u>75-699</u>	<u>EM-142, Level Transmitter LT-462, Installed Inplace Of LT-461 To Control LM-463F & LC-480, PCN75-1300.</u> J. Paris presented this new procedure. This procedure is to install level transmitter LT-462 in place of LT-461 to control LM-463F and LC-480. The Committee recommended approval of this procedure.
<u>75-719</u>	<u>RF-8.1, Step By Step Fuel Loading Sequence And Maps, PCN75-822.</u> J. Straight presented this procedure change. It was requested Sequence be removed from the title, also several minor changes correcting terminology. The Committee recommended approval of these changes.
<u>75-725</u>	<u>SM75-23, Installation Of Penetration Seals With Bisco Silicone Foam, PCN75-1263.</u> J. Straight presented this procedure change for review only. It requested after step 5.77 to add sign-off for R G & E Engineer & Date. The Committee reviewed this change.
<u>75-726</u>	<p><u>O-2.1, Normal Shutdown To Hot Shutdown, PCN75-1225, and PCN75-1242.</u> J. Straight presented these temporary changes for review;</p> <p><u>PCN75-1225</u> - requested steps 5.11.3, 5.21 & 5.24 be N.A.'d.</p> <p><u>PCN75-1242</u> - requested steps 5.1.2 thru 5.12 and 5.15 thru 5.24 be N.A.'d.</p> <p>The Committee reviewed these temporary changes.</p>
<u>75-727</u>	<u>PT-6.4, Excure/Incore Recalibration, PCN75-1268.</u> J. Straight presented this procedure change. It requested several minor changes for review. The Committee reviewed these changes.
<u>75-728</u>	<p><u>O-1.2, Plant From Hot Shutdown To Steady Load, PCN75-1262, 1264, 1259, 1273, 1241, 1251, 1258, and 1261.</u> J. Straight presented these procedure changes.</p> <p><u>PCN75-1262</u> - requested step 5.4.32 N.A. Fig. O.1.1A not attached to any O.1.2</p> <p><u>PCN75-1264</u> - requested;</p> <p>(1) add to step 5.5.12.1 "make sure blocking device is run upon valve 3425"</p>

FORC ITEM
NUMBER:

DESCRIPTION

75-728 (Cont.)

PCN75-1264 -

- (2) add to step 5.5.12.2 "make sure blocking device is run upon valve 3426"
- (3) add to step 5.5.12.3 "make sure blocking device is run upon valve 3427"
- (4) add to step 5.5.12.4 "make sure blocking device is run upon valve 3428"

PCN75-1259 - requested changes to be made to step 5.5.12.6, this was disapproved as these changes were made previously.

PCN75-1273, PCN75-1241, PCN75-1251, PCN75-1258 and PCN75-1261, were all temporary requests for review only to facilitate steps N.A.'d.

The Committee reviewed the temporary changes and requested approval of PCN75-1262, PCN75-1264 and disapproved PCN75-1259.

75-734

A-2.7, Calculating Core Quadrant Power Tilt, PCN75-1286.

J. Straight presented this procedure change for review.

Several changes were made to comply with Technical Specifications definition 9. The Committee reviewed this change.

75-738

S-4.1G, "A" Laundry & Hot Shower Tank Release To Discharge Canal, PCN75-1299.

E. Beatty presented this procedure change.

This procedure was written to incorporate S-4.1H, the procedure for the "B" tank. The Committee recommended approval of this procedure.

75-739

S-4.1Y, Laundry Tank Release Using Chemical Drain Pump, PCN75-1297.

E. Beatty presented this new procedure. The purpose of this procedure is to describe the valve line up to sample the laundry tank, and to discharge it to the canal after a release form has been prepared, using chemical drain tank. The Committee recommended approval of this procedure.

75-741

CP-462.2, Reinstate Of "A" Steam Generator Level Channel 642, PCN75-1317.

E. Beatty presented this procedure change. It requested under Instructions add the following;

5.4 Operations check normal level indication 462, place IC-466 back into the Automatic mode. Operations Initial _____

5.5 Operations place the bypass controller IC-480 from manual to Automatic if desired. Operations Initial _____

The existing steps 5.4 thru 5.7 make them steps 5.6 thru 5.9.

The Committee recommended approval of these changes.

PORC ITEM
NUMBER:

DESCRIPTION

75-742

CP-462.1, Defeat of "A" Steam Generator Level Channel "462", PCN75-1318. E. Beatty presented this procedure change. It requested under Instructions add the following;

5.3 Operations place HC-480 feedwater bypass controller from automatic to manual mode. Operations Initial _____

5.4 Operations place HC-466, main feedwater valve for loop "A" from automatic to manual mode. Operations Initial _____

The existing steps 5.3 thru 5.7 make them steps 5.5 thru 5.9.

The Committee recommended approval of these changes.

75-743

CP-461.1, General Defeat Procedure "461" Steam Generator Level, PCN75-1320. E. Beatty presented this procedure change. It requested to delete steps 5.3 and 5.4. The Committee recommended approval of this deletion. This item IS COMPLETE.

75-744

E. Beatty presented the following two procedures for temporary changes, for review only.

O-2.1, Normal Shutdown To Hot Shutdown, PCN75-1301, was to facilitate steps marked N.A.

O-1.2, Plant From Hot Shutdown To Steady Load, PCN75-1302. This requested to change step 5.5.7.6;

V4147A from open to CLOSED

V4147C from open to CLOSED

The Committee reviewed these temporary changes.

75-749A

O-1.2, Plant From Hot Shutdown to Steady Load, PCN75-1327, 1326. E. Beatty presented these procedure changes for review;

PCN75-1326 - step 5.4.54 calls for valves 3996, 4007 & 4008 to be opened - leave these valves closed at this time except 3996.

The Committee reviewed this change.

PCN75-1327 - requested to change step 5.5.7.6, the Committee reviewed this request and recommended disapproval as this was previously done by the above PCN75-1326.

FORC ITEM
NUMBER:DESCRIPTION75-751PT-6.4, Excure/Incore Recalibration, PCN75-1305.

E. Beatty presented this procedure change. It requested the following steps be changed to read;

- 4.3 - 190 steps
- 6.4 - approx. 175 steps
- 6.6 - approx. 175 steps
- 6.8 - approx. 190 steps
- 6.9 - remove "zero" and add "a stable value"

The Committee recommended approval of this change.

75-754

O-1.1C, Pre-Critical Technical Specification Check Sheet, PCN75-1323, 1325. E. Beatty presented this procedure change.

PCN75-1323 - requested in step 5.1.3 to change 4th item to read;
"S/G primary to secondary leakage <0.1 gpm"

The Committee recommended approval of this change.

PCN75-1325 - requested the same.

The Committee recommended disapproval of this PCN.

75-755

PT-22.10, Mechanical Manifold "C" Leakrate Test, PCN75-1332.

E. Beatty presented this procedure change for review only.
It was requested;

- (1) Change step 6.4 to read - "allow approximately 30 minutes for temperature stabilization before continuing with step 6.5"
- (2) Change step 6.9 to read - "allow pressure decay for at least one hour"

The Committee reviewed this change.

PORC ITEM
NUMBER:DESCRIPTION75-756

PT-5.30, Process Instrumentation Reactor Protection Channel Trip Test (Channel 3), PCN75-1322. E. Beatty presented the following procedure changes;

under Instructions add the following steps;

6.71 Operations place HC-480 feedwater bypass controller from automatic to manual mode Operations Initial _____

6.72 Operations place HC-466 Main feedwater valve controller "A" loop from automatic to manual mode. Operations Initial _____

number the existing 6.71 step 6.73 and change the number on remaining steps accordingly until old step 6.78. Add the following instructions which will be numbered as follows:

6.80 Operations verify proper indication exists on LI-462, Steam generator level. With normal indication, place HC-466 main feedwater controller "A" loop to automatic. Operations Initial _____

6.81 Operations place HC-480 bypass controller to position desired. Operations Initial _____

Number the remaining steps accordingly.

The Committee recommended approval of these changes.

75-757

PT-5.10, Process Instrumentation Reactor Protection Channel Trip Test, (Channel I), PCN75-1321. E. Beatty presented this procedure change. It was requested under Instructions to;

delete steps 6.65, 6.75, 6.76. Number remaining steps accordingly.

The Committee recommended approval of this change.

75-758

PT-2.3, Safeguard Valve Operation, PCN75-1310, 1313, 1311 and 1312. E. Beatty presented these procedure changes;

PCN75-1310 - requested step 6.5.7, add a place to sign off after step

PCN75-1313 - requested to add sign off line for notes: 1 thru 8.

PCN75-1311 - requested steps 6.1, 6.2, 6.3, 6.4, 6.5 all need spaces provided for sign off of the steps.

PCN75-1312 - requested to add sign off step for note before step 6.8 on page 11.

The Committee recommended approval of these changes.

PROC ITEM
NUMBER:

DESCRIPTION

- 75-759 PT-16, Auxiliary Feedwater System Flow Check, PCN75-1304.
E. Beatty presented this procedure change. It was requested;
step 6.1.10.1 (A motor driven auxiliary F.W. pump) MOV 4007
is to be left closed
step 6.2.10.2 (B motor driven auxiliary F.W. pump) MOV4008
is to be left closed
The Committee recommended approval of these changes.
- 75-760 O-3, Hot Shutdown With Xenon Present, PCN75-1306. E. Beatty
presented this procedure change. It requested step 5.1.1
change on to or, so statement reads "time of trip or shutdown"
The Committee recommended approval of this change.
- 75-763 E-18, Failure Of Spent Fuel Pit Cooling, PCN75-726.
E. Beatty presented this procedure to be eliminated, as it has
been incorporated into E-18.1 and E-18.2. The Committee re-
commended approval of this elimination.
- 75-764 A-2.4, Calculation Of Critical Rod Position, PCN75-1347. E. Beatty
presented this procedure change. This was a rewrite to clarify the
procedure. The Committee recommended approval of this rewrite with
modifications.
- 75-766 A-2.4.1, Calculation Of Hot Xenon Free Shutdown Margin, PCN75-1345.
E. Beatty presented this new procedure. This procedure provides the
method for calculating the Hot Xenon Free Shutdown Margin. The
Committee recommended approval of this procedure.
- 75-769 EM-143, Replacement Of Non-Controlling Alarm Bistable, PCN75-1397.
E. Beatty presented this new procedure. This procedure is to
outline repairs or replacement of Bistable _____ used only for
alarm purpose. The Committee recommended approval of this pro-
cedure. This item is NOT COMPLETE pending post-maintenance review.
- 75-770 E. Beatty presented the following procedures for temporary
changes for review only;
GS-21.0, Site Access Control (Security) & Personnel Identification.
PCN75-1343, requested to suspend section 3.5 thru 3.5.6.
O-2.2, Plant Shutdown From Hot Shutdown To Cold Shutdown, PCN75-1349.
This PCN was to facilitate steps marked N.A.
T-4F, Restoring 1B Feedwater Pump To Service After Maintenance,
PCN75-1285. This PCN was to facilitate steps marked N.A.
O-2.5, Plant Shutdown From Hot Shutdown To Cold Shutdown When
Condenser Steam Dump Is Unavailable, PCN75-1339, 1349.
PCN75-1349 - was to facilitate steps marked N.A.
PCN75-1339 - requested remainder of the procedure be marked N.A. and
to delete the procedure.

PROC ITEM NUMBER:	DESCRIPTION
75-770	The Committee reviewed the steps N.A.'d and recommended disapproval of the deletion of the procedure. <u>O-1.1D, Pre-Heatup Plant Requirement Check List, PCN75-1341,</u> requested to add steps 3.1.3 as follows; "plant status is such that O-1.1D is being performed in conjunction with O-1.1C" <u>M-37.20, Inspection & Repair Of Main Steam Isolation Valves, PCN75-1348 and 1381,</u> requested several temporary steps be added to document the modifications for review. The Committee reviewed these temporary change requests.
75-775	<u>PT-8, Reactor Trip Breakers A & B, PCN75-1174.</u> E. Beatty presented this procedure change. It requested to eliminate procedure for testing Reactor Trip Breakers A and B. The Committee recommended approval of this change. This item <u>IS COMPLETE.</u>
75-776	<u>PT-17.1, Area Radiation Monitors R-1 - R-9, PCN75-1379,</u> requested; 1) in steps 6.2.5.5; 6.3.5.5; 6.4.5.5; 6.5.5.5; 6.6.5.5 & 6.7.5.5, eliminate (.) period after the word "indication" and add the following to each step (AS INDICATED WITH SELECTOR SWITCH IN CHECK SOURCE POSITION). 2) in steps 6.8.5.5 & 6.9.5.5, eliminate (.) period after the word "LOCAL" and add the following to each step. (AS INDICATED WITH ALARM SETTING LEFT IN "OPERATE" POSITION). The Committee recommended approval of these changes.
75-778	<u>E-28, Reactor Coolant Leak, PCN75-1316,</u> requested to change step 4.5 to .1 gpm. The Committee recommended approval of this change.
75-779	E. Beatty presented the following procedures for deletion; <u>EM-1, Replacement Of Bistable for VCT Level Control LCV-1120,</u> PCN75-1350. <u>EM-6, AP Span Adjustment, PCN75-1352.</u> <u>EM-7, Removing An NIS Power Range Channel From Service, PCN75-1353.</u> <u>EM-12, Repairing Gasket Leak On RHR Return Check Valve To B Loop Leg 867B, PCN75-1355.</u> <u>EM-13, Work On Any Reactor Coolant Flow Module (One Channel At A Time, PCN75-1356.</u> <u>EM-15, Repacking Of Pressurizer Spray Valve Bypass, PCN75-1357.</u>

PROC ITEM
NUMBER:DESCRIPTION

- 75-779 EM-16, Repositioning Barton Feedwater Flow Meters, PCN75-1358.
- EM-19, Replace AT Trip Setpoint Channel Current Repeater, PCN75-1359.
- EM-21, High Head Safety Injection Check Valve Installation, PCN75-1360.
- EM-22, Repair Of Part Length Drive Conoseal I-5 And Replacement Of Pressurizer Safety Valve Flange Gasket, PCN75-1361.
- EM-25, Procedure To Change T Protection Component TM-405R 3 Sec. Filter, PCN75-1362.
- EM-27, Emergency Conversion From Sealed Reference Leg To Open Reference For LT-428, PCN75-1363.
- EM-28, Replacement Of Power Supply For Pressurizer Level Channel LT-426, PCN75-1364.
- EM-29, Returning 1A Reheater To Normal Line-Up After Inspection And Possible Repair, PCN75-1365.
- EM-32, Secondary E.M. Procedure For Welding Union On LAH-208h, PCN75-1366.
- EM-38, Replace Repeaters In Reactor Protection Channels, PCN75-1367.
- EM-39, Recovery Of Irradiation Speciman Basket, PCN75-1368.
- EM-47, Replacement Of Relay PC-482AX Located In SIAL Rack Compartment 1, PCN75-1369.
- EM-48, Replacement Of Bistable PC-482A Located In Protection Rack B2, PCN75-1370.
- EM-53, Retrieval Of Foreign Object In Reactor Core, PCN75-1371.
- EM-54, Modification To RCC Transport Basket, PCN75-1372.
- EM-63, Power Range Detector Replacement, PCN75-1373.
- EM-76, Restraint Removal From Turbine Control Valves 3462 & 3463, PCN75-1374.
- EM-89, Removal Of RCC Finger From Storage Can In Spent Fuel Pit, PCN75-1375.
- EM-109, Weld End Overlay Of Valves 287, 289 and 290, PCN75-1377.
- EM-104, Valve Wall Buildup On Valve 853B, PCN75-1376.
- S-4.1A, "B" Laundry & Hot Shower Tank Release To Discharge Canal, PCN75-1378.

The Committee recommended approval of these deletions.

PROC ITEM
NUMBER:DESCRIPTION75-780

EM-2, Replacement Of Bistable 431-B For Pressure Controller
PRV-431C, PCN75-1351. E. Beatty presented this procedure change.
It requested;

Add sign-offs for each step on Initial Conditions and Instructions.

&

Change 1985 psig statement in 4.15.1 to 2235 psig and 1935 psig in 3.2 to new set point.

The Committee recommended approval of these changes.

75-781

EM-115, Cleaning and Inspection Of The Containment Personnel
Hatch Equalizing Valve For The Intermediate Side, PCN75-1346.
E. Beatty presented this procedure change. It requested to change
step 5.6 to read;

5.6 Perform leak rate test on the containment personnel hatch
and verify that an acceptable leakage rate exists.

Results & Test Tech.

The Committee recommended approval of this change.

75-788

T-41A, Alignment Of Auxiliary Feedwater System Prior To Power
Operation, PCN75-998. E. Beatty presented these procedure changes.
PCN75-998 - requested step 5.41 change V4031 to V4301
requested step 5.52 remove V-4007 OPEN

PCN75-1054 - requested;

step 5.52 add MOV in front of 4007
add MOV in front of 4008

step 5.29 add MOV in front of 3505
step 5.30 add MOV in front of 3504

The Committee recommended approval of these changes.

75-789

S-2.2, Pressurizer Pressure & Spray Control, PCN75-1315.
E. Beatty presented this procedure change. It was requested
to change the setpoints to reflect the new pressure setpoints.
The Committee recommended approval of this change. This item
IS COMPLETE.

75-790

S-2.3A, Burping Pressurizer Relief Tank, PCN75-1382. E. Beatty
presented this procedure change. It was requested to add under
INITIAL CONDITIONS;

NOTE: The gas decay tank that is lined up to receive gas
must not be lined up should have sufficient volume
to receive all the gas to be vented. If not, switch

PROC ITEM
NUMBER:

DESCRIPTION

75-791

S-4.5, Sluicing Waste Condensate Polishing Demineralizer Spent Resin To Shipping Cask, PCN75-1223. E. Beatty presented this procedure change. It requested to change step 3.8 to read;

"the cask line _ _ _" not "lines". The Committee recommended approval of this change.

75-792

S-8A, Component Cooling Water System Startup And Normal Operation Valve Alignment, PCN75-1250. E. Beatty presented this procedure change. It requested step 5.1.12 should read "V-4617B closed". The Committee recommended approval of this change.

75-793

S-16, Safety Injection System, PCN75-1309. E. Beatty presented this procedure change. It requested;

Step 3.4 - change 35% to ~65%
Step 5.4 - change Value to Valve
Step 5.8 - change 1800 to 2000

The Committee recommended approval of this change.

75-794

S-16A, Safety Injection System Alignment, PCN75-1093. E. Beatty presented this procedure change. It was requested;

Step 5.1.27 - delete from step the following words ---- and remove fuses also words Fuses Out _____

75-795

PT-6.4, Excore/Incore Recalibration, PCN75-1391. E. Beatty presented this procedure change. This change was to supply proper rod position as required by reactor load and axial tilt limitations. The Committee recommended approval of this change.

75-800

M-11.5B, Major Mechanical Inspection Of Auxiliary Motor Driven Feed Pump 1B, PCN75-1344. E. Beatty presented this procedure change. It requested to change step 5.6.3 to read;

"tighten pump casing studs until secure; making a final pass to ensure all studs are tight"

The Committee recommended approval of this change.

FORM ITEM
NUMBER:

DESCRIPTION

75-803SM75-10.1, Installation Of Grounding Cable For AuxiliaryBuilding Addition, PCN75-1428

S. Spector presented this new procedure. This procedure describes the steps for installing a new section of grounding cable for grounding of the new auxiliary building addition and to allow removal of section of existing grounding cables temporarily for caisson installation. The Committee recommended approval of this procedure.

75-804

B. Snow presented the following temporary procedure changes for review only;

M-37.16A, Inspection & Maintenance of Manual Operated Grinnel Diaphragm Valves, PCN75-1396, requested steps 5.5.1 through 5.6 be marked N.A.

PT-2.1, Safety Injection System, Monthly Surveillance Requirement, PCN75-1383, requested step 4.2 be marked N.A.

RSSP-7.2, Calibration Procedure At Power T Average and Delta T Span Adjustment, PCN75-1421. Requested under Initial Conditions step 5.1 replace "or slightly" with "STEADY STATE" also step 5.3 replace "secured" with "as directed by chemist".

The Committee reviewed these changes.

75-807

M-37.2, Maintenance Of Pressurizer Relief Guard Valves, PCN75-1094.

B. Snow presented this procedure change. The Committee recommended disapproval of this PCN as the procedure was updated prior to this PCN request.

75-809

O-1.2, Plant From Hot Shutdown To Steady Load, PCN75-1392. B. Snow presented this procedure change. It requested step 5.4.54 to change valves 4007 & 4008 from open to CLOSED. The Committee recommended approval of this change.

PROC ITEM
NUMBER:

DESCRIPTION

- 75- 810 M-37.16H, Inspection & Maintenance Of Chapman-Crane Pressure Seal Tilting Disc Check Valve For Valve, PCN75-1415.
S. Spector presented this new procedure. This procedure is to describe the steps necessary to disassemble, inspect and repair and reassemble these valves. The Committee recommended approval of this procedure.
- 75- 811 M-11.5H, Removing The 1A Motor Driven Auxiliary Feed Pump From Service For Maintenance, PCN75-1414. S. Spector presented this new procedure. This procedure is to describe the steps necessary to remove the 1A motor driven feed pump and associated valves. The Committee recommended approval of this procedure.
- 75- 812 M-11J, Returning The 1A Motor Driven Auxiliary Feed Water Pump To Service After Maintenance, PCN75-1423. S. Spector presented this new procedure. This procedure is to describe the step to be used to restore the 1A motor driven auxiliary feed pump to service after it has been held for maintenance. The Committee recommended approval of this procedure.
- 75- 813 M-37.16J, Inspection And Maintenance Of The Motor Driven Auxiliary Feedwater Pump Recirculation Control Valve, PCN75-1422. S. Spector presented this new procedure. This procedure describes the steps necessary to disassemble, perform maintenance and reassemble Valve 4304 or 4310. The Committee recommended approval of this procedure.
- 75-815 M-37.20, Inspection & Repair Of Main Steam Isolation Valves, PCN75-1039, PCN75-1398. B. Snow presented these procedure changes.
PCN75-1039 - requested a new step be added;
5.2.8 - inspect valve internals and main steam header internals as necessary for cleanliness.
Q.C. Inspector _____
PCN75-1398 - requested to add words at end of item 5.2.14 as follows;
---and that the valve strokes to its proper full open position.
The Committee recommended approval of these changes.

PORC ITEM
NUMBER:

DESCRIPTION

75-816

M-40, Surveillance And Maintenance Of Hydraulic Snubbers, PCN75-1293 & 1405. B. Snow presented these procedure changes. It was requested;

Hydraulic Snubber List Page M-40:h - Remove asterics (*) on FW-85 and MS-22, change FW-8h Model to read HSSA-20

Sign Off on Page M-40:2

Add line for PORC Review Date _____

PCN75-1405 - requested another valve number be added.

The Committee recommended approval of this change.

75-818

B. Snow presented the following procedures for changes. It is requested the procedures be made applicable for A & B Motor Driven Auxiliary Feed Pumps, For _____ Pump, also to add a NOTE: "If installing new rotating impeller shaft assembly mark associated steps N.A."

M-11.5B, Major Mechanical Inspection Of The Auxiliary Feed Pump, PCN75-1416.

M-11.5C, Minor Mechanical Inspection Of Auxiliary Motor Driven Feed Pump, PCN75-1417.

The Committee recommended approval of these changes.

75-819

SC-3, Fire Emergency Plan, PCN75-1412. B. Snow presented this procedure change. There were several word changes requested to reflect actual policy and clarification. The Committee recommended approval of these changes.

75-821

A-2.3, Maximum Unit Power, PCN75-1411. B. Snow presented this procedure change. These changes were to supply data sheet, curves and tables for calorimetric with procedure. The Committee recommended approval of this change.

75-823

M-40.1, Hydraulic Snubber Unit Removal And Reinstallation Procedure (Bergen-Paterson), PCN75-1291. B. Snow presented this procedure change. It requested to remove (Bergen-Paterson) from the title also to add under Initial Conditions step 3.h, "Notify Q.C. Dept. _____". The Committee recommended approval of this change.

PORC ITEM
NUMBER:

DESCRIPTION

75-824

M-40.2, Hydraulic Snubber Unit Inspection & Repair Procedure (Bergen Paterson), PCN75-1292. B. Snow presented this procedure change. It requested under Initial Conditions, 3.3 add; "Notify Q.C. Dept. _____".

75-825

M-34, Removal & Reinstallation Of CVCS Relief Valve #209, PCN75-797. B. Snow presented this procedure change. It requested to add Reference:

2.2 RSSP-12, Testing Of Primary & Secondary Relief Valves On Test Stand.

The Committee recommended approval of this change.

75-826

M-11.5A, Removing The 1B Motor Driven Auxiliary Feed Pump From Service For Maintenance, PCN75-1413. B. Snow presented this procedure change. It was requested to change purpose to;

1.1 "To describe the steps necessary to remove the 1B motor driven feed pump and associated valves from service for maintenance"

5.9 delete the word "Locked"

Add signoffs

Maintenance Engineer _____

PORC Review Date _____

The Committee recommended approval of this change.

75-827

M-55.1, Repair Or Replacement Of Type W-2 Breaker Control Switches, PCN75-925. B. Snow presented the following procedure change. It requested to change sign off from "Maintenance Foreman" to Shift Foreman. The Committee recommended approval of this change. This item IS COMPLETE.

75-828

M-50.1, Isolation Of #12 Transformer, PCN75-1105. B. Snow presented this procedure change. It was requested to add to NOTE after 5.7;

NOTE: Relay settings shall not be changed without approval of the Maintenance Engineer

The Committee recommended approval of this change.

75-830

M-48.5, Isolation Of Bus 17, PCN75-1014. B. Snow presented this procedure change. It requested to change step 5.6.1 to read;

"Open and hold main incoming breaker on Bus 17 on MCC 1G, position "M"

The Committee recommended approval of this change.

RC ITEM
NUMBER:

DESCRIPTION

75-831

M-37.15, Replacing Or Adjusting Rubber Seat In Purge Supply And Exhaust Valves, PCN75-1095. B. Snow presented this procedure change. It requested to add to the purpose:

"or seat adjustment. If it is only necessary to adjust the seats then non-applicable steps may be marked N/A"

Add to title: Replacing "or adjusting rubber seat"

The Committee recommended approval of this change.

75-833

B. Snow presented the following procedures for change. It was requested the following note be added to each of these procedures;

NOTE: Relay settings shall not be changed without approval of the maintenance Engineer

M-48.2, Isolation Of Bus 14, PCN75-1110.

M-48.4, Isolation Of Bus 16, PCN75-1113.

M-48.5, Isolation Of Bus 17, PCN75-1109.

M-48.6, Isolation Of Bus 18, PCN75-1108.

M-60.5, 1A Emergency Diesel, Protective Relay Calibration & Trip Test, PCN75-1112, requested to add Precaution 4.2:

"Relay settings shall not be changed without approval of the maintenance engineer"

M-60.6, 1B Emergency Diesel Protective Relay Calibration And Trip Test, PCN75-1112, requested same as above.

The Committee recommended approval of this change.

75-835

SM75-27.1, Installation Of A Key Switch On The D.C. Power For MOV896 A & B, PCN75-1431. B. Snow presented this new procedure. This procedure is to describe the steps necessary to install a key switch on the D.C. power and verify proper isolation by operation of the switch for valves 896A and B to reduce the probability of a passive failure of the valves. The Committee recommended approval of this procedure.

75-836

SM75-33.1, Installation Of A Key Switch On The D.C. Power For MOV 852A and B, PCN75-1433. B. Snow presented this new procedure. This procedure is to describe the steps necessary to install a key switch on the D.C. power and verify proper isolation of same by operating the switch for valves 852A and B to reduce the probability of a passive failure of the valves. The Committee recommended approval of this procedure.

PROC ITEM
NUMBER:DESCRIPTION75-837S-13A, RHR System Lineup For Safety Injection, PCN75-1447.
B. Snow presented this procedure change. It requested;Step 5.2.1 after RV deluge Valve
add 852A Key Switch OFF

MOV 852A closed _____

Step 5.2.1 after RV deluge Valve
add 852B Key Switch OFF

MOV 852B closed _____

The Committee recommended approval of this change.

75-838S-16A, Safety Injection System Alignment, PCN75-1488. B. Snow
presented this procedure change. It requested after;5.1.2.20 add;
after 5.1.2.21 add;

V896A Key Switch Off _____

V896B Key Switch Off _____

The Committee recommended approval of this change.

75-839O-1.1C, Pre-Critical Technical Specification Check Sheet,
PCN75-1449. B. Snow presented this procedure change. It
requested;

Step 3.4, MOV 896A & B should read as follows:

MOV896A Open _____ Breaker Closed _____ Fuses In _____ Key Switch
Off _____

MOV896B Open _____ Breaker Closed _____ Fuses In _____ Key Switch Off _____

The Committee recommended approval of this change with modifications.

75-840E-1.2, Loss Of Coolant Accident, PCN75-1450. B. Snow presented this
procedure change. It requested to add NOTE after step 3.1.4;
NOTE: To close MOV 852A or B operator must place D.C. closing power
hold switch labeled 852A or B "Key Switch" in the ON position.Add NOTE after 896B of step 4.3.1.2; NOTE: To close 896A or B,
the D.C. power hold switch labeled 896A or B "Key Switch" must
be placed in the OPEN position.Add Note before step 4.4.5, NOTE: To exercise valve 852A or B operator
must place D.C. closing power hold switch labeled 852A or B,
"Key Switch" in the ON position.

The Committee recommended approval of these changes with modifications.

FORC ITEM
NUMBER:

DESCRIPTION

75-841

E-1.1, Safety Injection System Actuation, PCN75-1451. B. Snow presented this procedure change. It requested to change step 3.6 to read;

3.6 - Upon actuation of STS and opening of Valves 852A and 852B assure both valves have opened and the D.C. closing hold switch labeled "852A Key Switch" and "852B Key Switch" are still in the OFF position.

The Committee recommended approval of this change.

75-851

A-2.4.2, Calculation Of Cold Shutdown Boron Concentration, PCN75-1380. W. Backus presented this new procedure. This procedure provides the method for calculating the Cold Shutdown Boron Concentration. The Committee recommended approval of this procedure. This item IS COMPLETE.

75-852

M-40.4, Visual Leakage Inspection Of High Energy Piping, PCN75-1408. W. Backus presented this new procedure. The purpose of this procedure is to outline a method for visual leakage inspection of the high energy piping outside containment. The Committee recommended approval of this procedure.

75-853

W. Backus presented these procedure for deletions;

ST-71-3, Test On Cutting A Diesel Generating Unit Free From System When Carrying Auxiliary Load, PCN75-1175.

ST-72-2, Xenon Oscillation Test, PCN75-1176.

ST-72-3, Part Length Rod Removal Test - Safety Evaluation, PCN75-1177.

ST-73-3, Special Load Test For 40 Ton Auxiliary Building Overhead Crane (Main Hook) PCN75-1178.

ST-73-5, Xenon Transient Test, PCN75-1179.

ST-73-6, Measure Radiation Level Of Thimble Plugging Insert, PCN75-1180.

ST-73-10, To Test Integrity Of Control Air System To Steam Line Power Relief Valves, PCN75-1181.

ST-74-1, Reactor Core Decay Test, PCN75-1182.

ST-74-11, Moveable Detector Response Test, PCN75-1184.

The Committee recommended approval of these deletions.

75-854

M-11.5D, Electrical Inspection of 1B Auxiliary Motor Driven Pump, PCN75-1427. W. Backus presented this procedure change was requested to change the title to read;

"Electrical Inspections of The Motor Driven Auxiliary Feed Pumps For _____ Pump", also to add a new step 2.4 and a NOTE after instructions. The Committee recommended

PROC ITEM
NUMBER:

DESCRIPTION

75-855

M-49.1, Isolation Of L1A And L2A Busses, PCN75-1107. W. Backus presented this procedure change. It requested to add NOTE after 5.22 to read;

NOTE: Relay settings shall not be change without approval of the Maintenance Engineer

The Committee recommended approval of this change.

75-856

EM-45, Repair Of Blend System Valve FCV-110C, PCN75-1002. W. Backus presented this procedure change. It was requested the procedure be changed from EM-45 to M-37. The Committee recommended approval of this change.

75-857

RSSP-2.7, Safety Injection Sequence Timers, Train A & B, PCN75-960. W. Backus presented this procedure change. It requested step 6.21.5 be changed to read 52/CF1C, steps 6.26.6 should read Containment Fan 1C. The Committee recommended approval of this change.

75-858

RSSP-10.1, Conditions For Main Steam Safety Valve Test, PCN75-1114. W. Backus presented this procedure change. It was requested step 6.1 be changed to read;

"Reduce T avg to about 543°F by use of atmosphere steam relief, S/G blowdown rate, or by condenser steam dump"

The Committee recommended approval of this change.

75- 859

RF-9.9, Control Rod Drive Shaft Unlatching Tool Operating Instruction, PCN75-1280. W. Backus presented this procedure change. It requested to add to data sheets, pages 7-10;

Date Completed: _____

Completed By: _____

The Committee recommended approval of this change.

75-860

RF-9.8.2, Part Length (P/L) CRDM Drive Shaft Relatching, PCN75-718. W. Backus presented this procedure change. It requested in step 1.4 to change second sentence to read;

"each phase resistance should be ---"

The Committee recommended approval of this change.

PORC ITEM
NUMBER:

DESCRIPTION

75-861

RF-2, Reactor Refueling Outage Operations And Activities, PCN75-1281. W. Backus presented this procedure change. It was requested step 5.59 be changed to read;

"Quality Control Inspection preclosure reload verification in accordance with QCIP-2"

The Committee recommended approval of this change.

75-863

M-11.5F, Instrument And Control Inspection Of The Motor Driven Auxiliary Feed Pumps, For _____ Pump, PCN75-1425. W. Backus presented this procedure change. It requested to add to the data sheets, pages 5-10;

Date Completed: _____
Completed By: _____

The Committee recommended approval of this change.

75- 864

M-11.5F, Instrument And Control Inspection Of The Motor Driven Auxiliary Feed Pumps For _____ Pump, PCN75-1425.

&

M-11.5F Fitters Inspection Of Motor Driven Auxiliary Feed Pumps For _____ Pump, PCN75-1426. W. Backus presented these two procedures for changes. It was requested the titles be changed, also to have Shift Foreman and PORC Review Date added to the sign-offs. The Committee recommended approval of these changes with modifications.

75- 865

RF-8.4, Fuel & Core Component Movement In The Spent Fuel Pit, PCN75-1430. W. Backus presented this procedure change. It requested the existing appendix A be removed and replaced with a new one, also to attach location sheet of Spent Fuel Pit for information only. The Committee recommended approval of this change.

75- 866

O-5.2, Load Increases, PCN75-1432. W. Backus presented this procedure change. It requested;

- 1) Change step 5.7.1 to read: If the average axial flux difference deviates from it's target band, reduce reactor power to less than 90% of rated power.
- 2) Add to 5.7.2 & 5.8.1, if the computer is out of service.
- 3) Eliminate from step 5.8.2 - either immediately take a partial flux map per administrative procedure A-2.10.1 or A-2.10.2 or -----.

The Committee recommended approval of these changes.

ITEM
SER:

DESCRIPTION

367

EM-137, "A" Main Steam Line Weld Repairs, (Outside Containment)
PCN75-1276. W. Backus presented this procedure change. It requested;

1) Add to precautions;

4.5 When weld repairs, grinding or cutting are made on piping O.D., provide fire protection devices such as fire resistant blankets in the area of repairs.

2) Add step 5.4.1;

Q. C. Cleanliness Inspection of piping ID in area of weld repairs (If no repairs are made on the ID of piping welds, this step may be marked N.A.) Q.C. Inspector _____

The Committee recommended approval of these changes.

75-868

EM-140, Replacement Of Rod Drive Housing Cap, PCN75-1274.
W. Backus presented this procedure change. It was requested;

add new precaution 4.4 - Provide fire protective devices such as fire resistant blanket when grinding and welding

Reword step 5.3 as follows: Establish a clean work area by utilizing fire resistant blanket or equivalent in the work area to retain metal chips or grinding.

The Committee recommended approval of these changes.

75-869

EM-10B, Repair Of Leak On Shutdown To Reactor Coolant Filter, PCN75-1275. W. Backus presented this procedure change. It was requested;

Add precaution 4.2 - Provide adequate fire protective devices such as fire retardant blankets, as necessary, when grinding, cutting and welding.

Add new step 5.7.1 - Clean general area of all debris from welding and grinding

The Committee recommended approval of these changes.

PROC ITEM
NUMBER:

DESCRIPTION

75-870

EM-107, Inspection And Repair Of Safety Injection System Relief Valve 887, PCN75-924. W. Backus presented this procedure change. It requested to change step 5.4 to read;

5.4 - Remove relief valve and perform RSSP-12
"Testing of Primary and Secondary Relief Valves on Test Stand".

Change Procedure to M series (M-37)

The Committee recommended approval of these changes.

75-872

RSSP-2.1, Safety Injection Functional Test, PCN75-923, PCN75-878. W. Backus presented this procedure change.

PCN75-923 - was to facilitate steps N.A.'d.

PCN75-878 - This was disapproved as changes were previously done

The Committee reviewed PCN75-923 and recommended disapproval of PCN75-878.

75-873

EM-144, Power Supply PS-1 in IAC Rod Control Power Cabinet, PCN75-1456. C. Edgar presented this new procedure. This procedure is to describe the operations necessary to check and/or replace Power Supply PS-1 in the IAC Rod Control Power Cabinet. The Committee recommended approval of this procedure.

75-875

E. Beatty presented the following temporary procedure changes for review only. All PCN's were written to facilitate steps N.A.'d.

O-1.2, Plant From Hot Shutdown to Steady Load, PCN 75-1472

O-2.1, Normal Shutdown to Hot Shutdown, PCN 75-1474.

ST-75.4, Turbine Plant Start-up Program Following Plant Trip of 0956 Hours on 6/17/75, PCN 75-1473.

PT-2.3, Safeguard Valve Operation, PCN 75-1464.

PT-33.0, Startup Test Program, PCN 75-1410.

RF-8.4, Fuel & Core Component Movement In the Spent Fuel Pit, PCN 75-1453.

PROC ITEM
NUMBER:

DESCRIPTION75-876

SM-75-33.1, Installation of A Key Switch On the D.C. Power For MOV 852A & B, PCN 75-1440. E. Beatty presented this temporary procedure change for review.

Add Initial Condition:

3.4 Prior to holding a valve, all valves in the system that provide the duplicate function shall be tested to demonstrate operability

Add Precaution:

4.2 No valve may be inoperable for a period exceeding 12 hours.

Add Instruction:

5.1 Cycle Valve 852B for operability _____
Renumber Steps Accordingly
Tech Spec. Sect. 3.3

The Committee reviewed this change.

75-878

GS-21.0, Site Access Control (Security) and Personnel Identification, PCN 75-1435. E. Beatty presented the following procedure changes. It requested:

Step 3.5.1.3 - Change GAI personnel

Step 3.5.3.3 - to; by the RC&E Project Manager

Also some minor changes to correct typing errors.

The Committee recommended approval of these changes.

75-879

GS-31.0, Door Alarm System, PCN 75-1463. E. Beatty presented this procedure change. It requested Step 3.13 be changed to read: Ginna Station Event Report (A-25.1)

75-880

SM 75-27.1, Installation of A Key Switch on the D.C. Power For MOV 896 A & B, PCN 75-1441. E. Beatty presented the following procedure changes. It requested:

Insert new Step 5.20 as follows:

To continue procedure for V-896B refer back to Step 5.1 through Step 5.8, before continuing with Step 5.21.

After new Step 5.20, renumber the rest of step consecutively.

The Committee recommended approval of this change.

PORC ITEM NUMBER:	DESCRIPTION
75-882	<p>E. Beatty presented the following new CP's.</p> <p><u>CP-5.0, Calibration Alignment of T'Avg and Delta T at Zero Power Channel 1, PCN 75-1442.</u> The purpose of this procedure is to verify the calibration of the T'Avg and Delta T amplifiers at zero power using the actual R.T.D. measurement and calculating a true T'Avg and Delta T value for Dana Amplifier zero.</p> <p><u>CP-6.0, Calibration Alignment of T'Avg and Delta T at Zero Power Channel 2, PCN 75-1443.</u> The purpose of this procedure is to verify the calibration of the T'Avg and Delta T amplifiers at zero power using the actual R.T.D. measurement and calculating a true T'Avg and Delta T value for Dana amplifier zero.</p> <p><u>CP-7.0, Calibration Alignment of T'Avg and Delta T at Zero Power Channel 3.</u> The purpose of this procedure is to verify the calibration of the T'Avg and Delta T amplifiers at zero power using the actual R.T.D. measurement and calculating a true T'Avg and Delta T value for Dana Amplifier zero.</p> <p><u>CP-8.0, Calibration Alignment of T'Avg and Delta T Zero Power Channel 4.</u> The purpose of this procedure is to verify the calibration of the T'Avg and Delta T amplifiers at zero power using the actual R.T.D. measurement and calculating a true T'Avg and Delta T value for Dana Amplifier zero.</p>
The Committee recommended approval of these procedures.	
75-883	<p><u>S-12.2, Operator Action In The Event of Indication of Sufficient Increase In Leakage, PCN 75-1433.</u> E. Beatty presented this new procedure. The purpose of this procedure describes the steps to be taken in the event of a significant increase in leakage on any of our leak detection systems. The Committee recommended approval of this procedure.</p>
75-884	<p><u>S-12.3, Operator Action For Significant Leakage That Cannot Be Located, PCN 74-1436.</u> E. Beatty presented this new procedure. This procedure describes the steps to be taken if a significant increase in leakage cannot be identified using S-12.2. The Committee recommended approval of this procedure with modifications.</p>
75-885	<p><u>EM-144, Power Supply PS-1 In 1AC Rod Control Power Circuit PCN 75-1459, PCN 75-1470.</u> E. Beatty presented these procedure changes.</p>
PCN 75-1459	<p>Add Steps 5.5.1 and 5.7.1 as follows:</p> <p>5.5.1 Install jumper from PS-2 Terminal 4 to Neutral Bus.</p> <p>5.7.1 Remove jumper installed in Step 5.5.1</p>

PROC ITEM
NUMBER:DESCRIPTION

- 75-886 PT-2.3, Safeguard Valve Operation, PCN 75-1454. E. Beatty presented this procedure change. Several new steps were added under Precautions. The Committee recommended approval of this change.
- 75-887 A-2.2, 1/M Curves, PCN 75-1458. E. Beatty presented this procedure change. It requested to revise 1/m plant in the procedure. The Committee recommended approval of this change.
- 75-888 E-28, Reactor Coolant Leak, PCN 75-1452. E. Beatty presented this procedure change. It was requested in Step 2.4.2 to change words "pressurizer relief valves" to read "containment depressurization valves." The Committee recommended approval of this change.
- 75-889 S-4.1G, Laundry Release, PCN 75-1455. E. Beatty presented this procedure change. It was requested to change Step 5.19 to read "start laundry pump controller for the selected tank to be released." The Committee recommended approval of this change.
- 75-890 E-1.2, Loss of Coolant Accident, PCN 75-1462. E. Beatty presented this procedure change. It was requested to delete Step 4.3.1.1 and renumber accordingly. The Committee recommended approval of this change.
- 75-897 Abnormal Occurrence 75-12, Failure Of Undervoltage Relay During Monthly Surveillance Test. C. Hartlieb reported to the Committee that during the performance of the Monthly Surveillance Procedure, PT-9, "Undervoltage and Underfrequency Protection" on July 25, 1975, upon opening the knife switches for the solenoid coil of primary undervoltage device 273/11B, it was discovered that the device armature did not drop down to its de-energized position. As a result of this failure to operate, the Reactor Trip Undervoltage Protection signal was not generated this relay. The Committee made the following recommendations:
- 1) All four undervoltage relays be cleaned
 - 2) Upon returning to service procedure PT-9 be performed again
- J. Witte presented Corrective Action Report 1029 covering this abnormal occurrence.

PROC ITEM
NUMBER:

DESCRIPTION75-898

E. Beatty presented the following new procedures;

M-11.13, Inspection & Maintenance Of Goulds Pumps, PCN75-1502.

This procedure is to be used for the inspection and maintenance of Goulds Pumps, Model 3196-STD and 3197-STD or similar centrifugal units.

CP-5.10, Calibration Alignment Of T'Avg. & Delta T at 70% Power or Greater For Loop A, Unit 1, PCN75-1465.

This procedure is to verify the calibration of T'Avg. and Delta T amplifiers at 70% power or greater using the actual R.T.D. measurement then calculating a true T'Avg. and Delta T value.

CP-7.10, Calibration Alignment of T'Avg & Delta T

At 70% Power, Or Greater For Loop B, Unit 1, PCN75-1467.

This procedure is to verify the calibration of T'Avg. and Delta T amplifiers for Loop B, Unit 1, at 70% power or greater using the actual R.T.D. measurements then calculating a true T'Avg and Delta T value.

CP-8.10, Calibration Alignment Of T'Avg & Delta T At 70% Power, Or Greater For Loop B, Unit 2, PCN75-1468.

This procedure is to verify the calibration of T'AVG and Delta T amplifiers, Loop B, Unit 2, at 70% power or greater using the actual R.T.D. measurement then calculating a true T'AVG and Delta T value.

ST75.5, Monitor Six Steam Generator Levels Simultaneously From All Four Reactor Protection Channels, PCN75-1439. This procedure is to provide information thru recording of six narrow range steam generator levels as requested per Westinghouse.

The Committee recommended approval of these new procedures. This item IS COMPLETE.

75-899

E. Beatty presented the following temporary procedure changes for review only.

O-2.1, Normal Shutdown To Hot Shutdown, PCN75-1494, to facilitate steps marked N.A.'d.

ST-75-4, Turbine Plant Start-Up Program Following Plant Trip Of 0956 Hours On 6/17/75, PCN75-1493, was to facilitate step 4.2 N.A.'d.

P-9, Precautions, Limitations And Setpoints P-9 Radiation Monitoring System, PCN75-1481, requested R-12 alarm point is 30K, present reading ~27K alarm should be raised to 35K.

The Committee reviewed these changes.

PROC ITEM
NUMBER:

DESCRIPTION

75-900 0-1.2, Plant From Hot Shutdown To Steady Load, PCN75-1495, 1492.
E. Beatty presented these procedure changes.

PCN75-1495 - requested several steps N.A. 'd also step 5.4.5.4
read valve MOV 4007 closed.

This was temporary for review only.

PCN75-1492 - requested sign-off lines be added to steps 5.4.2 and
5.4.6.13.

The Committee reviewed PCN75-1495 and recommended approval of
PCN75-1492.

75-906 S-23.2.2, Containment Purge Procedure, PCN75-1488. E. Beatty
presented these procedure changes. It was requested;

Add step 3.3 - "A containment release permit has been
completed by Health Physics _____"

Under step 5.2 - eliminate "R10A on containment ----"
add; Install new filters on R10A (Notify H.P. Tech.) _____
add; R-10A valved to containment vent sample
line (notify H.P. Technician) _____

Under step 5.3 - after "close PEVO" add;
"change filters on R10A - - Notify H.P. Tech. _____"

Eliminate old appendix and add new one.

The Committee recommended approval of these changes.

75-909 0-1.10, Pre-Heatup Plant Requirement Check List, PCN75-1491.
E. Beatty presented this procedure change. It requested to
add step 3.1.3 to read;

"Plant at hot shutdown following reactor trip
after turbine trip"

The Committee recommended approval of this change.

75-910 S-15.1, Flux Mapping Normal Procedure, PCN75-1489, 1489.
E. Beatty presented these procedure changes.

PCN75-1489 - requested step 5.54 be deleted

PCN75-1490 - requested step 5.25 to add "Note: Detector A, Path #2
is plugged. For this path only, DO NOT BY-PASS, Turn
Detector A path selector to path #2 and turn detector
A operator selector to OFF. For pass #3 turn detector
A path selector to path #3 and turn Detector A, Operator
Selector to Normal"

The Committee recommended approval of these changes.

PROC ITEM NUMBER:	DESCRIPTION
<u>75-917</u>	<u>CP-6.10, Calibration Alignment of T'Avg & Delta T At 70% Power Or Greater, Loop A, Unit 2, PCN75-1466.</u> E. Beatty presented this new procedure for Delta T and T'Avg. Span for Reactor Protection, Channel TT. The Committee recommended approval of this procedure.
<u>75-919</u>	<u>PP-16, Auxiliary Feedwater System Flow Check, PCN75-1514.</u> E. Beatty presented this procedure change. It requested in step 6.1.10.1 MOV 4007 pump discharge valve is to be left closed. The Committee recommended approval of this change.
<u>75-922</u>	<u>M-37.16J, Inspection & Maintenance Of The Motor Driven Auxiliary Feed Pump Recirculation Control Valve, PCN75-1482.</u> E. Beatty presented this procedure change. It was requested under Initial Conditions to change step 3.2 to read "Secure air supply to valve positioner and remove sensing line" Step 5.0 under Instructions: Change step 5.13 to read; "Sensing line installed and air supply to positioner turned on" The Committee recommended approval of these changes.
<u>75-924</u>	<u>RP-2, Reactor Refueling Outage Operations And Activities, PCN75-1997, 75-1498.</u> E. Beatty presented these procedure changes. <u>PCN75-1997</u> - requested two new steps be incorporated into this procedure. Step 5.59.1 - Inventory fuel in core by comparison of Vido Tape to Fuel Status Records Step 5.108 - Inventory fuel in spent fuel pit by verifying there is a fuel assembly in each location indicated on the fuel status records <u>PCN75-1498</u> - Step 5.109, Inventory fuel in new fuel storage by comparing fuel assembly number and location against the Fuel Status Records. The Committee recommended approval of these item changes.
<u>75-928</u>	<u>PP-2.3, Safeguard Valve Operation, PCN75-1067.</u> E. Beatty presented this procedure change. It requested to delete steps 6.6 thru 6.6.8 and incorporate them into PP-3. The Committee recommended approval of this change.
<u>75-929</u>	<u>PP-3, Containment Spray Pumps And NaOH Additive Systems, PCN75-1515.</u> E. Beatty presented this procedure change. It requested steps from PP-2.3 be incorporated into this procedure. The Committee recommended approval of this change.

PROC ITEM
NUMBER:

DESCRIPTION

75-930

S-23.2.2, Containment Purge Procedure, PCN75-1513. E. Beatty presented this procedure change. It requested;

- 1) add to step 5.1 - Calculation will be completed by Health Physicist on Containment purge release form supplied by H.P.
- 2) Change 5.2.16 to read: "Monitor R-10A, 11 & 12 to satisfy that purge satisfies release limits. Record R-10A, 11 & 12, wind speed and direction on Containment purge release form"
- 3) Change 5.3.9, Insert RLO between "Reset _____ alarm"

The Committee recommended approval of these changes with modifications. This item IS COMPLETE.

75-931

GS-21.0, Site Access Control (Security) And Personnel Identification, PCN75-1527. E. Beatty presented this procedure change. It requested in step 3.5.1.1 to change with escort to without escort. The Committee recommended approval of this change. This item IS COMPLETE.

75-932

E. Beatty presented the following procedures for deletion;

RSSP-7.1, Zero Power T Average and Delta T Alignment, PCN75-1501.

RSSP-7.2, At Power T Avg. & Delta T Span Adjustment, PCN75-1469.

ST-72.1, Hydrogen Peroxide Addition To Reactor Coolant During Cooldown, PCN75-1506.

ST-73.4, "DB-75, DB-50 and DB-25 Circuit Breaker Overcurrent Trip Device Test and/or Replacement", PCN75-1503.

ST-73-8.1, Vibration Testing Program Isolation of Steam Dump System For Vibration Data Aquisition, PCN75-1504.

&

ST74.3, Freeze Plug on 3" Service Water Return From The Motor Coolers, C.V. Penetration Cooler & Safety Injection Pump Bearing Cooler, PCN75-1505.

The Committee recommended approval of these deletions.

75-933

EM-145, Immediate Boration Integrator YM-113, PCN75-1534. D. Francis presented this new procedure. This procedure is to provide maintenance on YM-113. The Committee recommended approval of this procedure with modifications.

PROC ITEM
NUMBER:

DESCRIPTION

75-943

A-1, Radiation Control Manual, PCN 75-1542. E. DeMeritt presented several changes to update the manual. The Committee recommended approval of these changes with minor modifications.

75-945

A-2-3, Maximum Unit Power, PCN 75-1528. S. Bullock presented this procedure change. It requested:

Remove last three lines from Page 2 (Calorimetric Sheet): Q.C. Review; Received Central Records; Non-Permanent Disposition Date.

The Committee recommended approval of this procedure.

75-948

SM 75-15, Auxiliary Building Ventilation High Radiation Interlock. G. Canori of Engineering presented the Design Criteria and Safety Analysis for this SM. The Committee performed the Safety Evaluation and found that the modification does not involve an unreviewed safety question and that Technical Specifications are not affected.

75-949

SM 75-30.1, Reactor Trip Underfrequency Protection Bus 11A, PCN 75-1553. D. Gent presented this new procedure. The purpose of this procedure is to outline the steps for removing and replacing protection relays 811/11A and 812/11A with a new model KF relay to reduce probability of a relay malfunction. The Committee recommended approval of this procedure.

75-950

SM 75-30.2, Reactor Trip Underfrequency Protection Bus 11B, PCN 75-1554. D. Gent presented this new procedure. The purpose of this procedure is to outline the steps for removing and replacing protective relays 811/11B and 812/11B with a new model KF relay, to reduce probability of a relay malfunction. The Committee recommended approval of this procedure.

75-953

S-4.1E, Waste Condensate Release, PCN 75-1523. E. Beatty presented this temporary change for review only. It requested:

On Page 2 following valves changed:

9108	Closed	9110	Open
9107	Closed	9109	Open
1771	Closed	1794	Open
2612	Closed		
1773	Closed		

The Committee reviewed this change.

75-955

RF-28, Disposal of Irradiated Flow Mixers, PCN 75-1533. E. Beatty presented this new procedure. The purpose of this procedure is to describe the steps necessary to move irradiated flow mixers from the spent fuel pit to shielded waste shipping drum. The Committee recommended approval of this procedure.

PROC ITEM
NUMBER:DESCRIPTION

- 75-958 M-11.19, Inspection and Maintenance of Crane-Deming Pumps, PCN 75-1517. E. Beatty presented this new procedure. This procedure is to be used for the inspection and maintenance of Crane-Deming Pumps, Model 3361 and 3065. The Committee recommended approval of this procedure.
- 75-959 M-11.18, Inspection and Maintenance of Buffalo Forge Pumps, PCN 75-1518. E. Beatty presented this new procedure. This procedure is to be used for the inspection and maintenance of Buffalo Forge Pumps, Model 3 x 1/2 AB. The Committee recommended approval of this item.
- 75-960 M-11.17, Inspection and Maintenance of Weinman Pumps, PCN 75-1519. E. Beatty presented this new procedure. This procedure is to be used for the inspection and maintenance of Weinman Pumps, Model WE3300 U and 1 1/2-21CB-2A. The Committee recommended approval of this item.
- 75-961 M-11.16, Inspection and Maintenance of Durco-Dunrion Pumps, PCN 75-1520. E. Beatty presented this new procedure. This procedure is to be used for the inspection and maintenance of Durco-Dunrion Pumps, Model 3X2S - 10/74. The Committee recommended approval of this procedure.
- 75-962 M-11.15, Inspection and Maintenance of Pacific Pumps, PCN 75-1521. E. Beatty presented this new procedure. This procedure is to be used for the inspection and maintenance of Pacific Pumps, Model 6L-SVC. The Committee recommended approval of this procedure.
- 75-964 P-9, Radiation Monitoring System, PCN 75-1543. E. Beatty presented this change. It requested to change Page 6 to agree with the procedure. The HP Department will review P-9 for adequacy of the alarm setpoint and alarm basis. The Committee recommended approval of this change.
- 75-965 A-2.3, Maximum Unit Power, PCN 75-1524. E. Beatty presented this procedure change. It requested to add:
Step 3.5 regarding computer being out of service.
The Committee recommended approval of this change with modifications.
- 75-967 O-1.2, Plant From Hot Shutdown to Steady Load, PCN 75-1531. E. Beatty presented this procedure change. It requested:
On Page 4, under second "PRECAUTION", third line, change 9% ΔK/K (900 pcm) to .5% ΔK/K (500 pcm)
The Committee recommended approval of this change.

<u>PROC ITEM NUMBER:</u>	<u>DESCRIPTION</u>
<u>75-968</u>	<u>M-11.13, Inspection and Maintenance of Goulds Pumps, PCN 75-1532.</u> E. Beatty presented this procedure change. It requested: On Page M-11.13:3-Add Mechanic Foreman signoff. The Committee recommended approval of this change.
<u>75-969</u>	<u>PT-23.24, Containment Isolation Valve Leak Rate Testing Reactor Support Coolers (Inlet and Outlet). PCN 75-1536.</u> E. Beatty presented this procedure change. It requested to make minor changes to update the procedure. The Committee recommended approval of these changes.
<u>75-970</u>	<u>PT-23.30, Containment Isolation Valve Leak Rate Testing Auxiliary Coolant System Excess Letdown (Supply and Return) PCN 75-1537.</u> E. Beatty presented this procedure change. It requested to make minor changes to update the procedure. The Committee recommended approval of these changes.
<u>75-971</u>	<u>PT-23.19, Containment Isolation Valve Leak Rate Testing Safety Injection System, PCN 75-1540.</u> E. Beatty presented this procedure change. It requested to make minor changes to update the procedure. The Committee recommended approval of these changes.
<u>75-972</u>	<u>PT-23.50A, Containment Isolation Valve Leak Rate Testing Containment Post Accident Air Sample (Clean Int. Bldg.) PCN 75-1541.</u> E. Beatty presented this procedure change. It requested to make minor changes to update the procedure. The Committee recommended approval of these changes.
<u>75-973</u>	<u>PT-23.40, Containment Isolation Valve Leak Rate Testing Auxiliary Steam Supply and Condensate Returns PCN 75-1544.</u> E. Beatty presented this procedure change. It requested to make minor changes to update the procedure. The Committee recommended approval of these changes.
<u>75-974</u>	<u>S-4.1Y, Laundry Tank Release Using Chemical Drain Pump, PCN 75-1545.</u> E. Beatty presented this procedure change. It requested to make minor changes to correct the procedure. The Committee recommended approval of these changes.
<u>75-975</u>	<u>PT-23.51B, Containment Isolation Valve Leak Rate Testing "B" Hydrogen Recombiner (Pilot and Main) PCN 75-1548.</u> E. Beatty presented this procedure change. It requested to make minor changes to update the procedure. The Committee recommended approval of these changes.

FORC ITEM
NUMBER:

DESCRIPTION

- 75-976 O-5.1, Load Reductions, PCN 75-1460. E. Beatty presented this procedure change. It requested:
- Delete steps 5.6 (all) and 5.7 (all)
Add in place copy of Tech. Spec. sections 3.10.2.9 - 3.10.2.12
- The Committee recommended approval of this change.
- 75-977 O-5.2, Load Increases PCN 75-1461. E. Beatty presented this procedure change. It requested:
- Delete precautions in step 5.1.4
Delete steps 5.7 (all) and 5.8 (all)
Add in place copy of Tech. Spec. sections 3.10.2.9 - 3.10.2.12
- The Committee recommended approval of this change.
- 75-981 S-23.22, Containment Purge Procedure, PCN 75-1556. E. Beatty presented this temporary procedure change. It requested:
- Step 5.3.9 N/A this step.
- The Committee reviewed this change. The Committee recommended the procedure be changed to allow performance of this step as directed by the Health Physicists.
- 75-982 SC-1.7C, Monitoring Site Radiation Level By TLD, PCN 75-1558.
E. Beatty presented this procedure change. It requested:
- Change Figure I as new marked copy
- Add new Table 1 on Site Environmental Locations.
- The Committee recommended approval of this change.
- 75-983 SC-1.7B, Field Determination of Iodine or Particulate, PCN 75-1557.
E. Beatty presented this procedure change. It requested:
- Step 3.4.2 Add, Remove end cap, to beginning of sentence
- Step 3.4.3 Remove (*) and substitute 4.5×10^{-5}
- * at bottom of page can be removed.
- The Committee recommended approval of these changes.
- 75-984 A-54.3, Cutting and Welding Permit, PCN 75-1525. J. Noon presented this new procedure. The purpose of this procedure is to outline the authorization and use of the Ginna Station Cutting and Welding Procedure. The Committee recommended approval of this procedure. /

FORC ITEM
NUMBER:

DESCRIPTION

75-990

PP-12.2, Emergency Diesel Generator 1B, PCN75-1586 and PCN75-1589.
E. Beatty presented these procedure changes for review only.
PCN75-1586 - requested steps 6.18 through 6.23 and step 6.31 be marked N.A. as data numbers not taken as they did not continue with Diesel Generator operation. Stopped Diesel Generator after Sync. to Bus 17.

PCN75-1589 - requested steps 6.18 through 6.23 be marked N.A. as only closure of Bus 16 breaker was tested.

75-991

P-9, Precautions, Limitations and Setpoints; P-9, Radiation Monitoring System, PCN75-1561. E. Beatty presented this procedure change for review only. It requested;

- a) change step 2.3.2.3 from alarm setting of 3×10^4 cpm to 3.5×10^4 cpm
- b) change page P-9:6 under R-12 alarm setpoint from 1×10^4 cpm to 3.5×10^4 cpm

It was recommended by PORC that this be made a permanent change of setpoint to 35K cpm. Health Physics Supervisor stated that 40K cpm would be approximately MPC.

75-992

E. Beatty presented the following new CP procedures:

CP-31.0, Calibration and/or Maintenance Of Source Range N-31, PCN75-1574.

CP-31.1, General Defeat Procedure For N-31, PCN75-1573.

CP-31.2, General Reinstate Procedure For N-31, PCN75-1575.

CP-31.3, Calibration Of Source Range N-31, PCN75-1576.

CP-31.4, Calibration Of Source Range N-31, Bistable Relay Drivers, PCN75-1577.

CP-32.0, Calibration and/or Maintenance Of Source Range N-32, PCN75-1572.

CP-32.1, General Defeat Procedure For N-32, PCN75-1571.

CP-32.2, General Reinstate Procedure For N-32, PCN75-1570.

CP-32.3, Calibration Of Source Range N-32, PCN75-1569.

&

CP-32.4, Calibration Of Source Range N-32, Bistable Relay Drivers, PCN75-1568.

These procedures replaced PT6.1A. The Committee recommended approval of these CP's.

FORC ITEM
NUMBER:

DESCRIPTION

75-993 PP-6.1A, Calibration Procedure, Source Range Channels, PCH75-1546.
E. Beatty presented this procedure change. It was requested to revise this procedure to A-30 format. FORC recommended disapproval of this request as a new CP series has been written to replace this procedure. It was recommended this procedure be eliminated.

75-996 O-6, Operation And Process Monitoring, PCH-75-1562. E. Beatty presented this procedure change. It was requested to add to instructions;

"Do not exceed differential pressure limits on H₂ cooler of 5.2 psid or condensate cooler of 10 psid. If necessary reduce load"

Reason for this is to prevent damage to coolers. The Committee recommended approval of this change.

75-997 PP-3, Containment Spray Pumps And NaOH Additive System, PCH75-1560.
E. Beatty presented this procedure change. It requested to remove step 6.1.7 and make step 6.1.8 new 6.1.7. Reason for this is Valve 873B needs to be left closed for testing NaOH system operability. Valve is opened by step 6.13. The Committee recommended approval of this change.

75-998 RSSP-2.2, Diesel Generator Load And Safeguard Sequence Test, PCH75-1578.
E. Beatty presented this procedure change. It was requested under Initial Conditions to add step 4.10 to read;

"Multi pen recorder and/or events marker used for monitoring the Safeguard Bus voltages and the operations of the Safeguard train equipment"

TYPE EQUIP. USED _____ :

DATE CALIBRATED _____ :

Reason for this change is to improve procedure flexibility. The Committee recommended approval of this change.

75-999 RSSP-2.5, Service Water Pump Discharge Check Valves, PCH75-1567.
E. Beatty presented this procedure change. It was requested the following be added at end of procedure to update the procedure.

"Review/Approval FORC _____"

The Committee recommended approval of this change.

PROC ITEM
NUMBER:

DESCRIPTION

75-1000

RSSP-2.6, Emergency Diesel Start & Breaker Closure Time, PCN75-1581.
E. Beatty presented this procedure change. It requested to insert a new step under Initial Conditions to read;

4.4 - Visicorder or equivalent used to secure voltage trace.

TYPE EQUIP. USED _____:

DATE CALIBRATED _____:

Also to add at the end of the procedure "PROC Review _____"
This request is to update the procedure. The Committee recommended approval of these changes.

75-1001

RSSP-2.7, Safety Injection Sequence Timing Train A & B, PCN75-1582. E. Beatty presented this procedure change. It requested under Initial Conditions to add step 4.5 to read;

4.5 - Visicorder or equivalent used for monitoring indications.

TYPE EQUIP. USED _____:

DATE CALIBRATED _____:

This is to update the procedure. The Committee recommended approval of this change.

75-1002

RSSP-7.0, Control Rod Drop Test, PCN75-1583. E. Beatty presented this procedure change. It requested after step 4.5 to insert;

TYPE OF EQUIP. USED _____:

DATE CALIBRATED _____:

This is to update the procedure. The Committee recommended approval of this change.

75-1004

A-2.4.1, Calculation Of Hot Xenon Free Shutdown Margin, PCN75-1579.
E. Beatty presented this procedure change. It requested to;

Delete "Note" after step 3.2.3

Re-number Fig. 3 to Fig. 1

Re-number Fig. 4 to Fig. 2

Step 3.2.1 refer to Fig. 1

Step 3.2.2 refer to Fig. 2

Step 3.1 refer to Fig. 1

Reason for change is to renumber figures sequentially to eliminate any confusion. The Committee recommended approval of this change.

PROC ITEM
NUMBER:

DESCRIPTION

75-1005

0-3, Hot Shutdown With Xenon Present, PCN75-1580. E. Beatty presented the following procedure changes. It was requested to change the following steps to read;

Step 4.5 ----- At least - 2.45% AK/K
Step 5.5 ----- Less than - 2.45% AK/K
Step 5.5.3 --- To or greater than 2.45% AK/K

Reason for this change is that Figure 3.10-2 was taken out of Technical Specifications. The Committee recommended approval of this change.

75-1007

S-3.4K, Releasing Monitor Tank To Discharge Canal, PCN75-1567. E. Beatty presented this procedure change. This was rewritten to conform to A-30 format and the procedure reviewed. The Committee recommended approval of this change.
COMPLETE.

75-1008

M-15, Diesel Generator Maintenance, PCN75-1591. E. Beatty presented this procedure change. Several changes were made to the procedure to update it to conform to A-30 format. The Committee recommended approval of this change.

75-1009

T-18B, Turbine Main Steam Stop Valves Test, PCN75-1590. E. Beatty presented this procedure change. It requested a note be added between steps 5.3 and 5.4 to read;

NOTE: If the test is to be performed using test circuitry, perform steps 5.4 through 5.8 and mark steps 5.9 to 5.9.26 N.A. If the test is to be performed mechanically, steps 5.4 through 5.8 may be marked N.A. and proceed to step 5.9

also to add new steps 5.9 to 5.9.26 to the procedure to permit testing of the turbine stop valves without using stop valve test circuitry. The Committee recommended approval of this change.

75-1010

MT-12.2, Emergency Diesel Generator 1B, PCN75-1593. E. Beatty presented this procedure change for review. This was to document why step 5.21 was marked N.A. The reason was that the diesel was not run for 1/2 hour, but was run only long enough to check breaker closure on Bus 16. The Committee reviewed this change.

PORC ITEM NUMBER:	DESCRIPTION
<u>75-1013</u>	<u>EM-147, Replace Steam Generator Level Control LC-463F And/Or LC-473F, PCN75-1608.</u> J. Paris presented this new procedure. This procedure is for changing out steam generator level controller LC-463F/or LC-473F. Reason is to isolate feedwater control problem on A Steam Generator Loop. The Committee recommended approval of this procedure.
<u>75-1017</u>	<u>EM-148, Level Transmitter LT-461 to Control LP-463F and LC-480 in place of Level Transmitter LT-462, PCN75-1614.</u> Mr. Paris presented this procedure. The purpose of the procedure is to describe steps necessary to accomplish the recommendations of PORC Item 75-1016. The Committee recommended approval of this procedure.
<u>75-1019</u>	<u>O-5.1, Load Reductions, PCN75-1610.</u> E. Beatty presented this procedure change. It was requested to corrected a typing error in step 5.9. It should read "-----refer to O-5.2 load ----" The Committee recommended approval of this change.
<u>75-1020</u>	<u>RF-46, Special Nuclear Material Physical Inventory, PCN75-1601.</u> E. Beatty presented this new procedure. This procedure is to conduct a physical inventory of special nuclear material possessed at Ginna Station. This procedure is to comply with 10CFR 70.51 (d). The Committee recommended approval of this procedure.
<u>75-1022</u>	<u>PP-16, Auxiliary Feedwater System Flow Check, PCN75-1604.</u> E. Beatty presented this temporary change for review only. It requested in step 6.1.10.1, MOV 4007, pump discharge valve is to be left closed. Reason for this is the pump discharge check valve is leaking causing pump casing to heat up. Maintaining MOV 4007 closed blocks off leakage path to casing. The Committee reviewed this change.
<u>75-1029</u>	<u>P-9, Precautions, Limitations And Setpoints, P-9 Radiation Monitoring System, PCN75-1607.</u> E. Beatty presented this procedure change. It was requested to raise R20 alarm to 30K. The present setpoint of 20K gives intermittent alarm as background is 18 to 19K. Background activity increase is due to plate-out on SFP heat exchanger takes. The Committee recommended approval of this change.
<u>75-1030</u>	<u>Replacement Of Under Voltage Relays On 4 KV Bus 11A & 11B.</u> J. Scheetz presented this Design Criteria and Safety Analysis to the Committee. PORC reviewed and recommended approval of this Design Criteria. The Committee performed the Safety Evaluation and found that the modification does not involve an unreviewed safety question and that Technical Specifications are not affected.

PORC ITEM NUMBER:	DESCRIPTION
75-1040	<p><u>E-14, High Reactor Coolant Activity, PCN 75-1613.</u> B. Snow presented this procedure change. It requested in step 3.2.1 to change the words "is returning" to "has returned", as it is the requirement of the Technical Specifications that activity be within limits in 48 hours. The Committee recommended approval of this change.</p>
<u>75-1042 -</u>	<p><u>PT-3, Containment Spray Pump and NaOH Additive System, PCN75-1615.</u> B. Snow presented these procedure changes. It was requested to recorrect steps under "Containment Spray Pumps and NaOH Additive System" as indicated below;</p> <ol style="list-style-type: none"> <li data-bbox="386 611 1389 674">1. Step 6.4.3 in reference to "NOTE" - eliminate 6.3.4 and insert 6.4.4. <li data-bbox="386 709 1466 741">2. Step 6.7.7, insert sign-off line which is missing from step. <p>This is to correct typing errors. The Committee recommended approval of these changes.</p>
<u>75-1044</u>	<p><u>GS-10.0, Security Guards Actions During A Radiation Emergency, PCN75-1623.</u> B. Snow presented this procedure change. It was requested to add step 3.1.5 to read;</p> <p>"lock security office entrance door upon leaving for emergency survey center"</p> <p>This is to comply with change to SC-1.3C. The Committee recommended approval of this change.</p>
<u>75-1045</u>	<p><u>GS-11.0, Security Guard Actions During A Medical Emergency, PCN75-1672.</u> B. Snow presented this procedure change. It requested to add new step;</p> <ol style="list-style-type: none"> <li data-bbox="381 1339 1381 1402">3.4 - All security personnel involved in the emergency must carry a walkie-talkie <p>This is to improve communications during a medical emergency. The Committee recommended approval of this change.</p>
<u>75-1046</u>	<p><u>GS-12.0, Security Guard Actions During A Fire, PCN75-1621.</u> B. Snow presented this procedure change. It requested a new step 3.4 be added;</p> <ol style="list-style-type: none"> <li data-bbox="381 1669 1463 1732">3.4 All security personnel involved in the emergency must carry a walkie-talkie. <p>This is to improve communications during a fire. The Committee recommended approval of this change.</p>

PORC ITEM
NUMBER:

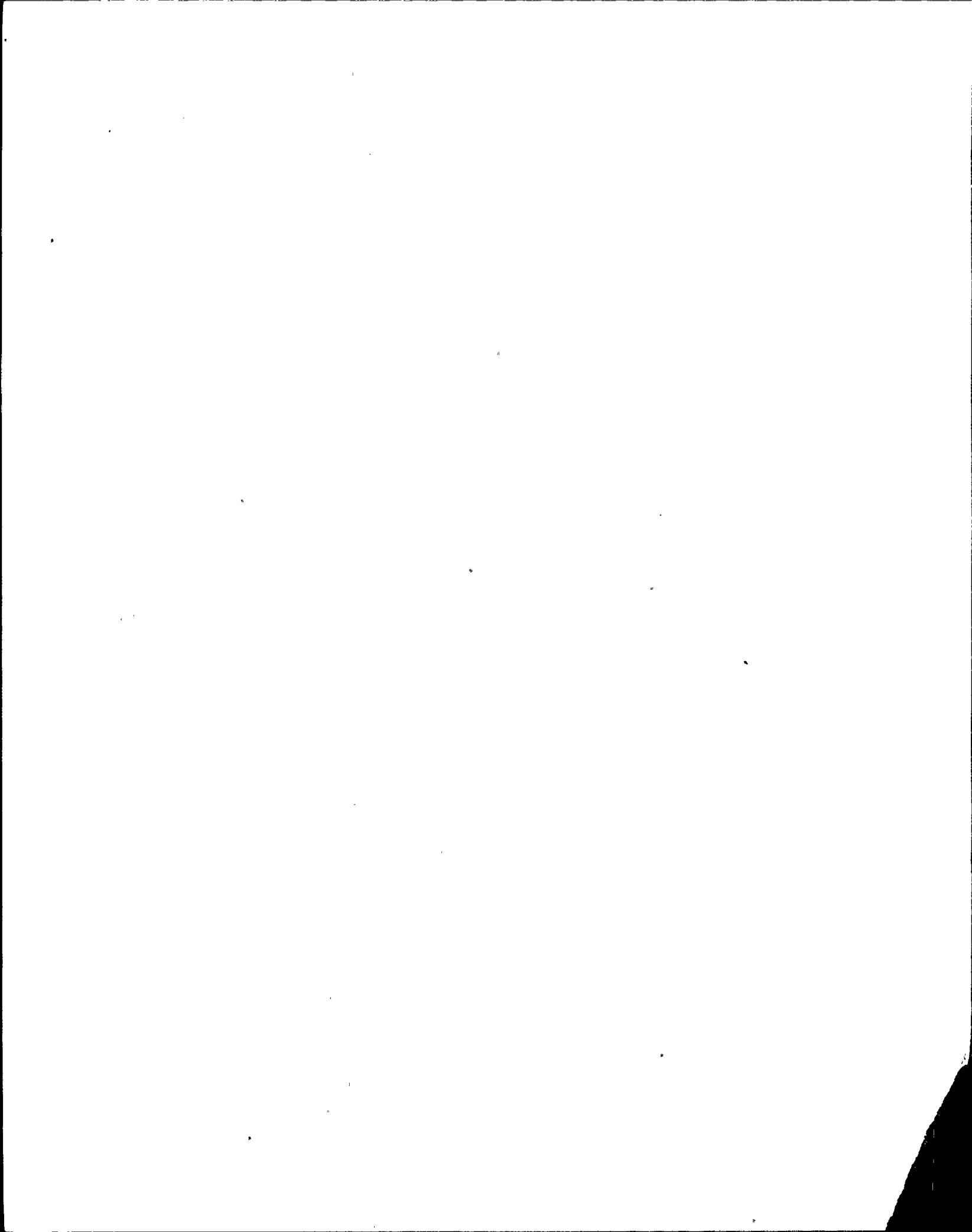
DESCRIPTION

- 75-1050 GS-21.0, Site Access Control (Security) And Personnel Identification, PCN75-1618. B. Snow presented this procedure change. It requested to add new step 3.1.1.5 to read;
- "Visitors to the site must read and sign the visitor instruction form at the security office"
- The Committee recommended approval of this change with modifications.
- 75-1054 S-23.2.2, Containment Purge Procedure, PCN75-1626.
B. Snow presented these procedure changes. It requested to add step 5.2.5 - "Set R-11 alarm set point at 1×10^5 cpm"
Step 5.2.7 change to read;
- "R011 and R012"
- This is to comply with P-9 and new computer addresses. The Committee recommended approval of these changes.
- 75-1055 O-6.2, Main Control Board System Status Verification, PCN75-1656.
B. Snow presented this procedure change. It requested in step 5.4.3 to change AOV 200A from closed to OPEN or as desired. This is because the procedure specifies all orifices valves to be closed. The Committee recommended approval of this change.
- 75-1056 PT-5.10, Process Instrumentation Reactor Protection Channel Trip Test, (Channel 1), PCN75-1638. B. Snow presented this procedure change which requested two new steps be added to incorporate the fact that the Controlling Transmitter for the A S/G feedwater level controller (LC-463F) was changed from Transmitter LT-462 to LT-461. The Committee recommended approval of these changes.
- 75-1057 PT-5.30, Process Instrumentation Reactor Protection Channel Trip (Channel 3), PCN75-1637. B. Snow presented these procedure changes. It requested steps 6.71, 6.72, 6.80 and 6.81 be deleted. The reason for this change is that the controlling transmitter for the A S/G feedwater level controller LC-463F was changed from transmitter LT-462 to LT-461.
- The Committee recommended approval of these changes.
- 75-1058 EM-149, Check Out Lift Circuitry For Control Rod G-5 In 2 BD Power Cabinet, PCN75-1685. C. Edgar presented this new procedure. This procedure is for checking the lift circuit of Rod G-5 in Rod Control Power Cabinet 2-BD. The Committee recommended approval of this procedure.

PROC ITEM
NUMBER:

DESCRIPTION

- 75-1062 PT-12.2, Emergency Diesel Generator 1B, PCN75-1643. E. Beatty presented this temporary procedure change. It requested to change the procedure to check operability of Bus 16 breaker. The Committee reviewed this change.
- 75-1064 M-40.5, Hydraulic Snubber Unit Inspection & Repair, (ITT Grinnell), PCN75-1676. E. Beatty presented this new procedure. This procedure outline the necessary steps for disassembly and reassembly of the hydraulic snubber units for purposes of inspection or repair. The Committee recommended approval of this procedure.
- 75-1066 A-2.4.2, Calculation Of Cold Shutdown Boron Concentration, PCN75-1687. E. Beatty presented this procedure change. It was requested to change steps 3.3 and 3.4 to;
"M¹ day/MTu to MW¹ day/MTu as the abbreviation was incorrect"
The Committee recommended approval of this change.
- 75-1069 E. Beatty presented the following CP'S requesting the same changes be made on each;
CP462.1, Defeat Of "A" Steam Generator Level Channel 462, PCN75-1642.
CP462.2, Reinstate Of "A" Steam Generator Level Channel 462, PCN75-1641.
CP-461.2, General Reinstate "461" Steam Generator Level, PCN75-1640.
The change requested general defeat section of the procedure to delete steps 5.3 and 5.4 as the controlling transmitter for level control changed from LT-462 to LT-461.
The Committee recommended approval of this change.
- 75-1070 E-23.1, Malfunction Of #1 Reactor Coolant Pump Seal, PCN75-1652. E. Beatty presented this procedure change. It requested a change to shut the pump down in 30 minutes instead of existing 24 hours when #1 seal fails. This is to conform to Westinghouse recommendation on control leakage reactor coolant pumps. The Committee recommended approval of this change.
- 75-1072 CP-17.1, Calibration Of Area Radiation Monitoring System Detectors, (R1-R9), PCN75-1670. E. Beatty presented this procedure for deletion. This procedure was succeeded by CP-209. The Committee recommended approval of this deletion.



PROC ITEM
NUMBER:

DESCRIPTION

- 75-1073 E-6.1, Loss Of Component Cooling During Power Operation, PCN75-1634. E. Beatty presented this procedure change. It requested step 4.5 be changed from 42% to 50% as P-8 setpoint is 50%. The Committee recommended approval of this change.
- 75-1074 M-11.10, Major Inspection Of Service Water Pump, PCN75-1662. E. Beatty presented this temporary procedure change for review only. This PCN was to facilitate steps marked N.A. as this procedure was used for minor inspection and lubrication. The Committee reviewed this change.
- 75-1075 M-40.2, Hydraulic Snubber Unit Inspection And Repair Procedure, PCN75-1677. E. Beatty presented this procedure change. It requested that in the NOTE under step 4.3 a spelling error be corrected from "it it" to "if it". The Committee recommended approval of this change.
- 75-1076 E. Beatty presented the following M procedures for changes;
- M-11.14, Inspection & Maintenance Of Ingersol Rand Pumps, PCN75-1663, 75-1669 - requested to change step 3.4 to read;
- 3.4 Notify Quality Control for maintenance inspection on component cooling water pumps (SN 0567-318, 319) and containment spray pumps (SN 0667 - 154, 155).
- PCN75-1669 - requested step 3.1.2 to read "gland leakage" not seal leakage.
- Step 5.8 should read; "Lubricate pump bearing per manufacturers instruction"
- Reason for this is not all pumps are equipped with seals, not all bearings are oil lubricated.
- M-11.17, Inspection & Maintenance Of Weinman Pumps, PCN75-1664. This requested step 5.8 be changed to read;
- "Lubricate pump bearing per manufacturer's instructions"
- This is because pump bearings are grease lubricated.
- M-11.18, Inspection & Maintenance Of Buffalo Force Pumps, PCN75-1665, requested step 5.8 be changed to read;
- "Lubricate pump bearings per Buffalo Force Bulletin 3321-B"
- Change step 3.1.2 to read; "gland leakage". This is as bearings are grease lubricated, glands are packed with packing.
- M-11.19, Inspection & Maintenance of Crane-Deming Pumps, PCN75-1666, requested step 5.8 be change to read;
- "Lubricate pump bearings per manufacturers instructions"

PROC ITEM
NUMBER:

DESCRIPTION

- 75-1077 - A-5h.3, Cutting And Welding Permit, PCN75-1654. E. Beatty presented this procedure change. This change requested several steps be added to the instructions to improve and clarify the procedure. The Committee recommended approval of this change with modifications.
- 75-1079 - RSSP-2.2, Diesel Generator Load & Safeguard Sequence Test, PCN75-1653. E. Beatty presented this procedure change. It was requested to have several steps added to update procedure and for improvement of instructions. The Committee recommended approval of these changes.
- 75-1080 E. Beatty presented the following CP's for the same changes to page 3, Data Sheet, to reflect the current PT-1 values for Delta T set-point program.
- CP-10.0, Calibration And/Or Maintenance Of Delta Flux Controllers, PCN75-1688.
- CP-20.0, Calibration And/Or Maintenance Of Delta Flux Controllers, PCN75-1689.
- CP-30.0, Calibration And/Or Maintenance Of Delta Flux Controllers, Channel 3, PCN75-1690.
- CP-40.0, Calibration And/Or Maintenance Of Delta Flux Controllers, PCN75-1691.
- The Committee recommended approval of these changes.
- 75-1081 PT-5.10, Process Instrumentation Reactor Protection Channel Trip Test (Channel 1), PCN75-1667. E. Beatty presented this procedure change. It was requested in step 6.89 (SF \leq FWS) be changed to read (SF \leq FWF) also in step 6.99 to add "if desired" for better operation.
- 75-1082 SC-1.7A, Emergency Radiation Monitoring Procedure, PCN75-1655. E. Beatty presented this procedure change. It requested off-site team #1, step 6F, off-site team #2, step 6F, off-site team #3, to change emergency phone numbers to update the procedure. The Committee recommended approval of these changes.
- 75-1085 PT-6.4, Excure/Incore Recalibration, PCN75-1648. E. Beatty presented these procedure changes. It requested to add two new steps to read;
- Step 6.1.1 - Enter value in K4021 of "-1" this inhibits axial tilt program.
- Add new step 6.11 - Enter value in K4021 of "0" this restarts axial tilt program.
- Technical Specifications allow inhibiting axial tilt program during recalibration. The Committee recommended approval of this change.

<u>PROC ITEM NUMBER:</u>	<u>DESCRIPTION</u>
<u>75-1087</u>	<u>PT-22.4, Equipment Hatch Between Door Volume Leakrate Test, PCN75-1693.</u> E. Beatty presented this procedure change for review only. This PCN was to facilitate steps marked N.A. The Committee reviewed this change.
<u>75-1089</u>	<u>S-26.4, Computer Rod Position Alarms, PCN75-1682.</u> E. Beatty presented this new procedure. This procedure is to provide description of the computer rod position alarm program which is designed to alarm when a control rod is more than 7.5 inches from its bank position. The Committee recommended approval of this new procedure.
<u>75-1090</u>	<u>A-57, Computer Technical Specification Related Programs, PCN75-1681.</u> E. Beatty presented this procedure change. This procedure is to define responsibilities and requirements for implementation of plant computer programs related to Technical Specifications requirements. The Committee recommended approval of this new procedure.
<u>75-1092</u>	<u>M-55.3, Removal, Inspection And Replacement Of Control Rod Drive Mechanism Coil Stack, PCN75-1696.</u> E. Beatty presented this new procedure. This procedure provides the guidelines for maintenance or replacement of a Magnetic Control Rod Drive Mechanism Coil Stack. The Committee recommended approval of this procedure.
<u>75-1097</u>	<u>S-4.1E, Waste Condensate Release, PCN75-1703.</u> E. Beatty presented this procedure change. It requested to change step 5.12 to read "Obtain a sample in a new bottle after allowing the sample line to purge for about 30 seconds". This is to specify sampling procedure. The Committee recommended approval of this change.
<u>75-1099</u>	<u>S-4.1G, "A" or "B" Laundry And Hot Shower Tank Release To Discharge Canal, PCN75-1701.</u> E. Beatty presented this procedure change. It requested to change step 5.13 to read; "Start laundry pump and recirc. tank for about 1/2 hour. Obtain a sample in a new bottle after purging the sample line for about 30 minutes. If the tank is to be transferred to the R.O. Unit, the sample bottle doesn't have to be new". This is to specify sampling procedure. The Committee recommended approval of this change.

PORC ITEM
NUMBER:

DESCRIPTION

75-1101

S-4.A, Liquid Waste Process Startup, PCN75-1635, 75-1704.
E. Beatty presented these procedure changes.

PCN75-1635 - requested correct spelling (WASTE) be made in step 5.1.2.

PCN75-1704 - requested to change step 5.1.8 to "after about 2 hours, open sample valve 1734 and purge the line for about 1 minute before obtaining sample".

This is to specify sampling procedure. -The Committee recommended approval of these changes.

75-1102

P-1, Precautions, Limitations and Setpoints P-1 Reactor Control And Protection System, PCN75-1644, PCN75-1645. E. Beatty presented these procedure changes.

PCN75-1644 - requested the following temporary changes for review only;

2.14.5.3.5 - Level Controller (Level Compensation)

2.14.5.3.6 - LC-463F & LC-473F

Filter lag time constant 7.2 sec. - Filter lag time constant 28 sec.

Proportional band 45% - Proportional bank 65%

2.14.5.3.7 - Flow compensation FC-466A & FC-476A

Proportional band 150% - Proportional band 150%

Reset Time Constant 30 min. - Reset Time Constant 30 min.

The Committee reviewed these changes.

PCN75-1645 - requested several sections of the procedure be deleted. Reason for this is due to changes in plant characteristics that make it necessary to adjust the proportional band and reset of a controller for optimum control. The Committee recommended approval of this change.

75-1103

M-37.10, Repair Of Blend System Valve FCV-110B. E. Beatty presented these procedure changes.

PCN75-75-1713 - was for review only. This PCN was to facilitate several steps marked N.A.

The Committee reviewed this request. The second request was in step 5.5.1 to include valve 368 to be closed before opening 367. The Committee recommended approval of this change.

PROC ITEM NUMBER:	DESCRIPTION
<u>75-1104</u>	<p><u>M-37.16, Repair Of Blend System Valve FCV-110C, PCN75-1712.</u> E. Beatty presented this procedure change for review only. This was to facilitate steps marked N.A., as only stroke adjustment was performed at this time.</p>
<u>75-1105</u>	<p><u>PT-16, Auxiliary Feedwater System Flow Check, PCN75-1711.</u> E. Beatty presented this procedure change for review only. This was to facilitate step marked N.A. as MOV 4007 is being kept closed due to leakage into pump casing. The Committee reviewed this change.</p>
<u>75-1106</u>	<p><u>O-1.1, Plant Heat Up From Cold Shutdown To Hot Shutdown, PCN75-1716.</u> E. Beatty presented this procedure change. It requested to;</p> <p>1) Add a new step 5.18 "Verify #1 seal leakoff \leq 0.3 GPM" as read on Fl-175 and Fl-176.</p> <p style="text-align: center;">Fl-175 - 1A RCP \leq 0.3 GPM _____ Fl-176 - 1B RCP \leq 0.3 GPM _____</p> <p>The Committee recommended approval of this change. This item <u>IS COMPLETE.</u></p>
<u>75-1107</u>	<p><u>S-2.1A, Pre-Startup Line-Up Of Reactor Coolant Pumps, PCN75-1715.</u> E. Beatty presented this procedure change. It requested;</p> <p>(1) Change step 5.4.6 to read "\leq 0.3 GPM #1 seal leakoff flow indication on RCP'S as read on Fl-175 and Fl-176."</p> <p>Reason for this is the minimum flow is needed before RCP pumps are started to protect seal faces from running dry. The Committee recommended approval of this change.</p>
<u>75-1112</u>	<p><u>GS-17.0, Accountability of Personnel During Emergency Conditions, PCN75-1735.</u> L. Brodie presented this new procedure. This procedure outlines security force responsibilities to ensure accountability of all personnel within the protected area, during an emergency condition. The Committee recommended approval of this procedure.</p>

PORC ITEM
NUMBER:DESCRIPTION75-1113

J. Brodie presented the following SC procedures;

SC-1.1, Radiation Emergency Procedures General Information,
PCN75-1736.SC-1.2, Local Radiation Emergency Procedure, PCN75-1741.SC-1.3A, Site Radiation Emergency (Shift Foreman & Control Room)
PCN75-1742.SC-1.3C, Site Radiation Emergency (Non-Operating Personnel)
PCN75-1743.SC-1.3B, Site Radiation Emergency (Emergency Coordinator & Survey
Center Assignees), PCN75-1752.SC-1.5, Evacuation Procedure, PCN75-1738.SC-1.7A, Emergency Radiation Monitoring Procedure, PCN75-1732.SC-1.7B, Field Determination Of Iodine Or Particulate
PCN75-1739.SC-1.8, Post Accident Environmental Sampling, PCN75-1744.SC-1.10, Planning, Conduct & Evaluation Of A Radiation Emergency
Drill, PCN75-1698.SC-1.11, Post Accident Re-Entry And Recovery Plan, PCN75-1737.SC-1.12A, Immediate Call List, PCN75-1709, 75-1733.SC-1.13, Estimating Off-Site Doses, PCN75-1748.SC-1.14A, Accountability Of Personnel, PCN75-1734.SC-1.14B, Search & Rescue Operation, PCN75-1649 & PCN75-1749.SC-1.15, Inspection Of Emergency Equipment, PCN75-1726.SC-1.16, Transfer Of Emergency Survey Center To Station 13A,
PCN75-1751.SC-1.7D, Emergency Radiation Monitoring Off-Site Survey Team,
#9D (Red), PCN75-1727.SC-1.7E, Emergency Radiation Monitoring On-Site Team #9E, (Blue)
PCN75-1728.SC-1.7F, Emergency Radiation Monitoring Off-Site Survey Team #9F,
(Green), PCN75-1729.

PORC ITEM
NUMBER:

DESCRIPTION

75-1113 SC-1.7G, Emergency Radiation Monitoring On-Site Survey Team #9G (Yellow), PCN75-1730.

SC-1.7H, Emergency Radiation Monitoring Off-Site Team #9H, (Orange) PCN75-1731.

Several of these change requests were to clarify the procedures and some were new procedures separating SC-1.7A into separate ones to help clarify survey teams instructions. The Committee recommended approval of these changes and new procedures.

75-1124 M-37.39, Temporary Repair Of Main Steam Safety Valve Flange Leak Using Furmanite America, Inc. Process, PCN75-1760.

J. Noon presented this new procedure. This procedure describes the steps necessary to facilitate a temporary repair of a Main Steam Safety Valve Flange Leak. The Committee recommended approval of this procedure.

75-1125 M-37.39, Temporary Repair Of Main Steam Safety Valve Flange Leak Using Furmanite America Inc. Process Valve No. _____, PCN75-1768. L. Brodie presented this temporary procedure change to be made permanent. Several changes were incorporated to allow for changes in method on injection sealer. The Committee recommended approval of these changes.

75-1114 M-37.39, Temporary Repair Of Main Steam Safety Valve Flange Leak Using Furmanite America Incorporated Process Valve No. PCN75-1772. B. Snow presented this temporary procedure change. This was to facilitate steps N.A.'d. Reason was the repairman was back to continue to try and stop additional leak. The Committee reviewed this change.

75-1115 PT-5.20, Process Instrumentation Reactor Protection Channel Trip Test (Channel 2), PCN75-1671. B. Snow presented this procedure change. It requested the following changes be made to improve clarity of the procedure.

Step 6.107 - Channel 1 test alarm should read; Channel 2 test alarm

Step 6.108 add; I/C Technician Initial _____

The Committee recommended approval of these changes. This item IS COMPLETE.

75-1116 RSSP-1.5, Valve Interlock Verification - Feedwater Isolation, PCN75-1754. B. Snow presented the following procedure changes. It was requested to change step 6.24 to read;

"Place the following Hand/Automatic transfer controllers to the manual position (MCB) and reduce manual control setting to zero"

This is to clarify what control is to be adjusted. The Committee recommended approval of this change.

PORC ITEM
NUMBER:

DESCRIPTION

75-1117

T-27.1, 1A Emergency Diesel Generator Pre-Start Alignment, PCN75-1767. B. Snow presented the following procedure change. It requested to change step 6.2.8 to read;

"Ensure the following switches in 1A Battery Room are on (1A Main D.C. Distribution Panel)"

Reason for this is the switches for the A Diesel Generator are on the 1A Main D. C. distribution panel. The Committee recommended approval of this change.

75-1118

PT-5.30, Process Instrumentation Reactor Protection Channel Trip Test (Channel 3), PCN75-1672, 1673, & 1674. B. Snow presented the following procedure changes to improve clarity of the procedure.

PT-5.30:4 - Step 6.19, Channel 2 should read Channel 3 & Omit word "TRIP"

PT-5.30:5 - Step 6.22, "goes out" should read "COMES ON"

PT-5.30:5 - Step 6.30, T should read ΔT

PT-5.30.6 - Step 6.35, T should read ΔT and add;
(1.71 °F; .912 MA = 40.40 MA + .912 MA = 41.31 MA).

Record on data sheet.

PT-5.30:9, Step 6.54 - add "Switch T/401B, located in S.D. Rack
NOTE: Deviation alarms may occur when switching occurs.

PT-5.30:9, Step 6.55 - add "(switch T/405F, located in R.I.L. rack)"
NOTE: Deviation alarms may occur when switching occurs

PT-5.30:13, Step 6.87 -add I/C Technician Initial _____

PT-5.30.16, Step 6.107 should read: Place feedwater by-pass valve controller HCV-481 to position desired.

PT-5.30:16, Step 6.111 add I/C Technician Initial _____

The Committee recommended approval of these changes.

75-1120

M-40.5, Hydraulic Snubber Unit Inspection and Repair Procedure, (147 Grinnel), PCN75-1725. B. Snow presented this procedure change. It was requested to change the last sentence of step 5.1.0 to read;

"Also note materials and their associated purchase order numbers used in rebuilding the snubber"

This is to maintain traceability of parts from procurement through installation as required by the Q.C. program. The

PORC ITEM
NUMBER:

DESCRIPTION

75-1122

PT-5.40, Process Instrumentation Reactor Protection Channel Trip Test, (Channel 4), PCN75-1675. B. Snow presented the following procedure changes. It requested the following changes be made;

- Step 6.17 - Remove word "TRIP"
- Step 6.48 - Add I/C Technician Initials _____
- Step 6.69 - "Low-Low Logic" should be "Low Level Logic"
- Step 6.70 - "Low-Level Logic" should read "Low-Low Logic"

The Committee recommended approval of these changes. This is to clarify and update the procedure.

75-1123

S-12.2, Operator Action In The Event Of Indication Of Significant Increase In Leakage, PCN75-1659, 75-1750. B. Snow presented these procedure changes;

PCN75-1659 - Requested to change step 1 to read;

"Containment purge system status change in the last 7 days"

PCN75-1750 - Requested to change step 1 to;

"Change 24 hours to 48 hours"

PORC recommended disapproval of these changes. B. Snow was assigned to investigate this further.

75-1128

A-52.2, Control Of Locked Valve Operation, PCN75-1660. B. Snow presented these procedure changes. It requested;

- 1) Add locked valve #1701 to locked valve list (valve #1701 locked closed) key number E509
- 2) Change lock number E-626 on valve 1762 to E-473 on page A-52.2:6

Reason for change #1 was that procedures call for 1701 to be locked closed. #2 is the lock that was cut off due to faulty mechanism and replaced by E-473.

The Committee recommended approval of these changes.

75-1129

M-11.14, Inspection And Maintenance Of Ingersol Rand Pumps, PCN75-1759. B. Snow presented this procedure changes;

Change 3.2 and 3.4 as follows:

- 3.2 - add; (Use procedure M-11.2 for Containment Spray Pump Isolation).
- 3.4 - If work is to be performed on the Component Cooling Water Pumps (Mod. No. 8 X 14 SD) or the Containment Spray Pumps (Mod. No. 4 X 11A) notify Quality Control.

This is to clarify and update the procedure. The Committee recommended approval of these changes.

PORC ITEM NUMBER:	DESCRIPTION
<u>75-1130</u>	<p><u>M-11.15, Inspection & Maintenance Of Pacific Pumps, PCN75-1756.</u> (RHR Pumps) B. Snow presented this procedure change. This procedure was rewritten as only "Pacific pumps are the RHR pumps". The Committee recommended approval of this rewrite. This item <u>IS COMPLETE.</u></p>
<u>75-1131</u>	<p><u>M-11.9B, Isolation And Maintenance Of 1B RHR Pump, PCN75-1757.</u> B. Snow presented this procedure for deletion. This procedure has been incorporated in M-11.15. The Committee recommended approval of this deletion.</p>
<u>75-1132</u>	<p><u>GS-10.0, Security Guards Actions During A Radiation Emergency, PCN75-1755.</u> B. Snow presented these procedure changes. It requested;</p> <p>Add: 3.1.3 "If required, follow procedures outlined in sections 3.2.2.6; 3.2.2.7, and 3.2.2.8."</p> <p>3.2.2.6 - Open emergency survey center, by breaking glass in key box and obtaining key</p> <p>3.2.2.7 - Activate tape recorder and test all communications equipment for operability</p> <p>3.2.2.8 - Standby to operate communication equipment as instructed until relieved by designated personnel (Emergency Tag #14)</p> <p>Reason for this is to change guard instructions to agree with SC-1.</p>
<u>75-1135</u>	<p><u>RSSP-1.3, Valve Interlock Verification Chemical Volume And Control System, PCN75-1753.</u> B. Snow presented these procedure changes. Several changes were requested to improve step sequence, clarify locations and update format. The Committee recommended approval of these changes.</p>
<u>75-1136</u>	<p><u>S-26.1, Computer Program Check, PCN75-1746.</u> B. Snow presented this procedure change. It requested in step 5.4 to change the last sentence to read;</p> <p>"----- Event Report SHALL also -----"</p>

This change is to insure uniform operator compliance with procedure. The Committee recommended approval of this change.

PROC ITEM
NUMBER:

DESCRIPTION

- 75-1137 S-26.2, Computer Out-Of-Service, PCN75-1745 and PCN75-1747.
B. Snow presented this procedure change. It was requested to add to Instructions;
- "Transfer reactor coolant pump bearing temperature monitoring from computer to recorder RK-30A"
- also to change the word should to SHALL in step 5.2. The Committee recommended approval of this change.
- 75-1138 EM-34, Repair Of Diaphragm Valve 365A (Blender to RWST Isolation Valve), PCN75-1724. B. Snow presented this procedure change. This procedure was put in A-30 format and made the 2 1/2 years review of the procedure. The Committee recommended approval of this change.
- 75-1141 EM-150, Temporary Repair Of Auxiliary Feedwater System Manual Isolation Valve 4012, PCN-75-1799. J. Noon presented this new procedure. This procedure describes the steps necessary to utilize the Furmanite process for sealing the water-steam leak at the body to bonnet joint of Valve 4012. This procedure is for temporary repair of Auxiliary Feedwater System Manual Isolation Valve 4012. The Committee recommended approval of this procedure.
- 75-1142 PT-22, Containment Penetration Leak Testing, PCN 75-1796
D. Gent presented this procedure change. It requested steps 22.18 and 22.19 be deleted and steps 22.21 and 22.22 be added:
- 22.21 - Mechanical Penetration-
Manifold "L" Leakrate Test
- 22.22 - Fuel Transfer Flange
Leakrate Test
- Reason for these changes are to correct error in numbering of procedure and to expand Initial Conditions. The Committee recommended approval of these changes.
- 75-1143 PT-22.21, Mechanical Manifold "L" Leakrate Test, PCN 75-1764. D. Gent presented this new procedure. This procedure outline steps necessary to perform a leakrate test of the mechanical penetrations on Penetration Pressurization System Manifold "L". This procedure is to provide steps for leakrate test. The Committee recommended approval of this procedure.
- 75-1144 PT-22.22, Fuel Transfer Flange Leakrate Test, PCN 75-1765.
D. Gent presented this new procedure. This procedure outlines the steps necessary to perform a leakrate test of the Fuel Transfer Flange penetration on Penetration Pressurization system manifold "L". This procedure is to provide steps for leakrate test. The Committee recommended approval of

PORC ITEM NUMBER:	DESCRIPTION
<u>75-1146</u>	<p><u>S-26.1, Computer Program Check, PCN 75-1775.</u> L. Brodie presented this procedure change. It requested to add Step 5.8.3:</p> <p>"Multiply value by 100/actual % power."</p> <p>Reason for this is to correct for power reduction. The Committee recommended approval of this change.</p>
<u>75-1147</u>	<p><u>P-9, Precautions, Limitations, and Setpoints P-9 Radiation Monitoring System, PCN 75-1773.</u> L. Brodie presented this temporary procedure change for review. It requested "temporary increase of setpoint on RMS Channel 19, to 20K > background." Reason for this is to blowdown Steam Generator's during ST 75-6. The Committee reviewed this temporary change.</p>
<u>75-1148</u>	<p>L. Brodie presented the following CP's. Several minor changes were requested to update and improve existing procedures.</p> <p><u>CP-2.0, Calibration and/or Maintenance On Rod Position Indication System, PCN 75-1781.</u></p> <p><u>CP-2.3.0, Rod Position Indication System "Shutdown Bank," PCN 75-1782.</u></p> <p><u>CP-2.4.0, Rod Position Indicator System "Part Length", PCN 75-1784.</u></p> <p><u>CP-2.5.0, Rod Position Indicator System "Bank A", PCN 75-1786.</u></p> <p><u>CP-2.6.0, Rod Position Indicator System "Bank B", PCN 75-1788.</u></p> <p><u>CP-2.7.0, Rod Position Indicator System "Bank C", PCN 75-1790.</u></p> <p><u>CP-2.8.0, Rod Position Indicator System "Bank D," PCN 75-1792.</u></p> <p>The Committee recommended approval of these changes.</p>
<u>75-1127</u>	<p><u>M-5.2, Maintenance Removal Or Repair Of Reactor Head Control Rod Drive Cables and Assemblies.</u> (refer to PORC Item #75-1058, CAR 104?) J. Roon presented his Post-Maintenance review. This procedure was performed on 10/11/75. Copy of Dr. Latz's report is attached to these minutes.</p>

PROC ITEM
NUMBER:

DESCRIPTION

- 75-1073 E-6.1, Loss Of Component Cooling During Power Operation, PCN75-1634. E. Beatty presented this procedure change. It requested step 4.5 be changed from 42% to 50% as P-8 setpoint is 50%. The Committee recommended approval of this change.
- 75-1074 M-11.10, Major Inspection Of Service Water Pump, PCN75-1662. E. Beatty presented this temporary procedure change for review only. This PCN was to facilitate steps marked N.A. as this procedure was used for minor inspection and lubrication. The Committee reviewed this change.
- 75-1075 M-40.2, Hydraulic Snubber Unit Inspection And Repair Procedure, PCN75-1677. E. Beatty presented this procedure change. It requested that in the NOTE under step 4.3 a spelling error be corrected from "it it" to "if it". The Committee recommended approval of this change.
- 75-1076 E. Beatty presented the following M procedures for changes;
- M-11.14, Inspection & Maintenance Of Ingersol Rand Pumps, PCN75-1663, 75-1669 - requested to change step 3.4 to read;
- 3.4 Notify Quality Control for maintenance inspection on component cooling water pumps (SN 0567-318, 319) and containment spray pumps (SN 0667 - 154, 155).
- PCN75-1669 - requested step 3.1.2 to read "gland leakage" not seal leakage.
- Step 5.8 should read; "Lubricate pump bearing per manufacturers instruction"
- Reason for this is not all pumps are equipped with seals, not all bearings are oil lubricated.
- M-11.17, Inspection & Maintenance Of Weinman Pumps, PCN75-1664. This requested step 5.8 be changed to read;
- "Lubricate pump bearing per manufacturer's instructions"
- This is because pump bearings are grease lubricated.
- M-11.18, Inspection & Maintenance Of Buffalo Force Pumps, PCN75-1665, requested step 5.8 be changed to read;
- "Lubricate pump bearings per Buffalo Force Bulletin 3321-1"
- Change step 3.1.2 to read; "gland leakage". This is as bearings are grease lubricated, glands are packed with packing.
- M-11.19, Inspection & Maintenance of Cranc-Deming Pumps, PCN75-1666, requested step 5.8 be change to read;

"Lubricate pump bearings per manufacturers instructions"
as there are different lubricants. The Committee recommended

PROC ITEM
NUMBER:

DESCRIPTION

75-1158

PP-15, Containment Purge Supply And Exhaust Damper Closure Time, PCH 75-1843. C. Hartlieb presented this new procedure. This procedure is to provide instructions to secure chart traces from which closure time of the containment purge supply and exhaust dampers may be obtained. The Committee recommended approval of this procedure.

75-1159

B. Snow
~~in. Snow~~ presented the following CP procedures, each one is a procedure for the calibration of the rod position indication system for different banks during an at power condition.

CP-2.3.1, Rod Position Indication System "Shutdown" Bank at Power Alignment, PCH 75-1783.

CP-2.4.1, Rod Position Indication System "Part-Length" Bank At Power Alignment, PCH 75-1785.

CP-2.5.1, Rod Position Indication System "A" Bank At Power Alignment, PCH 75-1787.

CP-2.6.1, Rod Position Indication System "B" Bank at Power Alignment, PCH 75-1789.

CP-2.7.1, Rod Position Indication System "C" Bank At Power Alignment, PCH 75-1791.

CP-2.8.1, Rod Position Indication System "D" Bank

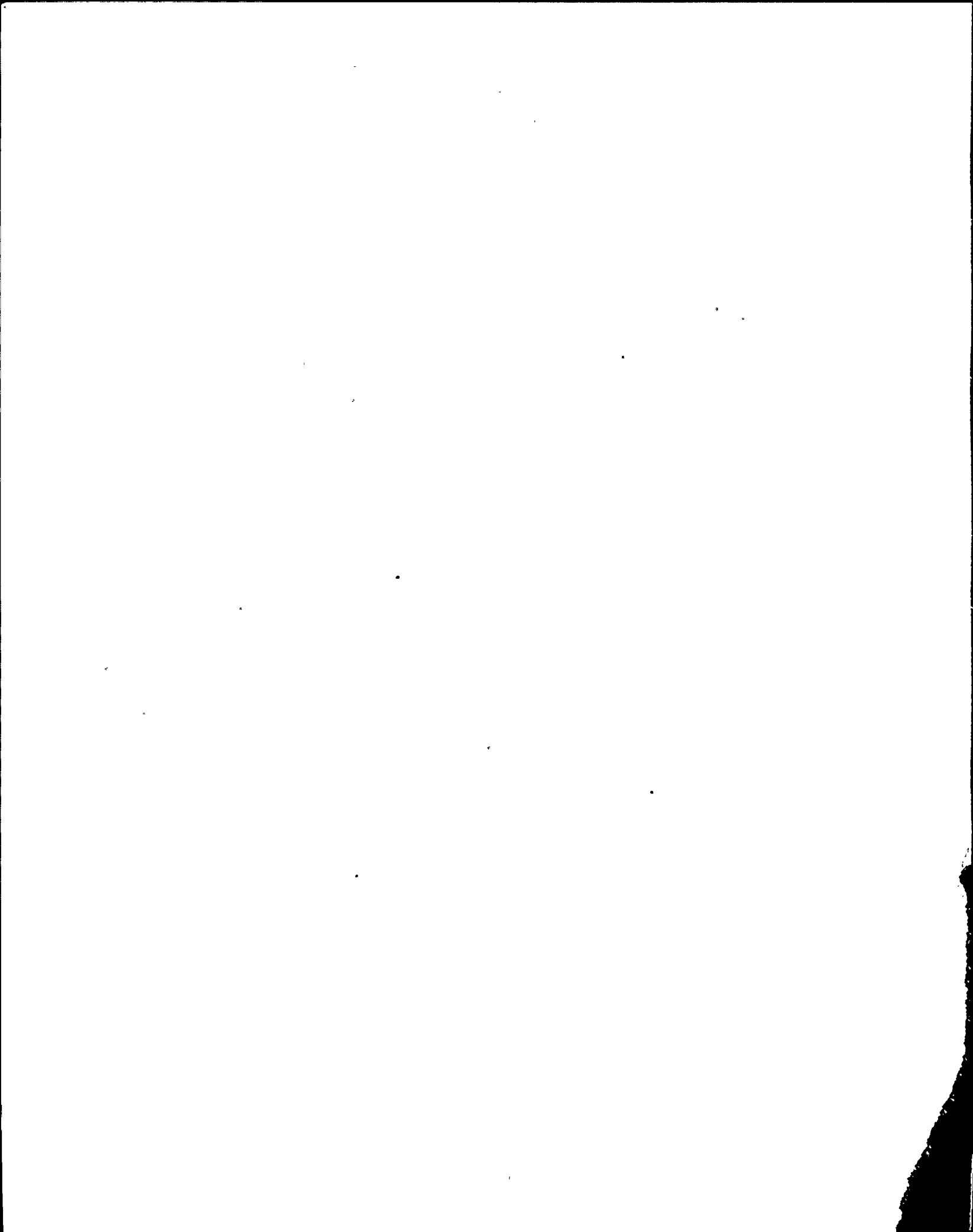
CP-2.9, Rod Position Indication System A.C. to D.C. Correlation Check, PCH 75-1794.

CP-2.10, Rod Position Indication System Maintenance Calibration Check, PCH 75-1795.

The Committee recommended approval of these procedures.

75-1160

EM-151, Maintenance On Non-Regenerative Heat Exchanger Pressure Circuit For PCV-135, PCN75-1853. J. Noon presented this new procedure. This procedure is for troubleshooting the non-regenerative HX exchanger control circuit, P-135. Reason for this procedure is that PCV-135 was not operating properly in auto.



PROC ITEM
NUMBER:

DESCRIPTION

75-1165

EM-136, Maintenance of Non-Regenerative Heat Exchanger Component Cooling Water Valve TCV-130, PCN 75-1851, PCN 75-1851 - L. Depew presented these procedure changes.

PCN 75-1851 - requested to add to step 5.13 "charging pump may be returned to automatic if desired"

Reason for this is operator only needs to adjust HCV-123 to control charging pump speed. The Committee recommended approval of this change with modification.

PCN 75-1852 - requested a sign-off line be added for shift foreman at the bottom of page.

The Committee reviewed this change as the line does appear on the original however did not show up after running copies

75-1168

PT-16, Auxiliary Feedwater System Flow Check, PCN 75-1841.

L. Depew presented this temporary procedure change. It was requested Step 6.1.10.1 read; "should leave MOV 4007 closed."

The Committee reviewed this change.

75-1171

M-37.39, Temporary Repair Of Main Steam Valve Flange Leak Using Furmanite America Incorporated Process Valve No. PCN 75-1836. L. Depew presented this temporary procedure

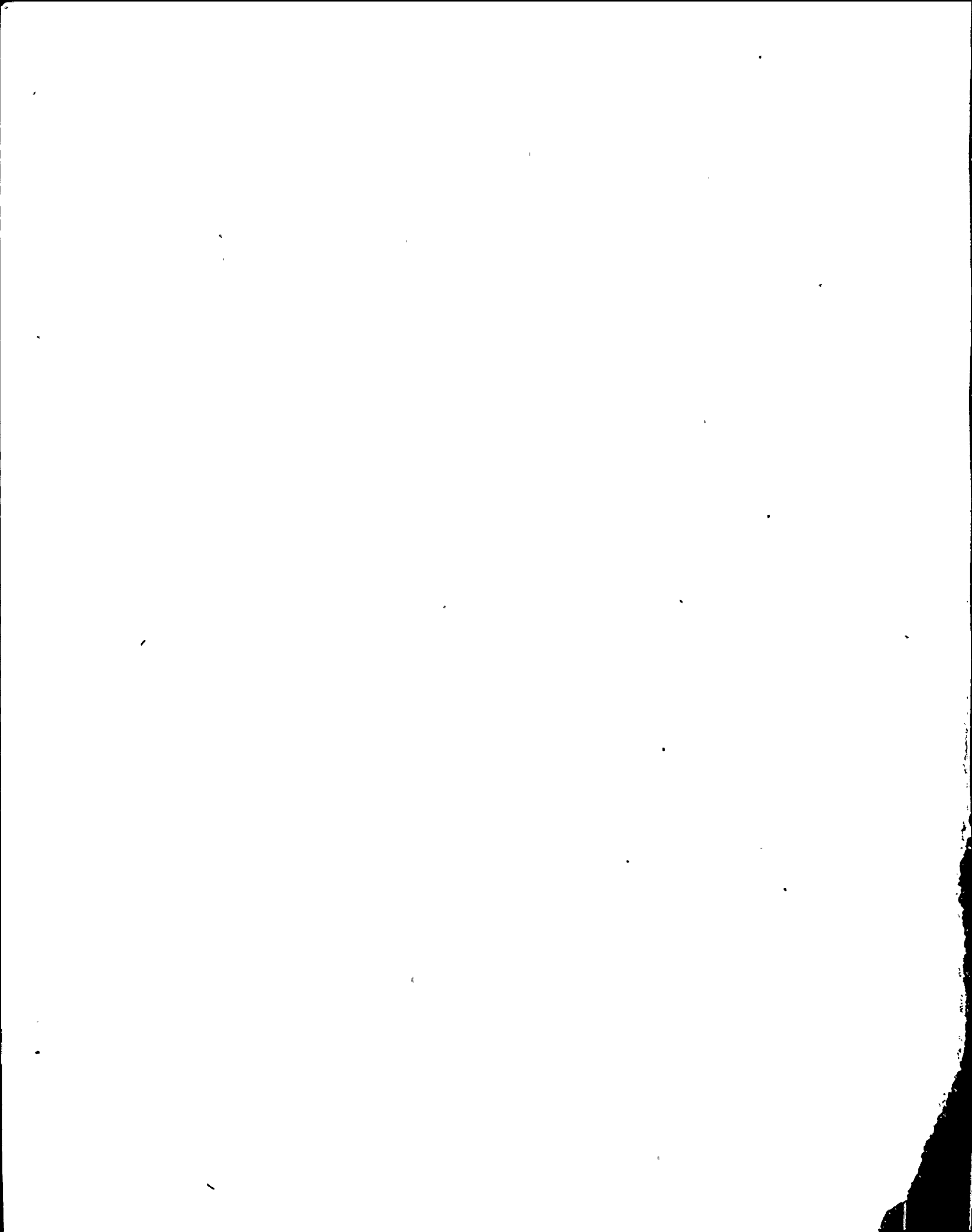
change. It requested steps 5.4, 5.5 and 5.6 be N.A.'d. Reason for this is furmanite personnel are going to continue attempting to stop the leak. The steps made N.A. had previously been completed. The Committee reviewed this change.

75-1178

PR-4, Relay Calibration And Trip Test - 4160/480 Volt Station Transformers, Transformer # _____, PCN75-1779. L. Depew presented this new procedure. This procedure is to provide instructions for calibration and maintenance of protective relays associated with the 4160/480 Volt Station Service Transformers. The Committee recommended approval of this procedure.

75-1179

PR-5, Relay Calibration & Trip Test 4160 Volt Auxiliaries, PCN75-1778. L. Depew presented this new procedure. This procedure is to provide instructions for calibration and maintenance of protective relays associated with the 4KV Auxiliary circuits. The Committee recommended approval of this procedure.



PROC ITEM
NUMBER:

DESCRIPTION75-1165

EM-136, Maintenance of Non-Regenerative Heat Exchanger Component Cooling Water Valve TCV-130, PG. 75-1851, PCN 75-1851 - L. Depew presented these procedure changes.

PCN 75-1851 - requested to add to step 5.13 "charging pump may be returned to automatic if desired"

Reason for this is operator only needs to adjust HCV-123 to control charging pump speed. The Committee recommended approval of this change with modification.

PCN 75-1852 - requested a sign-off line be added for shift foreman at the bottom of page.

The Committee reviewed this change as the line does appear on the original however did not show up after running copies

75-1168

PT-16, Auxiliary Feedwater System Flow Check, PCN 75-1841. L. Depew presented this temporary procedure change. It was requested Step 6.1.10.1 read; "should leave MOV 4007 closed."

The Committee reviewed this change.

75-1171

M-37.39, Temporary Repair Of Main Steam Valve Flange Leak Using Furmanite America Incorporated Process Valve No. PCN 75-1836. L. Depew presented this temporary procedure change. It requested steps 5.4, 5.5 and 5.6 be N.A.'d. Reason for this is furmanite personnel are going to continue attempting to stop the leak. The steps marked N.A. had previously been completed. The Committee reviewed this change.

75-1178

PR-4, Relay Calibration And Trip Test - 4160/480 Volt Station Transformers, Transformer # _____, PCN75-1779. L. Depew presented this new procedure. This procedure is to provide instructions for calibration and maintenance of protective relays associated with the 4160/480 Volt Station Service Transformers. The Committee recommended approval of this procedure.

75-1179

PR-5, Relay Calibration & Trip Test 4160 Volt Auxiliaries, PCN75-1778. L. Depew presented this new procedure. This procedure is to provide instructions for calibration and maintenance of protective relays associated with the 4KV Auxiliary circuits. The Committee recommended approval of this procedure.

FORC ITEM .
NUMBER:

DESCRIPTION75-1165

EM-136, Maintenance of Non-Regenerative Heat Exchanger Component Cooling Water Valve TCV-130, PCN 75-1851, PCN 75-1851 - L. Depew presented these procedure changes.

PCN 75-1851 - requested to add to step 5.13 "charging pump may be returned to automatic if desired"

Reason for this is operator only needs to adjust HCV-123 to control charging pump speed. The Committee recommended approval of this change with modification.

PCN 75-1852 - requested a sign-off line be added for shift foreman at the bottom of page.

The Committee reviewed this change as the line does appear on the original however did not show up after running copies

75-1168

PT-16, Auxiliary Feedwater System Flow Check, PCN 75-1841.

L. Depew presented this temporary procedure change. It was requested Step 6.1.10.1 read; "should leave MOV 4007 closed."

The Committee reviewed this change.

75-1171

M-37.39. Temporary Repair Of Main Steam Valve Flange Leak Using Furmanite America Incorporated Process Valve No. PCN 75-1836. L. Depew presented this temporary procedure

change. It requested steps 5.4, 5.5 and 5.6 be N.A.'d. Reason for this is furmanite personnel are going to continue attempting to stop the leak. The steps made N.A. had previously been completed. The Committee reviewed this change.

75-1178

PR-4, Relay Calibration And Trip Test - 4160/480 Volt Station Transformers, Transformer # _____, PCN75-1779. L. Depew presented this new procedure. This procedure is to provide instructions for calibration and maintenance of protective relays associated with the 4160/480 Volt Station Service Transformers. The Committee recommended approval of this procedure.

75-1179

PR-5, Relay Calibration & Trip Test 4160 Volt Auxiliaries, PCN75-1778. L. Depew presented this new procedure. This procedure is to provide instructions for calibration and maintenance of protective relays associated with the 4KV Auxiliary circuits. The Committee recommended approval of this procedure.

Annual Report #1101 #11 procedure changes. (continued)
Inclusive

POFC ITEM
NUMBER:

DESCRIPTION

- 75-1181 M-37.40, Maintenance On Motor Driven Auxiliary Feedwater Pumps- Discharge Valves - MOV 4007 and MOV 4008, PCN75-1860. L. Depew presented this new procedure. This procedure is to describe the steps necessary to perform maintenance of MOV4007 or MOV4008. The Committee recommended approval of this procedure with modifications.
- 75-1184 A-54.3, Cutting And Welding Permit, PCN75-1859. L. Depew presented this procedure change. Several changes were requested to reflect the new Cutting and Welding permit. The Committee recommended approval of these changes.
- 75-1185 M-11.5G, Returning The 1B Motor Driven Auxiliary Feedwater Pump To Service After Maintenance, PCN75-1862. L. Depew presented this procedure for the 2 1/2 year review. The only change requested was that it be put in A-30 format. The Committee recommended approval of this change.
- 75-1188 PT-5.40, Process Instrumentation Reactor Protection Channel Trip Test (Channel 4), PCN75-1858. L. Depew presented this procedure change. It was requested on data sheet under transmitter number to change;
- Feedwater Flow/Stm FT-466 and FT-464 to read;
- "Feedwater Flow/Stm FT-475 and FT-477"
- This is to update the procedure. The Committee recommended approval of this change.
- 75-1189 CP-2.0, Calibration And/Or Maintenance Of The Rod Position Indication System, PCN75-1857. L. Depew presented this procedure change. It was requested steps 5.2 through 5.2.7 be deleted. This is because these steps for defeating the channel are covered under procedure CP-2.1. The Committee recommended approval of this change.
- 75-1190 PT-2.3, Safeguard Valve Operation, PCN75-1657, 75-1856. L. Depew presented these procedure changes;
- PCN75-1657 - requested that all NOTES referred to in the procedure be typed out and to assign all NOTES a number sequence.
- PCN75-1856 - several changes were requested to separate systems, number steps, add cautions and generally improve the format of the procedure.
- The Committee recommended approval of these changes.

PROC ITEM NUMBER:	DESCRIPTION
<u>75-1191</u>	<p><u>M-60.5, Protective Relay Calibration And Trip Test, 1A Emergency Diesel, PCN75-1770.</u> L. Depew presented this procedure change. It requested several changes to improve instructions and to redesignate as PR-2 so that Diesel Relaying is included in a new series of PR procedures. The Committee recommended approval of these changes.</p> <p><u>M-60.6, Protective Relay Calibration And Trip Test - 1B Emergency Diesel, PCN75-1771.</u> L. Depew presented this procedure change. Several changes were requested to improve instructions and to re-designate as PR-3 so that Diesel Relaying is included in a new series of PR procedures. The Committee recommended approval of these changes.</p>
<u>75-1192</u>	<p><u>A-52.2, Control Of Locked Valve Operation, PCN75-1798.</u> L. Depew presented this procedure change. It requested valve 5763 be added to the locked valve list. In addition it was requested that valves 5974, 5948 and 5948A under B Diesel, valves 5973, 5947 and 5947A under A Diesel be added to the locked valve list. The Committee recommended approval of these changes with modifications.</p>
<u>75-1193</u>	<p><u>A-52.5, Control Of Limiting Conditions For System Specifications, PCN75-1845.</u> L. Depew presented this procedure change. It requested to change step 3.6 to read -----"IF time ---" also "place plant in <u>HOT</u> shutdown mode". This is to correct spelling errors. The Committee recommended approval of these changes.</p>
<u>75-1195</u>	<p><u>M-11.13, Inspection & Maintenance Of Goulds Pumps, PCN75-1844.</u> L. Depew presented this procedure change. It requested to change;</p> <p>Step 5.3 to read "uncouple or detach pump motor" Change step 5.9 to read "Lubricate pump and/or motor bearing per manufacturers instructions" Change step 5.11 to read "recouple or assemble motor to pump"</p> <p>This is because motors are attached differently on some models and bearings vary as to type of lubricant.</p> <p>The Committee recommended approval of this change.</p> <p><u>PCN75-1897</u> - requested the same as PCN75-1844.</p> <p>The Committee reviewed this PCN.</p>

PROC ITEM
NUMBER:

DESCRIPTION75-1196

EM-150, Temporary Repair Of Auxiliary Feedwater System Manual Isolation Valve 4012, PCN75-1835, 75-1837. L. Depew presented these procedure changes.

PCN75-1837 - Requested several changes to allow injection of Furmanite Compound into the void between yoke and bonnet.

PCN75-1835 - requested in step 5.1 to change the wording "bonnet yoke" also to change it to read "deep and then continue with 3/16" diameter hole into". This is to update the procedure. The Committee recommended approval of these changes.

75-1200

L. Brodie presented the following SC procedures for changes;

SC-1.17, Communications, PCN75-1917. It was requested step 3.1.4 be added to read;

"The plant P.A. system has been installed at the Emergency Survey Center to provide hard wire communications between Emergency Coordinator and Control Room"

SC-1.15, Inspection Of Emergency Equipment, PCN75-1916. It requested to add a new step 2.4 to read;

"Ion cleaner (defogger) for masks"

SC-1.14A, Accountability Of Personnel, PCN75-1915. Several changes were requested to clarify responsibilities of personnel in the procedure.

SC-1.12B, Station Call List, PCN75-1913. Several addresses and phone numbers were changed to update the procedure.

SC-1.12A, Immediate Call List, PCN75-1912. Changes were made to update the call list by adding the names of additional trained personnel.

SC-1.13, Estimating Off-Site Doses, PCN75-1914. Several changes were requested to correct typing errors and to clarify the procedure.

SC-1.11A, Immediate Re-entry, PCN75-1911. Changes were to add a reference to the procedure to clarify some of the steps in the procedure.

SC-1.9, In-plant Radiation Monitoring, PCN75-1910. Several changes were requested to clarify the use of respirator, self-contained breathing apparatus and to add a precaution relative to the limited air supply of a Scott Air Pack.

SC-1.7II, Emergency Radiation Monitoring Off-Site Team, #9II (Orange), PCN75-1909. Several changes were requested to clarify a step in the procedure, to eliminate steps in the procedure and to correct typing errors.

PROC ITEM
NUMBER:DESCRIPTION75-1200 (Cont.)

SC-1.7G, Emergency Radiation Monitoring On-Site Survey Team #9G (Yellow) Procedure, PCN75-1908. Several changes were requested to clarify some steps in the procedure and to correct typing errors.

SC-1.7F, Emergency Radiation Monitoring Off-Site, PCN75-1907. Several changes were requested to clarify some steps in the procedure and to correct typing errors.

SC-1.7E, Emergency Radiation Monitoring On-Site Survey Team #9E (Blue) Procedure, PCN75-1906. Several changes and additions were requested to clarify the procedure and to correct typing errors.

SC-1.7D, Emergency Radiation Monitoring Off-Site Survey Team #9D (Red), PCN75-1905. Several changes were requested to clarify the procedure and to correct typing errors.

SC-1.7B, Determination Of Iodine Or Particulate, PCN75-1904. This requested to add a new step 3.4.4.1 to add conversion calculations for thyroid dose.

SC-1.7A, Emergency Radiation Monitoring Procedure, PCN75-1903. This procedure was rewritten to explain the SC-1.7A procedure series.

SC-1.3C, Site Radiation Emergency, (Non-Operating Personnel) PCN75-1902. This requested a new step be added to allow the Office Supervisor to obtain tag 14 out of order.

SC-1.3, Site Radiation Emergency, (Emergency Coordinator & Survey Center Assignees), This procedure was rewritten to clarify tag assignments and Emergency Coordinators responsibilities. This request was approved with modifications.

SC-1.3A, Site Radiation Emergency (Shift Foreman And Control Room), PCN75-1900. Several changes were requested to make filling out of plant status easier and with less confusion.

SC-1.2, Local Radiation Emergency Procedure, PCN75-1899. This request was to correct a spelling error in step 3.2.4b.

The Committee recommended approval of all these changes.

75-1201

M-38.1, Diesel Fire Pump Engine Isolation For Maintenance And Inspection, PCN75-1922. D. Hamelink presented this new procedure. This procedure is for isolation of diesel fire pump engine for maintenance and inspection. The Committee recommended approval of this procedure.

75-1202

M-63.1, Containment Recirculation Fan Cooler Unit Maintenance, PCN75-1923. D. Hamelink presented this new procedure. This procedure is to provide the steps necessary for maintenance of the containment recirc-fan-cooler units. The Committee recommended

PROC ITEM NUMBER:	DESCRIPTION
75-1203	<u>RSSP1.2, Valve Interlock Verification - Safety Injection System, PCN75-1875.</u> L. Depew presented this procedure change. It requested after step 2.4, Valve Traveling _____ after step 2.4, Valve Traveling _____ red light, to add (MOV 825A and B _____ exhibits a red light only). This is to make a procedural correction. The Committee recommended approval of this change.
75-1206	<u>S-3.6, Auxiliary Building Heat Tracing System Operation, PCN75-1895.</u> L. Depew presented this procedure change. Several changes were made to provide information for breaker numbers for respective heat tracing circuits and to incorporate information from OSO-74-4. The Committee recommended approval of these changes.
75-1207	<u>Annunciator H-21, PCN75-1855.</u> L. Depew presented this procedure change. It was requested to change the alarm points on H-21 Cond. Hotwell level to Hi ~32" and Lo ~18". The Committee recommended disapproval of this change as this could not be done because the alarm setpoints are not adjustable.
75-1208	<u>T-36.1, Station Service Water-Header Valve Alignment For Two Loop Operation, PCN75-1898.</u> L. Depew presented this procedure change. Several changes were requested to add valves and improve and correct nomenclature. The Committee recommended approval of these changes.
75-1216	<u>PT-16, Auxiliary Feedwater Pump Flow Check, PCN75-1918.</u> L. Depew presented this procedure change for review only. It was requested step 6.1.10.1 be marked N.A. Reason for this is the valve is being maintained closed due to back leakage through the check valve. The Committee reviewed this change.
75-1217	<u>SC-1.7C, Monitoring Site Radiation Level By LTD, PCN75-1921.</u> L. Depew presented this procedure change. It was requested to replace the old map with a new updated one which is clearer and has correct TLD station numbers. The Committee recommended approval of this change.
75-1218	<u>M-43.0, Steam Generator Maintenance Test And Repair Procedures, PCN75-1718.</u> L. Depew presented this procedure change. It requested to add as Appendix I to the procedure a table of specific surveillance duties for the H.P. Technician and Job Foreman. This will provide better management surveillance controls for the steam generator inspection procedure. The Committee recommended approval of this change.
75-1220	<u>E-6.1, Loss Of Component Cooling During Power Operation, PCN75-1924.</u> L. Depew presented this procedure change. It was requested step 1.4 be deleted as alarm is on normally. The Committee recommended approval of this deletion.
75-1221	<u>P-9, Precautions, Limitations And Setpoints P-9 Radiation Monitoring System, PCN75-1882.</u> L. Depew presented this procedure change. It requested R-4 alarm setpoint be change to 150 Mr/hr to provide a more practical setting. The Committee recommended approval of this change.

PORC ITEM
NUMBER:DESCRIPTION

75-1222

PT-15, Containment Purge Supply And Exhaust Damper Closure Time, PCN75-1876, PCN75-1881. L. Depew presented these procedure changes.

PCN75-1876 requested;

- 1) Add step 6.1.1 - Open Containment Depressurization valves and verify containment pressure on PI-944 at MCB is equal to 0 psig.
- 2) Add new step 6.7.7 as follows:
6.7.7 - Close Containment Depressurization valves _____
- 3) Change existing step 6.7.7 to step 6.7.8

Reason for these changes are to ensure no flow out of containment through open purge supply and exhaust valves.

PCN75-1881 - requested;

- 1) Add new steps 6.4.2.1, 6.5.2.1, 6.6.2.1 and 6.7.2.1 as follows:
"Connect input leads of a third channel of trace recorder across dampers red indicating light _____"
- 2) Change steps 6.4.5, 6.5.5, 6.6.5 and 6.7.5 from (green light on) to (RED LIGHT OUT)
- 3) Change steps 6.4.1, 6.5.1, 6.6.1 and 6.7.1 so that words (wire //PVP....etc.) follow wordscontrol switch

Reason for these changes are to add additional monitoring and correct method of connection.

The Committee recommended approval of these changes.

75-1225

RSSP-2.7, Safety Injection Sequence Timers, Train A & B, PCN75-1926. L. Depew presented these procedure changes. Several changes were requested to incorporate Test Control, clarify steps and add data sheets. The Committee recommended approval of these changes.

75-1226

M-11.11, Inspection And Maintenance Of Crane Chempumps, Series G, PCN75-1919. L. Depew presented this procedure change. It requested step 3.4 be changed to read;

"Q.C. should be notified if it is a Boric Acid Pump"

This is to clarify applicability to Q.A. components. The Committee recommended approval of this change.

PORC ITEM
NUMBER:DESCRIPTION

75-1222

PT-15, Containment Purge Supply And Exhaust Damper Closure Time, PCN75-1876, PCN75-1881. L. Depew presented these procedure changes.

PCN75-1876 requested;

- 1) Add step 6.1.1 - Open Containment Depressurization valves and verify containment pressure on PI-944 at MCB is equal to 0 psig.
- 2) Add new step 6.7.7 as follows:
6.7.7 - Close Containment Depressurization valves _____
- 3) Change existing step 6.7.7 to step 6.7.8

Reason for these changes are to ensure no flow out of containment through open purge supply and exhaust valves.

PCN75-1881 - requested;

- 1) Add new steps 6.4.2.1, 6.5.2.1, 6.6.2.1 and 6.7.2.1 as follows:
"Connect input leads of a third channel of trace recorder across dampers red indicating light _____"
- 2) Change steps 6.4.5, 6.5.5, 6.6.5 and 6.7.5 from (green light on) to (RED LIGHT OUT)
- 3) Change steps 6.4.1, 6.5.1, 6.6.1 and 6.7.1 so that words (wire //PVP....etc.) follow wordscontrol switch

Reason for these changes are to add additional monitoring and correct method of connection.

The Committee recommended approval of these changes.

75-1225

RSSP-2.7, Safety Injection Sequence Timers, Train A & B, PCN75-1926. L. Depew presented these procedure changes. Several changes were requested to incorporate Test Control, clarify steps and add data sheets. The Committee recommended approval of these changes.

75-1226

M-11.11, Inspection And Maintenance Of Crane Chempumps, Series G, PCN75-1919. L. Depew presented this procedure change. It requested step 3.4 be changed to read;

"Q.C. should be notified if it is a Boric Acid Pump"

This is to clarify applicability to Q.A. components. The Committee recommended approval of this change.

PROC ITEM
NUMBER:

DESCRIPTION

75-1229

L. Depew presented the following new CP procedures;

CP-209.1, Calibration of Eberline Model, PMC-4B, PCN75-1877.
This is to describe the steps necessary to calibrate the portal monitor Model PMC-4B.

CP-209.2, Calibration of Tracerlab Model PPM-8 Portal Monitor, PCN75-1878. This is to describe the steps necessary to calibrate the Tracerlab Model PPM-8 portal monitor.

CP-209.3, Calibration Of Eberline Model HFM-3 Hand And Foot Monitor, PCN75-1879. This procedure is to describe the steps necessary to calibrate the Eberline Model HFM-3 Hand And Foot Monitor.

CP-209.4, Calibration of Nuclear Measurements Corporation Portable Air Monitors, PCN75-1880. This procedure is to describe the steps necessary to calibrate NMC Portable Air Monitor. The Committee recommended approval of these procedures.

75-1230

S-2.1, Reactor Coolant Pump Operation, PCN75-1714. L. Depew presented this procedure change. It requested to add new step 9 on, "B" Initial Condition to read;

"Reactor Coolant Pumps #1 seal leakoff flow \geq
0.3 gpm as read on F1-175 and F1-176"

Procedure was also rewritten to conform to A-30 format. The Committee recommended approval of these changes.

75-1232

S-9F, Transferring Spent Fuel Pit Transfer Canal To The CVCS Holdup Tank, Waste Holdup Tank Or Monitor Tanks, PCN75-1932.
L. Depew presented this procedure change. It requested to change valve number in step 5.2.3 from 1794 to 1795G. The Committee recommended approval of this change.

75-1233

S-9E, Draining Spent Fuel Pit Transfer Canal To Refueling Water Storage Tank, PCN75-1933. L. Depew presented this procedure change. It requested in step 5.6 to change valve number 1794 to 1795G and in step 3.4 to correct a typing error, "This canal" should read "THE canal". The Committee recommended approval of this change.

75-1237

P-9, Precautions, Limitations and Setpoints P-9 Radiation Monitoring System, PCN 75-1970. L. Depew presented this temporary procedure change for review only. It requested:

2.2.2.1 change 5mR/hour to 2mR/hour

2.3.2.4 change 3×10^3 cpm to 1×10^3 cpm

PORC ITEM
NUMBER:DESCRIPTION75-1238EM-152, Repair of Main Steam Insolation or Check Valve Cover Gasket Leak Using "Furmanite Process", PCN 75-1969.

D. Hamelink presented this new procedure along with the safety analysis on EWR 1430. This procedure describes the steps necessary for using "Furmanite" to seal steam leaks on the main steam line isolation valves cover gaskets and the main steam line check valve cover gaskets. The Committee performed the Safety Evaluation and found that this does not involve an unreviewed safety question and that Technical Specifications are not affected.

The Committee recommended approval of this procedure.

75-1241

B. Snow presented the following procedures for review only;

PT-16, Auxiliary Feedwater System Flow Check, PCN75-1971, requested step 6.1.10.1 be N.A.'d because the valve is being maintained closed due to leakage through check valve 4009 back to pump casing.

PT-12.1, Emergency Diesel Generator 1A, PCN75-1990, was to facilitate steps N.A.'d.

M-37.39, Temporary Repair Of Main Steam Safety Valve Flange Leak Using Furmanite America Inc. Process Valve No. . . . , PCN75-1973, was to facilitate steps 5.4 through 5.6 marked N.A. to allow Furmanite personnel to continue attempts to stop the leak.

The Committee reviewed these changes. This item IS COMPLETE.

75-1242

E-33, Quadrant Power Tilt, PCN75-1646. B. Snow presented this procedure for 30 month review. Several changes were requested to comply with Technical Specifications and A-30 format. The Committee recommended approval of these changes.

75-1244

B. Snow presented the following procedures for deletion;

M-60.5, Protective Relay Calibration & Trip Test (1A Diesel), PCN75-1954. This procedure has been replaced with PR-2.

M-60.6, Protective Relay Calibration & Trip Test (1B Diesel), PCN75-1955. This procedure was replaced with PR-3.

M-22, Securing & Restoring To Service the N₂ Supply System For Maintenance, PCN75-1956. This procedure was replaced with S-4.2.11.

M-2H, Waste Condensate Polishing Demineralizer & Resin Replacement, PCN75-1957. This procedure was replaced with S-4.5.2.

A-2.10.1, Partial Flux Maps Using Computer, PCN75-1940.

&

A-2.10.2, Partial Flux Maps With Computer Out Of Service, PCN75-1941. These were requested to be deleted as partial flux maps are no longer used.

PROC ITEM
NUMBER:

DESCRIPTION

75-1245

M-11.5H, Removing The 1A Motor Driven Auxiliary Feed Pump From Service For Maintenance, PCN75-1953. B. Snow presented this temporary procedure change to be made permanent. It was requested step 5.4 be change to read; "Hold shut valve 4357"
The Committee recommended approval of this change with modifications.

75-1247

M-11.13, Inspection And Maintenance Of Goulds Pumps, PCN75-1966. B. Snow presented these procedure changes. There were four changes requested, the first three change requests were done previously in a PCN, therefore, the Committee disapproved these. The fourth request was to add a step between 5.10 and 5.11 to read;

"verify proper motor rotation, if necessary" and to add a shift formeman sign off line.

The Committee recommended approval of these changes.

75-1248

B. Snow presented the following procedures which were requested to be redesignated;

A-2.2 - 1/M Curves, PCN75-1943 to O-1.2.1

A-2.4.2 - Cold Shutdown Boron Concentration Calculation, PCN75-1947, to O-2.2.1.

A-2.4, Critical Rod Position Calculation, PCN75-1945 to O-1:2.2.

A-2.4.1, Hot Xenon Free Shutdown Margin Calculation, PCN75-1946, to O-3.1.

A-2.9, Reactivity Changes, PCN75-1950 to O-6.5.

A-2.5, Correcting Measured Boron Concentration To Reference Full Power Expected Boron Concentration, PCN75-1948, to O-6.6.

The Committee recommended approval of these changes.

75-1249

B. Snow presented the following procedures for deletion;

A-2.2, I/M Curves, PCN75-1958, replaced with O-1.2.1.

A-2.4, Calculation of Critical Rod Position, PCN75-1960, replaced by O-1.2.2.

A-2.4.1, Calculation Of Hot Xenon Free Shutdown Margin, PCN75-1961, replaced by O-3.1.

A-2.4.2, Calculation of Cold Shutdown Boron Concentration, PCN75-1962, replaced by O-2.2.1.

A-2.5, Correcting Measured Boron Concentration to Reference Full Power Expected Boron Concentration, PCN75-1963, replaced by O-6.6.

A-2.9, Changes In Reactivity, PCN75-1965, replaced by O-6.5.

The Committee recommended approval of these deletions.



PROC ITEM
NUMBER:

DESCRIPTION

75-1251.

A-54.3, Cutting And Welding Permit, PCN75-1968. B. Snow presented this procedure change. Upon review of the procedure it was requested to add to step 3.2.5;

"The Control Room will be notified of the area where burning and welding permits have been issued prior to the job commencing."

The Committee recommended approval of this addition.

75-1254

EM-153, Service Water Pump 1A Breaker Auto-Close Circuit Check, PCN75-1994. D. Gent presented this new procedure which describes the steps necessary for checking the continuity and proper operation of the Service Water Pump 1A Breaker Auto-Close Circuit. The Committee recommended approval of the procedure.

75-1256

B. Snow presented these two procedures for the same changes.

0-1.2, Plant From Hot Shutdown To Steady Road, PCN 75-1952.

0-2.2, Plant Shutdown From Hot Shutdown To Cold Shutdown, PCN75-1951.

The change requested that the Reference in each of the above procedures be changed from A-2.4 to 0-1.2.2. This is to update the procedure. The Committee recommended approval of this change. This item IS COMPLETE

75-1257

B. Snow presented the following changes to be made in the following procedures;

PT-6.3.1, Power Range Nuclear Instrumentation System, Channel 41, PCN75-1988.

PT-6.3.2, Power Range Nuclear Instrumentation System, Channel 42, PCN75-1985.

PT-6.3.3, Power Range Nuclear Instrumentation System, Channel 43, PCN75-1986.

PT-6.3.4, Power Range Nuclear Instrumentation System, Channel 44, PCN75-1987.

Several changes were made to the procedures to improve data collection and clarify steps. The Committee recommended approval of these changes.



<u>PROC ITEM NUMBER:</u>	<u>DESCRIPTION</u>
<u>75-1258</u>	<p><u>SM75-7.21, HVAC Backfits, PCN75-1978.</u> B. Snow presented this procedure for the following changes;</p> <p>Step 2.1.3 Delete this reference. Steps 2.1.11 Change 5⁴ shts. to shts. 1 & 2. Steps 2.1.12, 2.2.7, & 2.3.7 change shts. 0-4 to 0-6, add new reference 2.5 Specification 21⁴ "HVAC Backfits".</p> <p>The reasons for these changes are due to drawing changes and previous omissions.</p> <p>The Committee recommended approval of these changes.</p>
<u>75-1259</u>	<p><u>S-3.1E, 1A Boric Acid Storage Tank Isolation/Restoration, PCN75-1979.</u> This was redesignation of M-12A into S series.</p> <p><u>S-3.1F, 1B Boric Acid Storage Tank Isolation, PCN75-1981.</u> This was redesignation of M-12B into S series.</p> <p><u>S-4.5.3, Boric Acid Evaporator Condensate Demineralizer, Resin Replacement, PCN75-1983.</u> This was redesignation of M-2F into S series.</p> <p>B. Snow presented the above procedure changes. The Committee recommended approval of these changes.</p>
<u>75-1260</u>	<p>B. Snow presented the following procedures for deletion;</p> <p><u>M-12A, Isolation/Restoring 1A Boric Acid Storage Tank, PCN75-1980,</u> replaced by S-3.1E.</p> <p><u>M-12B, Isolation/Restoring 1B Boric Acid Storage Tank, PCN75-1982,</u> replaced by S-3.1F.</p> <p><u>M-2F, Resin Replacement Procedure Boric Acid Evaporator Condensate Demineralizer, PCN75-1984,</u> replaced by S-4.5.3.</p> <p>The Committee recommended approval of these deletions.</p>
<u>75-1262</u>	<p><u>M32.1, DB-25, DB-50 & DB-75 Circuit Breaker Maintenance & Overcurrent Trip Device Test And/Or Replacement Procedure, PCN75-1992.</u> B. Snow presented this procedure change. It requested to change step 5.19 number from 63.23 to 60.23. This is to correct a typing error.</p> <p>The Committee recommended approval of this change.</p>
<u>75-1268</u>	<p><u>M-11.4.2, Isolation of 1B Changing Pump for Maint. and Restoring to Service After Maintenance, PCN75-2022.</u> L. Depew presented this procedure change. It requested to delete steps 5.1.10 and add steps 5.1.10 thru 5.1.14 to the procedure. Reason for this change is to allow weld repair on drain line. The Committee recommended approval of this change.</p>

PORC ITEM
NUMBER:DESCRIPTION75-1269

RSSP 2.2, Diesel Generator Load And Safeguard Sequence Test, PCN75-2028. L. Depew presented these procedure changes. It requested to make changes to "Diesel Generator Load and Safeguard Sequence Test" procedure to make general improvements for clarity and typing corrections. The Committee recommended approval of these changes with modifications.

75-1272

S-4.1L, Reverse Osmosis Unit Startup and Shutdown, PCN75-2027. L. Depew presented the following procedure changes;

Insert a new step: 5.15.5;

Before proceeding to restore valve alignment to isolate the Laundry tank flow path to the R.O. unit feed tank, place the selector switch to Manual to prevent inadvertant start of the Laundry pump against a dead head.

The reason for this is to save the pump. With R.O. feed tank being less than 60% and Laundry tank greater than 10%, the pump will start on auto, and it would be unwise to have system isolated. The Committee recommended approval of the changes.

75-1273

SM75-5.21, Core Drilling For MCC For Standby Auxiliary Feedwater Pump, PCN75-2003. L. Depew presented the following procedure changes;

Change References 1 - Drawing //DWD D-281-013 Detail B.

Under Instructions 1., Change the drawing number to D281-013.

The drawing change is by GAI and the procedure change is by NUCAM.

The Committee recommended approval of these changes.

75-1274

EM-152, Repair of Main Steam Isolation or Check Valve Cover Gasket Leak Using "Furmanite" process. L. Depew presented the following changes;

PCN75-2001 requested to;

Change: Step 5.8 to Step 5.11.

Add: Step 5.8: "Drill 5/16" Holes Centered Between The Studs, As Necessary"

Add: Step 5.9: "Tap 5/16" Holes and Install Adaptor Shutoffs"

Add: Step 5.10: "Inject Furmanite Compound Into Nozzles Around the Joint"

The reason for these changes is to allow more sealant to be injected into joint.

PORC ITEM
NUMBER:DESCRIPTION

75-1274

PCN75-1997 requested to;

Step 4.6 - change 3/16" to 1/8"

Step 5.2 - change 3/16" to 1/8"

NOTE: Size change has been verbally approved by J. Hutton who performed original Safety analysis.

The reason for these changes is to reduce steam leaking thru drilled holes. The Committee recommended approval of these changes.

75-1275

E-14, High Reactor Coolant Activity, PCN75-2000. L. Depew presented this procedure change. This procedure is to change units in 1.1 from 20 Mc. per gram to 2.0 μ ci per gram because the units shown indicate Millicuries in place of Microcuries. The Committee recommended approval of this change.

75-1277

PT-2.3, Safeguard Valve Operation, PCN75-1995. L. Depew presented this procedure change to add sign off line to step 6.17.7. This is to provide step for sign-off. The Committee recommended approval of this change.

75-1278

RSSP-3, Verification of Emergency Start Logic For Auxiliary Feedwater Pumps, PCN75-2018. L. Depew presented these procedure changes. It requested to make changes to procedure "Verification of Emergency Start Logic Auxiliary Feedwater Pumps" for correction of typing errors and general clean up of procedure to add valve numbers and equipment locations. The Committee recommended approval of these changes with modifications.

75-1279

P-8, Precautions, Limitations and Setpoints P-8 Waste Disposal System. L. Depew presented the following changes;

PCN75-1996 requested;

P-8 Step 2.20.23 PICA-1025 Vent header

Pressure 0.5-15 psig Hi ALM 1025A 2.0 psig

Low ALM 1025A .5 psig

Hi control 1025B 1.8 psig

Low control 1025B 1.2 psig

The reason for this is to update P-8 to system design.

PCN75-2008 requested;

To incorporate Operation Standing Order E-73.44 to this procedure. The Committee recommended approval of these changes.

PROC ITEM NUMBER:	DESCRIPTION
<u>75-1281</u>	<u>0-6.8, Logging, MVAR, Electrical and Tie Line Readings, PCN75-2006.</u> L. Depew presented this new procedure. This is to provide a procedure on Station 13 MVAR Readings, daily electrical log, and tie line readings. The Committee recommended approval of this change.
<u>75-1282</u>	<u>0-6.7, Weekly Alarm Status Check, PCN75-2021.</u> L. Depew presented this new procedure to provide a means of maintaining a status of alarms, and to provide a means of determining if proper and timely actions are being taken to correct the alarm conditions. The Committee recommended approval of procedure.
<u>75-1284</u>	<u>SC-2, Site Contingency Plan Sc-2 Adverse Weather Emergency Plan, PCN75-2010.</u> L. Depew presented this procedure change, adding what to do with wind below 40 mph. but greater than 25 mph. The Committee recommended approval of this change with modifications.
<u>75-1286</u>	<u>E-4, Station Blackout Operation PCN75-2011.</u> L. Depew presented this procedure change. It was requested to incorporate Operation Standing Order G-73-28 to this procedure. The Committee recommended approval of this change.
<u>75-1287</u>	L. Depew presented the following 2 procedures for the same changes; <u>Emergency Diesel Generator 1B PCN75-2032.</u> <u>Emergency Diesel Generator 1A, PCN75-2033.</u> Both PCN's requested additions to the procedure data sheet incorporating approximate values. The Committee recommended approval of these changes.
<u>75-1288</u>	L. Depew presented the following 2 procedures for the same changes; <u>N.I.S. Intermediate Range Channels, PCN75-2030.</u> <u>Source Range Nuclear Instrumentation System, PCN75-2031.</u> Both PCN requests are to add the Channel numbers on both the title sheet and data sheet. The Committee recommended approval of these changes.
<u>75-1290</u>	<u>T4- Operating Instructions T-4 Main Feedwater Pump Operation, PCN75-2013.</u> L. Depew presented this procedure to add Operation Standing Order G. 73-52 to this procedure. The Committee recommended approval of these changes.
<u>75-1291</u>	<u>M-11.3, Waste Evaporator Feed Tank Pump Replacement, PCN75-2036.</u> L. Depew presented this procedure to describe the isolation and replacement of the Feed Tank Pump. The Committee recommended approval of these changes with modifications.

PORC ITEM
NUMBER:

DESCRIPTION

75-1294SC-1, Site Contingency Plan SC-1 Radiation Emergency Plan, PCN75-1936 PCN75-2038. L. Depew presented the following procedure changes:PCN75-1936 requested;

Item 8.0, 2nd Paragraph - delete words "... the severity of the occurrence was..." and replace with the following:
"off-site consequences of the occurrence were"

The reason for this change is to eliminate the need for USNRC concurrence in cases where only fire department or ambulance first aid were called to provide support.

PCN75-2038 requested several changes be made to incorporate experience of recent drill. The Committee recommended approval of these changes.

75-1295

M-11.11, Crane Chempumps, Series G, Inspection and Maintenance, PCN75-2035. L. Depew presented this procedure change to provide for isolation of pump. The Committee recommended approval of this change.

75-1299

M-45.1A, Inspection of Safeguard Motor for 1A Service Water Pump, PCN75-2068. L. Depew presented this

procedure change for review only. This PCN was to facilitate the steps marked NA because they were not required to uncouple the pump. The procedure was used to check the pump and motor for-free roll and to check the thrust bearing rotor assembly. The Committee reviewed this change.

75-1301

GS-10.0, Security Guard Actions During A Radiation Emergency, PCN75-2064. L. Depew presented the following procedure changes;

3.1.4 - delete word "film" to read "retain their badges"

3.2.2.1 - same

add: 3.8 - Re-entry to the site following a site evacuation

shall be in accordance with existing entry procedure.

3.8.1 - Personnel who have retained their badges, as instructed, must show their badge to security personnel upon re-entry.

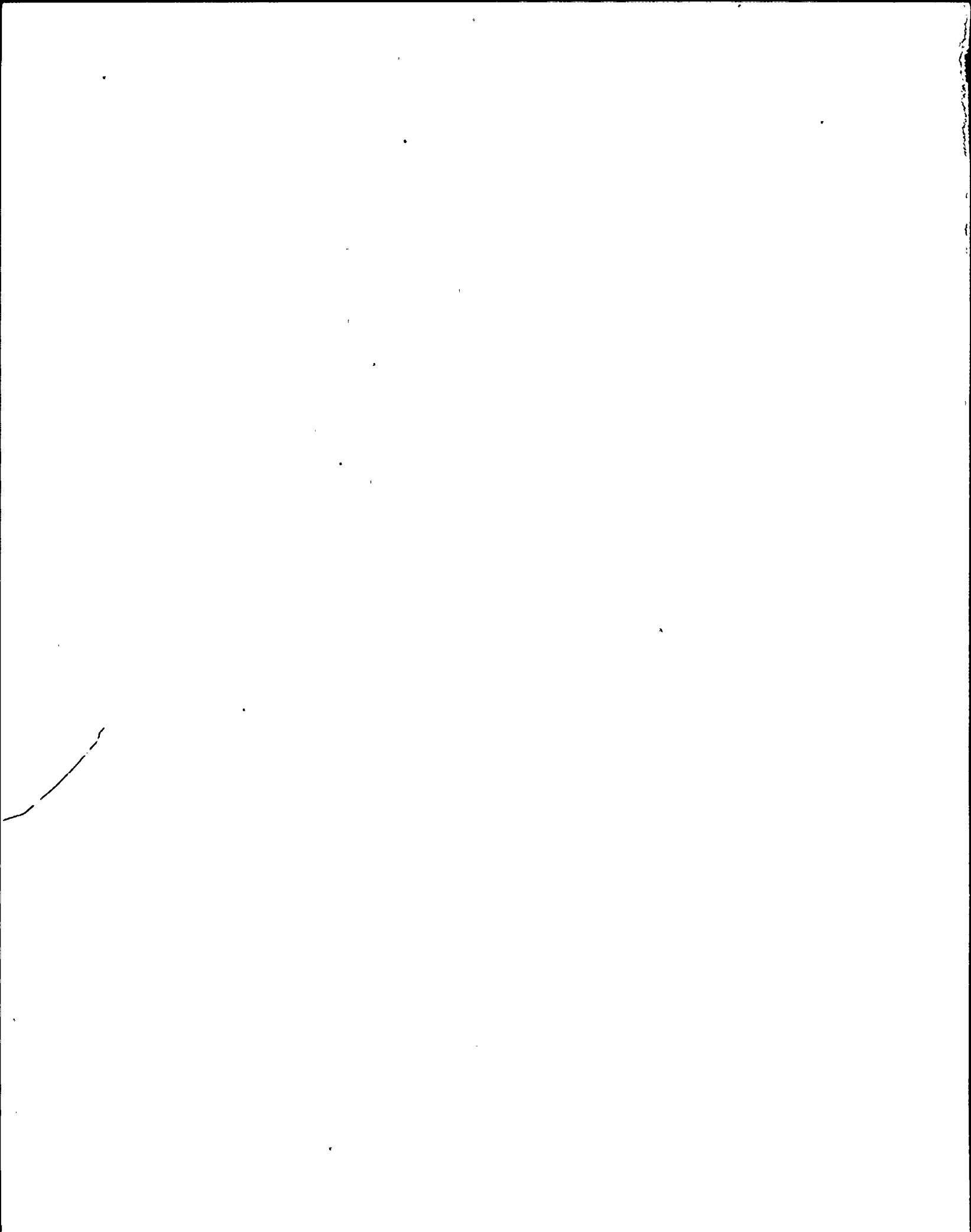
The reason for these changes is to satisfy AFCAR audit 75-71:TS finding 3. The Committee recommended approval of these changes. This item IS COMPLETE.

75-1302

GS-21.0, Access Control (Security) And Personnel I.D., PCN75-2065. L. Depew presented this procedure change. It requested that step 3.1.3.6 be changed to read;

Visitors shall be escorted, except for the top floor of the Service Building and the office area in the warehouse during normal working hours. During off-hours an escort is required for all areas.

The reason for this procedure change is to improve off-hour security per request of operating personnel. The Committee recommended approval of this change.



PORC ITEM
NUMBER:DESCRIPTION75-1304

GS-31.0, Door Alarm System, PCN75-2043. L. Depew presented this procedure change by adding a new step 3.4.1.2 to read;

Communication is established between alarm centers by activating alarm point #30 and #31. Point #30 is activated by pushing the "Acknowledge" button on the printer panel of the master unit. This button should be depressed until point #30 activates (<2 sec.) The Security force activates Point #31 from the guard house in the same manner.

The reason for this change is to meet requirements of Regulatory guide 1.17. The Committee recommended approval of this change.

75-1305

0-6.3, Maximum Unit Power, PCN75-1944. As a result of a Q.A. audit, L. Depew presented this procedure change for Redesignation of A-2.3 - Maximum Unit Power to 0-6.3. The Committee recommended approval of this procedure change.

75-1306

0-6.4, Core Quadrant Power Tilt Calculation, PCN75-1949. As a result of a Q.A. audit, L. Depew presented this procedure change for redesignation of A-2.7, Core Quadrant Tilt Calculation, to 0-6.4. The Committee recommended approval of this procedure change.

75-1307

L. Depew presented the following procedures for deletion:

A-2.3, Maximum Unit Power, PCN75-1959. It has been redesignated 0-6.3.

A-2.7, "Calculating Core Quadrant Power Tilt", PCN75-1964. It has been redesignated 0-6.4.

The Committee recommended approval of these deletions.

75-1308

S-12.4, RCS Leakage Surveillance Record Instructions, PCN75-2016.

L. Depew presented this new procedure to provide instruction for the use of the reactor coolant system leakage surveillance record form 49-61 and to ensure proper leak rate documentation. The Committee recommended approval of this new procedure.

75-1309

0-6.9, Operating Limits For Ginna Station Transmission, PCN75-2020.

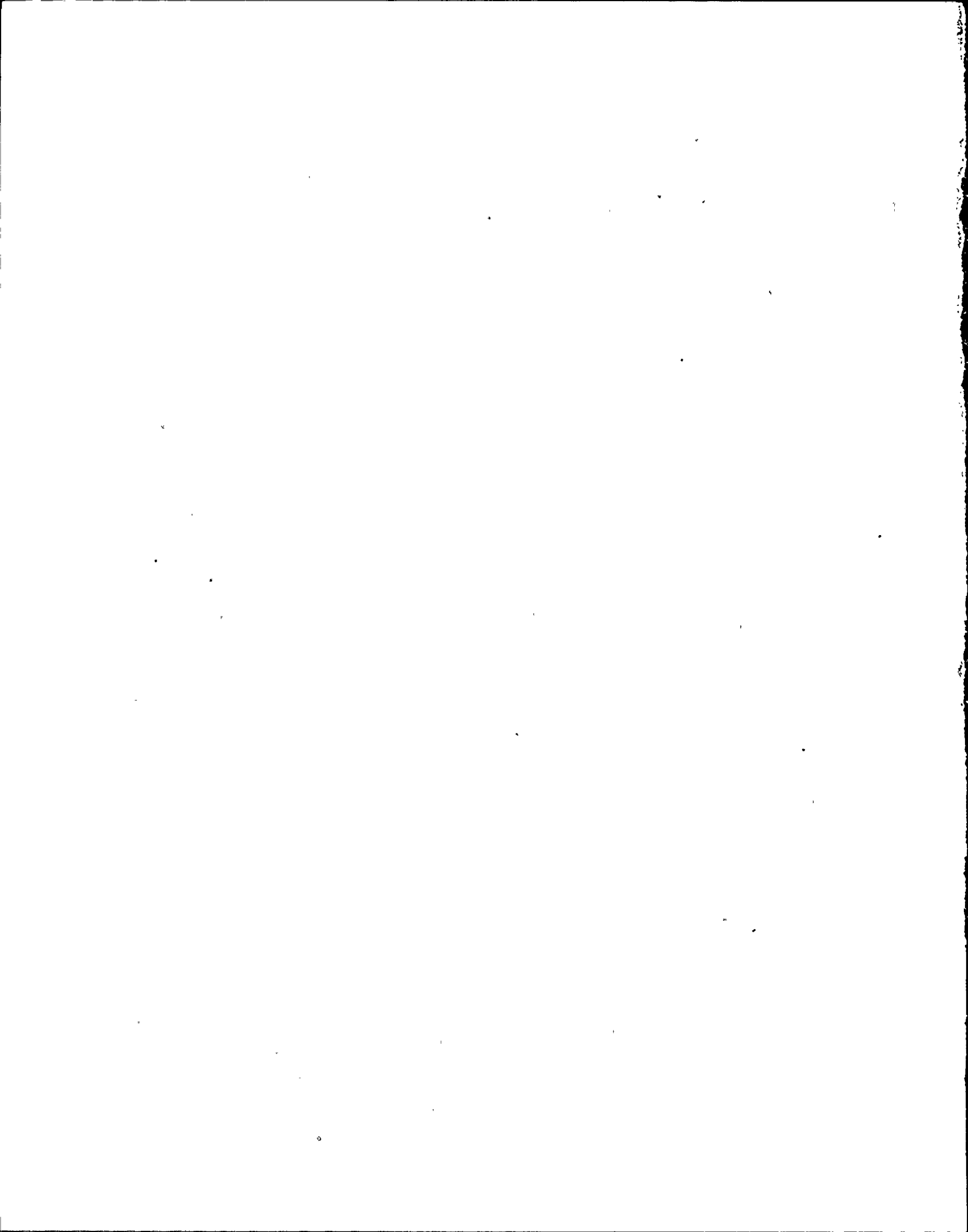
L. Depew presented this new procedure to describe the operating limits for Ginna Station Transmission Circuits and to incorporate Operation Standing Order D.73-55 into the new procedure. The Committee recommended approval of the new procedure.

PROC ITEM NUMBER:	DESCRIPTION
75-1310	<p><u>SC-1.10, Planning Conduct and Evaluation of A Radiation Emergency Drill SC-1.10, PCN75-2042.</u> L. Depew presented this procedure change. It is requested to add the following words to step 3.2.5.1;</p> <p>"He shall also instruct <u>all</u> observers to ensure the guard house door is closed tight after opening the door"</p> <p>The reason for this change is to ensure that the door is kept locked for security requirements. The Committee recommended approval of this procedure change.</p>
75-1311	<p>L. Depew presented the following procedure changes;</p> <p><u>SC-1.7E, Emergency Radiation Monitoring On-Site Survey Team #9 (Blue) Procedure, PCN75-2041.</u> It requested to add the following to step 3.7.9.1;</p> <p>"Ensure guard house door is closed and locked after using it"</p> <p>The reason for this is to maintain security.</p>
	<p><u>SC-1-7G, Emergency Radiation Monitoring On-Site Survey Team #9G (Yellow) Procedure, PCN75-2040.</u> It requested to add step 3.7.7.1;</p> <p>"Ensure guard house door is closed and locked after using it"</p> <p>The reason for this is to maintain security.</p> <p>The Committee recommended approval of these changes.</p>
75-1312	<p><u>SC-1.11, Post-Accident Re-Entry And Recovery Plan, PCN75-1935.</u></p> <p>PCN75-1935 requested to change item 3.2 to read as follows:</p> <p>The Emergency Coordinator shall have the responsibility to determine that the facility and/or surroundings are safe. If off-site conditions are such that off-site support assistance were required, the RG&E Nuclear Audit and Review Board, the Plant Operating Review Committee, and the USNRC must mutually agree that the site is safe and the plant can then be returned to normal operation.</p> <p>The reason for this is to be consistent with SC-1. The Bureau of Radiation Health should only be concerned with matters beyond the site boundary. The Committee recommended approval of this change.</p>
75-1313	<p><u>PP-16, Auxiliary Feedwater System Flow Check, PCN75-2039.</u></p> <p>L. Depew presented these procedure changes. It requested;</p> <p>Delete old step 4.1 - As it reads at present</p> <p>Add new step 4.1 - The auxiliary feedwater system is aligned for safeguards operation.</p> <p>The reason for this is that we should be running a complete check off each time, prior to performing PP-16 on these pumps. -207-</p> <p>The Committee recommended approval of these changes</p>

PORC ITEM
NUMBER:

DESCRIPTION

- 75-1315 P-9, Precautions, Limitations and Setpoints P-9 Radiation Monitoring System, PCN75-2047. J. Depew presented these procedure changes. Four new steps were added to this procedure to incorporate information from an Operation Standing Order concerning Buses 10A and 10B. The Committee recommended approval of these changes.
- 75-1318 M-32.1, DB-25, DB-50, and DB-75 Circuit Breaker Maintenance and Overcurrent Trip Device Test and/or Replacement Procedure, PCN75-2060. L. Depew presented the following procedure changes;
- Step 2.1 - change 63-23 to 60-23
Step 5.1.4 - change 63-25 to 60-23
Step 5.1.9 - change 63-23 to 60-23
- The reason for these changes is to correct a typing error. The Committee recommended approval of these changes.
- 75-1321 L. Depew brought the following item to the Committee's attention. PCN75-1434 was presented to the PORC at Meeting #103-75, but was inadvertently omitted from the minutes.
- P-9, Radiation Monitoring System Setpoints, PCN75-1434(Temporary). E. Beatty presented the procedure change. "R-12 setpoint is now 10K, present activity is 9K. Suggest raising setpoint to 30K, Ed DeMeritt concurs with this." The Committee agreed with the change, and it is to be made permanent. The reason for this change is to prevent the CV ventilation isolation signal at the alarm point (of 10K).
- 75-1322 O-6, Operation and Process Monitoring, PCN75-2045. L. Depew presented this procedure change. Several changes were requested to incorporate Operation Standing Orders. The Committee recommended approval of these changes.
- 75-1327 S-14, Area and Process Radiation Monitoring System, PCN75-7056. L. Depew presented this procedure change. It requested to;
- Change note after step 5.1.13 to read: "Step 5.1.13 may be considered N.A. if system was in operation and PT-17.1 has been completed within one month."
- Change note after Step 5.6 to read: "Step 5.6 may be considered N.A. if system was in operation and PT-17.2 has been completed within one month.
- The reason for these changes is to correct the wrong step number in each note. The Committee recommended approval of this change.



PORC ITEM
NUMBER:DESCRIPTION

- 1331 S-26.2, Computer Out-of-Service, PCN75-2044. L. Depew presented this procedure change. It requested the following changes;
- Step 2.2 should read "GS-22.0, Computer Guard Check".
- Add - 5.1.1 - Record time the computer fails and time it is returned to service in the control room Official Record.
- 5.3 - Add - "File these with the computer log sheets to fill in for the time period. There was no print out."
- The Committee recommended approval of these changes.
- 75-1336 SM-75-18.2, Pre-Startup Checkout of Temporary Blowdown System, PCN75-2082. S. Spector presented this new procedure, which was Pre-PORC'd, to outline the steps that must be performed prior to startup of the temporary blowdown system.
- 75-1337 SM-75-18.3, Initial Startup of Steam Generator Blowdown Heat Recovery System, PCN75-2075. S. Spector presented this new procedure, which was Pre-PORC'd, to describe the steps necessary to perform final check-out of the steam generator blowdown heat recovery system. This item is NOT COMPLETE pending post-maintenance review.
- 75-1338 L. Depew presented the following procedures for review:
- EM-112, Inspection and Repair of 1A Gas Decay Tank Inlet Valves, PCN75-2120.
- EM-116, Replacement of the Clip-on Strainers for the Service Water Pumps, PCN75-2098.
- EM-120, Replacing Valve 2233 on the Non-Regenerative Heat Exchanger Drain Line, PCN75-2107.
- EM-125, Repair of 1A Steam Generator Blowdown Sample Isolation Valve CV-76 (5735), PCN75-2113.
- EM-138, New Fuel Element Index Hole Reaming, PCN75-2192.
- EM-141, Excess Letdown Line Relocation, PCN75-2108.
- EM-142, Level Transmitter LP-462 Installed in Place of LP-461 to Control LM-463F and LC-480, PCN75-2109.
- The Committee recommended that the above procedures be deleted as they are no longer needed.

PROC ITEM NUMBER:	DESCRIPTION
<u>75-1344</u>	<p><u>S-3.1B, Pre-Operational Line Up of Boric Acid System, PCN75-2081.</u> L. Depew presented this temporary procedure change to be made permanent. It requested the following changes;</p> <p>Add a reference 2.2 Flow diagram CVCS Sheet #1 Add 5.2.1 - Batch Tank outlet valve 375c closed 5.4.2 - Add - (If cover on filter and no leaks evident, assume valve closed) 5.4.4 - Change to: 348A 5.4.5.1 - Change to PT-108 5.8 - Change to M. O. immediate boration valve MOV-350 closed Add 5.8.1 - Isolation valve for MOV-350, 348B open Add 5.8.2 - MOV 350/RMW flush valve 353 locked closed Add 5.9.1 - MOV 350/RMW flush vent 348c closed & capped Add 5.16.1 - Check filter bottom for possible leakage (see step 5.4.2) Add 5.19 - MOV-350 fuses in _____ Breaker closed _____</p> <p>The reason for this change is to provide better guidance for Pre-Operational line up. The Committee reviewed these temporary changes and recommended that they be made permanent.</p>
<u>75-1349</u>	<p><u>PT-2.3, Safeguard Valve Operator, PCN75-2139.</u> L. Depew presented this procedure change. It requested to delete steps 6.10.6, 6.10.6.1, and 6.10.6.2. The reason for this is that both C.C.W. Heat Exchangers are to remain in service. The Committee recommended approval of this change.</p>
<u>75-1350</u>	<p><u>M-12.1, Boric Acid Flow Transmitter-PT-110, Removal and Installation, PCN75-2063.</u> L. Depew presented this new procedure to provide a procedure for the removal of PT-110 for maintenance and/or replacement. The Committee recommended approval of this new procedure.</p>
<u>75-1351</u>	<p><u>K-32, Annunciator Card File, PCN75-2079.</u> L. Depew presented this PCN. It requested to add to Annunciator K-32, the temporary blow-down heat recovery Hi temperature alarm. This will convert this alarm window to a dual function; the C.V. air dryer, which is only used during refueling, and the above Heat Exchanger which is used when plant is on the line. The reason for this is there are no spare annunciator windows for the temporary blowdown Heat Exchanger.</p>
<u>75-1352</u>	<p><u>PT-6.2A, N-35, N.I.S. - Intermediate Range Channels Alignment and/or Bistable Adjustment Procedure, PCN75-2169.</u> L. Depew presented this temporary procedure for review only. It requested that Steps 5.6 through 5.21.10 and steps 5.23 through 5.23.13 are to be marked N/A. The reason for this is that only the steps to adjust high level rod stop, and high level trip setpoints were necessary. The Committee reviewed this change.</p>

FORC ITEM
NUMBER:

DESCRIPTION

- 75-1353 PT-6.2A, N-36, N.I.S - Intermediate Range Channels Alignment and/or Bistable Adjustment Procedure, PCN75-2170. L. Depew presented this temporary procedure for review only. It requested that Steps 5.6 through 5.21.10 and Steps 5.23 through 5.23.13 are to be marked N/A. The reason for this is that only the steps to adjust high level rod stop and high level trip setpoints were necessary. The Committee reviewed this change.
- 75-1358 S-14.1, Radiation Monitoring and Related Systems Daily Plot Requirements, PCN75-2049. L. Depew presented this new procedure to describe the daily plotting requirements and how the plotting shall be done. This is to incorporate Operations Standing Order 73-12 into an S procedure. The Committee recommended approval of this new procedure.
- 75-1365 S-8, Component Cooling Water System, PCN75-2181. L. Brodie presented this procedure change. It requested to add Step 3.6 as follows;
- 3.6 Maintain both C.C.W. heat exchangers in service with service water throttled to keep C.C.W. Heat Exchanger outlet temperature $\geq 50^{\circ}\text{F}$.
- The reason for this is to ensure proper operating policy for C.C.W. system. The Committee recommended approval of this change.
- 75-1368 O-5.1, Load Reductions, PCN75-2184. L. Brodie presented this procedure change. It requested to change Step 5.2 to read; "Add Boric Acid to the Reactor Coolant System (Refer to S-3.1, Step 5.4, Boron Concentration Controls." The reason for this is that Procedure S-3.1 was updated and the step number was changed. The Committee recommended approval of this change. This item IS COMPLETE.
- 75-1369 O-6.7.1, Plant Alarm Panels Test, PCN75-2195. L. Brodie presented this new procedure to describe use of panel test log form in the testing of plant alarm panels. The reason is to provide a procedure for panel test log forms. The Committee recommended approval of this change. This item IS COMPLETE.
- 75-1373 SC-1.1, Radiation Emergency Procedures General Information, PCN75-2201. L. Brodie presented this procedure change. It requested to add to step 3.2.2.8 the number of the H. P. procedures. The reason is the number is missing. The Committee suggested the procedure specify that the Health Physics Section may be responsible rather than use the words of the PCN request. The Committee recommended approval of this change.

PROC ITEM NUMBER:	DESCRIPTION
75-1375	<u>M-11.4.2, Isolation of 1B Charging Pump for Maintenance and Restoring to Service After Maintenance, PCN75-2193.</u> L. Brodie presented this temporary procedure for review only to repair leak on "B" charging Pump Drain Line. The Committee reviewed this change.
75-1381	<u>M-11.24, Main Feed Pump Inspection and Maintenance, PCN75-1172.</u> L. Brodie presented this new procedure to be used for the inspection and maintenance of Worthington Type 14 WNCB-131 pumps. Certain steps may not apply to all maintenance; and those steps may be marked N/A. The Committee recommended approval of this new procedure.
75-1382	<u>M-11.4.6, Charging Pump Stuffing Box Replacement, PCN75-2022.</u> L. Brodie presented this new procedure to outline the steps for replacement of the stuffing boxes on a charging pump. The Committee recommended approval of this new procedure.
75-1383	<u>M-11.4.7, Charging Pumps Stuffing Box Assembly Hotshop Inspection and Maintenance, PCN75-2142.</u> L. Brodie presented this new procedure to be used for the inspection and maintenance (Rebuilding) of charging pump stuffing box assemblies in the Hot Shop. The Committee recommended approval of this new procedure.
75-1384	<u>M-56.1, Power Range NIS Detection Replacement, PCN75-2161.</u> L. Brodie presented this new procedure to describe the steps required for replacement of a NIS Power Range detector. The Committee recommended approval of this new procedure.
75-1385	<u>S-14.2, RMS Check Source Data, PCN75-2048.</u> L. Brodie presented this procedure. It was disapproved. This procedure is not required by Technical Specifications or operations.
75-1390	<u>CP-35.0, Intermediate Range Channel N-35 Calibration and/or Maintenance, PCN75-2192.</u> L. Brodie presented this new procedure to describe a series of procedures and a data sheet necessary for calibration or maintenance work on Intermediate Range channel N-35. The reason for this is to change PT-6.2A to a calibration procedure. The Committee recommended approval of this new procedure.
75-1391	<u>CP-35.1, Intermediate Range N-35 General Defeat, PCN75-2231.</u> L. Brodie presented this new procedure for defeating Intermediate Range N-35 to perform calibration and/or maintenance. The reason for this is to change PT-6.2A to a Calibration Procedure. The Committee recommended approval of this new procedure.

<u>PORC ITEM NUMBER:</u>	<u>DESCRIPTION</u>
<u>75-1392</u>	<u>CP-35.3, Intermediate Range N-35 Calibration, PCN75-2233.</u> L. Brodie presented this new procedure to outline a procedure for calibration and/or maintenance on Intermediate approval of this new procedure. This item <u>IS COMPLETE.</u>
<u>75-1393</u>	<u>CP-35.4, Intermediate Range N-36 Bistable Relay Drivers Calibration, PCN75-2234.</u> L. Brodie presented this new procedure for calibration and/or maintenance on Intermediate Range Bistable Relay Drivers. The reason for this is to change PT-6.2A to a Calibration Procedure. The Committee recommended approval of this new procedure. <u>IS COMPLETE.</u>
<u>75-1394</u>	<u>CP-36.0, Intermediate Range Channel N-36 Calibration and/or Maintenance, PCN75-2191.</u> L. Brodie presented this new procedure to describe a series of procedures and a data sheet necessary for calibration or maintenance work on Intermediate Range channel N-36. The reason for this is to change PT-6.2A to a Calibration Procedure. The Committee recommended approval of this new procedure.
<u>75-1395</u>	<u>CP-36.1, Intermediate Range N-36 General Defeat, PCN75-2235.</u> L. Brodie presented this new procedure for defeating Intermediate Range N-36 to perform calibration and/or maintenance. The reason for this is to change PT-6.2A to a Calibration Procedure. The Committee recommended approval of this new procedure.
<u>75-1396</u>	<u>CP-36.2, Intermediate Range N-36 General Reinstate Procedure, PCN75-2236.</u> L. Brodie presented this new procedure for reinstating Intermediate Range N-36 from the defeat mode. The reason for this is to change PT-6.2A to a Calibration Procedure. The Committee recommended approval of this new procedure.
<u>75-1397</u>	<u>CP-36.3, Intermediate Range N-36 Calibration, PCN75-2237.</u> L. Brodie presented this new procedure for Calibration and/or maintenance on Intermediate Range N-36. The reason for this is to change PT-6.2A to a Calibration Procedure. The Committee recommended approval of this new procedure.
<u>75-1398</u>	<u>CP-36.4, Intermediate Range N-36 Bistable Relay Drivers Calibration, PCN75-2234.</u> L. Brodie presented this new procedure to outline a procedure for calibration and/or maintenance on Intermediate Range Bistable Relay Drivers. The Reason for this is to change PT-6.2A to a Calibration Procedure. The Committee recommended approval of this new procedure.
<u>75-1399</u>	<u>PT-6.2A, N.I.S. - Intermediate Range Channels Alignment and/or Bistable Adjustment Procedure, PCN75-2190.</u> L. Brodie presented this procedure for deletion because it is being replaced by CP-35 and CP-36. The Committee recommended approval of this deletion.
<u>75-1401</u>	<u>M-11.27, Component Cooling Pump Inspection and Maintenance, PCN75-2199.</u> L. Brodie presented this new procedure for the inspection and maintenance of component cooling pumps. The Committee recommended approval of this new procedure.

FORC ITEM
NUMBER:

DESCRIPTION

75-1404

PT-12.2, Emergency Diesel Generator 1B, PCH75-2240. L. Brodie presented this temporary procedure for review only. It requested to mark steps 6.22 to 6.3, and 6.13 to 6.24 N/A. The reason for this is that the Diesel Generator was started just to check the air pressure during start. The Committee recommended approval of this temporary change.

75-1405

P-9, Precautions, Limitations and Setpoints P-9 Radiation Monitoring System, PCH75-2242. L. Brodie presented this temporary procedure change for review only. It requested;

Change Step 2.3.2.11 from 6000 cpm to ~30,000 cpm.

Pg. P-9:5, R-1G - change 6000 cpm to ~30,000 cpm.

MPC for Iodine @ 100 GPM (approximately normal full blowdown) is equivalent to approximately 45,000 cpm.

The reason for this is to allow re-opening of the S/G blowdown valves following isolation. The Committee recommended approval of this temporary change.

ANNUAL REPORT #11

JANUARY 1, 1975 TO DECEMBER 31, 1975 INCLUSIVE

Annual Report of System Modification and Special Tests

SYSTEM
MODIFICATION
NUMBER: _____

TITLE

SM 74-2

Rod Position Indicator Pulse To Analog Converter Change

Description: Remove the two existing pulse to analog converters, and install the new solid state pulse to analog converter.

Safety Evaluation: This modification does not involve any unreviewed safety questions or Technical Specification changes.

SM 74-3

Repair Of Type W2 Breaker Control Switches

Description: Replace the pull lever in the W-2 switches with a new double pin lever.

Safety Evaluation: This modification does not involve any unreviewed safety question or Technical Specification changes.

SM 74-18

Steam Generator Platforms

Description: Fabricate and install permanent (removable) platforms below steam generators 1A & 1B for inspection, testing, and cleaning of Steam Generators.

Safety Evaluation: This modification does not involve any unreviewed safety question and does not require a change in Technical Specifications.

SM 74-23

Installation of the Modification for Control of the Boric Acid Supply to Safety Injection System

Description: Change the logic and timer controls for the refueling water storage tank to safety injection system valves 825A and 825B.

Safety Evaluation: This modification does not involve any unreviewed safety question and does not have an effect to the Technical Specifications.

SYSTEM
MODIFICATION
NUMBER:

SM 74-24

Containment Monitor Pump Trip Modification

Description: Install trip circuit so pump will stop on closure of inlet and outlet valves when monitor is in containment sample mode.

Safety Evaluation: This modification does not involve any unreviewed safety questions and does not require change of Technical Specifications.

SM 74-30

Steam Generator Blowdown Jumper

Description: Modify the Steam Generator Blowdown isolation valves (CV-70 and CV-71) control circuitry to permit opening of the valves when main feedwater pumps are out of service.

Safety Evaluation: This modification does not involve any unreviewed safety questions or Technical Specification changes.

SM 74-41

Reactor Trip Breaker Train A and B Undervoltage 20ET and 2OAST Monitor Circuit Installation

Description: Install monitor lights to verify proper operation and circuit makeup of the trip breakers and undervoltage trip scheme.

Safety Evaluation: This modification does not involve any unreviewed safety questions or Technical Specification changes.

SM 74-45

All Volatile Treatment Panel

Description: Installation of All Volatile Treatment Instrument panel.

Safety Evaluation: This modification does not effect Technical Specifications and does not involve any unreviewed safety questions.

SM 75-1

Instrumentation Relocation in Intermediate Building

Description: Reactor protection channel flow and pressure transmitters that sense main steam line pressure and main feedwater flow were moved to areas that are less vulnerable to pipe whip as a result of a major main steam or feedwater pipe rupture. Jet shielding was installed to protect the associated instrument tubing. Cabling of some containment instruments that could be affected by such a rupture were rerouted through the Auxiliary Building.

SYSTEM
MODIFICATION
NUMBER:

SM 75-1 (Con't) Safety Evaluation: This modification does not involve any unreviewed safety question and does not require Technical Specifications change.

SM 75-2 T.V. System in Containment

Description: Mount T.V. monitoring system, install a new electrical penetration, and run conduit and cables to allow control room observation of reactor coolant pumps, reactor vessel and charcoal filters.

Safety Evaluation: This modification does not involve any unreviewed safety question or Technical Specification changes.

SM 75-3 Reheater Steam Relief Vent Pipe, Support and Concrete

Description: Modification of reheater valve inlet pipe and reheater steam relief header discharge piping for relieve escaping steam dynamic load, and addition of new structural steel for increased support.

Safety Evaluation: This modification does not involve any unreviewed safety questions or any Technical Specification changes.

SM 75-4 Reroute of Steam Generator Blowdown Line

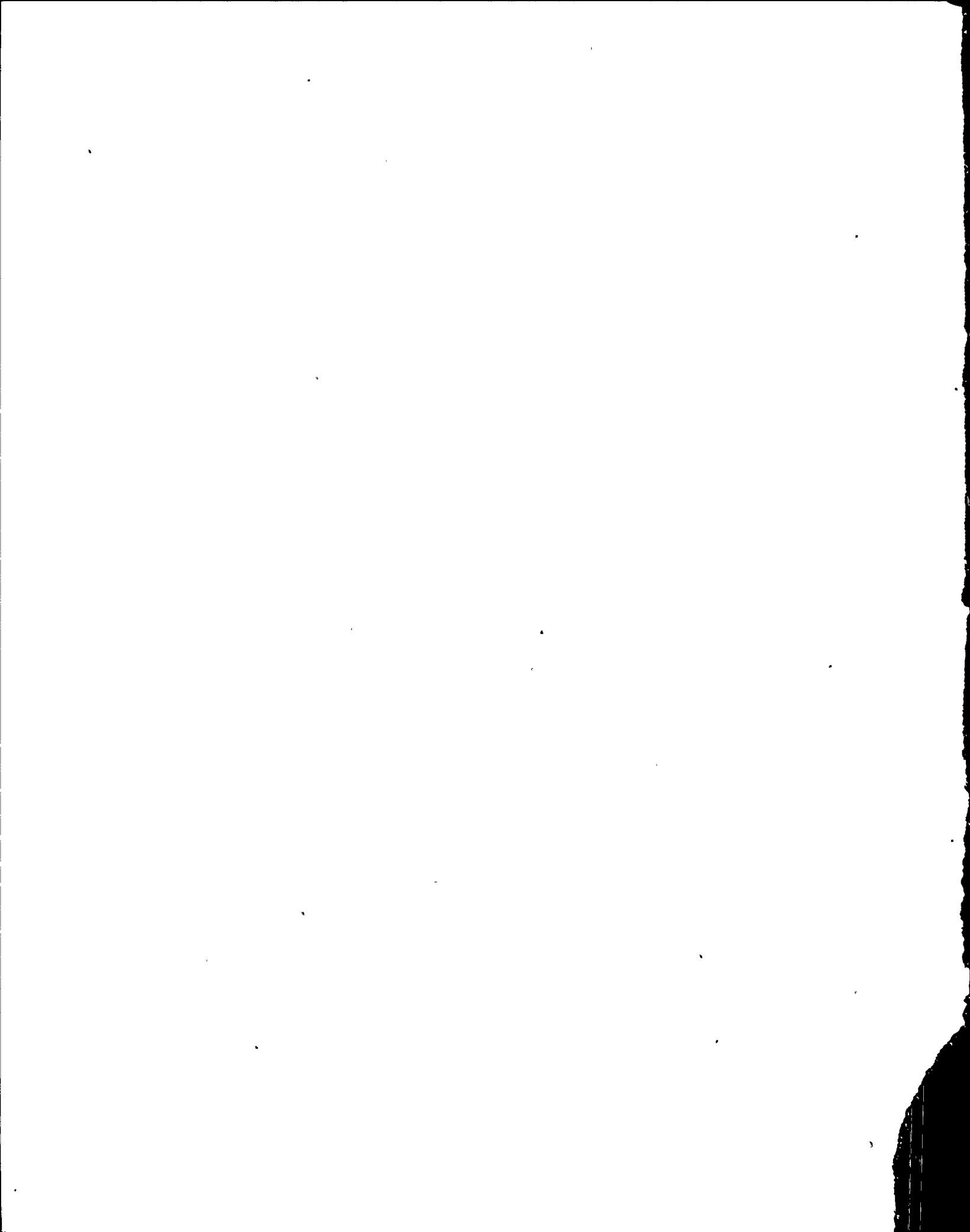
Description: Reroute portion of the Steam Generator Blowdown piping and install jet shields and line restraints.

Safety Evaluation: This modification does not involve any unreviewed safety question or Technical Specification changes.

SM 75-6 Service Water Redundant Return

Description: This modification consists of the addition of a redundant return line for the Service Water. This return line will service two component cooling water heat exchangers, the spent fuel pit heat exchangers, and the auxiliary building motor coolers.

Safety Evaluation: This modification does not involve any unreviewed safety question and does not require Technical Specifications change.



SYSTEM
MODIFICATION
NUMBER:

SM 75-7.1

Heating Ventilation and Air Conditioning Backfits for Control Room and Relay Room

Description: Installed duct work above control room ceiling and conduit and cable in the control room and the relay room areas.

Safety Evaluation: This modification does not involve any unreviewed safety question and does not require a change of Technical Specifications.

SM 75-8

Service Water Protection - Intermediate Building

Description: Install steel plate in the Intermediate building directly under the main steam header.

Safety Evaluation: The modification does not involve any unreviewed safety question or Technical Specification changes.

SM 75-10.1

Installation of Grounding Cable for Auxiliary Building Addition

Description: Installation and revision of grounding cable for grounding the new Auxiliary Building.

Safety Evaluation: This modification does not involve any unreviewed safety question and does not require a change of Technical Specifications.

SM 75-10.2

Installation of Auxiliary Building Addition Caissons

Description: Installation of caissons for the Auxiliary Building addition.

Safety Evaluation: This modification does not involve any unreviewed safety question and does not require a change of Technical Specification.

SM 75-10.21

Auxiliary Building Addition Excavation, Fill and Compaction

Description: Excavation, fill and compaction for the Auxiliary Building Addition.

Safety Evaluation: This modification does not involve an unreviewed safety question and does not require a change of Technical Specification.

SYSTEM
MODIFICATION
NUMBER:

SM 75-10.22

Auxiliary Building Addition Below Ground Installations

Description: Make below ground penetration and other necessary installation, prior to floor pad installation for the Auxiliary Building addition.

Safety Evaluation: This modification does not involve and unreviewed safety question and does not require a change of Technical Specification.

SM 75-11

Switch Control Board Recorders and Controllers to Instrument Bus 1C

Description: Switch of control board recorders and controllers power feed over to instrument bus 1C.

Safety Evaluation: This modification does not involve and unreviewed safety question and does not require a change in the Technical Specifications.

SM 75-16

Feedwater Thermowells Installations

Description: Install new thermowells between flow orifice and check valves in "A" and "B" feedwater lines.

Safety Evaluation: The installation was in a Non-QA pipe and is before the check valve. Due to this there is no safety problems and does not require a Technical Specification change.

SM 75-17

Condensate Reject Line Separation

Description: Separate Auxiliary Feedwater pump suction from condensate reject line.

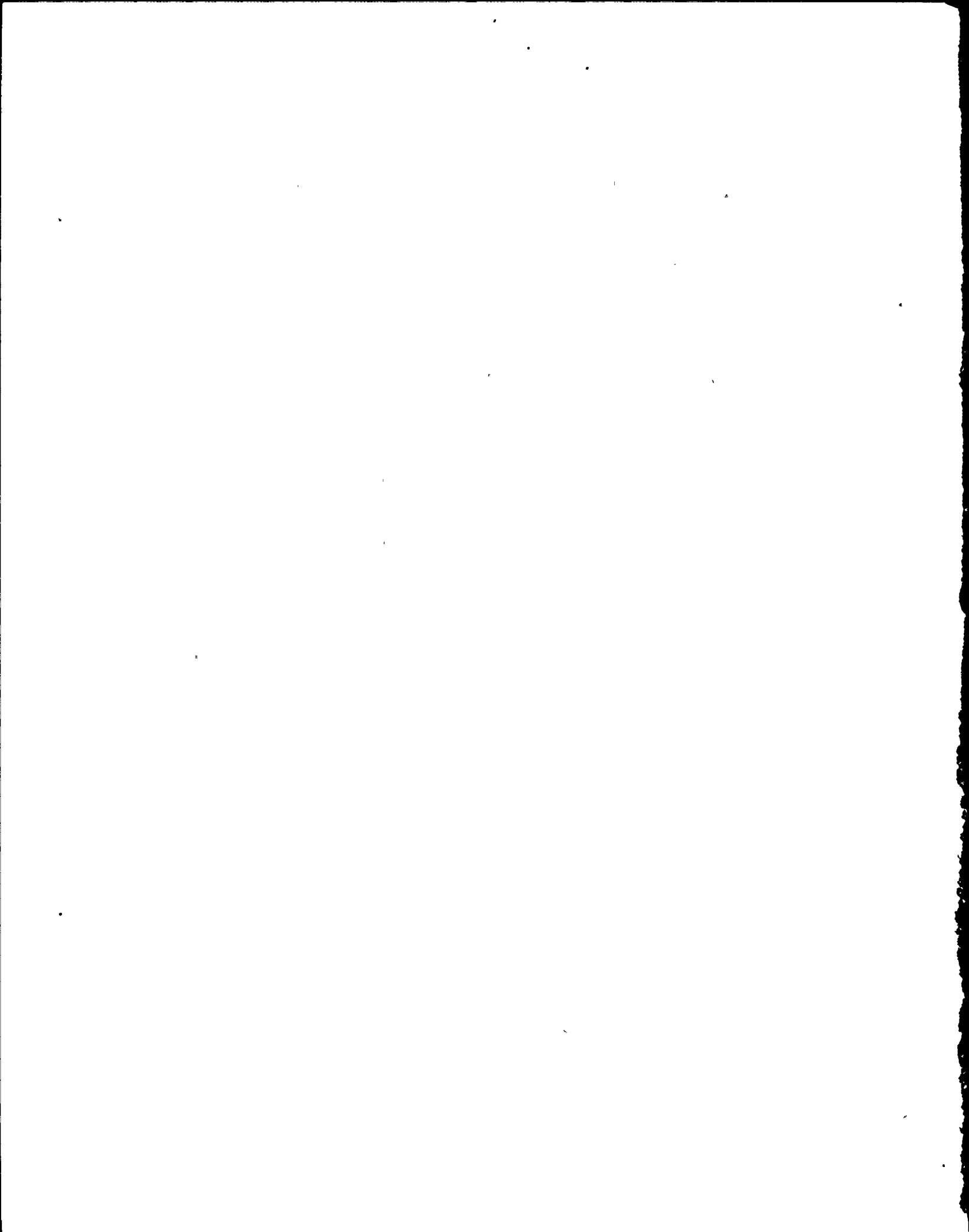
Safety Evaluation: This modification does not involve an unreviewed safety question and does not require a change in Technical Specifications.

SM 75-19

Pressurizer Jib Crane Extension

Description: Add a removable extension onto the pressurizer jib crane to allow work on the pressurizer safety valves.

Safety Evaluation: This modification did not involve any unreviewed safety questions or Technical Specification changes.



SYSTEM
MODIFICATION
NUMBER:

SM 75-20

Control Rod Drive Shaft Modification

Description: Machining of chamfers on control rod drive rod assemblies to correct an interference condition which is possible.

Safety Evaluation: This modification does not involve any unreviewed safety question and does require change of Technical Specifications.

SM 75-23

Accumulator Level Indicator Modification

Description: Remove and reinstall loop accumulator level transmitters to allow raising of their positions approximately 16.778".

Safety Evaluation: This modification does not involve any unreviewed safety questions and does not require change of Technical Specifications.

SM 75-24.1

Steam Generator Orifice Ring Modification

Description: Remove the existing orifice ring and install new orifice ring to steam generator swirl vanes.

Safety Evaluation: This modification does not involve and unreviewed safety question and does not require a change in Technical Specifications.

SM 75-24.2

Feedwater Ring Modifications on Steam Generators

Description: Enlarge hot leg holes and plug certain cold leg holes.

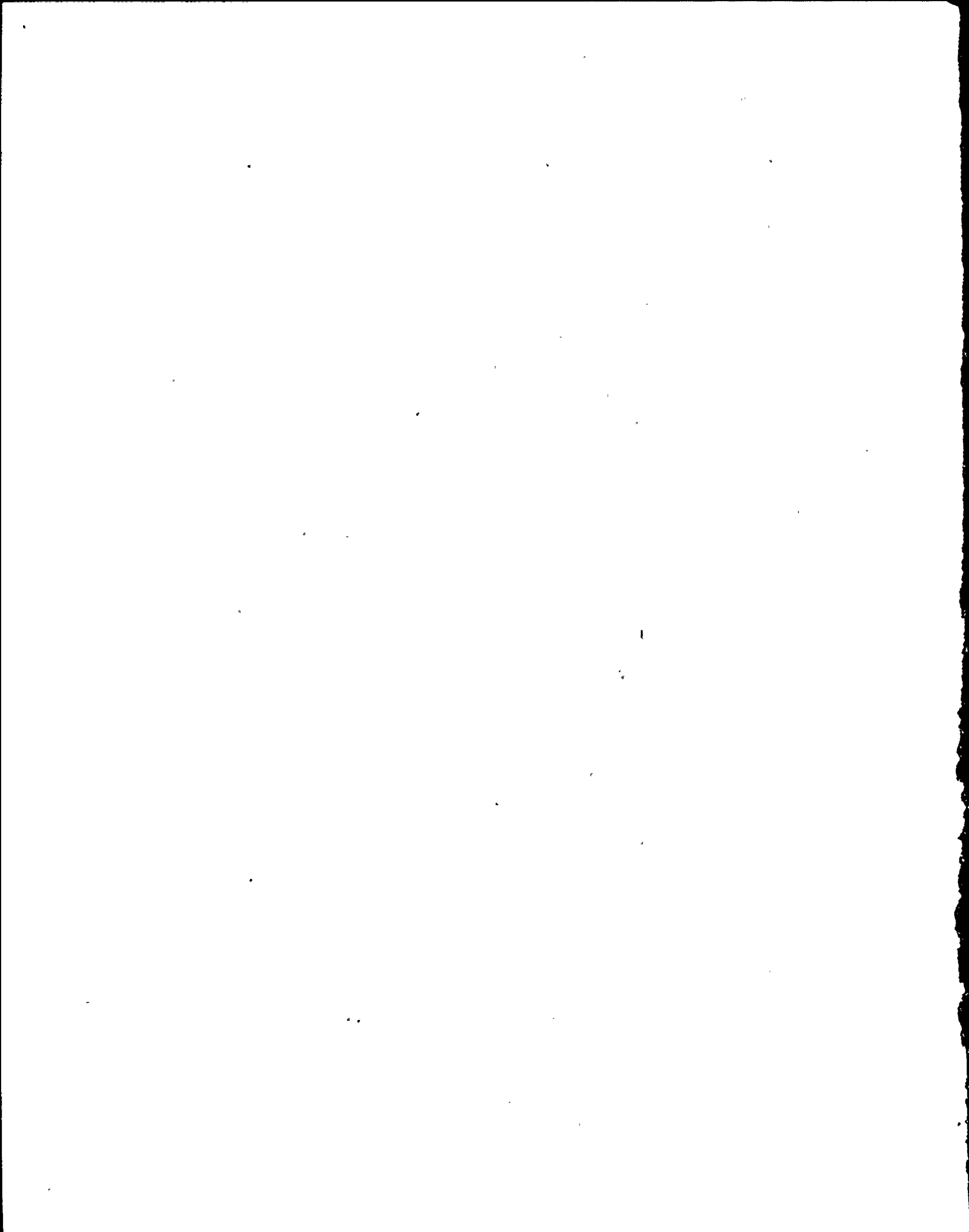
Safety Evaluation: This modification does not involve any unreviewed safety question and does not require a change in Technical Specifications.

SM 75-24.3

Downcomer Resistance Plate Removal for Steam Generators

Description: Remove feedwater resistance plate and support rods.

Safety Evaluation: This modification does not involve any unreviewed safety question and does not require a change in Technical Specifications.



SYSTEM
MODIFICATION
NUMBER: _____

SM 75-24.4

Tube Lane Blocking Device

Description: Install tube lane blocking device

Safety Evaluation: This modification does not involve any unreviewed safety question and does not require a change in Technical Specifications.

SM 75-27

Provide Key Switched on MCB for MOV's 896A & 896B

Description: Install key switch on DC power for MOV 896A & B to reduce passive failure probability.

Safety Evaluation: This modification does not involve any unreviewed safety question or Technical Specification.

SM 75-28

Additional Telephone Cables

Description: Install two conduits from outside through 1B Battery Room wall to allow installation of two new 50 pair telephone cables.

Safety Evaluation: This modification does not involve any unreviewed safety question or Technical Specification change.

SM 75-30

Installation of Solid State Underfrequency Relay
For 4KV Bus

Description: Remove and replace protective relays 811/11A, B and 812/11A, B, with a new model KF relay, to reduce probability of a relay malfunction.

Safety Evaluation: This modification does not involve any unreviewed safety question or Technical Specification change.

SM 75-32

Installation of Penetration Seals With BISCO Silicone
Foam

Description: Install BISCO silicone foam in floor and wall penetrations for prevention, limitation and control of fire, its hazards and damaging effects, where the protection of safety related systems, structures and equipment is required.

SYSTEM
MODIFICATION
NUMBER: _____

SM 75-32 (Con't) Safety Evaluation: This modification does not involve any unreviewed safety questions and does not require a change in Technical Specifications.

SM 75-33 Provide Key Switches on MCB for 852A & B

Description: Provide key switch on DC power for MOV 852A and B to de-energize valves.

Safety Evaluation: This modification does not involve any unreviewed safety question or Technical Specification change.

SM 75-35 Main Steam Isolation Valves Modification

Description: Remove upper seal on piston in actuator, machine approximately 1-3/8" off disc stop in valve body, and drill 1/8" bleed hole on upper chamber of actuator.

Safety Evaluation: This modification does not involve any unreviewed safety questions and does not require a change in Technical Specifications.

SM 75-36 Temporary Feedwater Control Valve Modification

Description: Install hydraulic dampening system on feedwater control valves.

SPECIAL
TEST
NUMBER: _____
Safety Evaluation: This modification does not involve any unreviewed safety question or Technical Specification change.

ST 73-4 Circuit Breaker Overcurrent Trip Device Test

Description: This test was conducted on the Safeguards Equipment to insure that the overcurrent devices would operate correctly. Several devices have been replaced and the testing is continuing.

Safety Evaluation: This test does not involve any unresolved safety question and does not require a change in the Technical Specifications.

ST 74-6.1 Installation of a Negative Sequence Recorder on the Generator Output

Description: Install a VP-840 watt transducer in the generator metering potential and current transformers for subsequent recorder hookup and tests.

SPECIAL TEST NUMBER:

ST 74-6.1
(Con't)

Safety Evaluation: The test did not involve any unresolved safety questions, and does not require a change in the Technical Specifications.

ST 74-6.2

Removal of Westinghouse Supplied Negative Sequence Recorder and Installation of RG&E Negative Sequence Monitor on the Generator Output

Description: To effect the steps necessary to remove a VP-840 watt transducer in the generator metering potential and current transformers for recorder hookup and tests, and installation of similar equipment.

Safety Evaluation: The test did not involve any unresolved safety questions, and does not require a change in the Technical Specifications.

ST 75-1

Rapid Fuel Assembly Power Escalation Test

Description: By positioning a control rod (G-9) in a region of relatively high linear power density, the flux will be depressed, decreasing the power density. During the conditioning, significant clad creepdown will take place. When the control rod is withdrawn, the linear power density will increase, causing rapid fuel pellet expansion. The expansion should cause increased clad stresses and the test should verify that vendor calculations are conservative. (See * below).

Safety Evaluation: Higher than normal stresses placed on fuel assembly E-24 cladding, but less than design values. The margin of safety was not reduced and there were no unreviewed safety questions. No Technical Specifications changes were necessary.

ST 75-3

Westinghouse Development Test MSR Reheater Scavenge Steam Vent Condenser

Description: Establish the necessary condition and steps which must be followed to enable the effective testing and evaluation of an SSVC which is installed in MSR-1A.

Safety Evaluation: This test does not involve any unreviewed safety question and does not require change of Technical Specifications.

*: Test was aborted prior to actual rapid-power-escalation when NRC Compliance determined an unreviewed safety question was involved.

SPECIAL
TEST
NUMBER: _____

ST 75-4

Initial Turbine Plant Start-Up Program Following
Plant Trip of 0956 Hours on 6/17/75

Description: To outline requirements for plant startup following plant trip of 0956 hours on 6/17/75.

Safety Evaluation: This test does not involve any unreviewed safety question and does not require a change of Technical Specifications.

ST 75-5

Monitor Six Steam Generator Levels Simultaneously
From All Four Reactor Protection Channels

Description: Record all six steam generator levels using most sensitive range on recorder without inducing external noise.

Safety Evaluations: This test does not involve any unreviewed safety question and does not require a change of Technical Specifications.

ST 75-6

Steam Generator Moisture Carry Over Measurement

Description: Determine the amount of moisture carry-over from the steam generators to feedwater using 24-Na tracers.

Safety Evaluation: This test does not involve any unreviewed safety question and does not require a change of Technical Specifications.

ANNUAL REPORT #11

JANUARY 1, 1975 TO DECEMBER 31, 1975 INCLUSIVE

Annual Report of System Modification and Special Tests

SYSTEM
MODIFICATION
NUMBER: _____

TITLE

SM 74-2

Rod Position Indicator Pulse To Analog Converter Change

Description: Remove the two existing pulse to analog converters, and install the new solid state pulse to analog converter.

Safety Evaluation: This modification does not involve any unreviewed safety questions or Technical Specification changes.

SM 74-3

Repair Of Type W2 Breaker Control Switches

Description: Replace the pull lever in the W-2 switches with a new double pin lever.

Safety Evaluation: This modification does not involve any unreviewed safety question or Technical Specification changes.

SM 74-18

Steam Generator Platforms

Description: Fabricate and install permanent (removable) platforms below steam generators 1A & 1B for inspection, testing, and cleaning of Steam Generators.

Safety Evaluation: This modification does not involve any unreviewed safety question and does not require a change in Technical Specifications.

SM-74-23

Installation of the Modification for Control of the Boric Acid Supply to Safety Injection System

Description: Change the logic and timer controls for the refueling water storage tank to safety injection system valves 825A and 825B.

Safety Evaluation: This modification does not involve any unreviewed safety question and does not have an effect to the Technical Specifications.

SYSTEM
MODIFICATION
NUMBER:

SM 74-24

Containment Monitor Pump Trip Modification

Description: Install trip circuit so pump will stop on closure of inlet and outlet valves when monitor is in containment sample mode.

Safety Evaluation: This modification does not involve any unreviewed safety questions and does not require change of Technical Specifications.

SM 74-30

Steam Generator Blowdown Jumper

Description: Modify the Steam Generator Blowdown isolation valves (CV-70 and CV-71) control circuitry to permit opening of the valves when main feedwater pumps are out of service.

Safety Evaluation: This modification does not involve any unreviewed safety questions or Technical Specification changes.

SM 74-41

Reactor Trip Breaker Train A and B Undervoltage 20ET and 20AST Monitor Circuit Installation

Description: Install monitor lights to verify proper operation and circuit makeup of the trip breakers and undervoltage trip scheme.

Safety Evaluation: This modification does not involve any unreviewed safety questions or Technical Specification changes.

SM 74-45

All Volatile Treatment Panel

Description: Installation of All Volatile Treatment Instrument panel.

Safety Evaluation: This modification does not effect Technical Specifications and does not involve any unreviewed safety questions.

SM 75-1

Instrumentation Relocation in Intermediate Building

Description: Reactor protection channel flow and pressure transmitters that sense main steam line pressure and main feedwater flow were moved to areas that are less vulnerable to pipe whip as a result of a major main steam or feedwater pipe rupture. Jet shielding was installed to protect the associated instrument tubing. Cabling of some containment instruments that could be affected by such a rupture were rerouted through the Auxiliary Building.

SYSTEM
MODIFICATION
NUMBER:

SM 75-1 (Con't) Safety Evaluation: This modification does not involve any unreviewed safety question and does not require Technical Specifications change.

SM 75-2 T.V. System in Containment

Description: Mount T.V. monitoring system, install a new electrical penetration, and run conduit and cables to allow control room observation of reactor coolant pumps, reactor vessel and charcoal filters.

Safety Evaluation: This modification does not involve any unreviewed safety question or Technical Specification changes.

SM 75-3 Reheater Steam Relief Vent Pipe, Support and Concrete

Description: Modification of reheater valve inlet pipe and reheater steam relief header discharge piping for relieve escaping steam dynamic load, and addition of new structural steel for increased support.

Safety Evaluation: This modification does not involve any unreviewed safety questions or any Technical Specification changes.

SM 75-4 Reroute of Steam Generator Blowdown Line

Description: Reroute portion of the Steam Generator Blowdown piping and install jet shields and line restraints.

Safety Evaluation: This modification does not involve any unreviewed safety question or Technical Specification changes.

SM 75-6 Service Water Redundant Return

Description: This modification consists of the addition of a redundant return line for the Service Water. This return line will service two component cooling water heat exchangers, the spent fuel pit heat exchangers, and the auxiliary building motor coolers.

Safety Evaluation: This modification does not involve any unreviewed safety question and does not require Technical Specifications change.

SYSTEM
MODIFICATION
NUMBER:

SM 75-7.1 Heating Ventilation and Air Conditioning Backfits for Control Room and Relay Room

Description: Installed duct work above control room ceiling and conduit and cable in the control room and the relay room areas.

Safety Evaluation: This modification does not involve any unreviewed safety question and does not require a change of Technical Specifications.

SM 75-8 Service Water Protection - Intermediate Building

Description: Install steel plate in the Intermediate building directly under the main steam header.

Safety Evaluation: The modification does not involve any unreviewed safety question or Technical Specification changes.

SM 75-10.1 Installation of Grounding Cable for Auxiliary Building Addition

Description: Installation and revision of grounding cable for grounding the new Auxiliary Building.

Safety Evaluation: This modification does not involve any unreviewed safety question and does not require a change of Technical Specifications.

SM 75-10.2 Installation of Auxiliary Building Addition Caissons

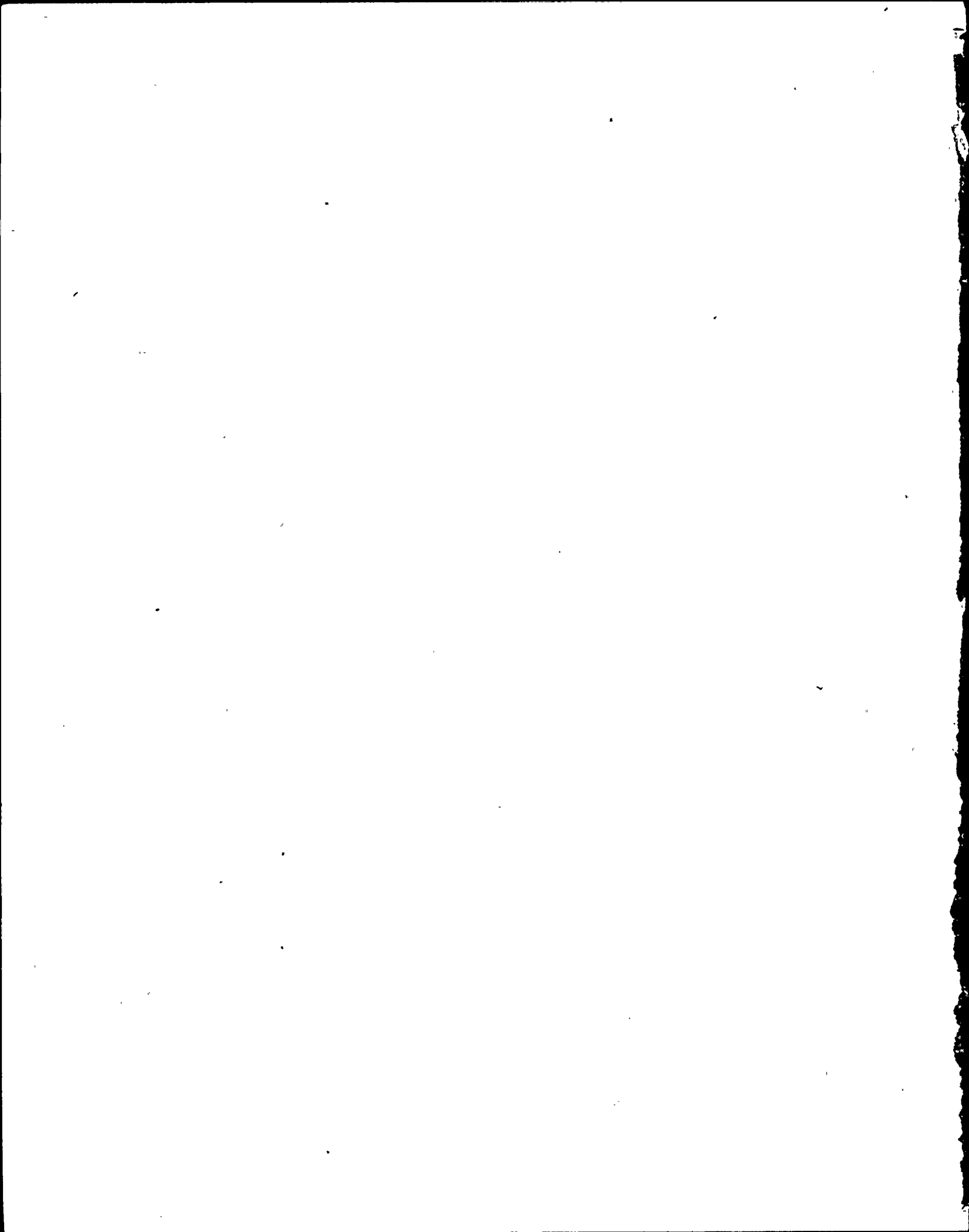
Description: Installation of caissons for the Auxiliary Building addition.

Safety Evaluation: This modification does not involve any unreviewed safety question and does not require a change of Technical Specification.

SM 75-10.21 Auxiliary Building Addition Excavation, Fill and Compaction

Description: Excavation, fill and compaction for the Auxiliary Building Addition.

Safety Evaluation: This modification does not involve an unreviewed safety question and does not require a change of Technical Specification.



SYSTEM
MODIFICATION
NUMBER: _____

SM 75-10.22

Auxiliary Building Addition Below Ground Installations

Description: Make below ground penetration and other necessary installation, prior to floor pad installation for the Auxiliary Building addition.

Safety Evaluation: This modification does not involve and unreviewed safety question and does not require a change of Technical Specification.

SM 75-11

Switch Control Board Recorders and Controllers to Instrument Bus IC

Description: Switch of control board recorders and controllers power feed over to instrument bus IC.

Safety Evaluation: This modification does not involve and unreviewed safety question and does not require a change in the Technical Specifications.

SM 75-16

Feedwater Thermowells Installations

Description: Install new thermowells between flow orifice and check valves in "A" and "B" feedwater lines.

Safety Evaluation: The installation was in a Non-QA pipe and is before the check valve. Due to this there is no safety problems and does not require a Technical Specification change.

SM 75-17

Condensate Reject Line Separation

Description: Separate Auxiliary Feedwater pump suction from condensate reject line.

Safety Evaluation: This modification does not involve an unreviewed safety question and does not require a change in Technical Specifications.

SM 75-19

Pressurizer Jib Crane Extension

Description: Add a removable extension onto the pressurizer jib crane to allow work on the pressurizer safety valves.

Safety Evaluation: This modification did not involve any unreviewed safety questions or Technical Specification changes.

SYSTEM
MODIFICATION
NUMBER:

SM 75-20

Control Rod Drive Shaft Modification

Description: Machining of chamfers on control rod drive rod assemblies to correct an interference condition which is possible.

Safety Evaluation: This modification does not involve any unreviewed safety question and does require change of Technical Specifications.

SM 75-23

Accumulator Level Indicator Modification

Description: Remove and reinstall loop accumulator level transmitters to allow raising of their positions approximately 16.778".

Safety Evaluation: This modification does not involve any unreviewed safety questions and does not require change of Technical Specifications.

SM 75-24.1

Steam Generator Orifice Ring Modification

Description: Remove the existing orifice ring and install new orifice ring to steam generator swirl vanes.

Safety Evaluation: This modification does not involve any unreviewed safety question and does not require a change in Technical Specifications.

SM 75-24.2

Feedwater Ring Modifications on Steam Generators

Description: Enlarge hot leg holes and plug certain cold leg holes.

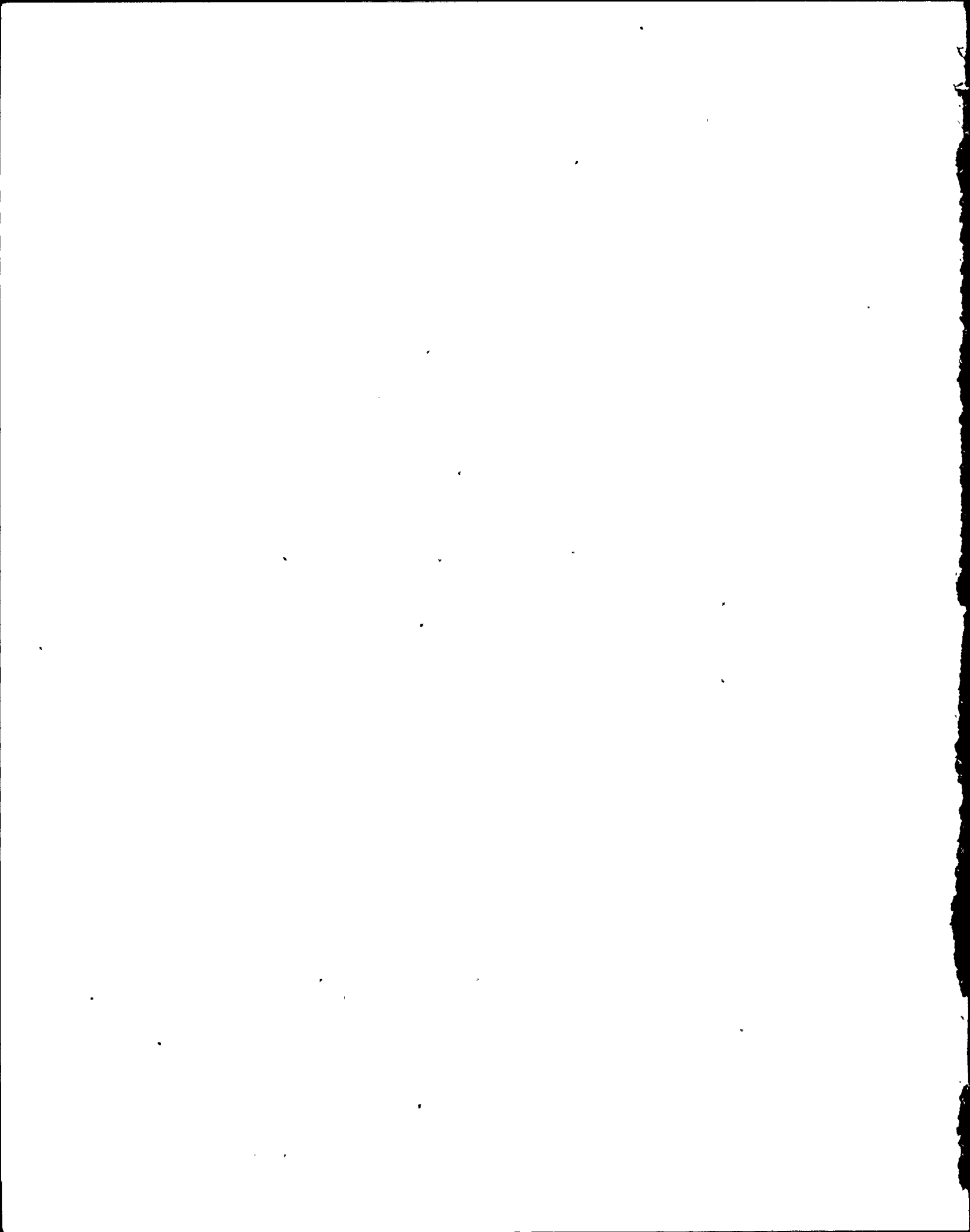
Safety Evaluation: This modification does not involve any unreviewed safety question and does not require a change in Technical Specifications.

SM 75-24.3

Downcomer Resistance Plate Removal for Steam Generators

Description: Remove feedwater resistance plate and support rods.

Safety Evaluation: This modification does not involve any unreviewed safety question and does not require a change in Technical Specifications.



SYSTEM
MODIFICATION
NUMBER: _____

SM 75-24.4 Tube Lane Blocking Device

Description: Install tube lane blocking device

Safety Evaluation: This modification does not involve any unreviewed safety question and does not require a change in Technical Specifications.

SM 75-27 Provide Key Switched on MCB for MOV's 896A & 896B

Description: Install key switch on DC power for MOV 896A & B to reduce passive failure probability.

Safety Evaluation: This modification does not involve any unreviewed safety question or Technical Specification.

SM75-28 Additional Telephone Cables

Description: Install two conduits from outside through 1B Battery Room wall to allow installation of two new 50 pair telephone cables.

Safety Evaluation: This modification does not involve any unreviewed safety question or Technical Specification change.

SM 75-30 Installation of Solid State Underfrequency Relay
For 4KV Bus

Description: Remove and replace protective relays 811/11A,B and 812, 11A,B, with a new model KF relay, to reduce probability of a relay malfunction.

Safety Evaluation: This modification does not involve any unreviewed safety question or Technical Specification change.

SM75-32 Installation of Penetration Seals With BISCO Silicone
Foam

Description: Install BISCO silicone foam in floor and wall penetrations for prevention, limitation and control of fire, its hazards and damaging effects, where the protection of safety related systems, structures and equipment is required.

SYSTEM
MODIFICATION
NUMBER:

SM 75-32 (Con't) Safety Evaluation: This modification does not involve any unreviewed safety questions and does not require a change in Technical Specifications.

SM 75-33 Provide Key Switches on MCB for 852A & B

Description: Provide key switch on DC power for MOV 852A and B to De-energize valves.

Safety Evaluation: This modification does not involve any unreviewed safety question or Technical Specification change.

SM 75-35 Main Steam Isolation Valves Modification

Description: Remove upper seal on piston in actuator, machine approximately 1-3/8" off disc stop in valve body, and drill 1/8" bleed hole on upper chamber of actuator.

Safety Evaluation: This modification does not involve any unreviewed safety questions and does not require a change in Technical Specifications.

SM 75-36 Temporary Feedwater Control Valve Modification

Description: Install hydraulic dampening system on feedwater control valves.

SPECIAL TEST NUMBER: Safety Evaluation: This modification does not involve any unreviewed safety question or Technical Specification change.

ST-73-4 Circuit Breaker Overcurrent Trip Device Test

Description: This test was conducted on the Safeguards Equipment to insure that the overcurrent devices would operate correctly. Several devices have been replaced and the testing is continuing.

Safety Evaluation: This test does not involve any unresolved safety question and does not require a change in the Technical Specifications.

ST 74-6.1 Installation of a Negative Sequence Recorded on the Generator Output

Description: Install a VP-840 watt transducer in the generator metering potential and current transformers for subsequent recorded hookup and tests.

SPECIAL TEST NUMBERS:

ST 74-6.1
(Con't)

Safety Evaluation: The test did not involve any unresolved safety questions, and does not require a change in the Technical Specifications.

ST 74-6 2

Removal of Westinghouse Supplied Negative Sequence Recorded and Installation of RG&E Negative Sequence Monitor on the Generator Output

Description: To effect the steps necessary to remove a VP-840 watt transducer in the generator metering potential and current transformers for recorded hookup and tests, and installation of similar equipment.

Safety Evaluation: The test did not involve any unresolved safety questions, and does not require a change in the Technical Specifications.

ST 75-1

Rapid Fuel Assembly Power Escalation Test

Description: By positioning a control rod (G-9) in a region of relatively high linear power density, the flux will be depressed, decreasing the power density. During the conditioning, significant clad creepdown will take place. When the control rod is withdrawn, the linear power density will increase, causing rapid fuel pellet expansion. The expansion cause increased clad stresses and the test should verify that vendor calculations are conservative. (See * below).

Safety Evaluation: Higher than normal stresses were placed on fuel assembly E-24 cladding, but less than design values. The margin of safety was not reduced and there were no unreviewed safety questions. No Technical Specifications changes were necessary.

ST 75-3

Westinghouse Development Test MSR Reheater Scavenge Steam Vent Condenser

Description: Establish the necessary condition and steps which must be followed to enable the effective testing and evaluation of an SSVV which is installed in MSR-1A.

Safety Evaluation: This test does not involve any unreviewed safety question and does not require change of Technical Specifications.

*: Test was aborted prior to actual rapid-power-escalation when NRC Compliance determined an unreviewed safety question was involved.

SPECIAL
TEST
NUMBER:

ST 75-4

Initial Turbine Plant Start-Up Program Following Plant Trip of 0956 Hours on 6/17/75

Description: To outline requirements for plant start-up following plant trip of 0956 hours on 6/17/75

Safety Evaluation: This test does not involve any unreviewed safety question and does not require a change of Technical Specifications.

ST 75-5

Monitor Six Steam Generator Levels Simultaneously From All Four Reactor Protection Channels

Description: Record all six steam generator levels using most sensitive range on recorder without inducing external noise.

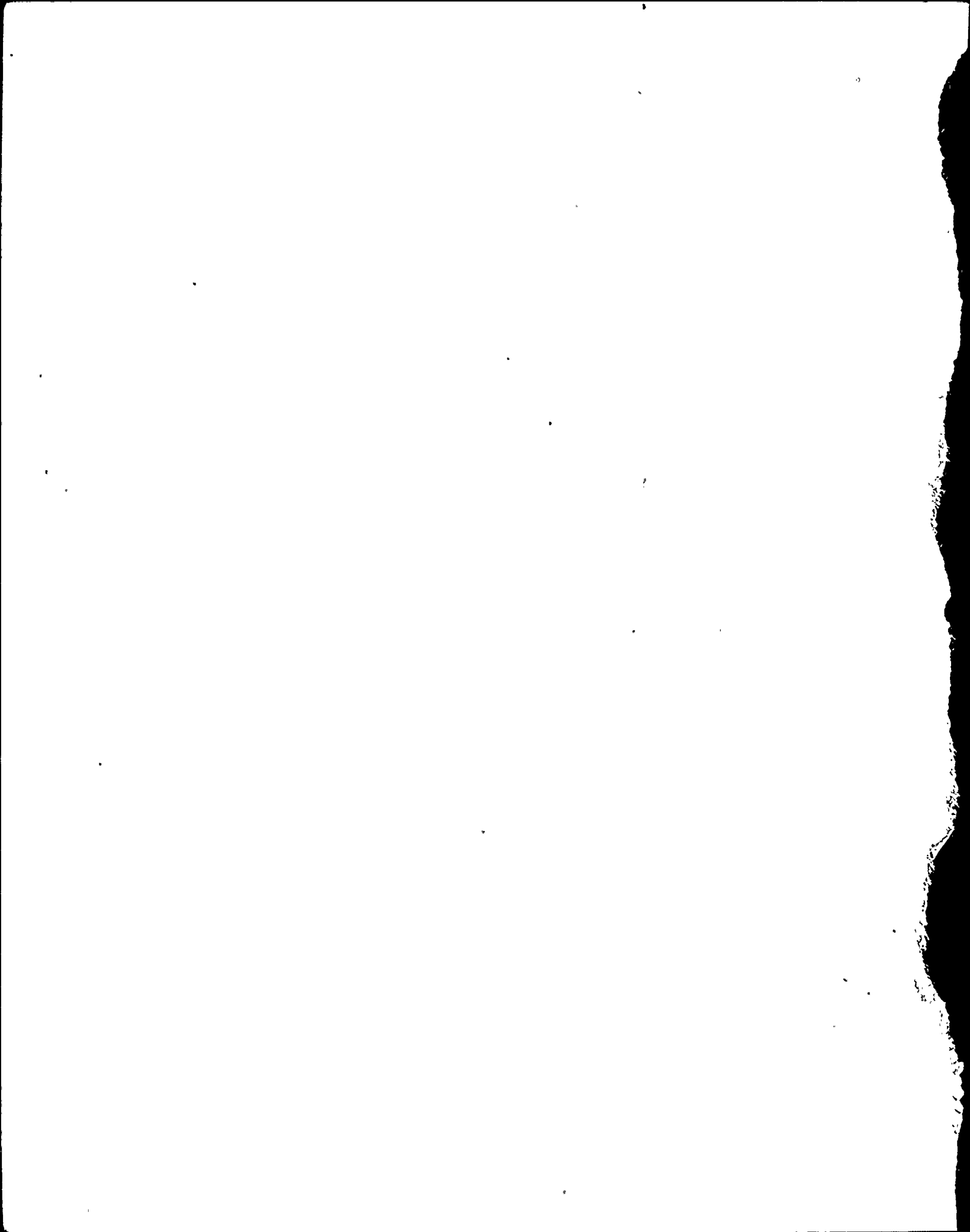
Safety Evaluations: This test does not involve any unreviewed safety question and does not require a change of Technical Specifications.

ST-75-6

Steam Generator Moisture Carry Over Measurement

Description: Determine the amount of moisture carry-over from the steam generators to feedwater using 24 Na tracers.

Safety Evaluation: This test does not involve any unreviewed safety question and does not require a change of Technical Specifications.



ANNUAL OPERATING REPORT #13

OF

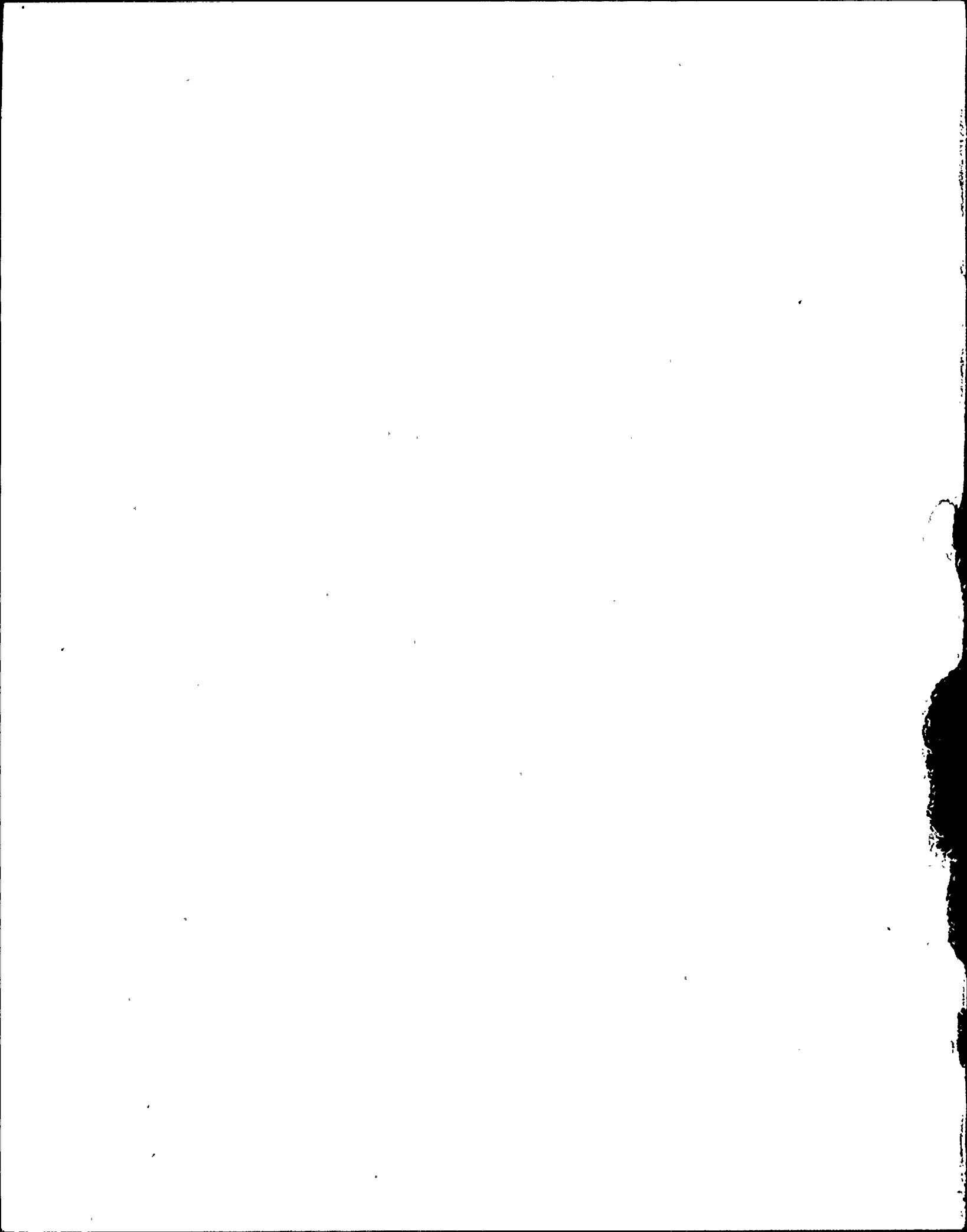
R. E. GINNA NUCLEAR STATION

ROCHESTER GAS & ELECTRIC CORPORATION

1977

DOCKET NO. 50-244

LICENSE NO. DPR-18

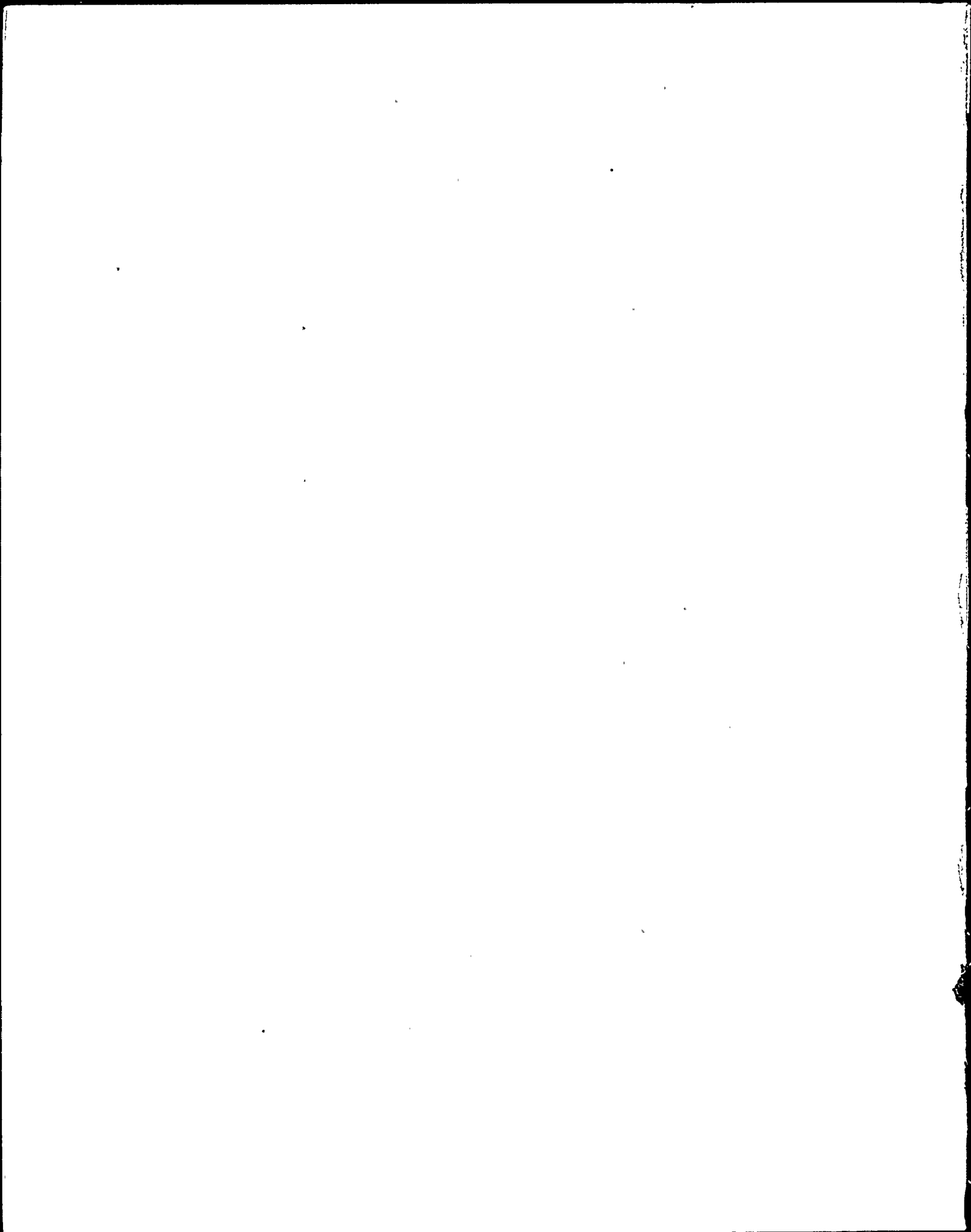


<u>TABLE OF CONTENTS</u>	<u>PAGE</u>
INTRODUCTION	1
HIGHLIGHTS	2
SUMMARY OF OPERATING EXPERIENCE	3
PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY-RELATED MAINTENANCE	4
Amendments to Facility License or Technical Specifications	5 - 23
Facility or Procedure Changes Requiring NRC Approval	24
Tests and Experiments Requiring NRC Approval	25
Other Changes, Tests, and Experiments	26 - 31
Corrective Maintenance of Safety Related Equipment	32
Mechanical and Electrical Maintenance	33 - 43
Instrumentation and Control Equipment Maintenance	44 - 49
LICENSEE EVENT REPORTS	50 - 72
OTHER EVENTS OF INTEREST	73
Fuel Performance	74
Fuel Data	75
Core Loading Pattern	76
Steam Generator Inspection(s)	77 - 78
Summary of Containment Integrated Leak Rate Tests	79
DATA TABULATIONS	80
Net Electrical Power Generation	81
Unit Shutdowns and Forced Power Reductions	82 - 83
Number of Personnel and Man/Rem Exposure by Work and Job Function	84 - 86
Plant Organization and Personnel	87
Operating Histogram	88
UNIQUE REPORTING REQUIREMENTS	89
Condenser Tube Maintenance	90
Primary, S/G Blowdown Chemistry	91
Leak Tests on Sources	92
Radioactive Effluents	93 - 94
Solid Waste	95 - 96
Environmental Monitoring	97 - 100
GLOSSARY	101

INTRODUCTION

R. E. Ginna Nuclear Station is a pressurized water reactor of 470 MW_e net maximum capacity, owned by Rochester Gas and Electric Corporation and is located in Ontario, New York. The nuclear steam supply system is a Westinghouse PWR. The Architect/Engineer was Gilbert Associates, and the constructor was Bechtel. The condenser cooling method is once-through, and Lake Ontario is the condenser cooling water source. The Plant is subject to license DPR-18, issued September 19, 1969 pursuant to Docket Number 50-244. The date of initial reactor criticality was November 9, 1969, and commercial generation of power began July 1, 1970.

This report was prepared by Frank W. Aman, 716/546-2700, Extension 291-237.



HIGHLIGHTS

The Robert Emmett Ginna Station operated at 100% power until January 11 of this report period when, due to a tube leak in the 1B2 condenser. Power was reduced to 46% while repairs were made. A turbine main steam stop valves test was also performed and the unit was returned to 100% power on January 12.

On January 25, power was reduced to 50% to effect repairs on a tube leak in the 1B1 condenser. The unit was returned to full power on January 26.

Power was reduced to 48% on February 13 to conduct the turbine steam stop valves test and returned to full power on February 19.

On March 21, power was reduced to 45% to effect repairs on a tube leak in the 1B1 condenser and to perform a turbine main steam stop valves test. The unit was returned to full power on March 26.

On April 15 the unit was placed in cold shutdown for annual overhaul and refueling. On May 22 the unit was taken critical but returned to hot shutdown on May 23 due to control rods K-7 and L-8 being inoperable. The unit commenced return to power on May 24 and reached full power on May 28.

The unit remained at full power until July 5 when it was taken to cold shutdown to effect repairs on a tube leak in the "B" steam generator. The unit was taken critical on July 13 and reached full power on July 14.

The unit was again taken to cold shutdown on August 4 to effect repairs on two inoperable snubbers on the "A" steam generator. The unit was returned to full power on August 8.

The unit was taken to hot shutdown on November 4 to effect repairs on an inoperable snubber on the "B" steam generator. The unit was returned to full power on November 6.

The unit was taken to hot shutdown to correct inoperable control rods on November 17 and returned to full power on November 19.

The unit was taken to hot shutdown on December 3 due to a leaking pipe tap on the residual heat removal system. The unit was returned to full power on December 4 and remained at full power through the balance of this report period.

SUMMARY OF OPERATING EXPERIENCE

The following is a chronological description of Plant Operations including other pertinent items of interest for the twelve month period ending December 31, 1977.

1/11	1-B-2 Condenser, Tube Leak - Also performed Turbine Main Steam Stop Valves Test.
1/25	1-B-1 Condenser, Tube Leak
2/13	Turbine Main Steam Stop Valves Test
3/21	1-B-1 Condenser, Tube Leak - Also performed Turbine Main Steam Stop Valves Test.
4/15-5/22	Annual refueling and maintenance.
5/24	Inoperable Control Rods K-7 and L-8
7/5-7/13	"B" Steam Generator - Tube Leak.
8/4-8/7	"A" Steam Generator - Two inoperable snubbers.
11/4-5	"B" Steam Generator - One inoperable snubber.
11/17	Inoperable Control Rods.
12/3	Leaking pipe tap - residual heat removal system.

PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND

SAFETY-RELATED MAINTENANCE

AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS

On April 7, 1977, The Nuclear Regulatory Commission issued Amendment #12 to R. E. Ginna Nuclear Power Plant Provisional Operating License No. DPR-18, revising Appendix A, paragraphs 3.1.2 and its basis Table 3.1.1, 3.1.2, 3.5.1, paragraph 4.3.1, Figures 6.2.1, 6.2.2, to read as follows:

3.1.2 Heatup and Cooldown Limit Curves for Normal Operation

3.1.2.1 The reactor coolant temperature and pressure and system heatup and cooldown rates (with the exception of the pressurizer) shall be limited in accordance with Figures 3.1-1 and 3.1-2 for the first 10.6 effective full power years.

Basis: Fracture Toughness Properties

Heatup and cooldown limit curves are calculated using the most limiting value of RT_{NDT} at the end of 10.6 effective full power years (EFPY). The 10.6 EFPY period is chosen such that the limiting RT_{NDT} at the 1/4T location in the core region is higher than the RT_{NDT} of the limiting unirradiated material. This service period assures that all components in the Reactor Coolant System will be operated conservatively in accordance with Code recommendations.

The highest RT_{NDT} of the core region material is determined by adding the radiation induced ΔRT_{NDT} for the applicable time period to the original RT_{NDT} shown in Reference (5). The fast neutron ($E > 1$ Mev) fluence at 1/4 thickness and 3/4 thickness vessel locations is given as a function of full power service life in Reference (5). Using the applicable fluence at the end of the 10.6 EFPY period for 1/4 thickness and the copper content of the material in question, the ΔRT_{NDT} is obtained from Reference (5). The ΔRT_{NDT} is more conservative than the value obtained from the second capsule of radiation surveillance program.

Heatup and Cooldown Limit Curves

Thus, the governing equation for the heatup-cooldown analysis is:

$$(1) \quad 2 K_{Im} + 1.25 K_{It} \leq K_{IR}$$

where: K_{Im} is the stress intensity factor caused by membrane (pressure) stress.

K_{It} is the stress intensity factor caused by the thermal gradients.

K_{IR} is provided by the Code as a function of temperature relative to the RT_{NDT} of the material.



Reference:

- (5) WCAP-8421, "Analysis of Capsule R from the Rochester Gas and Electric Corporation, R. E. Ginna Unit No. 1 Reactor Vessel Radiation Surveillance Program"

3.1-12 Amendment No. 12

If the specified shutdown margin is maintained, there is no possibility of an accidental criticality as a result of an increase in moderator temperature or a decrease of coolant pressure.

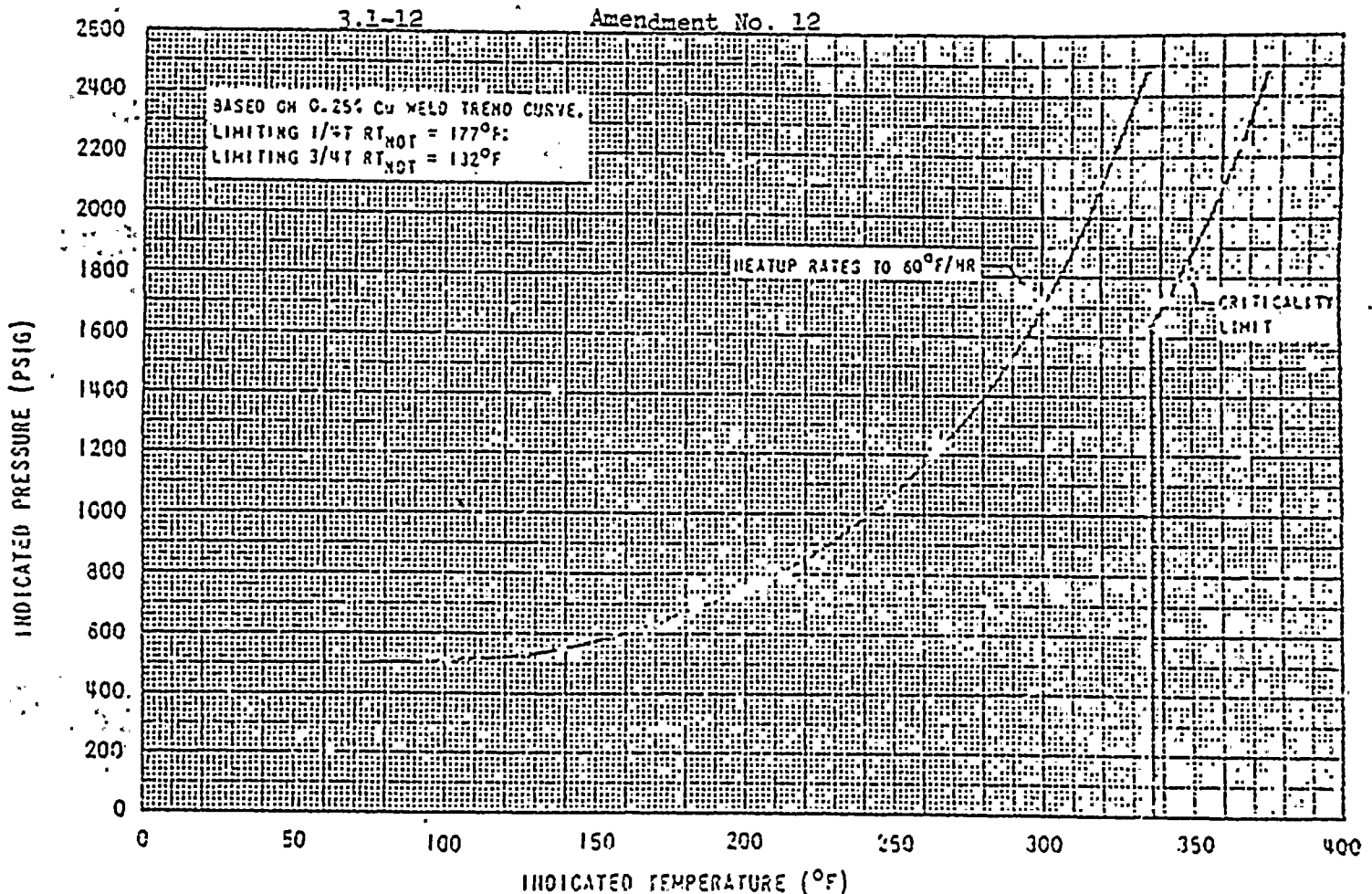
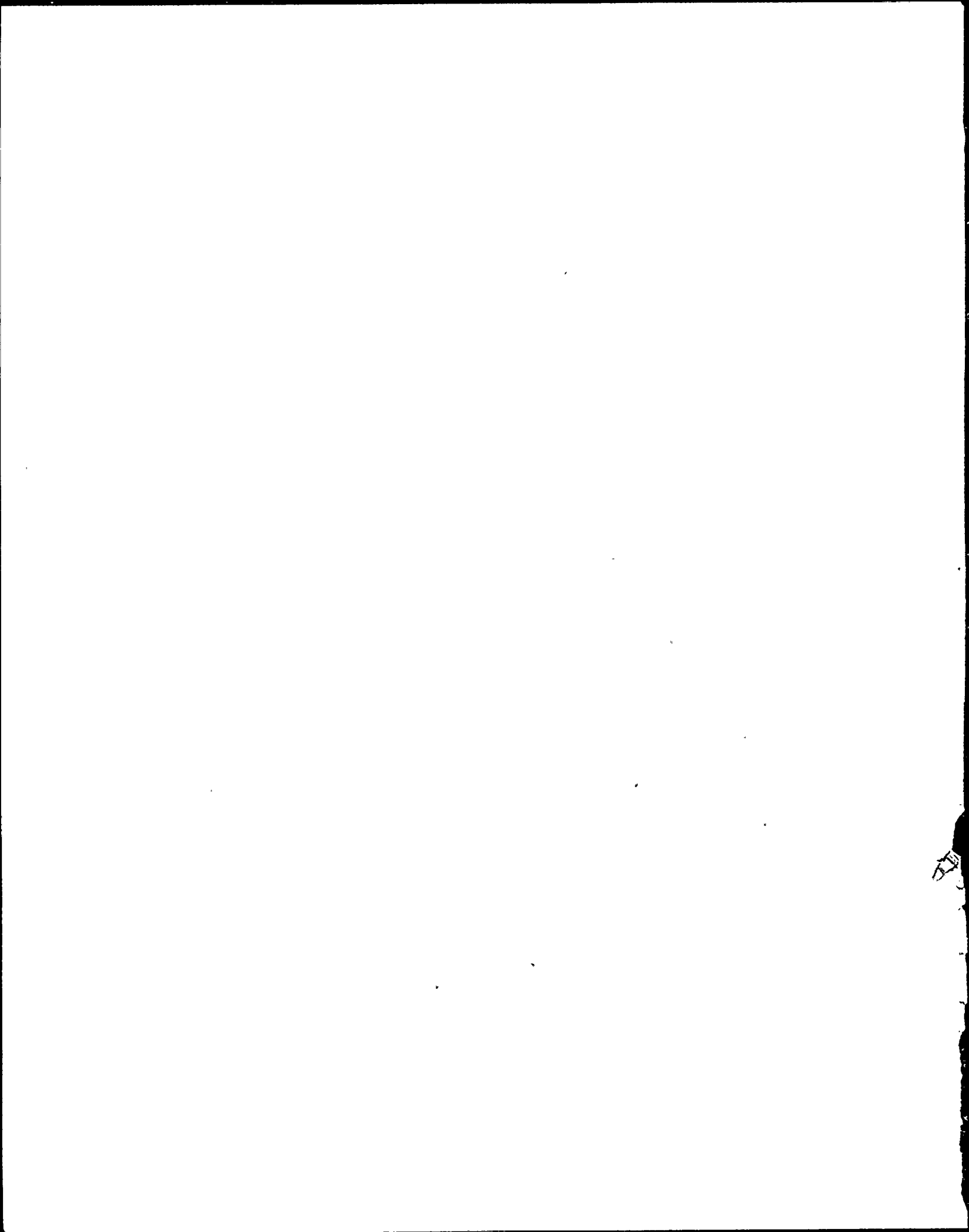


Figure 3.1-1 Reactor Coolant System Heatup Limitations Applicable for 10.6 Effective Full Power Years. $T_{ERROR} = 10^{\circ}F$, $P_{ERROR} = 60$ PSI



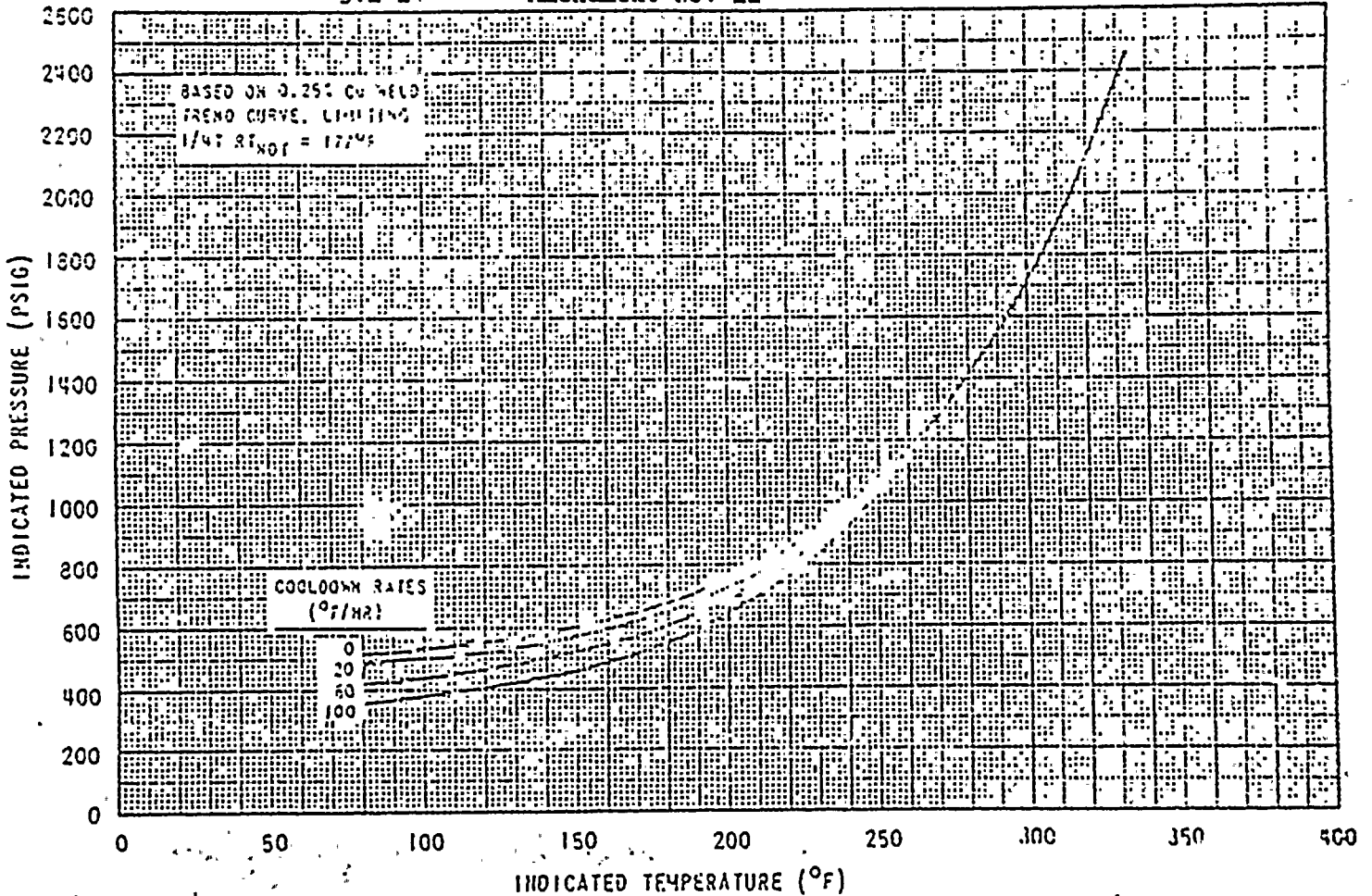


Figure 3.1-2 Reactor Coolant System Cutdown Limitations Applicable for 10.6 Effective Full Power Years. T_{ERROR} = 10°F, P_{ERROR} = 60 PSI

3.5-6
Amendment No. 12

16. Control rod misalignment monitors**

TABLE 3.5.1

a) Rod position deviation	1	-*	1 or Log individual rod positions once/hr, and after a load change of 10% or after 30 in. of control rod motion	-*	-**
b) Quadrant power-tic monitor (upper & lower ex-core neutron detectors)	1	-**	1 or Log individual upper & lower ion chamber currents once/hr & after a load change of 10% or after 30" of control rod motion	-*	-**

NOTE: 1: When block condition exists, maintain normal operation.

2: = Full Power

* Not Applicable

** If both rod misalignment monitors (a and b) are inoperable for 2 hours or more, the nuclear overpower trip shall be reset to 93% of rated power in addition to the increased surveillance noted.

*** If a functional unit is operating with the minimum operable channels, the number of channels to trip the reactor will be column 3 less column 4.

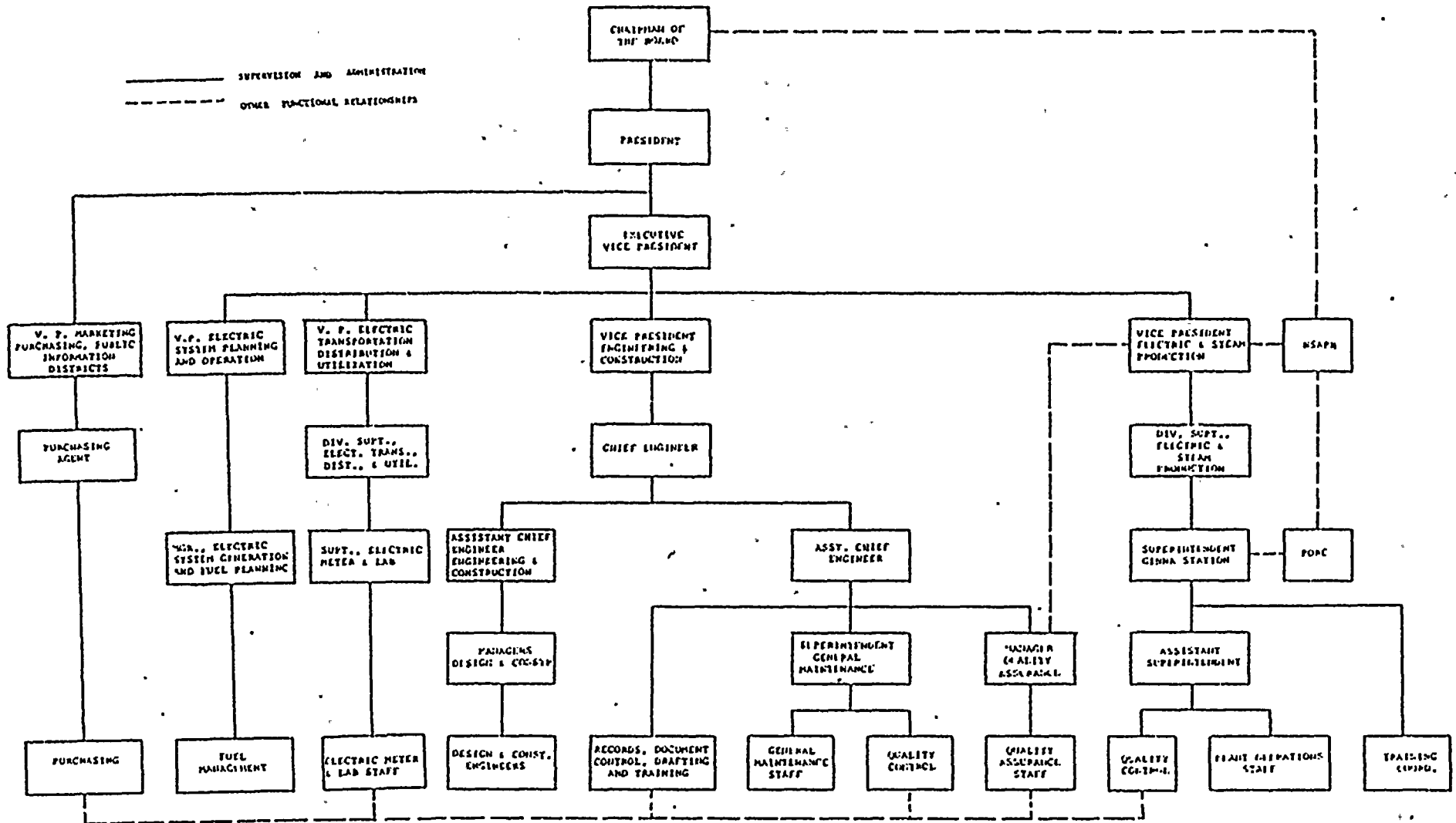
4.3.0 REACTOR VESSEL MATERIAL SURVEILLANCE PROGRAM

Specification:

4.3.1 The reactor vessel material surveillance testing program is designed to meet the requirements of Appendix H to 10 CFR Part 50. This program consists of the metallurgical specimens receiving the following test: tensile, charpy impact and the WOL test. These tests of the Radiation Capsule Specimens shall be performed as follows:

<u>CAPSULE</u>	<u>TIME TESTED</u>
V	End of 1st core cycle
R	End of 3rd core cycle
T	10 years, at nearest refueling
P	20 years, at nearest refueling
S	30 years, at nearest refueling
N	Standby

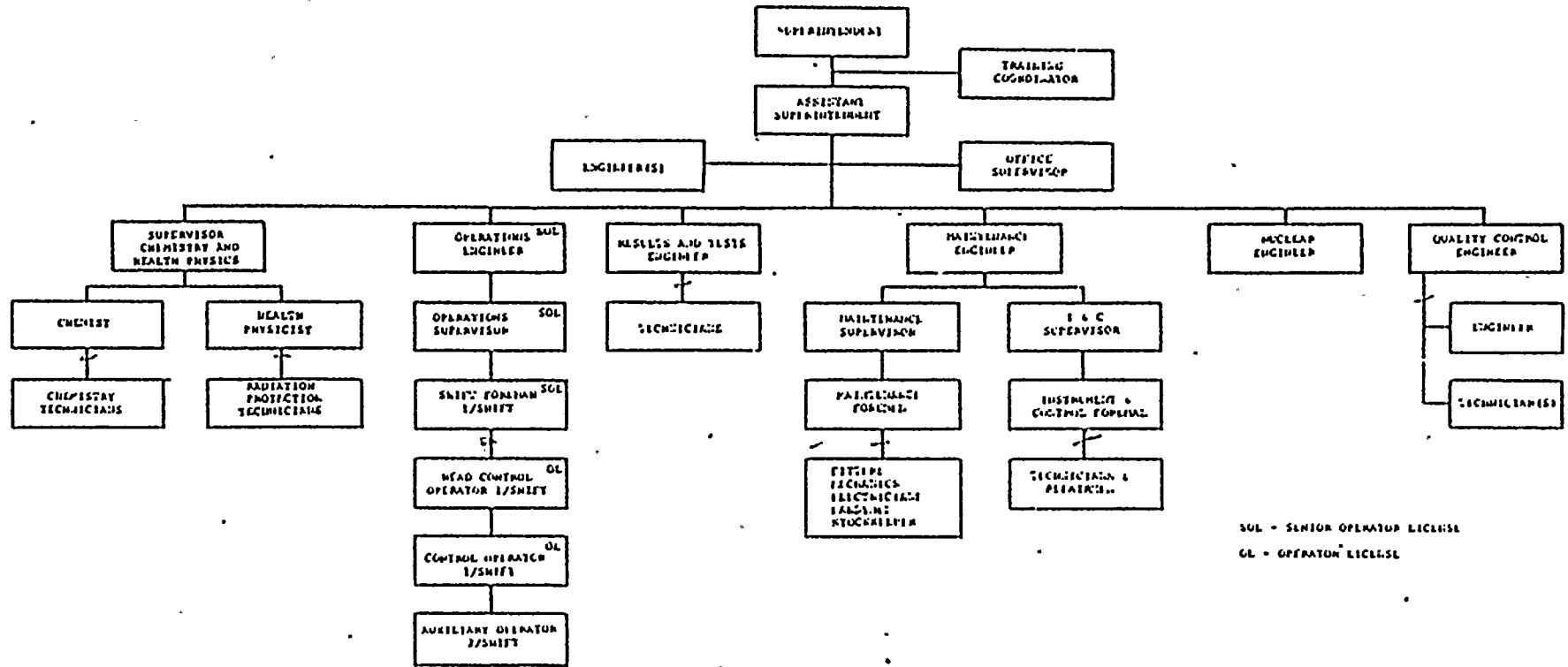
R.E.GINNA NUCLEAR POWER PLANT
MANAGEMENT ORGANIZATION CHART



6.2-2 Amendment No. 12

Figure 6.2-1

NUCLEAR LAW AND ELECTRIC COMPONENTS
CHINA STATION ORGANIZATION



SOL - SENIOR OPERATOR LICENSE
OL - OPERATOR LICENSE

Figure 6.2-2

6.2-3 Amendment No. 12

AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS

On May 17, 1977, the Nuclear Regulatory Commission issued Amendment #13 to The R. E. Ginna Nuclear Power Plant Provisional Operating License No. DPR-18, revising Appendix A, paragraph 4.2 to read as follows:

4.2 Inservice Inspection

Applicability

Applies to the inservice inspection of Quality Groups A, B and C Components, High Energy Piping Outside of Containment and Steam Generator Tubes.

Objectives

To provide assurance of the continuing structural and operational integrity of the structures, components and systems in accordance with the requirements of 10 CFR 50.55a(g).

Specification

- 4.2.1 The inservice inspection program for Quality Groups A, B and C Components, High Energy Piping Outside of Containment and Steam Generator tubes shall be in accordance with Appendix B of the Ginna Station Quality Assurance Manual. This inservice inspection program shall define the specific requirements of the edition and Addenda of the ASME Boiler and Pressure Vessel Code, Section XI, which are applicable for the forty month period of the ten year inspection interval. The program ten year inspection intervals shall be based on the following commencing dates.
- 4.2.1.1 The inspection interval for Quality Group A components shall be ten year intervals of service commencing on January 1, 1970.
- 4.2.1.2 The inspection intervals for Quality Group B and C Components shall be ten year intervals of service commencing with May 1, 1973.
- 4.2.1.3 The inspection intervals for the High Energy Piping Outside of Containment shall be ten year intervals of service commencing with May 1, 1973. The inspection program during each third of the first inspection interval provides for examination of all welds at design basis break locations and one-third of all welds at locations where a weld failure would result in unacceptable consequences. During each succeeding inspection interval, the program shall provide for an examination of each of the design basis break location welds, and each of the welds at locations where a weld failure would result in unacceptable consequences.

- 4.2.1.4 The inspection intervals for Steam Generator Tubes shall be specified in the "Inservice Inspection Program" for the applicable forty month period commencing with May 1, 1973.
- 4.2.1.5 Inservice Inspection of ASME Code Class 1, Class 2 and Class 3 components (Quality Groups A, B and C) shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a (g), except where specific written relief has been granted by the NRC pursuant to 10 CFR 50, Section 50.55a(g) (6) (i).

Basis:

The inservice inspection program provides assurance for the continued structural integrity of the structures, components and systems of Ginna Station. The program complies with the ASME Boiler and Pressure Vessel Code Section XI "Rules for Inservice Inspection of Nuclear Power Plant Components" as practicable, with due consideration to the design and physical access of the structures, components and systems as manufactured and constructed. This compliance will constitute an acceptable basis for satisfying the requirements of General Design Criterion 32, Appendix A of 10 CFR Part 50 and the requirements of Section 50.55a, paragraph g of 10 CFR Part 50.

On June 1, 1977, the Nuclear Regulatory Commission issued Amendment #14 to the R. E. Ginna Nuclear Power Plant Provisional Operating License No. DPR-18, revising Appendix A, Table 3.5-1, page 3, paragraph 3.13, Table 3.13-1 (new), Table 4.1.2 Paragraph 4.6.1, Paragraph 4.13, Paragraph 4.14 (new), Table 4.14-1 (new) to read as follows: - ..

3.13 Shock Suppressors (Snubbers)

Applicability:

Applies to the operability of all safety-related shock suppressors (snubbers) listed in Table 3.13-1.

Objective:

To specify the requirements for operability of shock suppressors (snubbers).

Specification:

- 3.13.1 The reactor shall not be made critical unless (1) all shock suppressors (snubbers) listed in Table 3.13-1 are operable.

3.13.2 Continued hot shutdown or power operation is permitted for a period up to 72 hours without the conditions of 3.13.1 being met. If the conditions of 3.13.1 are not met in the 72 hour period, then the reactor shall be in a cold shutdown condition within the next 36 hours.

Basis:

Shock suppressors (snubbers) are required to be operable to ensure that the structural integrity of the reactor coolant system and all other safety-related systems is maintained during and following a seismic or other event initiating dynamic loads.

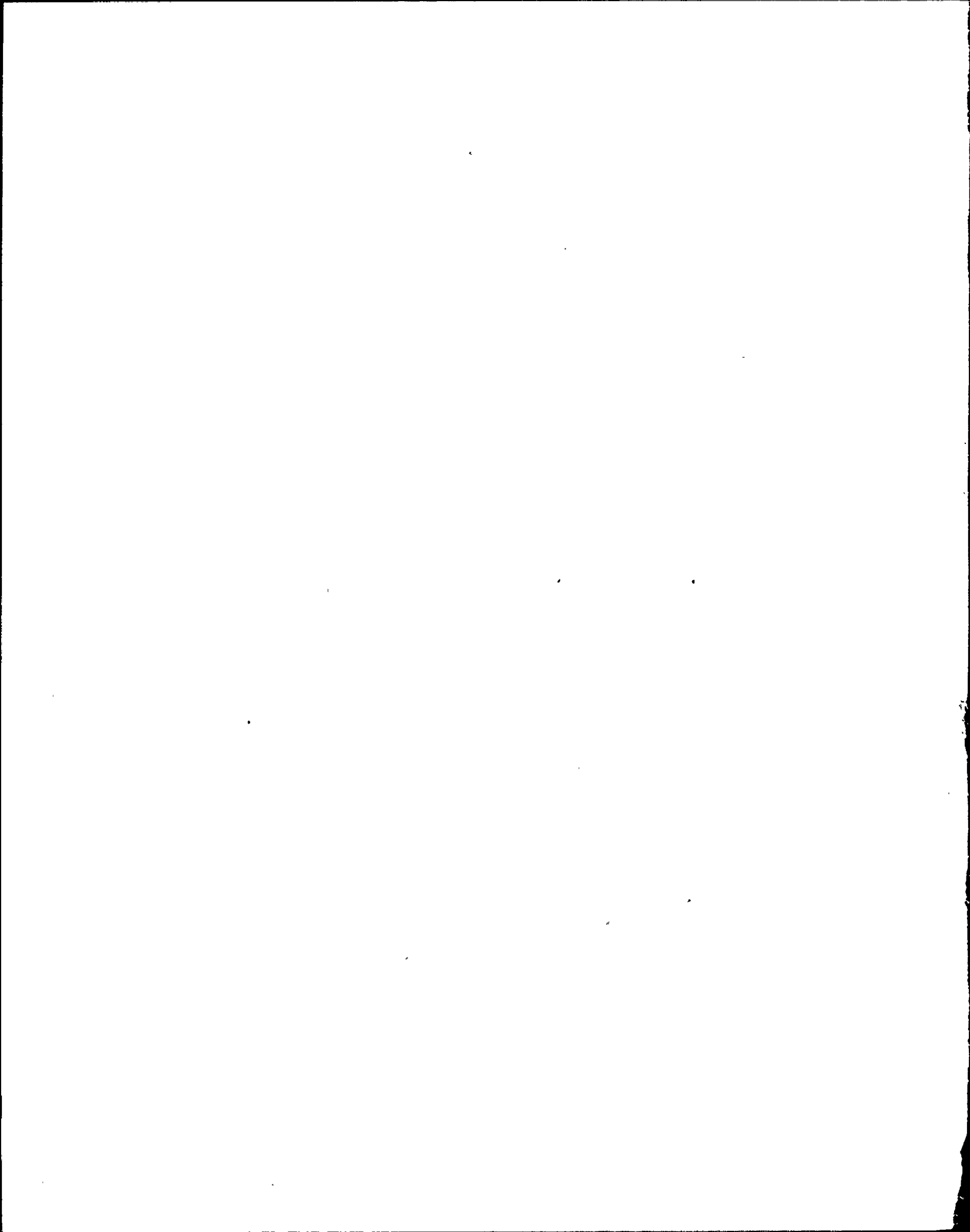


TABLE 3.5-1

FUNCTIONAL UNIT	1 NO. of CHANNELS	2 NO. of CHANNELS TO TRIP	3 MIN. OPERABLE CHANNELS	4 MIN. DEGREE OF REDUNDANCY	5 PERMISSIBLE BYPASS CONDITIONS	6 OPERATOR ACTION IF CONDITIONS OF COLUMN 3 or 5 CANNOT BE MET
17. Circulating Water Flood Protection						
a. Screenhouse	2	1	2 ⁺	*		Power operation may be continued for a period of up to 7 days with 1 channel inoperable or for a period of 24 hrs. with two channels inoperable.
b. Condenser	2	1	2 ⁺	*		Power operation may be continued for a period of up to 7 days with 1 channel inoperable or for a period of 24 hrs. with two channels inoperable.

NOTE 1: When block condition exists, maintain normal operation.

F.P. = Full Power

* Not Applicable

** If both rod misalignment monitors (a and b) are inoperable for 2 hours or more, the nuclear overpower trip shall be reset to 93% of rated power in addition to the increased surveillance noted.

*** If a functional unit is operating with the minimum operable channels, the number of channels to trip the reactor will be column 3 less column 4.

+ A channel is considered operable with 1 out of 2 logic or 2 out of 3 logic.

Table 3.13-1

Safety Related Shock Suppressors (Snubbers)

<u>Snubber Mark No.</u>	<u>Piping System</u>	<u>Accessible or Inaccessible (A or I)</u>	<u>High Radiation** Zone</u>	<u>Especially Difficult to Remove</u>
SGA-1	"A" Steam Generator	I	Yes	Yes
SGA-2	"A" Steam Generator	I	Yes	Yes
SGA-3	"A" Steam Generator	I	Yes	Yes
SGA-4	"A" Steam Generator	I	Yes	Yes
SGA-5	"A" Steam Generator	I	Yes	Yes
SGA-6	"A" Steam Generator	I	Yes	Yes
SGA-7	"A" Steam Generator	I	Yes	Yes
SGA-8	"A" Steam Generator	I	Yes	Yes
SGB-1	"B" Steam Generator	I	Yes	Yes
SGB-2	"B" Steam Generator	I	Yes	Yes

3.13-2 Amendment No. 14

Table 3.13-1 (Continued)

Safety Related Shock Suppressors (Snubbers)

<u>Snubber Mark No.</u>	<u>Piping System</u>	<u>Accessible or Inaccessible (A or I)</u>	<u>High Radiation** Zone</u>	<u>Especially Difficult to Remove</u>
SGB-3	"B" Steam Generator	I	Yes	Yes
SGB-4	"B" Steam Generator	I	Yes	Yes
SGB-5	"B" Steam Generator	I	Yes	Yes
SGB-6	"B" Steam Generator	I	Yes	Yes
SGB-7	"B" Steam Generator	I	Yes	Yes
SGB-8	"B" Steam Generator	I	Yes	Yes
FW-3	"A" Feedwater	A	No	No
FW-5	"A" Feedwater	I	No	Yes
FW-25	"A" Feedwater	A	No	No

-91-

Table 3.13-1 (Continued)

Safety Related Shock Suppressors (Snubbers)

<u>Snubber Mark No.</u>	<u>Piping System</u>	<u>Accessible or Inaccessible (A or I)</u>	<u>High Radiation** Zone</u>	<u>Especially Difficult to Remove</u>
AFW-25	"A" Aux. Feedwater	A	No	No
AFW-26	"A" Aux. Feedwater	A	No	No
AFW-27	"A" Aux. Feedwater	A	No	No
AFW-28	"A" Aux. Feedwater	A	No	No
AFW-29	"A" Aux. Feedwater	A	No	No
AFW-49	"A" Aux. Feedwater	A	No	No
FW-9	"B" Feedwater	I	No	Yes
FW-30	"B" Feedwater	A	No	No
FW-80	"B" Feedwater	A	No	No
FW-81	"B" Feedwater	A	No	No
FW-82	"B" Feedwater	A	No	No

Table 3.13-1 (Continued)

Safety Related Shock Suppressors (Snubbers)

<u>Snubber Mark No.</u>	<u>Piping System</u>	<u>Accessible or Inaccessible (A or I)</u>	<u>High Radiation** Zone</u>	<u>Especially Difficult to Remove</u>
FW-83	"B" Feedwater	A	No	No
AFW-10	"B" Aux. Feedwater	A	No	No
AFW-13	"B" Aux. Feedwater	A	No	No
MS-2	"A" Main Steam	I	No	Yes
MS-3	"A" Main Steam	I	No	Yes
MS-22	"A" Main Steam	A	No	No
MS-7	"B" Main Steam	I	No	Yes
MS-8	"B" Main Steam	I	No	Yes
MS-146 (Top)	"B" Main Steam	A	No	No
MS-146 (Bottom)	"B" Main Steam	A	No	No

3.13-5 Amendment No. 14

Table 3.13-1 (Continued)

Safety Related Shock Suppressors (Snubbers)

<u>Snubber Mark No.</u>	<u>Piping System</u>	<u>Accessible or Inaccessible (A or I)</u>	<u>High Radiation** Zone</u>	<u>Especially Difficult to Remove</u>
MS-147	"B" Main Steam	A	No	No
MS-148	"B" Main Steam	A	No	No
MS-159	"B" Main Steam	A	No	No
MS-160	"B" Main Steam	A	No	No
H-1	PRZR Relief	I	Yes	No
H-2	PRZR Relief	I	Yes	No
H-3	PRZR Relief	I	Yes	No
H-4	PRZR Relief	I	Yes	No
H-5	PRZR Relief	I	Yes	No
H-6	PRZR Relief	I	Yes	No
H-7	PRZR Relief	I	Yes	No
H-8	PRZR Relief	I	Yes	No

* Snubbers may be added to safety related systems without prior License Amendment to Table 3.13-1 provided that a proposed revision to Table 3.13-1 is included with the next License Amendment request.

** Modifications to this table due to changes in high radiation areas (during shutdown) shall be submitted to the NRC as part of the next License Amendment request.

3:13-6 Amendment 1a

TABLE 4.1-2

MINIMUM FREQUENCIES FOR EQUIPMENT AND SAMPLING TESTS

	<u>TEST</u>	<u>FREQUENCY</u>	<u>FSAR SECTION REFERENCE</u>
19.	Circulating Water Flood Protection Equipment	Calibrate Each Refueling Shutdown	
4.6.1	<u>Diesel Generators</u>		
	b. Automatic start of each diesel generator and automatic restoration of particular vital equipment, initiated by an actual loss of all normal AC station service power supplies together with a simulated safety injection signal. This test shall be conducted during each refueling shutdown to assure that the diesel-generator will start and following maximum breaker closure times after the initial starting signal for trains A and B will not be exceeded.		
			A B
	Diesel plus Safety Injection		20 sec. 22 sec.
	Pump plus RHR Pump		
	All breakers closed		40 sec. 42 sec.

4.14 Shock Suppressors (Snubbers)

Applicability:

Applies to the operability of all shock suppressors (snubbers) listed in Table 3.13-1.

Objective:

To specify the requirements for inspection and functional testing or shock suppressors (snubbers).

Specifications:

Visual Inspections:

4.14.1 Each hydraulic shock suppressor (snubber) with seal material fabricated from ethylene propylene or other materials demonstrated compatible with the operating environment, shall be determined operable by visual inspection in accordance with the inspection schedule in Table 4.14-1. Initiation of the Table 4.14-1 inspection schedule shall be made assuming the unit was previously at the 12 month inspection interval. The initial inspection shall be performed within 12 months of the effective date of this specification.

- 4.14.2 Each hydraulic shock suppressor (snubber) with seal material not fabricated from ethylene propylene or other material demonstrated compatible with the operating environment shall be determined operable at least once every 31 days by a visual inspection of the snubber.
- 4.14.3 The inspection of the hydraulic shock suppressors (snubbers) in 4.14.1 and 4.14.2 shall be a visual inspection and shall include inspection of the hydraulic fluid reservoir, fluid connections, and linkage connections to the piping or steam generators and anchors.
- 4.14.4 Shock suppressors (snubbers) in 4.14.1 may be categorized into two groups, "accessible" and "inaccessible". This categorization shall be based upon accessibility for inspection during reactor operation. These two groups may be inspected independently, but in accordance with the applicable schedule.
- 4.14.5 Inspections performed before that interval has elapsed may be used as a new reference point to determine the next inspection. However, the results of such early inspections performed before the original required time interval has elapsed (nominal time less 25%) may not be used to lengthen the required inspection interval. Any inspection whose results require a shorter inspection interval will override the previous schedule.
- 4.14.6 A hydraulic shock suppressor (snubber) is considered to be operable if the fluid reservoir level and the installation are in accordance with the manufacturer's instructions.

Functional Testing:

- 4.14.7 During each refueling outage, but at least once during each 18 month interval a representative sample of at least 10 hydraulic shock suppressors (snubbers) or 10% whichever is less, of all shock suppressors (snubbers) listed in Table 3.13-1, shall be selected and functionally tested. These hydraulic shock suppressors (snubbers) shall be tested in accordance with the original manufacturer's test requirements. Hydraulic shock suppressors (snubbers) greater than 50,000 lbs. capacity may be excluded from functional testing requirements. Hydraulic shock suppressors (snubbers) selected for functional testing shall be selected

on a rotating basis. Hydraulic shock suppressors (snubbers) identified as either being "especially difficult to remove" or in "high radiation zones" may be exempted from functional testing provided they were demonstrated operable during previous functional tests. Hydraulic shock suppressors (snubbers) found inoperable during functional testing shall be restored to operable status prior to being returned to service. For each hydraulic shock suppressor (snubber) found inoperable during these functional tests, an additional minimum of 10% of all hydraulic shock suppressors (snubbers) or 10, whichever is less, shall also be functionally tested until no more failures are found or all hydraulic shock suppressors (snubbers) have been functionally tested.

Basis:

The hydraulic shock suppressors (snubbers) are required to be operable to ensure that the structural integrity of the Reactor Coolant System and all other safety systems is maintained during and following a seismic or other event initiating dynamic loads. The only hydraulic shock suppressors (snubbers) excluded from this inspection program are those installed on non-safety related systems and then only if their failure or failure of the system on which they are installed would have no adverse effect on any safety related system.

The inspection frequency applicable to hydraulic shock suppressors (snubbers) containing seals fabricated from materials which have been demonstrated compatible with their operating environment is based upon maintaining a constant level of protection. Therefore, the required inspection interval varies inversely with the observed failures. The number of inoperable hydraulic shock suppressors (snubbers) found during an inspection determines the time interval for the next required inspection. To provide further assurance of hydraulic shock suppressor (snubber) reliability, a representative sample of the installed hydraulic shock suppressors (snubbers) will be functionally tested during each refueling shutdown, but at least every 18 months. Observed failures of these samples will require functional testing of additional units. To minimize personnel exposures, hydraulic shock suppressors (snubbers) installed in high radiation zones or in especially difficult to remove locations may be exempted from functional testing requirements provided the operability of hydraulic shock suppressors (snubbers) was demonstrated during functional testing at either the completion of their fabrication or at a subsequent date.

Table 4.14-1

Shock Suppressor (Snubber) Inspection Schedule

<u>Number of shock suppressors (snubbers) found inoperable during inspection or during inspection interval</u>	<u>Next required inspection interval</u>
0	18 months \pm 25%
1	12 months \pm 25%
2	6 months \pm 25%
3 or 4	124 days \pm 25%
5, 6, or 7	62 days \pm 25%
\geq 8	31 days \pm 25%

NOTE:

The above required inspection interval shall not be lengthened more than one step at a time.

FACILITY OR PROCEDURE CHANGES REQUIRING NRC APPROVAL

There were no procedural changes during this reporting period that required prior approval by The Nuclear Regulatory Commission.

TESTS AND EXPERIMENTS REQUIRING NRC APPROVAL

There were no tests or experiments during the reporting period which required prior approval by the Nuclear Regulatory Commission.

OTHER CHANGES, TESTS AND EXPERIMENTS

System Modification SM 75-22

System Modification SM 75-22, Inflatable Seal Ring. Presently a flat stainless steel plate is used to seal off area between the reactor vessel and the reactor cavity during refueling. Due to leakage from this seal, preparations were made so an inflatable ring could be used to provide a better seal.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 75-39

System Modification SM 75-39, Modification of Main Steam Isolation Valves. This modification changed the disc and disc stop of the valves to allow the disc to be fully horizontal when in the open position. This modification also included installation of a larger actuator system. This should help prevent spurious valve closures.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 75-41

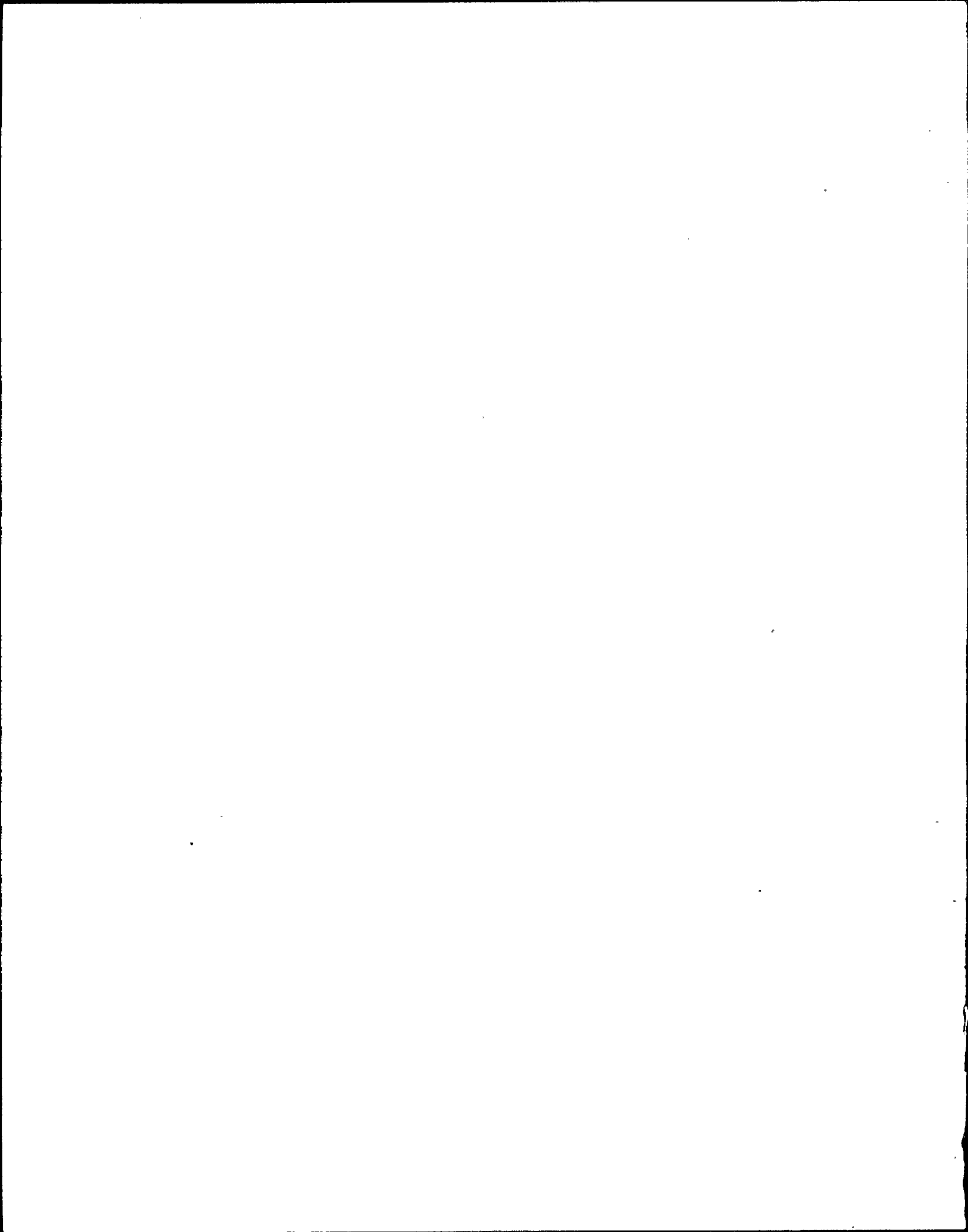
System Modification SM 75-41, Full Flow Condensate Demineralizers. This modification included the installation of demineralizers, condensate booster pumps, and associated equipment to allow 100% condensate processing to improve secondary side water purity.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 75-45

System Modification SM 75-45, Waste Evaporator Feed Pump Control System Modification. This modification included installing a low level shutoff of the waste evaporator feedtank for the recirculation pump. This provided assurance that the pump would stop during drumming of the waste when the tank was nearly empty. Presently there is a no level indication in the area the operator is stationed during drumming.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.



System Modification SM 76-13

System Modification SM 76-13, Sprinkler Head Relocation. This modification involved relocation of the sprinkler head in the stockroom and various shop offices to provide better fire protection for these areas.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 76-24

System Modification SM 76-24, Air Ejector Modification. This modification involves replacing the in-line monitor, that has been subject to failure due to moisture, with one that is external to the piping. All equipment for this radiation monitor including the readout module were replaced.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 76-29

System Modification SM 76-29, Main Feedwater Pump Recirculation. This modification involved installation of piping and valves to increase the recirculation capacity of these pumps from 8% to 25%. This is to be used during startup and shutdown of the pumps.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 76-30

System Modification SM 76-30, Grinnell Protomatic Valves. This modification involved replacement of the sleeve valves with piston type valves in the fire system. The piston valve is a vendor recommendation to improve reliability of the system.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 77-01

System Modification SM 77-01, Auxiliary Boiler Conversion. This modification involves temporary installation of a fuel supply tank and necessary piping to allow the house heating boiler to burn oil due to a shortage of natural gas.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 75-57

System Modification SM 75-57, Spent Fuel Pool Rack. This modification involved removing the existing racks and installation of a higher density rack to increase spent fuel storage capacity to 595 assemblies.

A change in the Plant Operating Technical Specifications was necessary and was approved by Amendment #11 to License DPR-18.

System Modification SM 75-58

System Modification SM 75-58, Spent Fuel Pool Cooling. This modification involved relocation of the spent fuel pool heat exchanger suction and discharge piping to insure better cooling of the spent fuel assemblies in the new high density racks.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 76-05

System Modification SM 76-05, Westinghouse Monitor Rack. This modification involved installation of a seismically restrained steel rack to house vibration monitoring equipment in the Control Room.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 76-08

System Modification SM 76-08, Sump Tank Pump Discharge Piping. This modification involved installation of piping and valves to provide an alternate discharge from the existing sump tank pumps. This new discharge piping is designed to provide a means for pumping the contents of the sump tank to temporary waste water processing equipment.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 76-11

System Modification SM 76-11, Spare #12 Transformer. This modification included the installation of a transformer. In the event of a failure to #12 transformer, the electrical connections can be made to allow use of the spare.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 77-03

System Modification SM 77-03, Boric Acid Piping Modification. Due to a leak found in this system, it was decided to replace certain areas of Schedule 10 piping with Schedule 40 and reduce areas of high stress. Also, a portion of the system was rerouted to a more accessible area to allow closer surveillance of the piping..

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 77-04

System Modification SM 77-04, Manipulator Crane Interlock on Hoist-Up Circuit. This modification involved installation of an interlock on the hoist-up circuit so this contact cannot be made unless the gripper is in the engaged position or the weight indicator reads less than the weight of a fuel assembly.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 77-05

System Modification SM 77-05, Personnel Hatch Packing Nut Keepers. This modification involves addition of restrainers on the teflon packing shaft seal nuts which are on the handwheel shafts of the containment personnel airlocks. This will prevent the packing gland nut from rotating with the handwheel shaft, thus preventing the nuts from loosening and allowing leakage.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 77-07

System Modification SM 77-07, Hydrogen Storage Room Modification. This modification involves installation of explosion bolt releases on the west wall of the H₂ and Co₂ bottle house. This will provide means for instant pressure relief from an explosion in the room.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 77-08

System Modification SM 77-08, Full Length Control Rod Sampling Resistor Modification. This modification concerns the 19 unused stationary coil current regulation sampling resistors in Power Cabinets 1AC, 2AC, 1BD, and 2BD of the Full Length Rod Control System. Fault current in any one of these unused sampling resistors could cause the inadvertent drop of two to three rods. Westinghouse Nuclear Service Division has therefore suggested that the feedback wire on the lower end of each unused resistor be removed. The wires are to be connected to the neutral bus, which is the neutral of the 4 wire, 260 VAC supply from the rod drive M. G. sets, and is common to the upper end of all sampling resistors.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 77-09

System Modification SM 77-09, Relief Valve Diaphragm Modification. This modification involves changing the diaphragm material in the operators of valves PCV-430, 431A, 431B, and 431C to a more suitable material for the environment.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 77-11

System Modification SM 77-11, Divert Line Relocation. This modification involves relocating a section of the letdown divert line to allow easier maintenance and repair on the piping.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 77-12

System Modification SM 77-12, Incore Thermocouple. This modification involved relocating three thermocouples from the discharge of the fuel assembly area to the top of the reactor head. This was performed to provide increased data on the reactor vessel upper head fluid temperature.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

System Modification SM 77-1621

System Modification SM 77-1621, Computer to Control Room Recorder. This modification involved installation of cable and recorder so the Control Room could have a recording of any parameter addressable on the computer.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant operating Technical Specifications is not necessary.

CORRECTIVE MAINTENANCE OF SAFETY RELATED EQUIPMENT

MECHANICAL AND ELECTRICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
1A Main Steam Isolation Valve	Corrective	N/A	Gasket Leak	Steam Leak at Cover Joint	Sealed using Furmanite Process EM-152
1A Charging Pump	Corrective	N/A	-	Leakage increase at east gland leak off	Repacked east gland, M-11.4.1 and M-11.4.6
1C Safety Injection Pump Bus 14 Breaker	Corrective	LER-77-01	Weak spring in secondary contact	Pump failed to start on first attempt during monthly surveillance test	Replaced #2 secondary contact. M-32.1.
1A Main Steam Isolation Valve 3517	Corrective	N/A	-	Gasket leakage	Reinjected gasket area with "Furmanite". Used EM-152.
1A Waste Gas Compressor	Corrective	N/A	-	Unit stalled out at 80 psig	Checked out seal water piping for blockage.
Fire System Valve 5171	Corrective	N/A	-	Cast yoke cracked	Valve bonnet yoke was replaced. Procedure M-64.1 was used.
NIS Channel N-41 power range	Corrective MR 77-381	N/A	N-41 drawer out of calibration	Redundant measurement N-41 reads high.	Recalibrated NIS Channel 41.

MECHANICAL AND ELECTRICAL MAINTENANCE

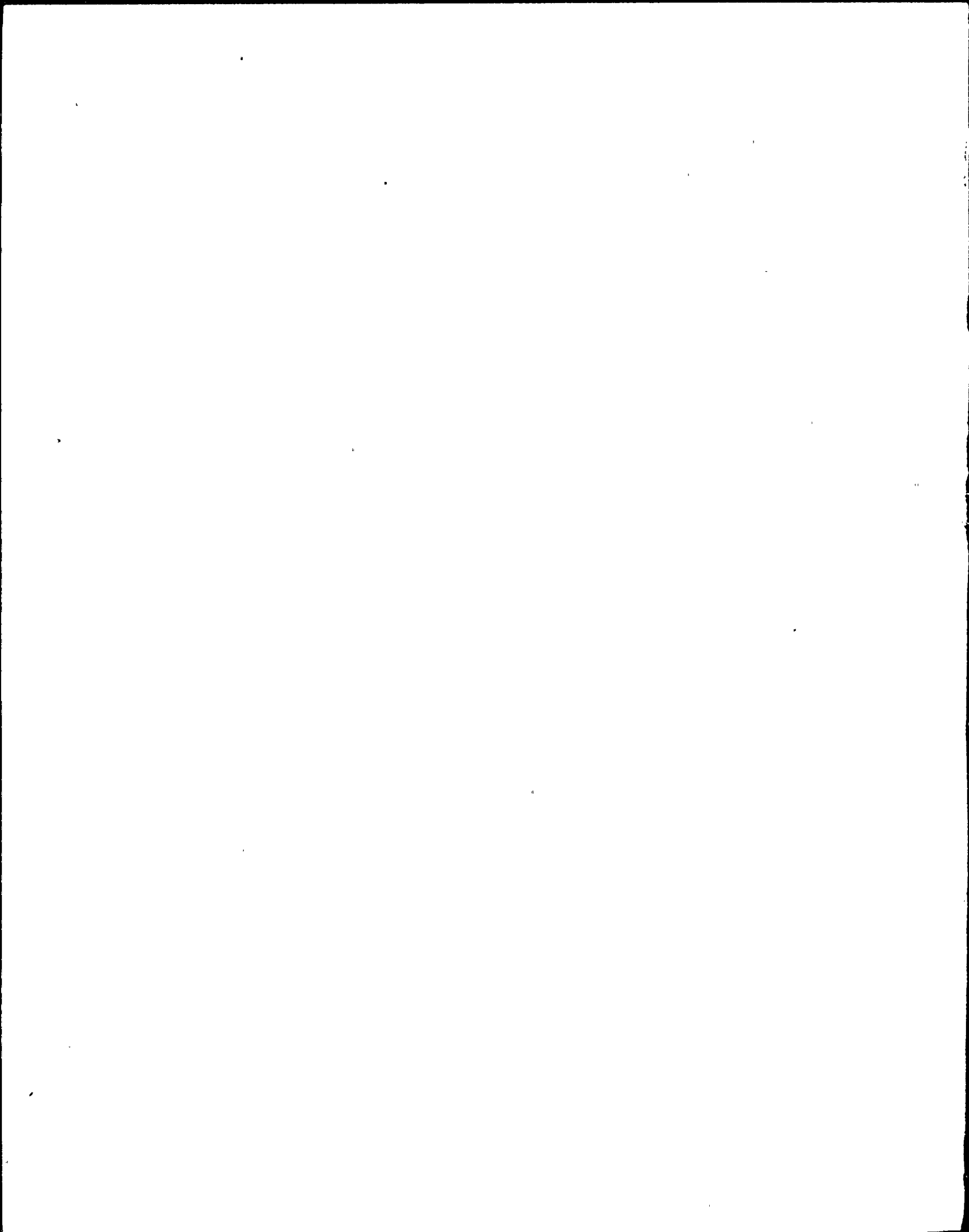
EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
RMS-R-17-Component Cooling Water Monitor	Corrective MR 77-393	N/A	Alarm setpoint drift	Reading high with no alarm	Adjusted alarm level set to proper value.
1D Gas Decay Tank Rupture Disc.	Corrective	-	-	Slow leak from tank back to vent header	Replaced rupture disc. Isolated per Procedure S-4.2.13.
1A Charging Pump	Corrective TC-77-616	N/A	Packing wear	Leakage at packing gland	Replaced charging pump stuffing box assembly using M-11.4.1 and M-11.4.6.
Acoustic Leak Detection System	Corrective	N/A	Calibration Drift	"Caution" would not clear on Point #5	Performed PT 19 (calibration of system).
Valve HCV-133 (Residual Heat Removal HX to Demineralizer)	Corrective MT 77-622	N/A	Transducer drift	Valve leaking thru	Calibrated and adjusted current to air transducer.
Pressurizer Power Operated Relief Valve 430	Corrective	N/A	Seat was cut	Leaking through	Inspected and installed new internals and cleaned up gasket seating areas.

MECHANICAL AND ELECTRICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
"A" Charging Pump	Corrective	N/A	Stuffing box needed repacking	Leakage	Repacked Stuffing Box.
"B" Boric Acid Tank Sample Valve 330B	Corrective	N/A	Boron Buildup	Plugged Valve	Cleaned valve and replaced diaphragm.
"A" Boric Acid Tank Sample Valve 330A	Corrective	N/A	N/A	Leaky bonnet	Replaced diaphragm and compressor.
Steam Generator Tubes	Preventive Eddy Current Inspection	LER 77-03	See steam generator report	See steam generator report	Approximately 2200 tubes were checked in each steam generator. Sludge lancing performed in both generators. Sludge profile determined in each generator. 13 tubes in "A" generator and 1 tube in "B" were plugged.
"A" Diesel Generator Solenoid Valve 5907	Corrective	N/A	N/A	Leaking through	Rebuilt valve.
"B" Steam Generator Blow-down Isolation Valve 5737	Corrective	N/A	Seat Cut	Excessive leakage past valve	Inspected and lapped valve.

MECHANICAL AND ELECTRICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
Main Steam Power Operated Relief Valve 3410	Corrective	N/A	Seat Cut	Leaking through	Inspected and lapped valve.
"B" Loop Sample Valve 955	Corrective	N/A	Bad Gasket	Valve leaking	Replaced gasket.
Letdown orifice valves 200B and 202	Corrective	N/A	Bad Gasket	Valves leaking	Replaced gasket.
Chemical Volume Control System Letdown Vent Valve 2234.	Corrective	N/A	N/A	Damaged Valve during ISI Hydro.	Replaced Valve.
"C" Charging Pump (Varidrive)	Corrective	N/A	Belt Wear	Belt Failure	Replaced belts.
Control Rod Drive Cables (K-7, L-8 inoperable)	Preventive	LER 77-04	Deterioration of cables under adverse conditions	Inoperable control rods	28 cables replaced. During physics testing, two were found switched. Made proper connections.
"A" & "B" Containment Spray Pumps	Preventive	N/A	N/A	N/A	Motor inspected - new bearings, Varnished "A" motor.



MECHANICAL AND ELECTRICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
"A" Steam Generator Blow-down Isolation Valve 3738	Corrective	N/A	Bad Seat	Excessive Leakage	New seat installed.
Hydraulic Snubbers AFW 10, 27, 28, 49	Functional Testing	N/A	Snubbers not properly set at factory	Bleed rates and lockup velocities unacceptable	Properly set lockup velocities and bleed rates on units.
Containment Purge Supply and Exhaust Valve	Corrective	N/A	Improper adjustment on rubber seats	Excessive Leakage	Adjusted seats.
			Broken operator spring	Would not operate	Replaced spring.
Diesel Generator Solenoid Valve 5907	Corrective	N/A	Welding slag in valve	Would not operate properly	Cleaned and rebuilt.
"A" Diesel Fuel Oil High Level Switch	Corrective	N/A	Float Broken	Would not operate properly	Repaired level float stem and linkages.
"C" Safety Injection Pump Breaker	Corrective	LER 77-07	Poor contacts	Failure to start off of Bus 14	Multi-amp test and replaced secondary contacts.
Main Steam Power Operated Relief Valve 3411	Corrective	N/A	Seat Cut	Leaking through	Lapped valve.
Main steam to Turbine Auxiliary Feedwater Pump MOV 3504	Corrective	N/A	Faulty Clutch Assembly	Inoperable limiter	Replace with spare limiter.

MECHANICAL AND ELECTRICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
Boric Acid Filter PI-108 Cutoff Valve 349B	Corrective	N/A	Failed diaphragm	Leaking	Replaced diaphragm.
"A" Charging Pump	Corrective	N/A	Pieces of diaphragm from valves in pump suction	Pump delivering approximately 1/2 flow.	Replaced valve disc and "O" rings
Charging Pump Discharge to Relief Valve 284	Corrective	LER 77-09	Inadequate weld buildup on socket weld	Pinhole leak	Weld cutout and reweld.
Auxiliary Spray to Pressurizer Valve AOV 296	Corrective	N/A	N/A	Packing leak	Repacked valve.
Charging to "A" cold leg valve 392B	Corrective	N/A	N/A	Packing leak	Repacked valve.
Gas decay tank reuse valve 1632	Corrective	N/A	Piece of steel caught in valve	Leaking through	Removed piece of steel lodged in valve.
Safety Injection Pump Discharge Check Valve 889A	Corrective	LER 77-12	Crud buildup	Stuck open	Cleaned and inspected.
Steam Generator blowdown Isolation Valve 5738	Corrective	N/A	Cut Plug	Excessive leaking	Replaced plug and stem.

MECHANICAL AND ELECTRICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
Pressurizer Liquid Space Sample Valve 953	Corrective	N/A	Cut gasket	Bonnet leak	Replaced bonnet gasket.
"B" Containment Spray Discharge Valve 860D	Corrective	N/A	Torque setting too low	Valve failed to close	Increased torque pressure setting.
Charging to "B" cold leg valve 294	Corrective	N/A	N/A	Packing leak	Repacked valve.
"B" Steam Generator	Corrective	LER 77-08	See steam generator report	Leaking tube	Approximately 400 tubes Eddy Current tested. 5 tubes plugged.
Rod Control System	Corrective	N/A	Blown fuse in 2AC cabinet	"Urgent Failure" Alarm	Replaced blown fuse.
Boric Acid Filter Bypass Valves 398A and 398B.	Corrective	LER 77-16	Diaphragm failure	Excessive leaking from bonnets	Bonnet and diaphragm assembly replacement.
"B" Loop Sample Isolation Valve AOV 966C	Corrective	N/A	N/A	Leaking through	Cleaned and inspected.
"A" Charging Pump	Corrective	N/A	N/A	Excessive leakage	Stuffing box assembly replacement.

MECHANICAL AND ELECTRICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
Blender supply to Refueling Water Storage Tank Valve 365A	Corrective	N/A	Failed Diaphragm	Bonnet leak	Diaphragm replacement
"A" Steam Generator hydraulic snubbers; 2 on north-east corner	Corrective	LER 77-13	Seal failure	Excessive leakage, loss of fluid in reservoir	Replaced seals and tested seals.
Rod Position Indication System	Corrective	N/A	Drawer D-10 failed causing CR 9 relay coil to burn out	Turbine runback	Replaced RPI drawer and relay coil.
Reactor Protection System	Corrective	N/A	Relays RT 2 and RT 4 had open coil circuits	Relays dropped out	Replaced relays.
"B" Charging pump discharge line to relief valve 284	Corrective	LER 77-18	Defect during weld installation	Leak	Ground out defect area and repaired. Performed penetrant test to verify repair.
"B" Charging Pump	Corrective	N/A	N/A	Stuffing box leaking	Installed spare stuffing box.
"B" Diesel Generator Tie Bus 16	Corrective	LER 77-19	Bent secondary control contact	Failure of "B" diesel breaker to close onto Bus 16	Tested, cleaned and replaced secondary contacts.

MECHANICAL AND ELECTRICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
"C" Charging Pump Discharge Drain Line to Valve 292E	Corrective	LER 77-20	Porosity or start/stop on initial weld root pass	Leak in weld	Installed, new section of piping.
NIS-42 Power Range	Corrective	N/A	Blown fuse while checking overpower setpoint	NC-306 High Range Bistable failure	Replaced bistable.
"E" Motor Driven Auxiliary Feedwater Pump Relief Valve 4022	Corrective	N/A	Cut seat	Leaking through	Installed new spring. Cleaned and lapped valve.
"C" Charging Pump - Varidrive	Corrective	N/A		Varidrive belts broke	Replaced belts.
"A" Diesel Generator	Corrective	LER 77-21	Governor set improperly	Failure of diesel to achieve rated power	Reset governor.
Radiation Monitoring System R-6	Corrective	N/A	Burned out transformer	Drawer inoperable	Replace transformer.
Fire System	Corrective	N/A	Fire booster tank fill valve positioner defective	Valve Cycling	Rebuilt positioner.

MECHANICAL AND ELECTRICAL MAINTENANCE

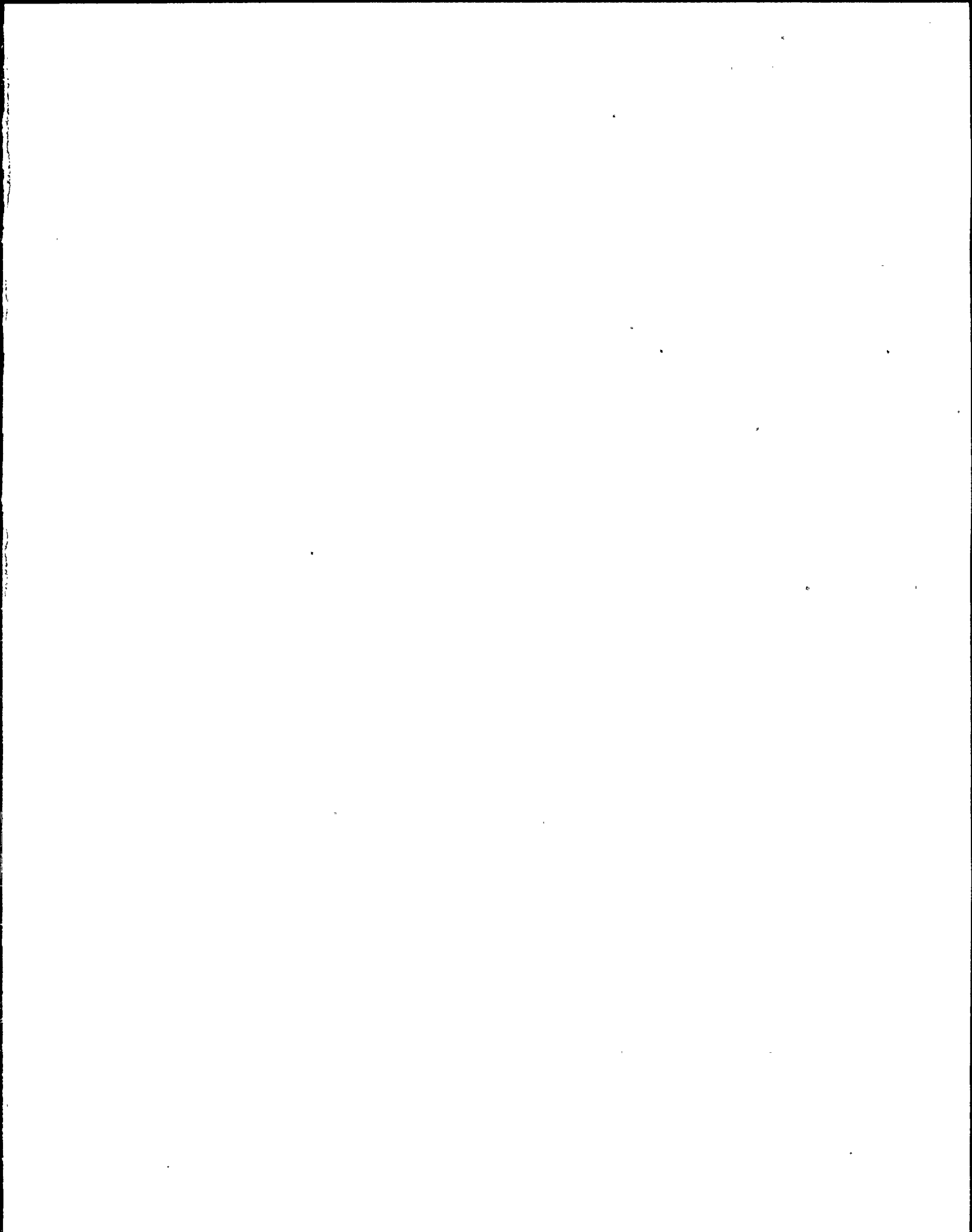
EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
"B" Steam Generator Snubber B-4	Corrective	LER 77-22	Seal wear	Excessive leakage snubber declared inoperable	Replaced seals and pressure tested unit. Procedure M 40.7.
Weld leak down stream of non-regenerative heat exchanger, adjacent to Valve 204B	Corrective	LER 77-25	Poor initial weld	Pinhole leak	Repaired weld per EM-191.
Steam line Atmospheric Power Relief Valves	Preventive & Corrective	N/A	Leaking Solenoid Valve on "B" valve.	Air Leak	Performed PT-20.1 (Control Air System Integrity Test) on CV-56 and CV-57. Replaced solenoid valve for Valve CV-57 ("B" Valve).
Radiation Monitoring System - R-2 Containment	Corrective MR 77-2659 &	N/A	Defective Drawer	Spurious alarms without meter moving upscale	Replaced with spare drawer.
Rod Control System	Corrective	Outage Number 127	"Urgent Alarm" in 2BD Power Cabinet	Could not withdraw Bank "D"	Replaced defective printed circuit cards.
Rod Control System	Corrective	N/A	"Urgent Alarm" in 1BD Power Cabinet	Could not withdraw Bank "D"	Reseated printed circuit cards.
Reactor Protection System	Corrective	N/A	Bistable PC-469A failed	"Steam Generator Lo-Lo Pressure single channel alert" alarm	Replaced bistable.

MECHANICAL AND ELECTRICAL MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
RHR Piping near Valve 718A	Corrective	LER 77-26	Improper seal weld on threaded nipple during construction	Pinhole leak	Weld repaired per EM-192.
"B" Charging Pump	Corrective	N/A	Normal Wear	Varidrive failure	Rebuilt varidrive per M-11.4.4.
Valve 252 RC filter outlet to RHR heat exchanger	Corrective	N/A	Hydro on RHR System	Leaking diaphragm	Replaced diaphragm per M-37.16A.
"B" Charging Pump	Corrective	N/A	Varidrive belt failure	Loss of charging flow	Replaced belts.
"B" Charging Pump	Corrective	N/A	Varidrive belt failure	Loss of charging flow	Replaced belts.
"C" Charging Pump	Corrective	N/A	Normal packing wear	Increased charging pump leak off rate	Repacked pump.
NaOH Spray Additive Valves 836A and 836B	Corrective	N/A	Valves not seating tightly	NaOH leaking into RWST	Cleaned and lapped seats, adjusted valve stroke per M-37.55.
Valve CV 58 (4561) Cooling Service Water to Containment	Corrective	N/A	Defective Solenoid	Valve could not be reset	Replaced solenoid coil.

Instrumentation and Control Equipment Maintenance

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
Boric Acid Tank Levels	Corrective	N/A	Partial Plug	LT-102 & LT-172 Drifted High	Performed PT-21. Reamed out all sensing lines.
Boric Acid Tank Levels	Corrective	N/A	Partial Plug	LT-171 Drifted High	Performed PT-21. Reamed out all sensing lines.
Boric Acid Tank Levels	Corrective	N/A	Partial Plug	LT-102 Drifted High	Performed PT-21. Reamed out all sensing lines.
Boric Acid Tank Levels	Corrective	N/A	Partial Plug	LT-172 Drifted High	Performed PT-21. Reamed out all sensing lines.
NIS Channel N-41 power range	Corrective	N/A	N-41 Drawer out of Calibration	Redundant measurement N-41 reads high	Recalibrated NIS Channel 41
RMS-R-17-component cooling water monitor	Corrective	N/A	Alarm setpoint drift	Reading high with no alarm	Adjusted alarm level set to proper value
RMS-R-19-steam generator blowdown monitor	Corrective	N/A	Check source box lever stuck	Reading was high	Freed up check source box lever.
Boric Acid Tank Levels	Corrective	N/A	Partial Plug	LT-106 Drifted High	Performed PT-21. Reamed out all sensing lines.



Instrumentation and Control Equipment Maintenance

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
Acoustic Leak Detection System	Corrective	N/A	Calibration Drift	"CAUTION" would not clear on Point #5	Performed PT-19 (calibration of system)
Fire System	Corrective	N/A	Fire water tank level control malfunctions	Fire service booster pump would not start automatically	Repaired fire water tank level controller.
Boric Acid Tank Levels	Corrective	N/A	Partial Plug	LT-106, LT-171, & LT-172 Drifted High	Performed PT-21. Reamed out all sensing lines.
Valve HCV-133 (Residual Heat Removal HX to demineralizer)	Corrective	N/A	Current to air transducer drift	Valve leaking thru	Calibrated and adjusted current to air transducer.
Boric Acid Tank Levels	Corrective	N/A	Partial Plug	LT-171 was high	Performed PT-21. Reamed out all sensing lines.
RMS-R-15-air ejector monitor	Corrective	N/A	Moisture in detector connector	Failing Low	Dried out connector and detector.
Fire System	Modification	N/A	SM 76-30.1		Modified Grinnell deluge valves to insure operation.
RMS-R-15-air ejector monitor	Corrective	N/A	Pre-amp connector damp	Spikes High	Dried out pre-amp connector.

Instrumentation and Control Equipment Maintenance

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
RMS-R-15-Air ejector monitor	Corrective	N/A	Defective Detector	Reading High	Replaced detector and recalibrated.
Boric Acid Tank Levels	Corrective	N/A	Bad tubing fitting	LT-102 would not come off zero	Performed PT-21. Reamed out all sensing lines.
RMS-R-15-Air ejector monitor	Corrective	N/A	Defective Detector	Reading High	Replaced detector and recalibrated.
Rod Control System	Corrective	N/A	Blown fuse in 2AC cabinet	"Urgent Failure" Alarm	Replaced blown fuse.
RMS-R-13 and R-14. Auxiliary Building gas and particular monitors	Corrective	N/A	Low-Flow	Low Flow Alarm	Replaced pump.
Fire Water Storage Tank	Corrective	N/A	Air leak in level controller	Fill valve cycles open and shut	Repaired air leak.
RMS-R-15-Air ejector monitor	Corrective	N/A	Faulty Detector	Reading High	Replaced detector and recalibrated.
RMS-R-15-Air ejector monitor	Corrective	N/A	Faulty Detector	High Alarm	Replaced detector and recalibrated.
RMS-R-15-Air ejector monitor	Corrective	N/A	Moisture in connector	Failed Low	Dried out detector and connector.

Instrumentation and Control Equipment Maintenance

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
Rod Position Indication System	Corrective	N/A	Drawer D-10 failed causing CR-9 Relay coil to burn out	Turbine runback	Replaced RPI Drawer and relay coil.
Reactor Protection System	Corrective	N/A	Relays RT-2 & RT-4 had open coil circuits	Relays dropped out	Replaced relays.
Boric Acid Tank Levels	Corrective	N/A	Partial Plug	LT-171 was high	Performed PT-21. Reamed out all sensing lines.
Boric Acid Tank Levels	Corrective	N/A	Partial Plug	LT-106 was high	Performed PT-21. Reamed out all sensing lines.
RMS-R-15-Air ejector monitor	Corrective	N/A	Bad Detector	Reading High	Replaced detector.
Boric Acid Tank Levels	Corrective	N/A	Partial Plug	LT-171 was high	Performed PT-21. Reamed out all sensing lines.
RMS-R-15-Air ejector monitor	Modification	N/A	N/A	N/A	Performed SM-76-24.3. Replaced R-15 with a new unit.
Boric Acid Tank Levels	Corrective	N/A	Partial Plug	LT-172 was high	Performed PT-21. Reamed out all sensing lines.

Instrumentation and Control Equipment Maintenance

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
Nuclear Instrumentation System	Corrective	N/A	Blown fuse while checking over-power setpoint	Turbine runback	Replaced bistable card.
Boric Acid Tank Levels	Corrective	N/A	Partial Plug	LT-106 was high	Performed PT-21. Reamed out all sensing lines.
Radiation Monitoring System - R-6	Corrective	N/A	Burned out transformer	Drawer inoperable	Replaced transformer.
Boric Acid Tank Levels	Corrective	N/A	Partial Plug	LT-102 was high	Performed PT-21. Reamed out all sensing lines.
Fire System	Corrective	N/A	Fire Booster Tank fill valve positioner defective	Valve Cycling	Rebuilt positioner.
Boric Acid Tank Levels	Corrective	N/A	Partial Plug	LT-171 was high	Performed PT-21. Reamed out all sensing lines.
Boric Acid Tank Levels	Corrective	N/A	Partial Plug	LT-106 was high	Performed PT-21. Reamed out all sensing lines.
Steam line atmospheric power relief valves	Preventive & Corrective	N/A	Leaking solenoid valve on "B" valve	Air Leak	Performed PT-20.1 on CV-56 and CV-57. Replaced solenoid valve for Valve CV-57 ("B" valve).

Instrumentation and Control Equipment Maintenance

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION		CORRECTIVE ACTION
			CAUSE	RESULTS	
Radiation Monitoring System R-2 Containment	Corrective	N/A	Defective Drawer	Spurious alarms without meter moving upscale	Replaced with spare drawer.
Rod Control System	Corrective	Outage Number 127	"Urgent Alarm" in 2BD power cabinet	Could not withdraw Bank "D"	Replaced defective printed circuit cards.
Rod Control System	Corrective	N/A	"Urgent Alarm" in 1BD power cabinet	Could not withdraw Bank "D"	Reseated printed circuit cards.
Reactor Protection System	Corrective	N/A	Bistable PG-469A failed	"Steam Generator Lo-Lo pressure single channel alert alarm"	Replaced bistable.
Valve CV-58. Cooling service water to containment	Corrective	N/A	Defective Solenoid	Valve could not be reset	Replaced solenoid coil.
Boric Acid Tank Levels	Corrective	N/A	Partial Plug	LT-106 was high	Performed PT-21. Reamed out all sensing lines.
Incore Flux Mapping System "C" Drive	Corrective	N/A	Bad encoder and driver	Detector drive would not stop in auto	Replaced encoder and driver

LICENSEE EVENT REPORTS

During the reporting period of January 1, 1977 through December 31, 1977, a total of 26 events, classified as Reportable Occurrences, took place requiring submittal of Licensee Event Reports to the NRC. The following is a compilation of narrative descriptions of those events:

LER 77-01,
1-03-77

"C" SI Pump Failed to Start

During monthly testing, the "C" SI pump failed to start manually from the Bus 14 breaker on the first attempt. The control switch was reset and the pump started on next three attempts. It also started successfully from bus 16. This incident is similar to 7 events involving bus 16. Complete inspection of circuit breaker and all circuitry in "close" circuit was made. The only problem found was a weak spring in a secondary contact assembly which would have caused intermittent faulty operation. Circuit breaker is Westinghouse Model DB-50, 600V A. C. Contact assembly was replaced. Pump will be started weekly with bus 14 breaker until the next scheduled monthly surveillance test. No danger to the health and welfare of the general public was identified.

LER 77-02
2-17-77

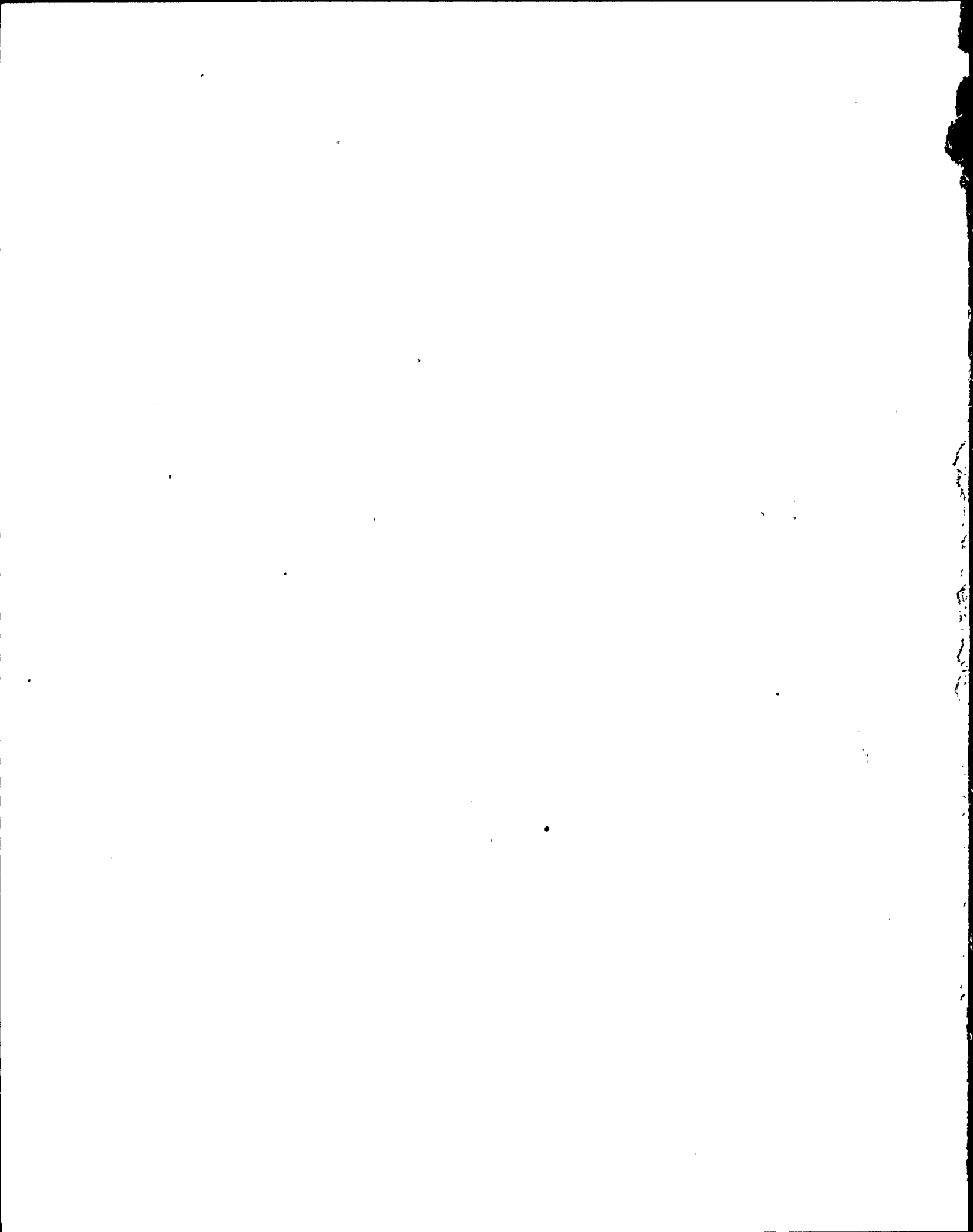
Reactivity Insertion Curve Used for Safety Analyses

As a result of review of analyses methods, Westinghouse notified RG&E that reactivity insertion curve, FSAR Fig. 14-1, used for safety analyses is not sufficiently conservative. All transients were re-evaluated using revised curve. Westinghouse determined that there was no impact on any transients except loss of flow transient for which 1.30 minimum DNBR criterion is still met. No danger to the health and welfare of the general public was identified.

LER 77-03
5-02-77

Abnormal Degradation of Steam Generator Tubes

During the planned Eddy Current inspection of the Steam Generator (SG) tubes, 13 tubes in the "A" Steam Generator and 1 tube in the "B" Steam Generator showed defects above the wastage criteria of 40%. These tubes have been plugged. Localized corrosion of tubes caused by concentrations of residual phosphates. A complete Eddy Current program description and report is attached. No danger to the health and welfare of the general public was identified.

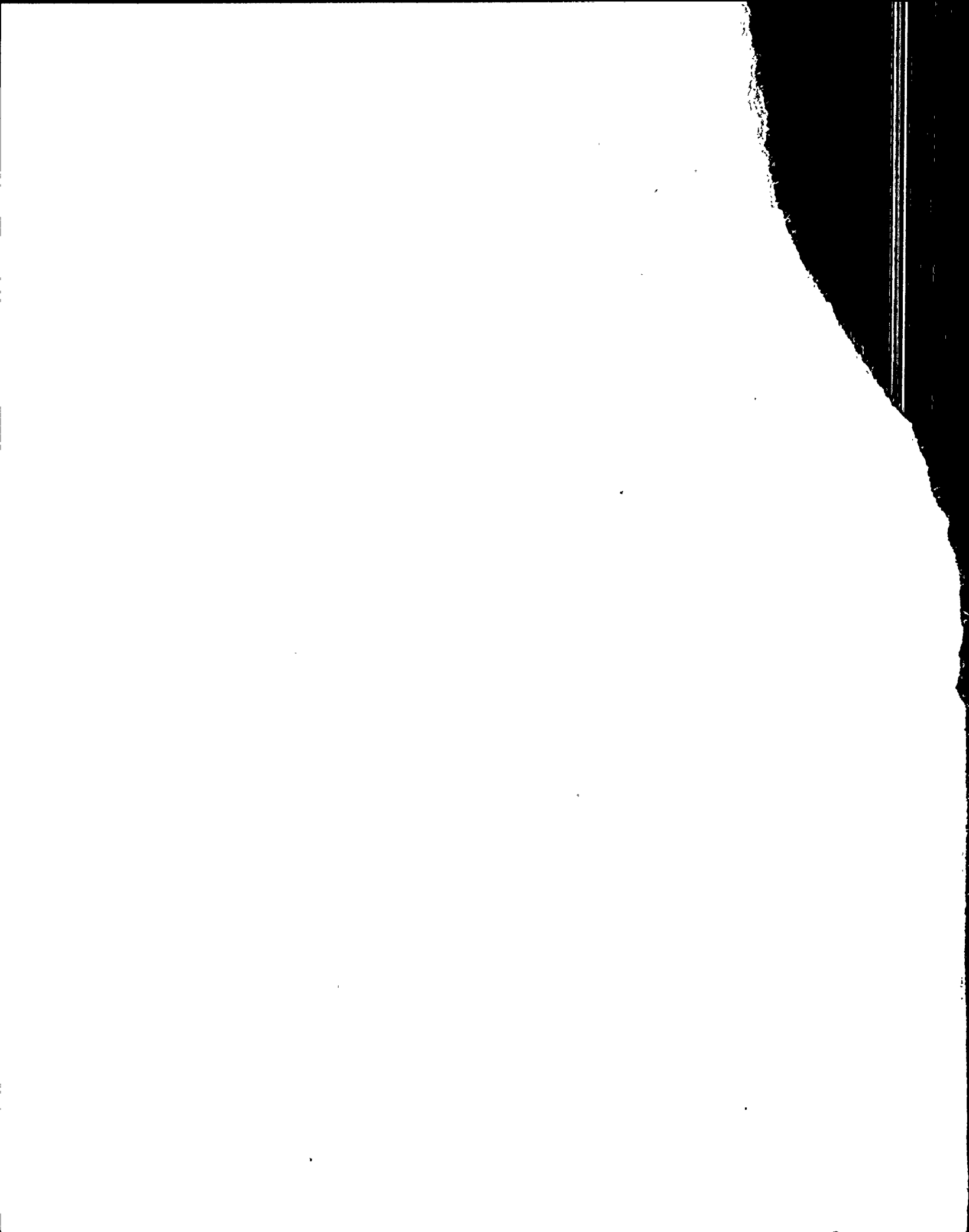


Ginna Station
Steam Generator Inspection
Final Report
May 11, 1977

Rochester Gas and Electric Corporation performed a planned inspection of Ginna Station steam generators from April 16, 1977 through April 23, 1977 in accordance with the Inservice Inspection Program as part of the annual refueling and maintenance outage. This inspection consisted of eddy current examinations at 400 KHz and 100 KHz to detect defects from corrosion or cracking, 400 KHz for dent evaluation, 25 KHz for sludge profiling and 3.5 KHz to verify support plate integrity.

The inspection included the following:

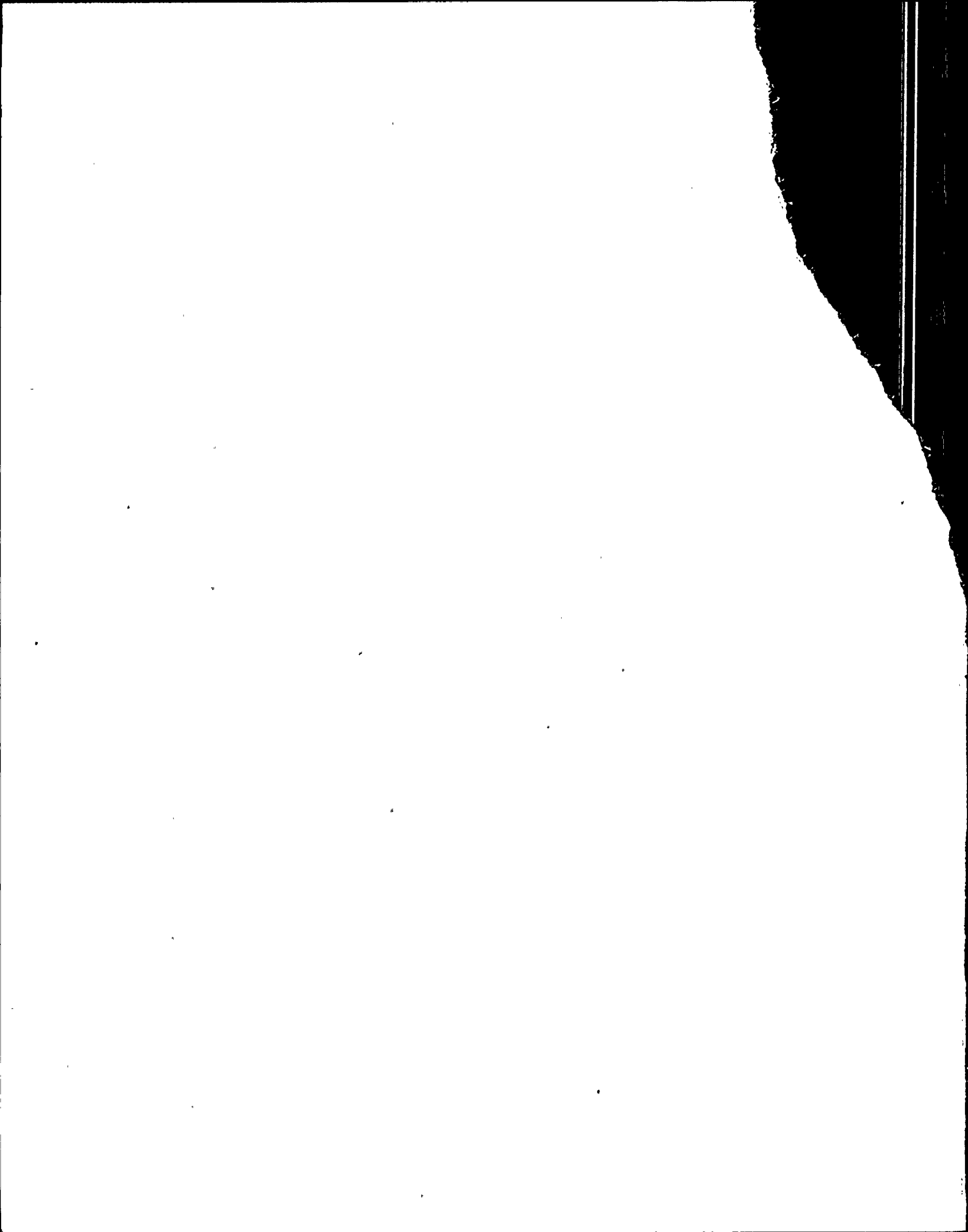
<u>Leg</u>	<u>Tubes</u>	<u>Frequency</u>	<u>Concern</u>	<u>Support</u>
A-Inlet	195	25 KHz	Sludge	1st
	1731	400 KHz	Defect	1st
	148	400 KHz	Defect	6th
	125	400 KHz	Defect	U-bend
	100	400 KHz	Dent	6th
	207	3.5 KHz	Supports	6th
	60	100 KHz	Defect	U-bend
A-Outlet	195	25 KHz	Sludge	1st
	268	400 KHz	Defect	6th
	100	400 KHz	Dent	6th
	207	3.5 KHz	Supports	6th



<u>Leg</u>	<u>Tubes</u>	<u>Frequency</u>	<u>Concern</u>	<u>Support</u>
B-Inlet	195	25 KHz	Sludge	1st
	1252	400 KHz	Defect	1st
	148	400 KHz	Defect	6th
	125	400 KHz	Defect	U-bend
	70	400 KHz	Dent	6th
	100	3.5 KHz	Support	6th
	60	100 KHz	Defect	U-bend
B-Outlet	195	25 KHz	Sludge	1st
	208	400 KHz	Defect	6th
	100	400 KHz	Dent	6th
	100	3.5 KHz	Support	6th

Results of these examinations are given in Figures 1 through 3 which includes the "A" inlet and the "B" inlet and outlet, respectively. Tables (1) and (2) of this report are included for comparison of the last five steam generator inspections.

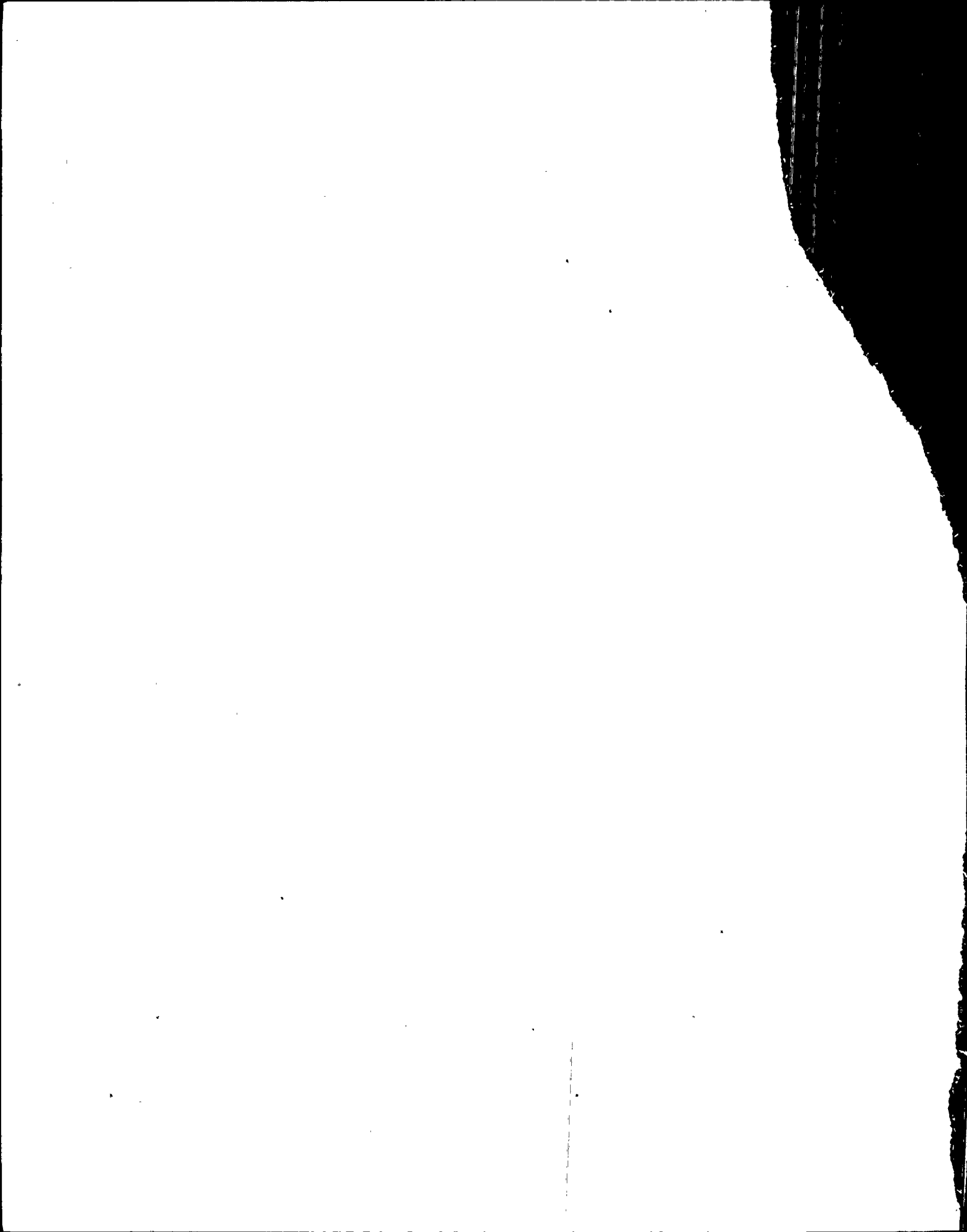
All of the eddy current indications were within the first few inches of tubing directly above the tube sheet with the exception of one in the U-bend area of the "B" steam generator. This indication in the "B" steam generator U-bend area was on a periphery tube and appears to be a one-of-a-kind construction-type defect of very small volume, 95% through the tube wall. The other indications with the exception of two, are postulated to be due to wastage, based on growth rates. The excepted two



tubes, in the "A" inlet, due to their large growth rate compared to the mean, may have been caused by concentrations of caustics. Confirmation of this is not possible since the eddy current examination method cannot differentiate between indications resulting from either wastage or stress corrosion cracking, but can accurately measure the maximum defect penetration to within normal statistical variation. The indications are seen only in those areas where indications have been previously noted, with the exception of the indication in the "B" steam generator U-bend area. There has been no expansion of the indication region to other areas of the steam generators. This inspection verified that there are not any other tubes with ID Indication in the periphery tubes associated with the wedge areas where there had been a leak in April 1976, (see Licensee Event Report 76-15).

The dent evaluation program performed on both steam generators revealed no dents above 10 mils with only a few of the tubes in each generator being involved. Along with the dent evaluation program, a support plate integrity examination was performed which did not reveal any problems.

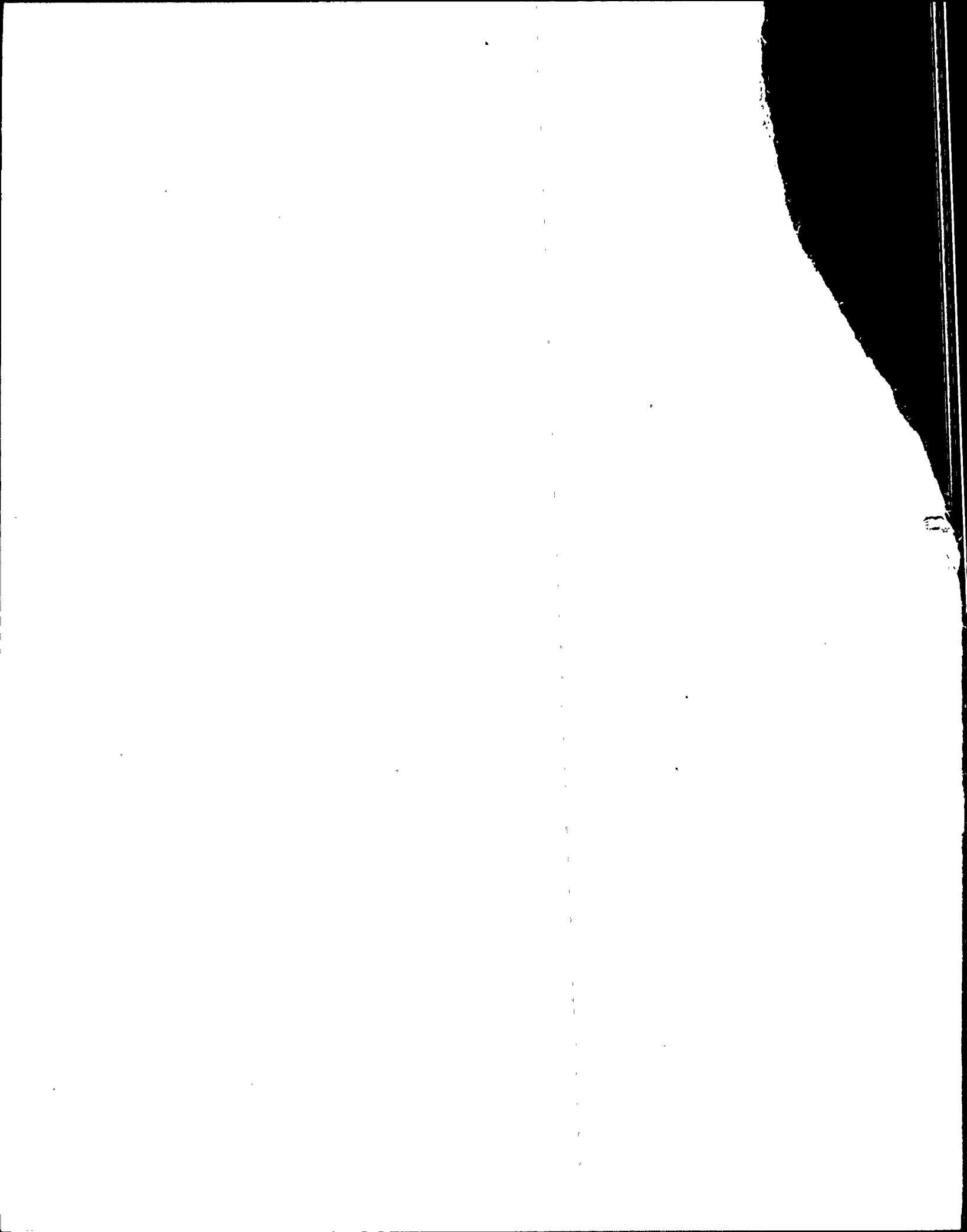
The small number of tubes in the "A" inlet which have experienced deterioration, inconsistent with the average, are believed to have deteriorated from the concentration of residual phosphates in the secondary side sludge deposits. These concentrations of phosphates are caused by the remaining traces of sludge deposits formed during the period of phosphate control of the secondary system water chemistry before the conversion to AVT chemistry control in November 1974.



Removal of residual phosphates from the secondary side over the past 31 months of operation has been accomplished by continuous steam generator blowdown and high pressure water lancing. The high pressure water lancing performed on the secondary side is designed to remove as much of the sludge as possible, which contains the undesirable residual phosphates and/or caustics. Blowdown samples taken during normal operation indicated only small amounts of phosphates present, although phosphates in the sludge could revert back into PO_4^{3-} and concentrate on the tube surfaces which, depending on the molar ratio, would result in acidic or caustic attack of the tube.

The corrective action taken to ensure the continued reliability of the steam generators includes the following:

- a. All tubes with eddy current indications of wall penetration greater than or equal to 40% were plugged.
- b. A thorough lancing of the secondary side of the tube sheets was performed in both steam generators to remove as much as possible any remaining phosphates and/or caustics contained within the sludge. Sludge lancing of both steam generators will be continued in an effort to keep sludge content to a minimum. The lancing, coupled with blowdown during startup and normal operation, should considerably reduce the probability of significant tube degradation during the plants' subsequent operation.



c. In addition, a modification of the plant's secondary condensate system is under construction which is designed to insure that the feedwater entering the steam generators will be of the highest purity. This modification will add in-line demineralizers to the condensate system, and is scheduled to be placed in service in August of this year.

Because it has been established that all but 14 tubes from both steam generators had less than 40% defect indications; because there were only a few tubes which experienced comparatively rapid degradation; and because sludge lancing should further reduce the probability of phosphate and caustic pockets forming, the steam generators are considered acceptable for uninterrupted use until the planned refueling outage in the Spring of 1978, approximately 1 year from the expected date of return to service of Ginna Station. An eddy current examination of the steam generators in accordance with the Inservice Inspection Program shall be performed during the 1978 refueling outage.

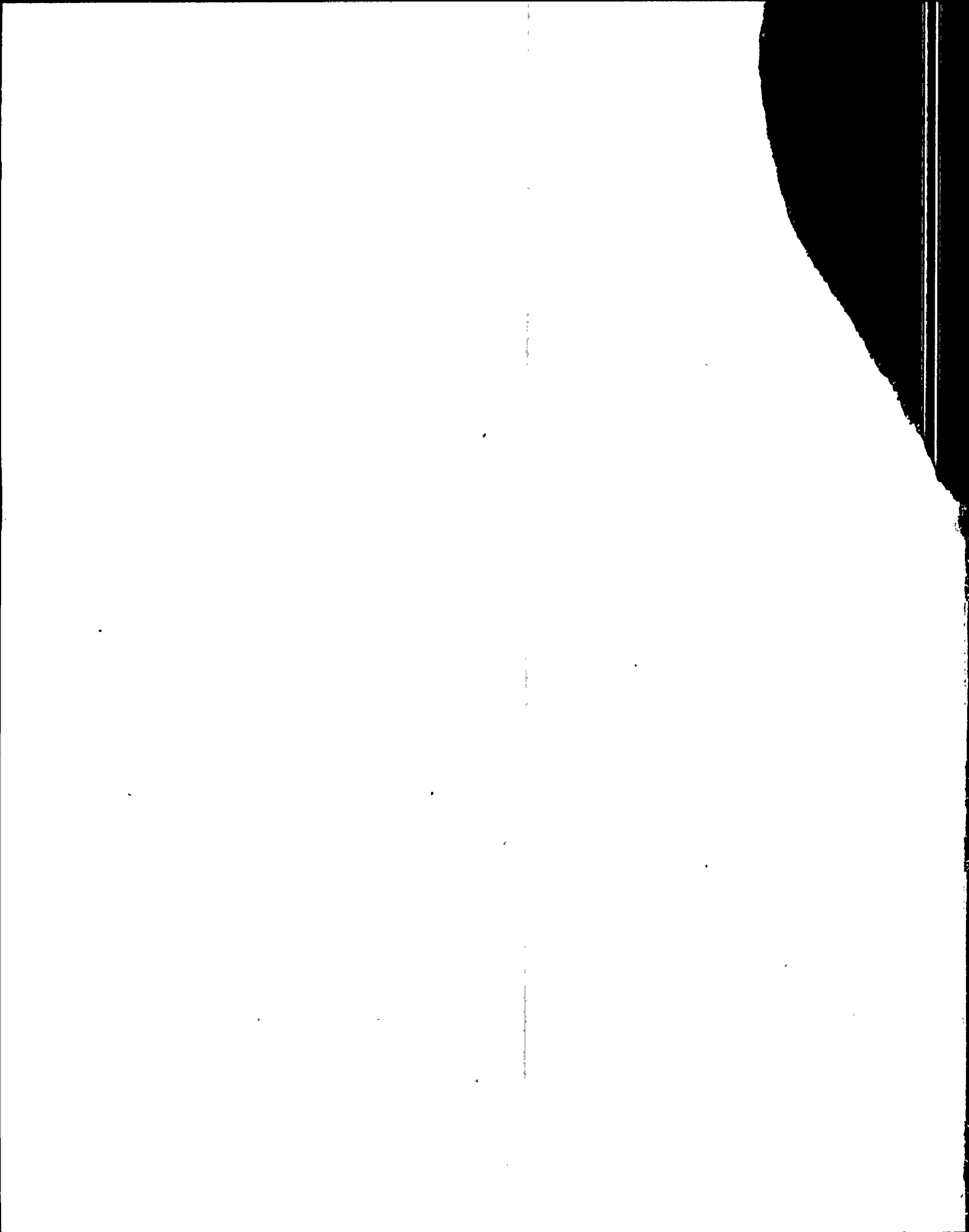


TABLE (1)

STEAM GENERATOR POSITION	INDICATION SIZE (t)	EXAMINATION DATE				
		FEB. 1974	NOV. 1974	MAR. 1975	FEB. 1976	APR. 1977
"A" INLET	< 20	329	631	655	230	730
	20-24	63	59	109	59	39
	25-29	50	46	63	47	37
	30-34	36	31	38	50	23
	35-39	14	25	27	31	8
	40-44	24	14	22*	19*	0
45-49	12	5	14*	8*	11*	
> 50	17	2*	10*	12*	2*	
Total "A" Inlet Tubes Examined At Each Inspection at 400KHZ		3260	1655	2174	3192	2003

STEAM GENERATOR POSITION	INDICATION SIZE (t)	EXAMINATION DATE				
		FEB. 1974	NOV. 1974	MAR. 1975	FEB. 1976	APR. 1977
"A" OUTLET	< 20	58	278	1	10	113
	> 20	0	0	0	0	0
Total "A" Outlet Tubes Examined At Each Inspection at 400KHZ		516	430	442	3192	268

* tubes were explosively plugged

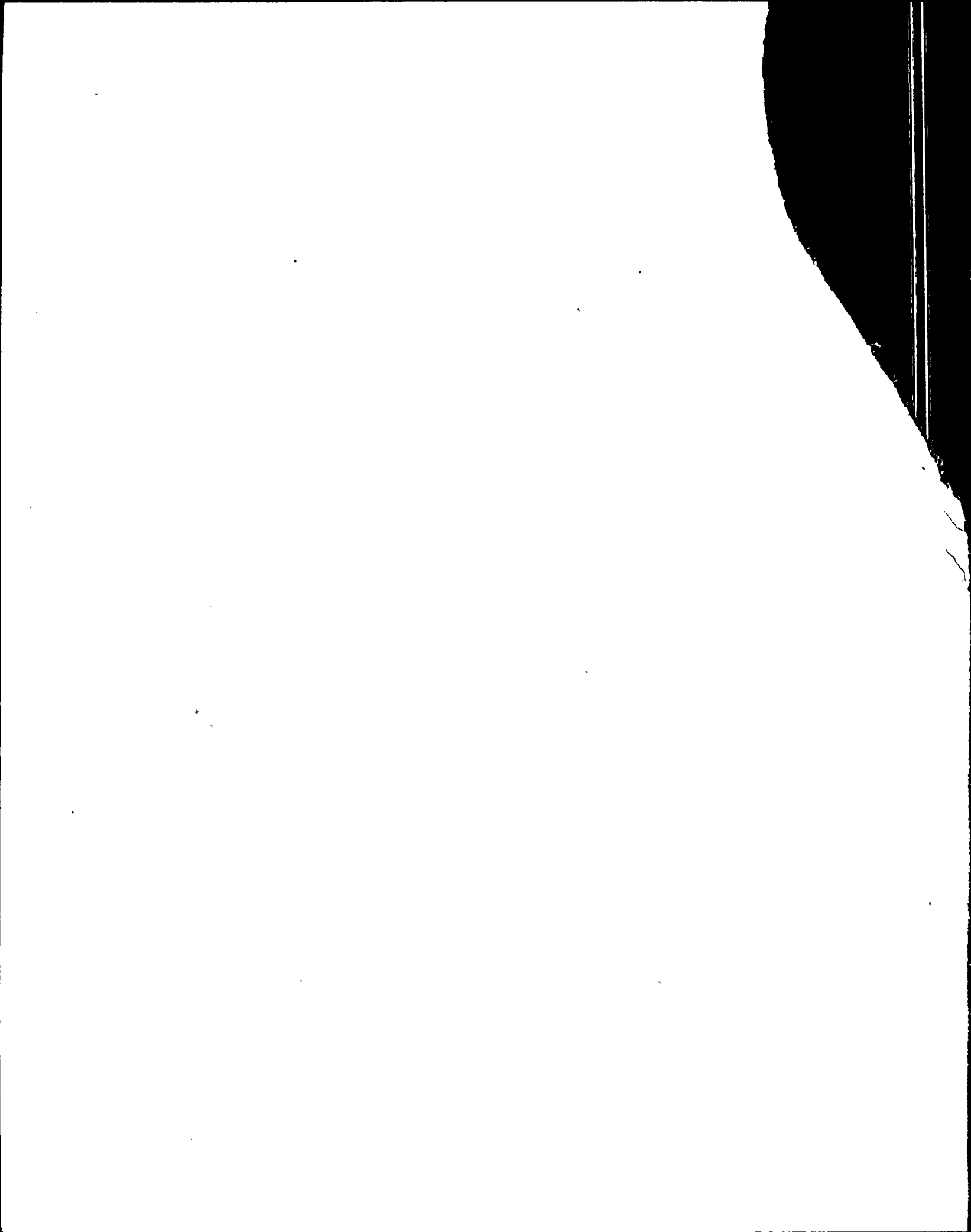


TABLE (2)

STEAM GENERATOR POSITION	INDICATION SIZE (%)	EXAMINATION DATE				
		FEB. 1974	NOV. 1974	MAR. 1975	FEB. 1976	APR. 1977
"B" INLET	< 20	21	490	411	764	719
	20-24	4	3	13	25	12
	25-29	2	4	10	8	8
	30-34	0	1	9	9	0
	35-39	0	1	5	3	1
	40-44	0	0	1*	1*	0
	45-49	0	0	0	0	0
	≥ 50	0	0	10*	1*	0
Total "B" Inlet Tubes Examined At Each Inspection at 400 KHZ		1098	675	1931	3247	1525

STEAM GENERATOR POSITION	INDICATION SIZE (%)	EXAMINATION DATE				
		FEB. 1974	NOV. 1974	MAR. 1975	FEB. 1976	APR. 1977
"B" OUTLET	< 20	0	0	0	1003	90
	≥ 20	0	0	0	2	1*
Total "B" Outlet Tubes Examined At Each Inspection at 400 KHZ		516	39	442	3247	268

NOTE: Two tubes in the "B" steam generator were explosively plugged in January 1976 and fifteen in April 76

* TUBES WERE EXPLOSIVELY PLUGGED

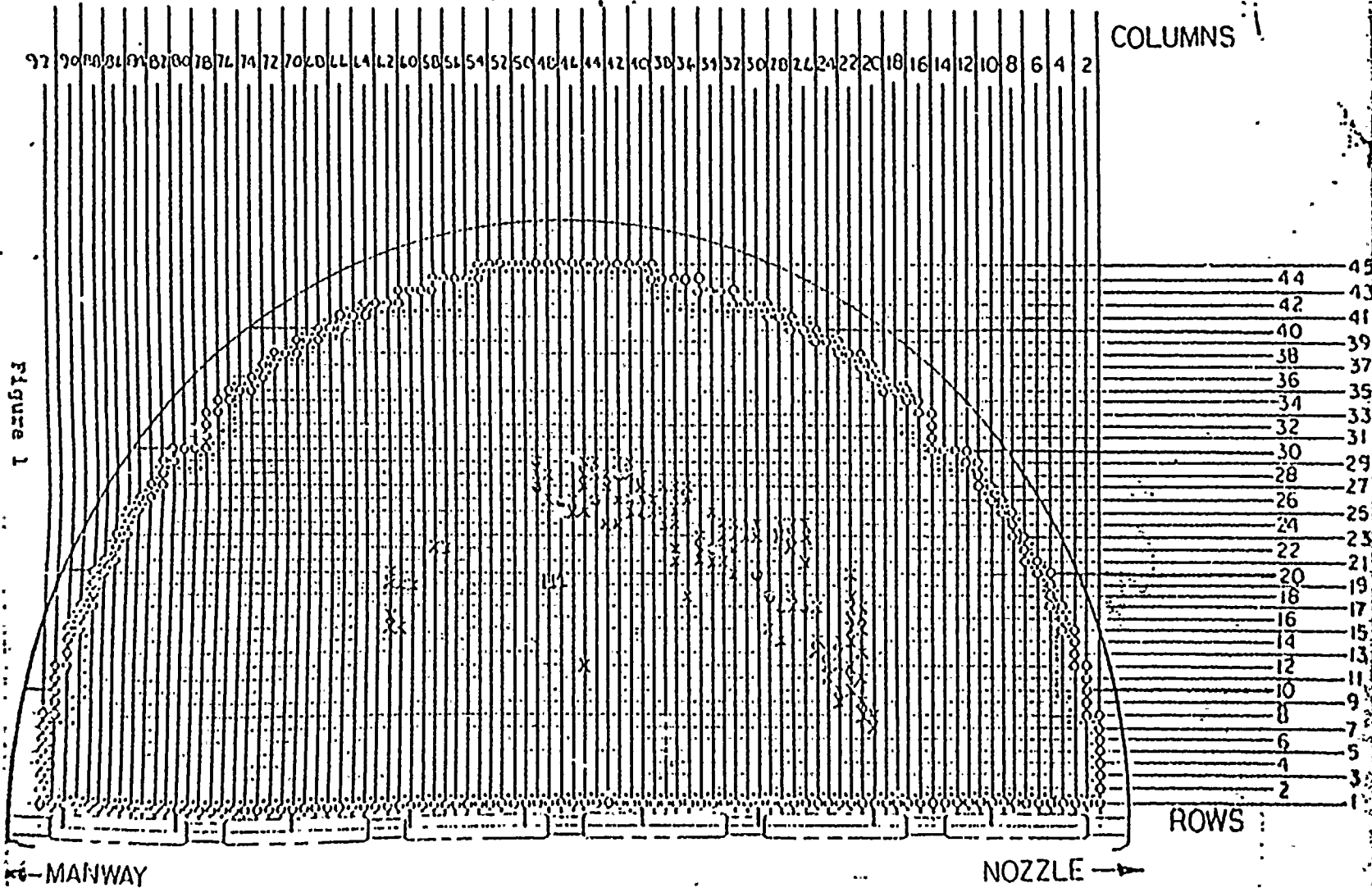


"A" INLET EDDY CURRENT INDICATIONS

RGLE Steam Generator

- X - 20% TO 39%
- - 40% TO 59%
- △ - 60% TO 79%
- - 80% TO 90%

91 87 81 85 83 81 79 77 75 73 71 69 67 65 63 61 59 57 55 53 51 49 47 45 43 41 39 37 35 33 31 29 27 25 23 21 19 17 15 13 11 9 7 5 3 1



- X - 20% TO 39%
- o - 40% TO 59%
- Δ - 60% TO 79%
- - 80% TO 90%

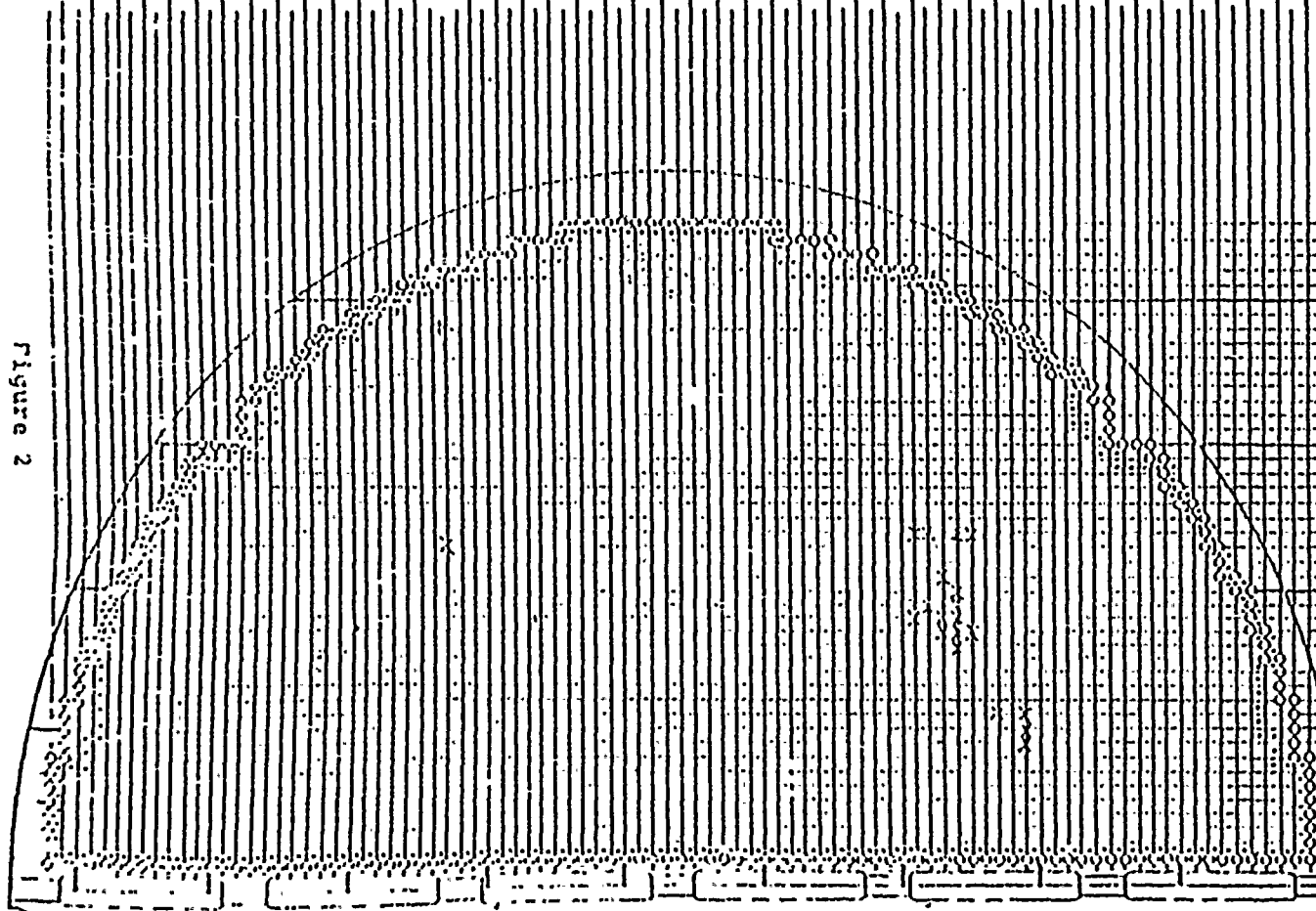
"B" INLET EDDY CURRENT INDICATION

RIGHT Steam Generator

11 10 9 8 7 6 5 4 3 2 1 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

11 10 9 8 7 6 5 4 3 2 1 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

COLUMNS



- 44
- 42
- 40
- 38
- 36
- 34
- 32
- 30
- 28
- 26
- 24
- 22
- 20
- 18
- 16
- 14
- 12
- 10
- 8
- 6
- 4
- 2

ROWS

MANWAY →

NOZZLE →

Figure 2

X-20% TO 39%
 O-40% TO 59%
 Δ-60% TO 79%
 □-80% TO 90%

EDDY CURRENT INDICATION

RG&E Steam Generator

71 81 01 15 03 04 77 77 75 73 71 69 67 65 63 61 59 57 55 53 51 49 47 45 43 41 39 37 35 33 31 29 27 25 23 21 19 17 15 13 11 9 7 5 3 1

72 70 00 01 02 03 04 05 06 07 08 09 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45

COLUMNS

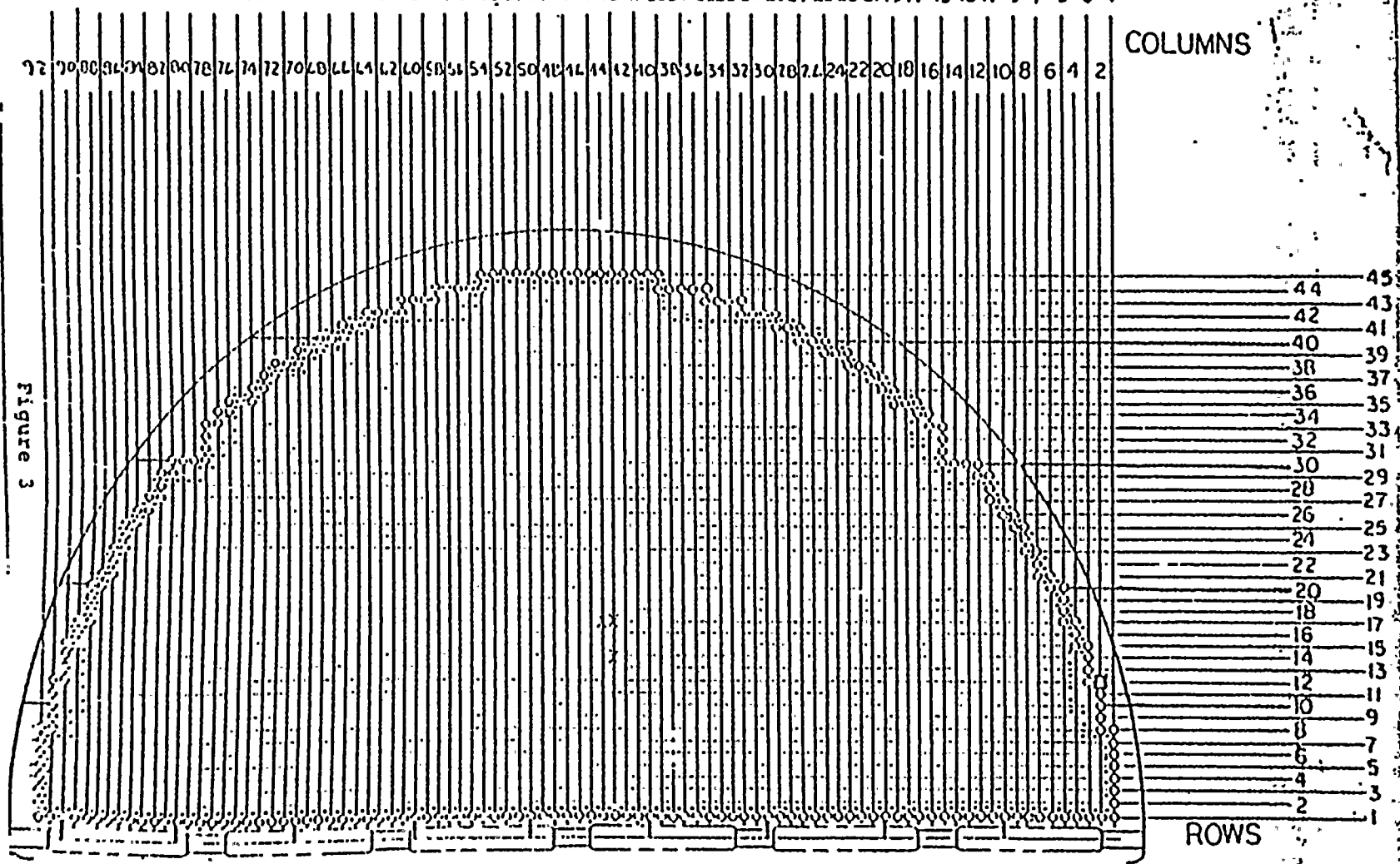
45
 44
 43
 42
 41
 40
 39
 38
 37
 36
 35
 34
 33
 32
 31
 30
 29
 28
 27
 26
 25
 24
 23
 22
 21
 20
 19
 18
 17
 16
 15
 14
 13
 12
 11
 10
 9
 8
 7
 6
 5
 4
 3
 2
 1

ROWS

Figure 3

13-MAINWAY

NOZZLE →



LER 77-04
5-23-77

Cable Reversal for Two Control Rods

During initial power escalation to 50% following refueling, a series of flux maps revealed a flux anomaly. The Unit was taken to hot shutdown, electricians verified reversed power and rod position indication (RPI) cables and restored cables to proper arrangement. During installation of new control rod power cables at reactor head, control and RPI cables for rods L-8 and K-7 were erroneously interchanged. After proper restoration, low power physics testing was continued and bank D and C worth were measured again. Proper rod response was verified by control rod exercise and further flux maps.

During any future maintenance, a Q. C. verification of power and RPI cables will be performed when cables are replaced. CAR 1126 was issued to document these actions. No danger to the health and welfare of the general public was identified.

LER 77-05
5-20-77

Reactor Coolant Chloride Concentration Exceeding Steady State Limit

During heatup after refueling, reactor coolant sample showed 0.16 chloride. Standby mixed bed demineralizer was put in service; sample frequency was increased. Chloride concentration returned to below 0.15 ppm in 17 hours. Release of chloride from old bed of resin. No further action to prevent recurrence is necessary. No danger to the health and welfare of the general public was identified.

LER 77-06
6-19-77

"C" Charging Pump Varidrive

During normal round, auxiliary operator noticed "C" charging pump varidrive belt (1 of 2) smoking. The "A" Charging Pump was inoperative. The "B" pump was started. The "C" pump was stopped and both belts were replaced. The belt is US Electrical Motors Varidrive part 84-2. No further corrective action was necessary. No danger to the health and welfare of the general public was identified.

LER 77-07
6-29-77

Bus 14 Breaker for "C" SI Pump

During the test, the "C" SI pump failed to start from bus 14 breaker on 2 attempts. The pump was started successfully with bus 16. Subsequently, the bus started on bus 14. This incident is similar to 7 events involving bus 16 and 1 event with bus 14. A replacement breaker was tested and installed. Inspection of the circuit breaker revealed no apparent cause for failure. The circuit breaker is Westinghouse Model DB-50, 500 V. A. C. It will be returned to Westinghouse for further investigation. No danger to the health and welfare of the general public was identified.

LER 77-08
7-05-77

"B" Steam Generator Leak

During normal operations the air ejector and blowdown radiation monitors alarmed. Analysis of the "B" Steam Generator blowdown verified iodine activity from a calculated leak rate of 0.006 gpm. The leaking tube was located with a hydrostatic test, and eddy current testing was performed to determine if a pattern of degradation existed. About 300 tubes were examined which included tubes in the area of the leaking tube and other selected areas with previous indications. These examinations revealed four additional tubes, in the area of the leaking tube, with similar (ID) indications. These four tubes and the leaking tube were explosively plugged. The failure of the tube R45C54 and the defect indications in tubes R44C55, 56, 57 and 58 are postulated to be caused by stress corrosion cracking from the inside surface of the tubes. These inside diameter cracks are theorized to be caused by a dent at the tube to tube sheet interface which induces stress into the tube wall resulting in cracking. There are not any chemical species postulated to be involved in this cracking phenomena due to the purity of the primary system's water, which flows through the tubes.

Corrective action taken included eddy current examination of surrounding tubes and tubes in areas where previous inservice inspections had revealed small defect indications. There were no other tubes identified, of those examined, which exhibited further degradation. Therefore, the failed tube and four others with similar (ID) indications were plugged. Further investigation is continuing with Westinghouse and RG&E personnel involvement studying the generic FWR tube failure problem. At the next refueling outage, an inservice inspection of the steam generator tubes will include tubes in the area of this failure. No danger to the health and welfare of the general public was identified.

LER 77-09
7-12-77

"B" Charging Pump Discharge Relief Piping Leak

During an operator inspection, a leak was detected in the "B" Charging Pump Discharge Relief Piping. With the Unit at cold shutdown and a redundant pump operable, the "B" pump was isolated. A probable cause was inadequate weld buildup on a socket weld during modification of the discharge. The pipe to flange weld was cut out and replaced per approved maintenance and welding procedures, and a hydro test was successfully completed. The present Q. C. program established since the modification, assures proper inspection of all safety-related socket welds to requirements of welding specifications. No danger to the health and welfare of the general public was identified.

LER 77-10
7-13-77

Redundant Service Water Discharge Line Support Design Error

See attached report, 4 pages. No danger to the health and welfare of the general public was identified.

LER 77-11
7-13-77

Containment Fan Cooler Technical Specifications

During the preparation of a charcoal filter testing Tech. Spec. change, it was determined that the Tech. Specs. on Containment fan coolers were not consistent with the design assumptions in the FSAR. The FSAR gives equipment assumed to operate to reduce containment pressure after DBA as 1 spray pump and 2 fan coolers. Tech. Specs. allow 1 of 4 fan coolers to be inoperable. In the event of blackout and one failed Diesel, only 1 fan cooler may be immediately available. Administrative controls have been revised to require operability of the necessary equipment, and a proposed Tech. Spec. change is being prepared. No danger to the health and welfare of the general public was identified.

LER 77-12
7-13-77

1A SI Pump Discharge Check Valve Leaking

During plant startup, decrease in 1A SI accumulator level was noted. Investigation showed 1A SI pump discharge check valve leaking, allowing water from accumulator to enter the RWST. 1A pump was isolated for valve repair; redundant systems were operable. After repair, the pump was tested and deemed operable. No danger to the health and welfare of the general public was identified.

LER

Redundant Service Water Discharge
Line Support Design Error

Event Description

Installation of a redundant house service water discharge line as part of the high energy backfit program began in 1975. Construction of the line and associated discharge structure was essentially completed in the spring of 1977, but the redundant discharge system has not been charged or placed in service. RG&E was advised on July 13, 1977 by the architect, Gilbert Associates, that based on subsequent review and analysis performed by them, the loading transmitted from the piping system to the auxiliary building structure through support SW-70 could produce, under design basis earthquake conditions, stresses in the structural support steel in excess of applicable allowable values.

On July 14 GAI informed RG&E that four additional supports (SW-69, SW-73, SW-78, SW-83) could produce stresses in the support structure in excess of applicable allowable values under conditions previously identified for SW-70.

The FSAR requires that the auxiliary building structure be designed and constructed as a seismic structure class 1. Gilbert Associates design control procedures require design analysis for safety related modifications at Ginna Station to be completed prior to issuance of construction drawings. Upon notification of this condition by the architect engineer on July 13, further work on redundant house service water line was stopped and the architect engineer was instructed to initiate a corrective design as well as perform a review of their design control program. The existing house service water discharge line was unaffected by these redundant service water supports and was operable during this period.

Cause Description

During the fall of 1974, the architect engineer, (GAI) began design of the redundant service water piping to the requirements of ASME, Section III and issued drawings for the majority of the pipe supports. These supports were installed during 1975 with the remainder, attendant upon final concept, being installed inside the Auxiliary Building during 1976. However, it was not until the latter part of 1976 that the GAI Structural Department began checking the support loads imposed on the structure by this change. In March 1977, GAI's project manager, advised RG&E that additional analysis and possible support modification to the structure might be required because of the added imposed pipe support loads. GAI believed at this time some modification to the structure was necessary, but the condition was safe as long as the pipe remained empty. Therefore, routine resolution of this task continued through the spring of 1977 since the line had not yet been placed in service.

On July 12, 1977, during a telephone conference between engineers at GAI and RG&E, the nature of the pipe support loads for SW-70 that were imposed on the structural steel of the auxiliary building was discussed, and GAI was requested to review their analysis, assuming that the redundant service water line was empty. GAI informed RG&E on July 13, 1977, that a preliminary calculation indicated that the allowable stresses at SW-70 are exceeded even if the pipe is empty and snow loads are present during the design basis earthquake. An immediate hold was placed on the redundant service water line. RG&E was advised by telephone on July 14, 1977, that a total of five (5) supports including SW-70 would have to be modified to meet the design criteria (i.e., line in service). Subsequently, GAI completed their

analysis and issued structural steel drawings, indicating reinforcements of existing structural steel and a new "A" frame support SW-70.

On July 21, 1977, GAI advised RG&E that final analysis concluded that the preliminary calculations were conservative and that the allowable stresses were not exceeded when the line is not in service (i.e., empty). It should be noted that the final analysis shows that, since the line was never filled, the structure including supports were never subjected to excessive loading nor would have exceeded allowable stresses for other postulated design basis loadings.

GAI's Corporate Quality Assurance Manual used to control the project requires that internal and external design/interface controls be established including review, approval, release, distribution and revision of design documents including design interfaces with all participating design organizations. Additionally, the "reject Procedure Manual and design criteria for this task include the requirement for Piping/Structural Engineering interface for pipe supports and their attachment structure. It has been concluded by GAI and RG&E that had project procedures been followed, adequate review of the support structural steel with associate support load imposed by the new redundant service water line would have been completed prior to release of construction drawings. Gilbert Associates has conducted a management review and has concluded that this incident is an isolated non-compliance with existing procedure.

To rectify the present situation and to assure no recurrence of this non-compliance, Gilbert has initiated the following action:

- (1) The technical design requirements for the service water line supports have been completed, with drawings having been transmitted for construction on July 18, 1977. An additional review of the piping modifications designed by

GAI for Ginna during this period has been initiated to assure no other interfaces have been missed. This review is expected to be completed by September 1, 1977.

- (2) Administratively, Gilbert has agreed to reindoctrinate affected project engineers and the key technical personnel assigned to the project in interface control between project disciplines. Gilbert's QA Division will perform an audit which will verify implementation of corrective action.

RG&E completed installation of the five (5) support modifications required on the redundant service water line on July 22, 1977. RG&E's QA Division plans to conduct a follow-up audit of GAI corrective action plan within sixty (60) days.

LER 77-13
8-02-77

"A" Steam Generator Snubbers

During the check of a 1 gal. reservoir for "A" Steam Generator snubbers reservoir, it was found empty. Upon testing of the seals, 2 of 8 snubbers showed excessive shaft seal leakage. During unit cooldown, the 2 snubbers were removed, seals replaced and pressure tested and the snubbers reinstalled. The snubbers are Anker-Holth, rated at 532,000 lb. tension and compression with displacement less than 1 inch in 20 min. A new test program includes the Steam Generator snubbers. No danger to the health and welfare of the general public was identified.

LER 77-14
8-09-77

Flux Difference Target Band Departure

Following reactor coolant dilution to compensate for Xenon buildup, control rods were inserted automatically, causing a departure from the flux difference target band. Boric acid was added, clearing the target band alarm in 7 minutes. A letter has been issued to all licensed personnel to remind them to continue to monitor the flux difference immediately after dilution. No danger to the health and welfare of the general public was identified.

LER 77-15
8-22-77

Spray Additive Dilution

During analysis of the spray additive tank sample, NaOH showed 29.5% concentration. A second analysis was performed to verify this. Per PORC recommendation, unit shutdown was started and NaOH concentration was increased to 30.3%. After verification, the unit was released for normal operations. Leakage through a check valve from the RWST diluted the NaOH tank. Sample frequency has been increased until valves are repaired at the next convenient opportunity. Leakage has been minimized by increasing tank pressure. No danger to the health and welfare of the general public was identified.

LER 77-16
8-23-77

Boric Acid Flow Path Isolation

During repair of boric acid filter bypass valves per approved emergency procedure, 1 or 2, boric acid flow paths from the boric acid tanks to the RCS was isolated. A path from RWST was available. Shutdown of the unit was in progress. Following diaphragm and bonnet replacements, the flow path was restored. Isolation of valves to be repaired resulted in reducing number of flow paths. No danger to the health and welfare of the general public was identified.

LER 77-17
8-24-77

NaOH Concentration Determination

During review of calculational procedure for NaOH concentration, an error was found and NaOH concentration was shown to be less than 30% by wt. 50% NaOH was added to tank increasing concentration to within specification by 1130 hours on 25 August 1977. The procedure was corrected by the Supervisor of Health Physics and Chemistry and reviewed by PORC. No danger to the health and welfare of the general public was identified.

LER 77-18
9-04-77

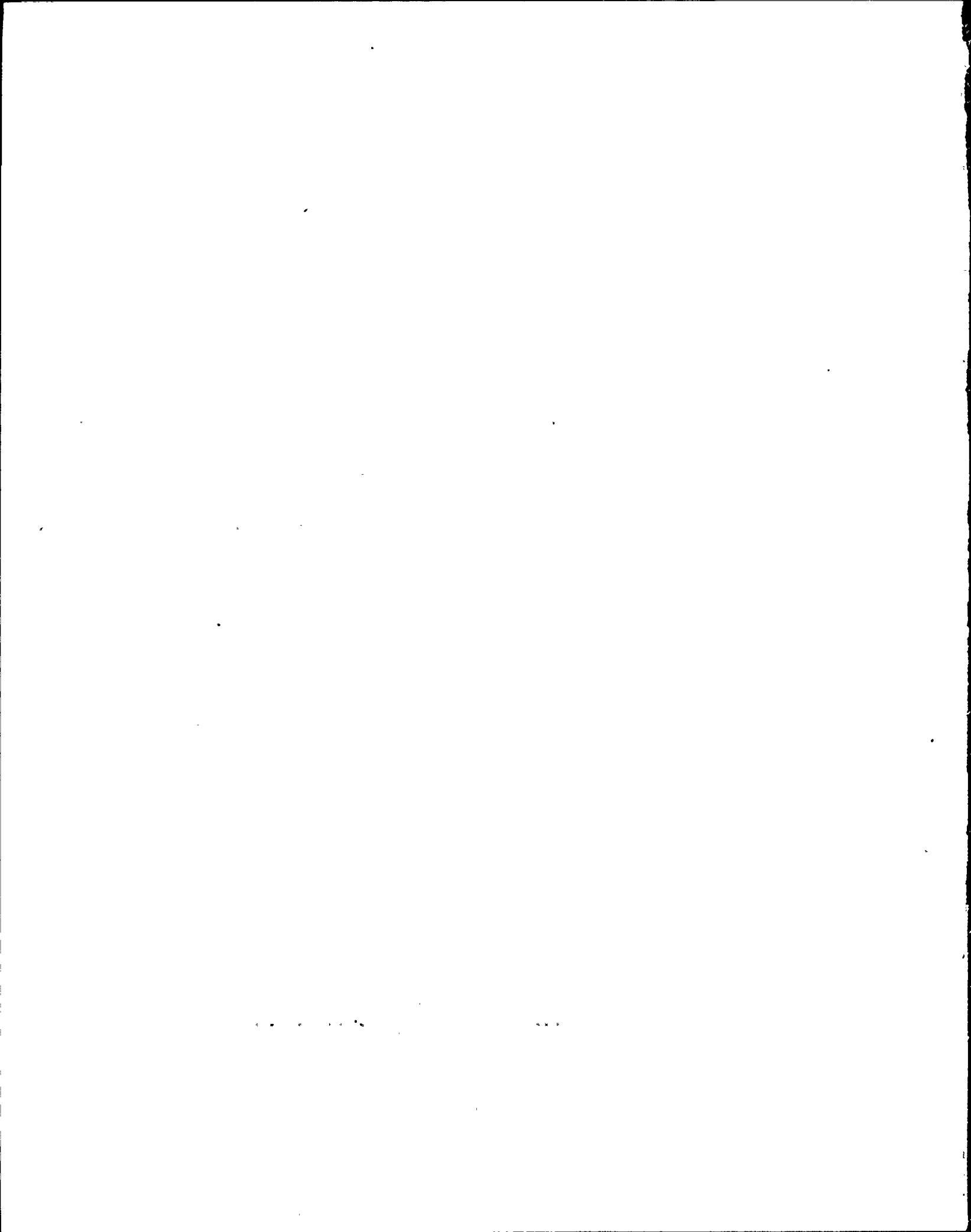
"B" Charging Pump Discharge Relief Piping Pinhole Leak

During an operator round, a pinhole leak was detected in the "B" Charging Pump Discharge Relief Valve Piping. With 2 other pumps operable, the "B" pump was isolated. A subsurface defect in the weld which was part of past repairs. No inadequacies have been noted in the welding or liquid penetrant testing procedures. The defect area was ground, cleaned, checked, rewelded and a liquid penetrant examination performed per approved maintenance and welding procedures. An inservice inspection for leakage was also performed. No danger to the health and welfare of the general public was identified.

LER 77-19
9-14-77

Bus 16 Circuit Breaker for 1B Emergency Generator

During a periodic test of the 1B Emergency Generator (EG), the bus 16 breaker from EG failed to close. The 1A EG was placed in service as required. The electricians inspected the breaker and found a secondary contact finger bent. A new secondary contact section was installed. The bent secondary contact finger is believed to be caused by improperly returning the breaker to service after being in the "rolled out" position. Training in proper performance of this operation will be given to the Auxiliary Operators. The breaker is a Westinghouse Model DB-75, 3000 amp. 600 volt. No danger to the health and welfare of the general public was identified.



LER 77-20
9-20-77

"C" Charging Pump Discharge Drain Line Pinhole Leak

During an operator round, a pinhole leak was detected in the "C" charging pump discharge drain line. With two other pumps operable, the "C" pump was isolated. The weld was cut out and replaced per approved maintenance and welding procedure. No danger to the health and welfare of the general public was identified.

LER 77-21
10-07-77

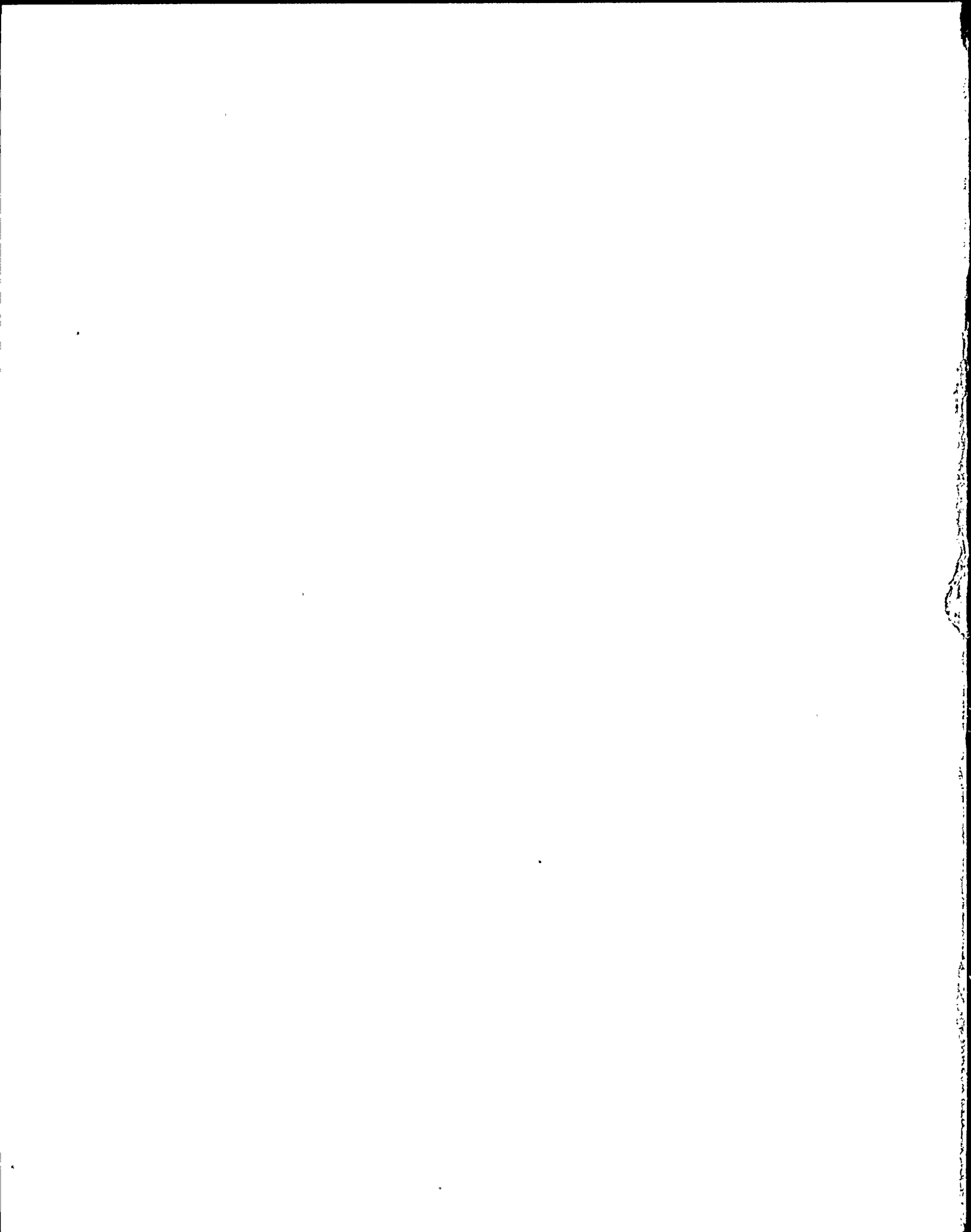
1A Governor Setting

While performing PT-12.1 Emergency Generator 1A, the 1A Emergency Generator (EG) could be loaded no more than 1450 kw. The 1A EG was declared inoperable, and the 1B EG was placed in service, constituting operation in a degraded mode permitted by a limiting condition for operation (T.S. 6.9.2.b (2)). Misreading governor hydraulic actuator speed setting and subsequent adjustment by unlicensed operator during pre-start check. Maintenance personnel corrected setting. PT-12.1 was completed satisfactorily. Setting verification was deleted from pre-start procedure. No danger to the health and welfare of the general public was identified.

LER 77-22
11-02-77

"B" Steam Generator (SG) Snubbers

During a check of the 1 gal. reservoir for the "B" Steam Generator (SG) Snubbers, the fluid reservoir was found empty. Visual examination of the units showed unit SGB-4 to be leaking fluid. During the unit cooldown, the snubber was removed, seals replaced and pressure tested and the snubber reinstalled. The other 7 units for "B" Steam Generator were operable. The snubbers are Anker-Holth Division, McDowell-Wellman Company rated at 532,00 lb. comp. and en. with disp. less than 1 in in 20 seconds. Tape and paint left on shaft by construction may have contributed to failure although no tape was noted in immediate area of seal. Tape and paint was removed from all other shafts on "B" Steam Generator units. Increased inspection and subsequent preventive maintenance plans on attached. No danger to the health and welfare of the general public was identified.



LER 77-23
11-16-77

Control Rod Urgent Failure Rod Stop

During steady load, a rod control urgent failure rod stop alarm lit and auto rod motion was stopped. Unit was shut down by boration, with rod insertion control available in bank mode. I & C Techs. examined the involved circuitry in LBD and 2BD power cabinets and the logic cabinet. Normal rod drop would not have been impeded upon trip signal. Random failure of 2 defective p. c. cards; Westinghouse slave cycler stationary decoder card, part #3359C62G02, and slave cycler logic card integrated circuit chip, Motorola Model MC6601 were replaced with qualified spares. All potentially faulty connections from logic cabinet to 2BD and cage were crimped or soldered. Rod system was successfully tested, and unit returned to operation. No danger to the health and welfare of the general public was identified.

LER 77-24
11-18-77

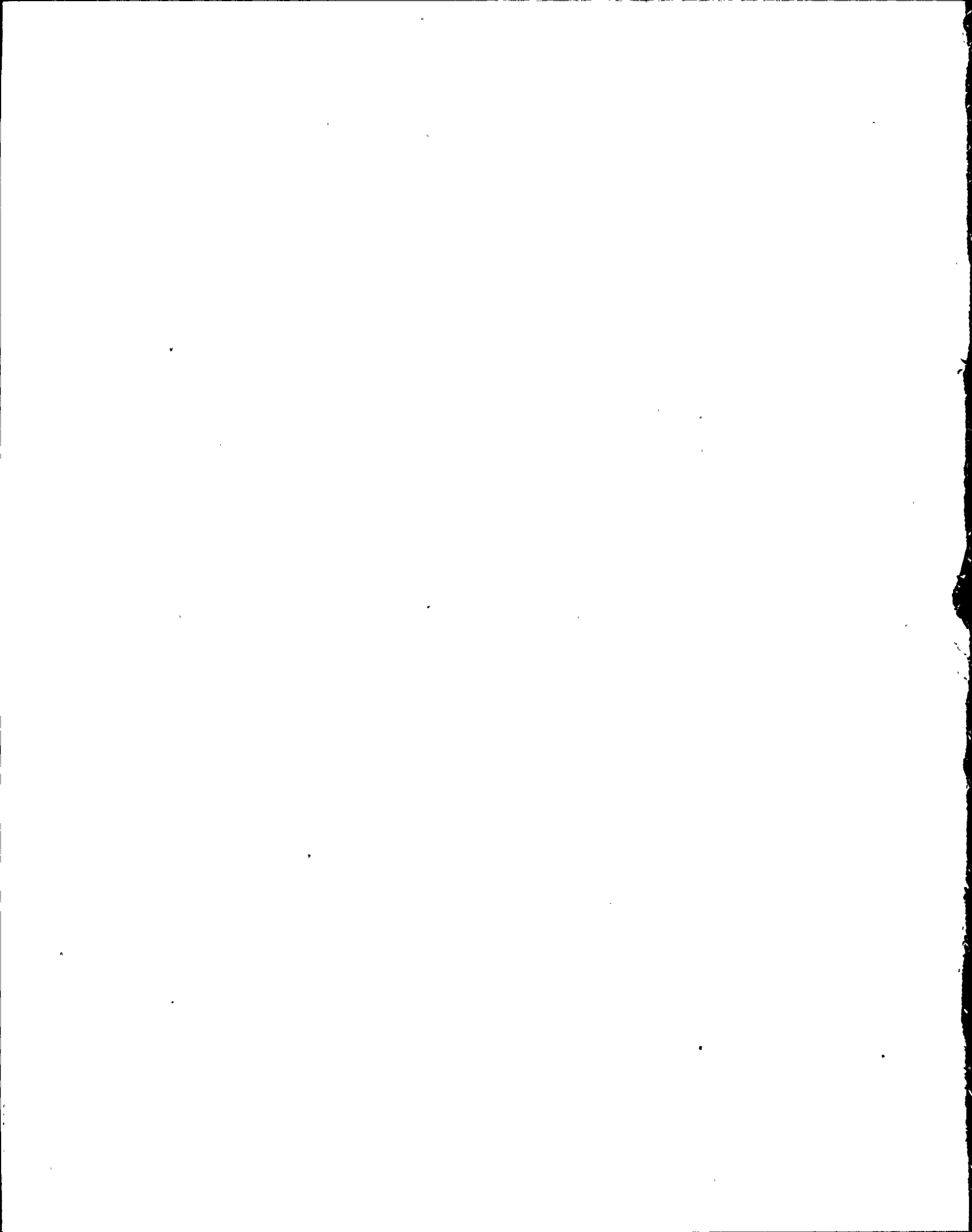
Control Rod Urgent Failure Rod Stop

During return to full load after rod control system maintenance, a rod control urgent failure rod stop alarm lit and auto rod motion was stopped. Unloading for unit shutdown was started by boration with rod insertion control in bank mode. All rods were capable of being tripped. When rod stop was cleared during investigation of LBD power cabinet unit unloading was stopped. Stationary regulation failure in LBD power cabinet apparently due to loose connectors. This was corrected when p. c. cards in LBD for B and D banks were swapped, and urgent failure condition cleared. They were returned to the original positions with no further problem. No danger to the health and welfare of the general public was identified.

LER 77-25
11-29-77

Non-Regenerative HX Outlet Piping Pinhole Leak

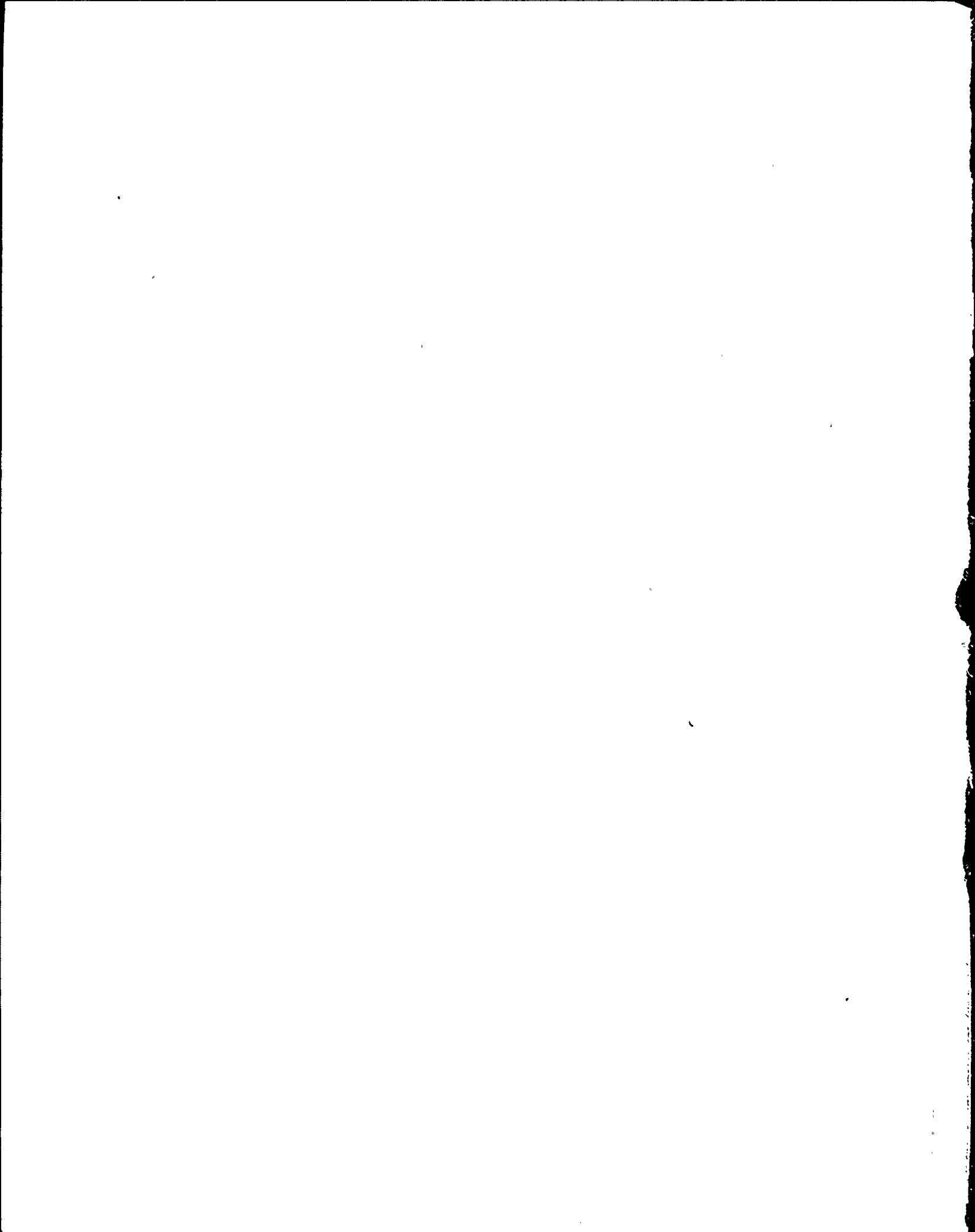
During increased surveillance due to increased activity noted on local continuous air monitor readings a small leak was found in nonregenerative HX outlet piping. No significant increase was noted on plant vent monitors. (T.S. 6.9.2.b.(4) Excess letdown was put in, and normal letdown isolated. Inclusion in a 2 in. socket weld during construction. Defect was ground out and weld was repaired per approved procedure, and dye penetrant exam was performed. As this is considered a random failure, no further action is required. No danger to the health and welfare of the general public was identified.



LER 77-26
12-02-77

RHR Pump Leak on Flow Orifice Inlet

During a RHR pump monthly test, a leak was noted on a flow orifice inlet tap seal welded threaded connection. Leak rate was estimated at 3 to 5 gpm at 125 psig. (T. S. 6.9.2.b.(4)). Unit was taken to hot shutdown at Superintendent's direction to isolate and repair leak as it was in nonredundant portion of return to RCS. A "Cold Lap" on seal weld applied by construction personnel allowed leakage when threaded connection began to leak. Defect was ground out and seal weld repaired and examined by dye penetrant exam. Hydro at 350 psig was performed successfully. This system is tested for leakage at 12 month intervals per Tech. Specs. No danger to the health and welfare of the general public was identified.



OTHER EVENTS OF INTEREST

FUEL PERFORMANCE

Primary system chemistry sampling from January 1, 1977 through December 31, 1977 showed no increases indicative of significant fuel cladding failure.

Periodic flux map data and incore program results indicated normal flux distribution and confirmed that hot channel factors were below technical specification limits.

Predicted and measured boron concentration at various cycle burn-up intervals were within 20 PPM boron, well below technical specification limits.

During this reporting period, the reactor was refueled. Cycle VI was terminated April 16, 1977. Thirty-two new fuel assemblies plus nine partially spent fuel assemblies from the spent fuel pit were loaded into the core. Cycle VII operation commenced May 21, 1977.

For major events of interest for the year 1977, see "Highlights" section of this report. Major events during 1977 include the following:

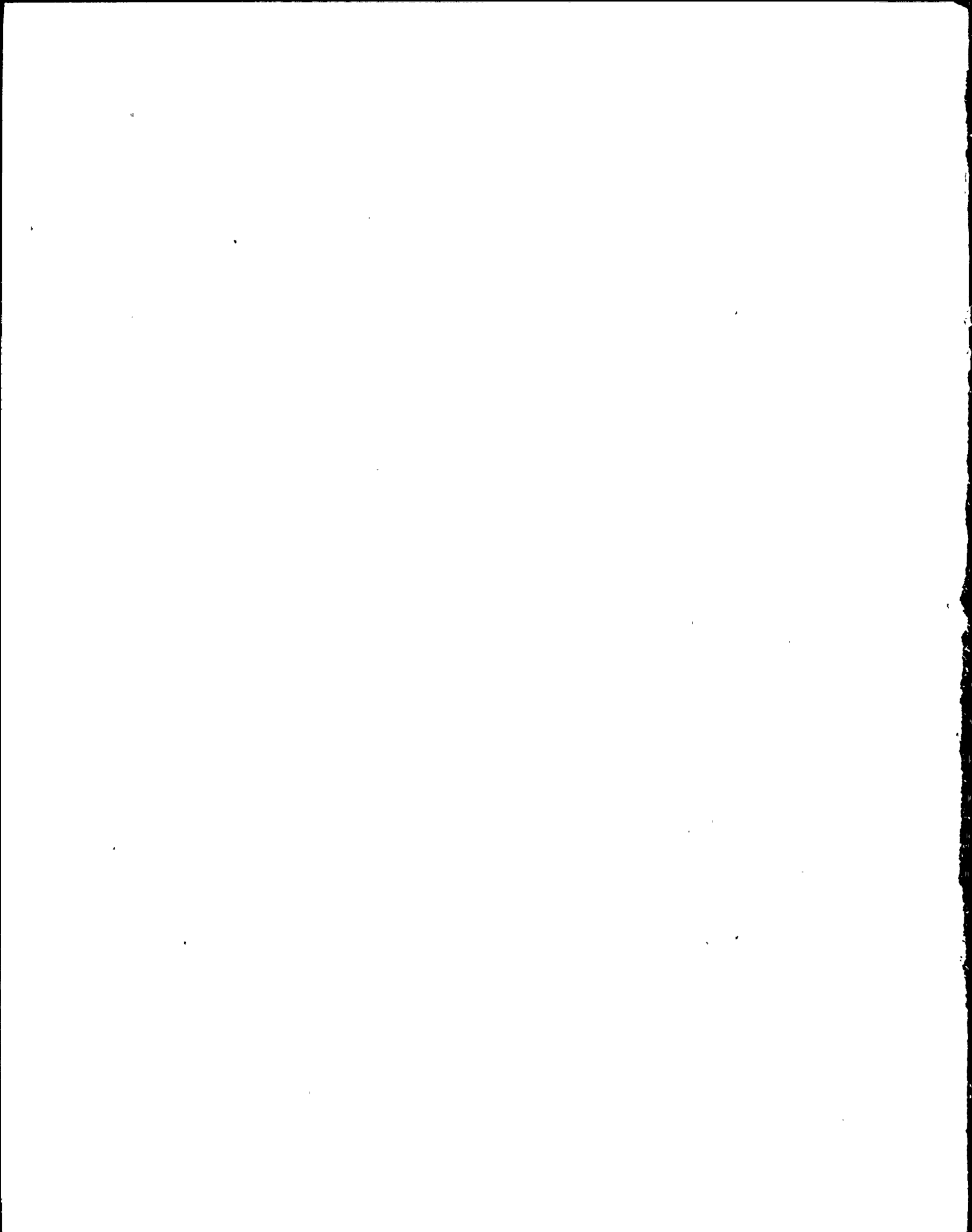
1. 5/23/77 - Inoperable Control Rods K-7 and L-8
2. 7/05/77 - "B" S/G Tube Leak
3. 8/04/77 - "A" S/G Two Inoperable Snubbers.
4. 11/04/77 - "B" S/G One Inoperable Snubber
5. 11/17/77 - Inoperable Control Rods
6. 12/03/77 - Leaking Pipe Tap - R.H.R. System

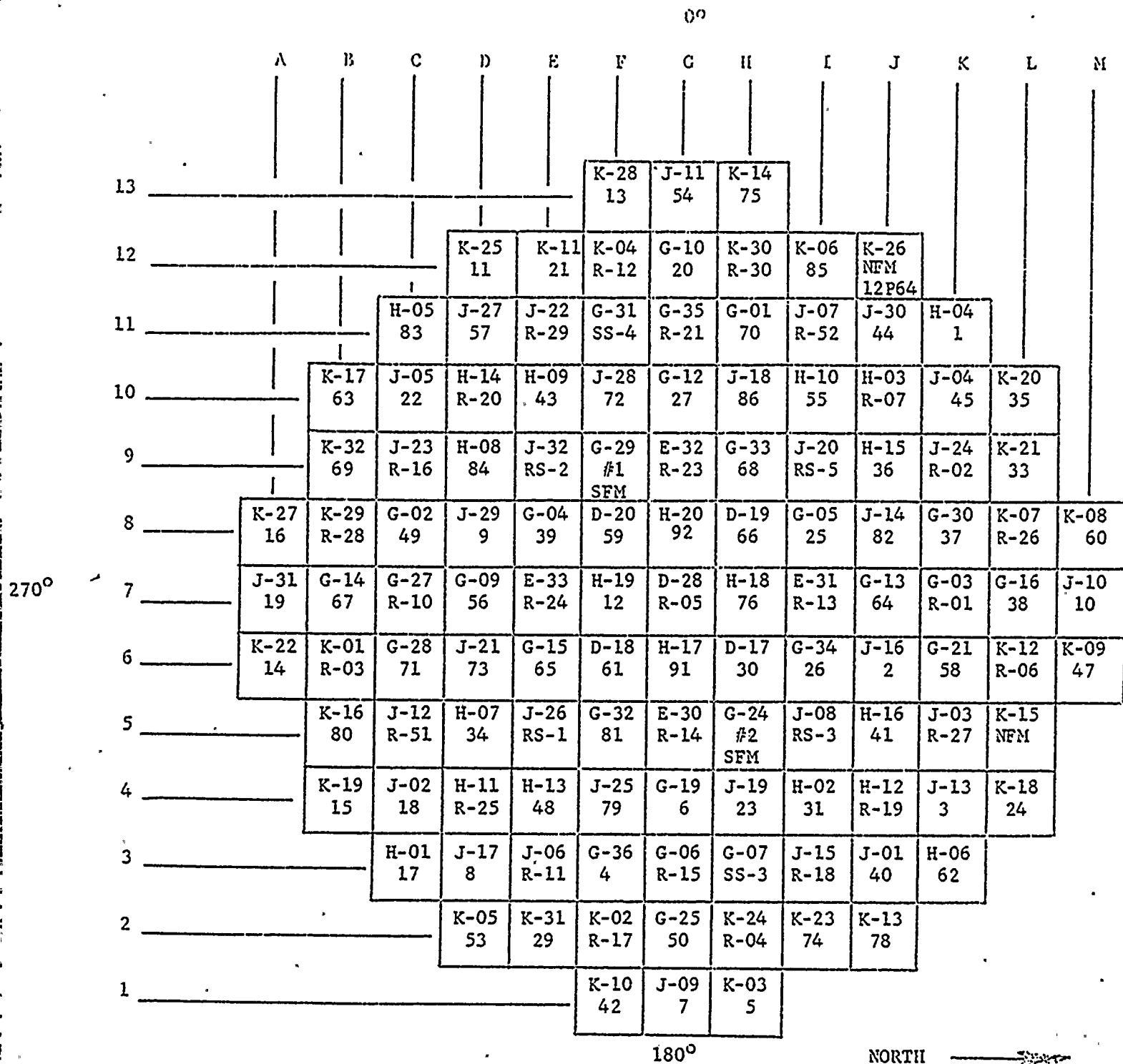
Eight fuel assemblies that were located in the \bar{z} baffle joint core region during cycle VI were transferred to the spent fuel pit for binocular observation. No unusual conditions were observed which would tend to indicate problems with the internals \bar{z} baffles' joint.

FUEL DATA

The following outline addresses topics in which the NRC has expressed interest:

	<u>Cycle VI</u>	<u>Cycle VII</u>
Fuel Vendor:	Westinghouse (117) Exxon Nuclear (2) Babcock & Wilcox (2)	Westinghouse (121)
Fuel Loading Data:	Core consists of 121 fuel assemblies. Each fuel assembly consists of 14x14 array of 179 fuel rods, 16 RCC guide thimbles and one instrument tube. Active fuel length 142 inches.	
Core Loading Map of Cycle VII:	See Figure 1	
Spent Fuel Pit Storage:	Licensed storage capacity = 595 Number of spent fuel assemblies in storage = 124	
Future Reloads:	Cycle VIII reload, 1978 refueling. 32 new fuel assemblies supplied by Exxon 3.1 w/o enrichment 94% TD, pellet density 142 inches active fuel length	





CYCLE: VII
RG&E GINNA STATION

XXX YYY

XXX = Fuel Assembly I.D.
 YYY :: R = RCC
 RS = Part Length Rod
 SS = Secondary Source
 No. = Flow Mixer
 SFM = Special Flow Mixer
 NFM = New Flow Mixer

STEAM GENERATOR INSPECTION

During the year 1977 Eddy Current testing was performed on two occasions. A regular program was completed in both Generators during the refueling outage in April. The plant was shutdown in July due to a leak detected in the B Generator and testing was performed in that Generator only at that time.

The scheduled inspection (See LER 77-03 for complete report) included 2003 tubes in the "A" Generator inlet at 400 KHZ for wall thickness degradation, 195 tubes at 25 KHZ for a sludge profile, 207 tubes at 3.5 KHZ to verify support integrity, and 185 tubes were examined over the U-bends (125 tubes at 400 KHZ and 60 tubes at 100 KHZ). A similar program was performed on the outlet side of the "A" Generator - a lesser number of tubes (286) being inspected at 400 KHZ and no U-bends were probed from the outlet side.

The inspection in the B Generator included 1525 tubes in the inlet side at 400 KHZ for wall thickness degradation, 195 tubes at 25 KHZ for sludge profile, 100 tubes at 3.5 KHZ to verify support plate integrity, and 185 tubes were examined over the U-bend. A similar program was performed in the outlet side with a lesser number (208) of tubes examined over the U-bends from the outlet side.

The tubes inspected at 400 KHZ are categorized as follows:

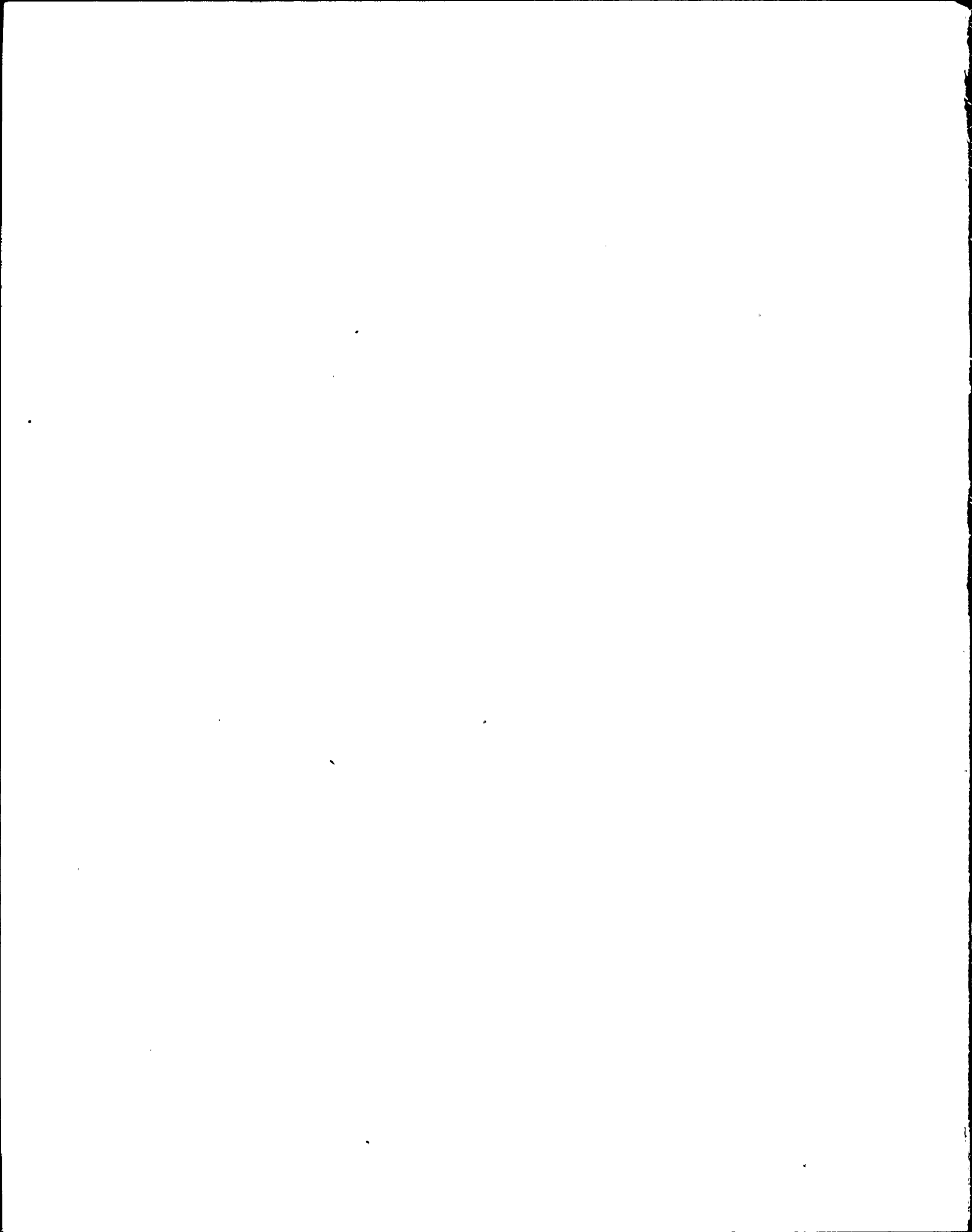
	A Generator		B Generator	
	inlet	outlet	inlet	outlet
Required sample	387	200	445	202
Examined first time	-	-	-	-
Baseline without E.C. indication	1166	155	785	177
Baseline with E.C. indication	837	113	740	91

Thirteen tubes in the A Generator showed defects greater than 40% and one tube in the B Generator.

Eddy Current test results
 Site: R.E. Ginna - Unit #1
 Test Frequency: 400 KHZ

Steam Generator A (inlet)
 Date: April 1977

Row	Column	Type of Indication	Location
17	26	O D Defect	2 inches above Tube Sheet
17	28	O D Defect	2 " " " "
18	29	O D Defect	2½ " " " "
19	47	O D Defect	2½ " " " "
19	48	O D Defect	2½ " " " "
20	30	O D Defect	2 " " " "
22	34	O D Defect	3½ " " " "
25	40	O D Defect	3 " " " "
26	44	O D Defect	1 " " " "
26	47	O D Defect	2 " " " "
27	49	O D Defect	1 " " " "
28	42	O D Defect	2 " " " "
29	49	O D Defect	1 " " " "



Eddy Current test results
Site: R. E. Ginna - Unit #1
Test Frequency: 400 KHZ

Steam Generator B (inlet)
Date: April 1977

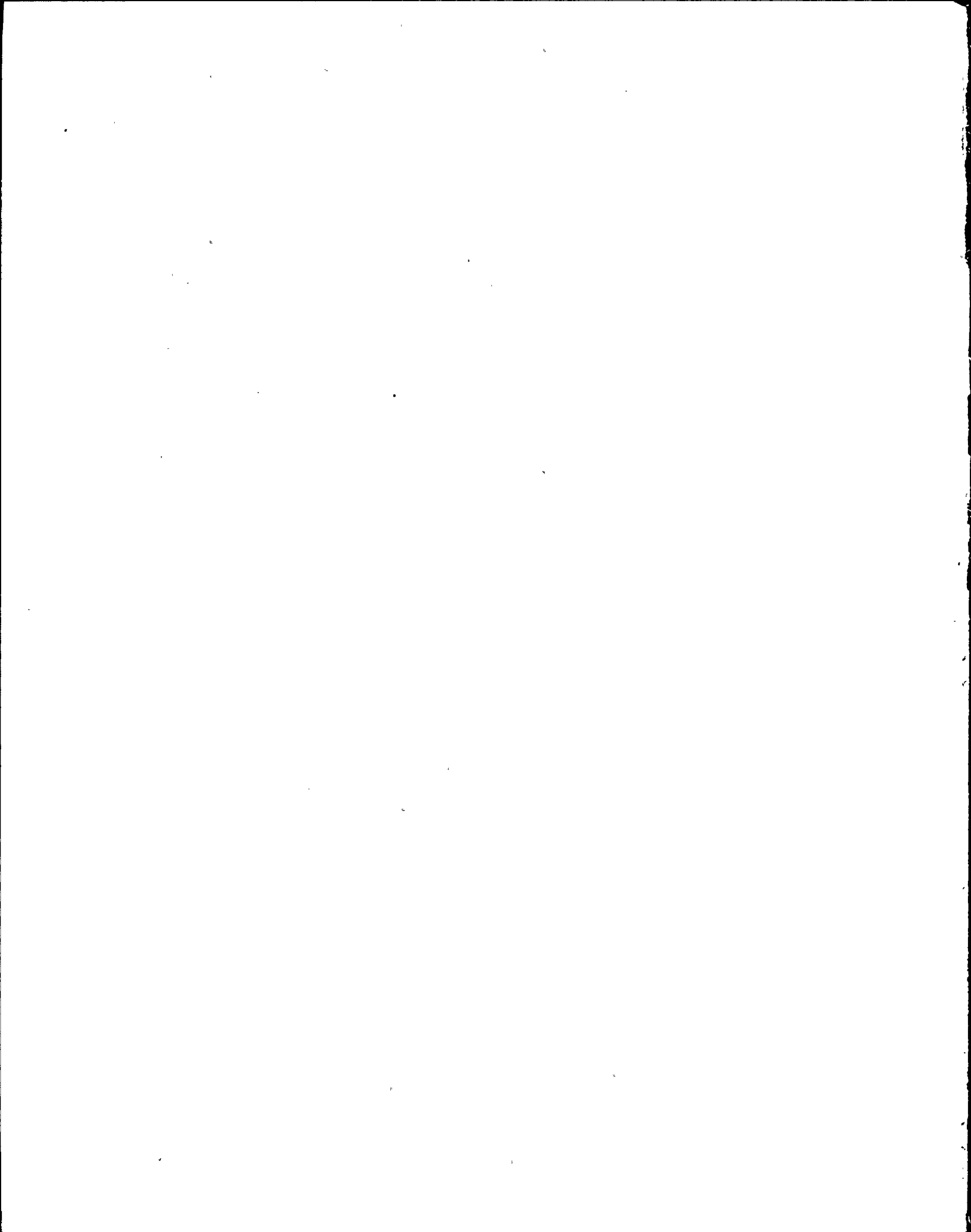
Row	Column	Type of Indication	Location
12	2	O D Defect	at top of U-bend

In July 1977 the plant was shutdown due to a primary to secondary side leak in the B Generator. The Tube failure was located in the inlet side at Row 45 Column 54 (by hydrostatic pressure) and verified by Eddy Current testing. Since this tube was located in a wedge area, all wedge areas were tested in addition to about 100 tubes in the area of the tube bundle that had shown some indications during the previous inspection. The leaking tube showed stress corrosion cracking from the inside surface of the tube. Four additional tubes in the same wedge area showed similar defects and these five tubes were plugged.

Eddy Current test results
Site: R. E. Ginna - Unit #1
Test Frequency: 400 KHZ

Steam Generator B (inlet)
Date: July 1977

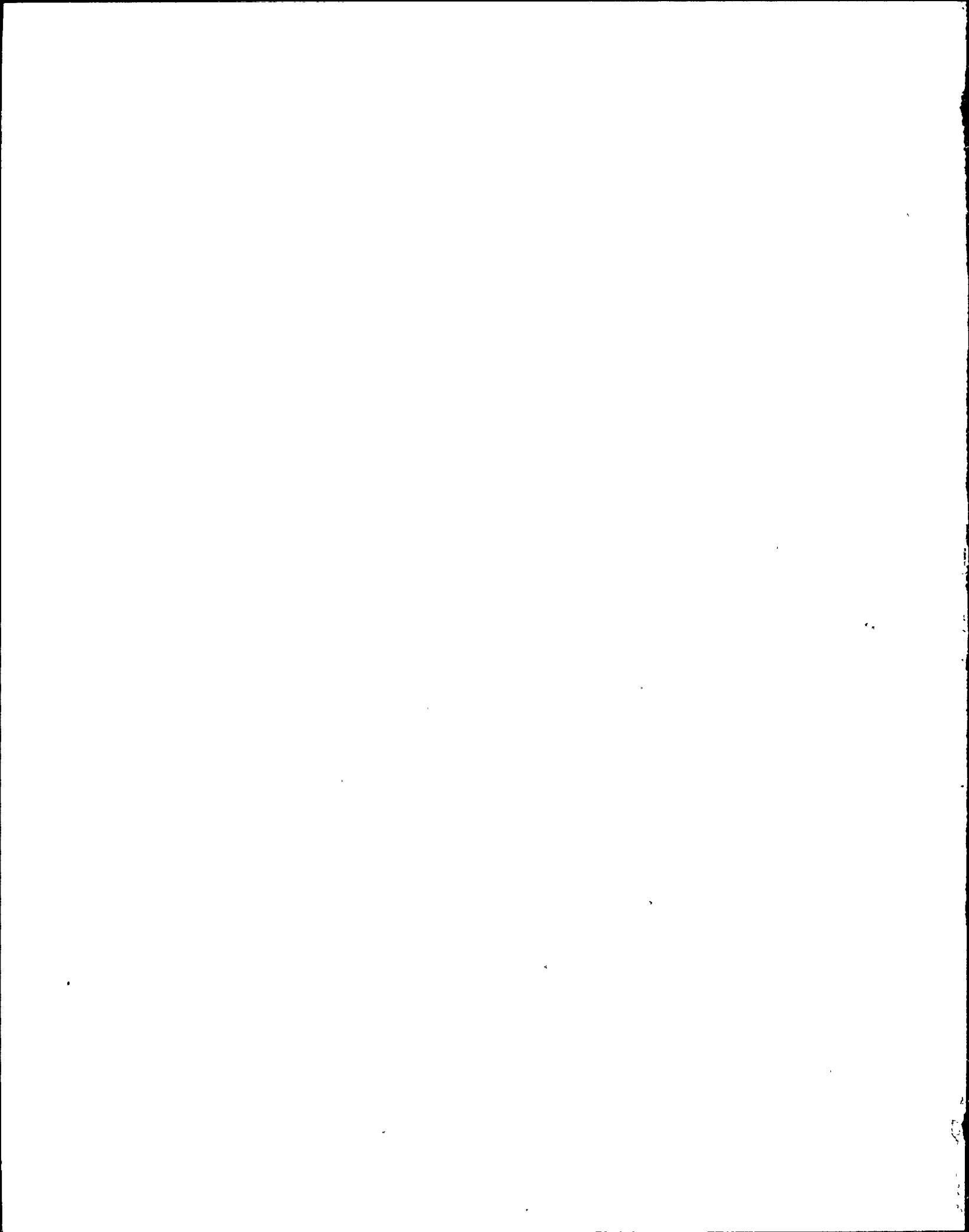
Row	Column	Type of Indication	Location
45	54	I D Defect	at the Tube Sheet
44	55	I D Defect	at the Tube Sheet
44	56	I D Defect	at the Tube Sheet
44	57	I D Defect	at the Tube Sheet
44	58	I D Defect	at the Tube Sheet



SUMMARY OF CONTAINMENT INTEGRATED LEAK RATE TESTS

The Reactor Containment Building Leakage Rate Test was not performed during 1977.

It is currently expected that this test will be conducted during the 1978 Refueling Outage.



DATA TABULATION

NET ELECTRICAL POWER GENERATION

	<u>YEAR TO DATE</u>	<u>CUMULATIVE</u>
Number of Hours Reactor Was Critical	7,569.11	54,038.62
Reactor Reserve Shutdown Hours	49.77	1,101.09*
Hours Generator on Line	7,490.75	52,518.38
Unit Reserve Shutdown Hours	0	8.5*
Gross Thermal Energy Generated (MWH)	11,081,808	68,437,042
Gross Electrical Energy Generated (MWH)	3,202,384	22,187,860
Net Electrical Energy Generated (MWH)	3,028,488	20,982,294
Reactor Service Factor	86%	76%
Reactor Availability Factor	87%	77%
Unit Service Factor	86%	74%
Unit Availability Factor	86%	74%
Unit Capacity Factor (using MDC)	74%	66%
Unit Capacity Factor (using design MWe)	74%	66%
Unit Forced Outage Rate	5%	11%

*Cumulative Data Commencing January 1, 1975

UNIT SHUTDOWNS AND POWER REDUCTIONS

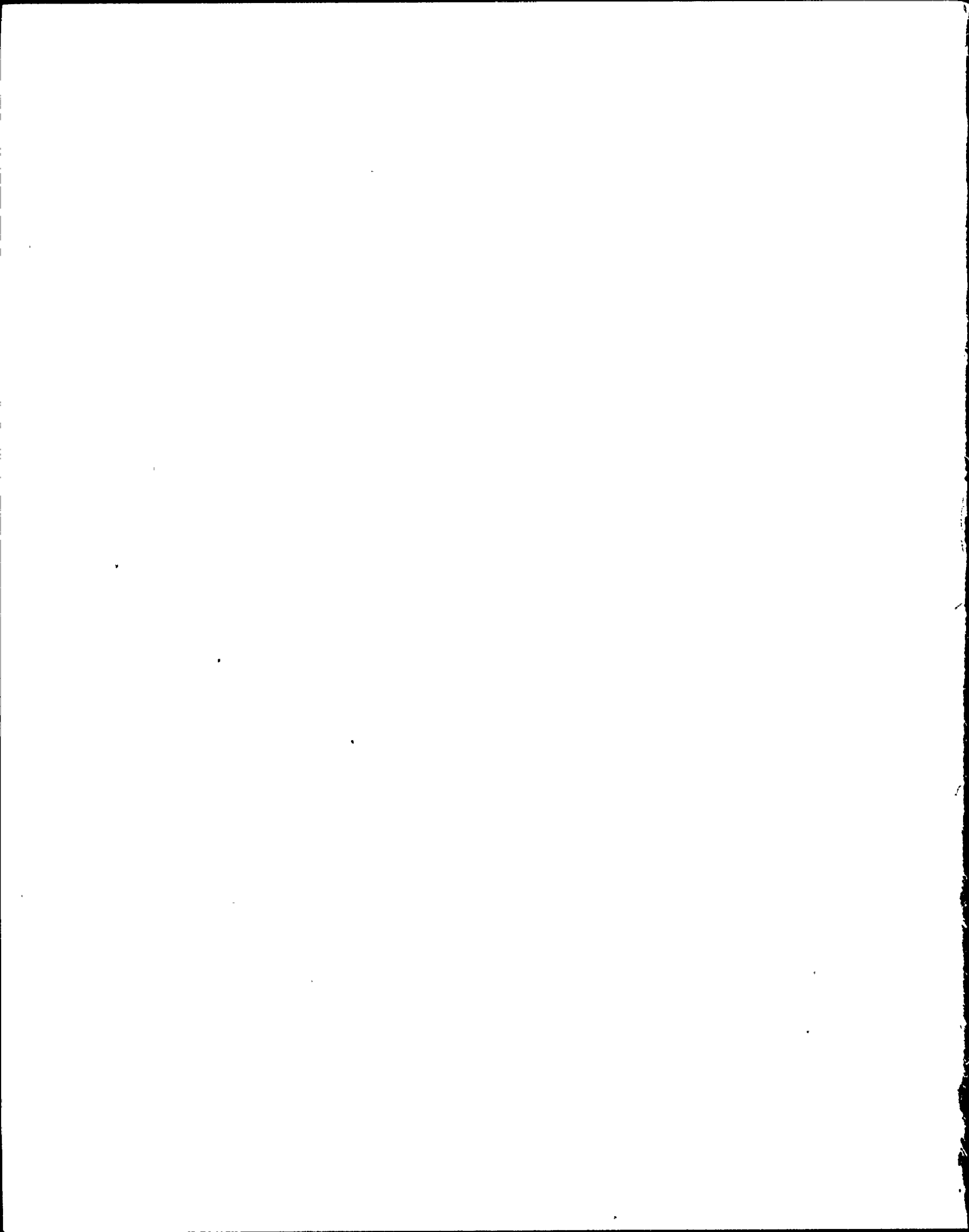
NO.	DATE	TYPE F-FORCED S-SCHEDULED	DURATION (HOURS)	REASON(1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER(2)	LICENSEE EVENT REPORTS	CORRECTIVE ACTIONS/COMMENTS
1	1-11-77	F	14.33	A	1		1-B-2 Condenser, tube leak. Also performed Turbine Main Stop Valves Test.
2	1-25-77	F	14.1	A	1		1-B-1 Condenser, tube leak.
3	2-13-77	S	8.92	B	1		Turbine Main Steam Stop Valves Test.
4	3-21-77	F	12.17	A	1		1-B-1 Condenser, tube leak. Also performed Turbine Main Steam Stop Valves Test.
5	4-15-77	S	360.25	C	1		Annual refueling and maintenance.
6	5-23-77	F	14.75	A	1		Inoperable Control Rods, K-7 and

(1) REASON:

A-Equipment Failure(Explain)
 B-Maint. or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training and
 License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

(2) METHOD:

1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)



UNIT SHUTDOWNS AND POWER REDUCTIONS

NO.	DATE	TYPE F-FORCED S-SCHEDULED	DURATION (HOURS)	REASON(1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER(2)	LICENSEE EVENT REPORTS	CORRECTIVE ACTIONS/COMMENTS
7	7-5-77	F	188	A	1		"B" Steam Generator - tube leak.
8	8-4-77	F	81	A	1		"A" Steam Generator - two inoperative snubbers.
9	11-4-77	F	48	A	1		"B" Steam Generator - one inoperable snubber.
10	11-17-77	F	30	A	1		Inoperable control rods.
11	12-3-77	F	17.5	A	1		Leaking pipe tap - residual heat removal system.

(1) REASON:

A-Equipment Failure(Explain)
 B-Maint. or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training and
 License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

(2) METHOD:

1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

Number of Personnel and Man-Rem Exposure by Work and Job Function

Work & Job Function	Number of Personnel (>100 mrem)			Total Man-Rem		
	Station Employees	Utility Employees	Contract Workers and Others	Station Employees	Utility Employees	Contract Workers & Others
Reactor Operations & Surveillance						
Maintenance Personnel	3	0	0	2.45	0	0
Operating Personnel	25	0	0	11.05	0	0
Health Physics Personnel	0	0	0	0	0	0
Supervisory Personnel	0	0	0	0	0	0
Engineering Personnel	0	0	0	0	0	0
Routine Maintenance						
Maintenance Personnel	36	150	83	30.59	83.40	63.51
Operating Personnel	0	0	0	0	0	0
Health Physics Personnel	8	1	7	6.53	0.27	2.48
Supervisory Personnel	2	3	0	1.73	1.50	0
Engineering Personnel	3	4	0	1.05	2.50	0
Inservice Inspection						
Maintenance Personnel	3	4	4	0.50	3.40	2.40
Operating Personnel	0	0	0	0	0	0
Health Physics Personnel	0	0	0	0	0	0
Supervisory Personnel	0	0	0	0	0	0
Engineering Personnel	0	0	0	0	0	0
Special Maintenance						
Maintenance Personnel	6	15	36	1.50	7.50	22.59
Operating Personnel	0	0	0	0	0	0
Health Physics Personnel	6	0	7	0.60	0	1.00
Supervisory Personnel	0	0	4	0	0	0.50
Engineering Personnel	0	0	2	0	0	0.50
Waste Processing						
Maintenance Personnel	15	0	12	8.00	0	15.00
Operating Personnel	10	0	0	9.50	0	0
Health Physics Personnel	6	0	5	3.50	0	1.20
Supervisory Personnel	0	0	0	0	0	0
Engineering Personnel	0	0	0	0	0	0
Refueling						
Maintenance Personnel	0	75	2	0	75.60	1.20
Operating Personnel	6	0	0	4.45	0	0
Health Physics Personnel	3	0	3	0.50	0	0.50
Supervisory Personnel	0	3	0	0	2.50	0
Engineering Personnel	1	0	0	1.79	0	0
TOTAL						
Maintenance Personnel	39	163	123	43.04	171.30	104.70
Operating Personnel	28	0	0	25.00	0	0
Health Physics Personnel	8	1	7	11.13	0.27	5.18
Supervisory Personnel	2	3	4	1.73	0.82	0.50
Engineering Personnel	5	7	2	2.84	5.20	0.50
Grand Total	82	174	136	83.74	177.67	110.88

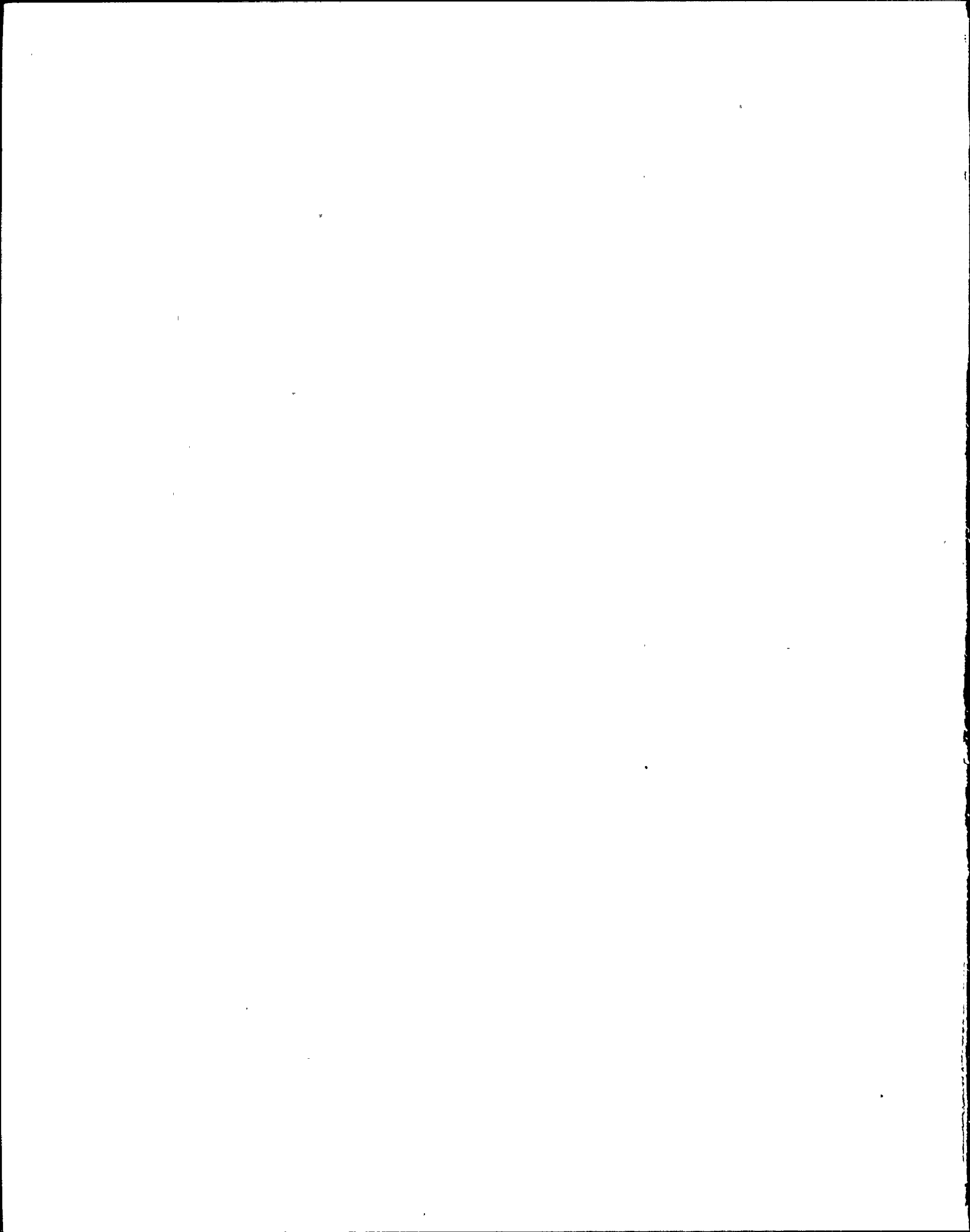
Paragraph 6.9.1b 2f

During 1977 there were 5 forced reductions in power. At no time were releases from the plant greater than 10% of the allowable annual limit.

During July 5 - 13 outage for a steam generator tube inspection 22 persons exceeded 500 mRem. The highest dose received by a person was 1400 mRem and other doses were distributed as follows:

500 - 750 mRem	6 persons
750 - 1000 mRem	7 persons
1000 - 1500 mRem	9 persons

There was no excessive exposure associated with the other 4 forced power reductions.



ANNUAL REPORT #13

JANUARY 1, 1977 THRU DECEMBER 31, 1977 INCLUSIVE

6.9.1.b.3

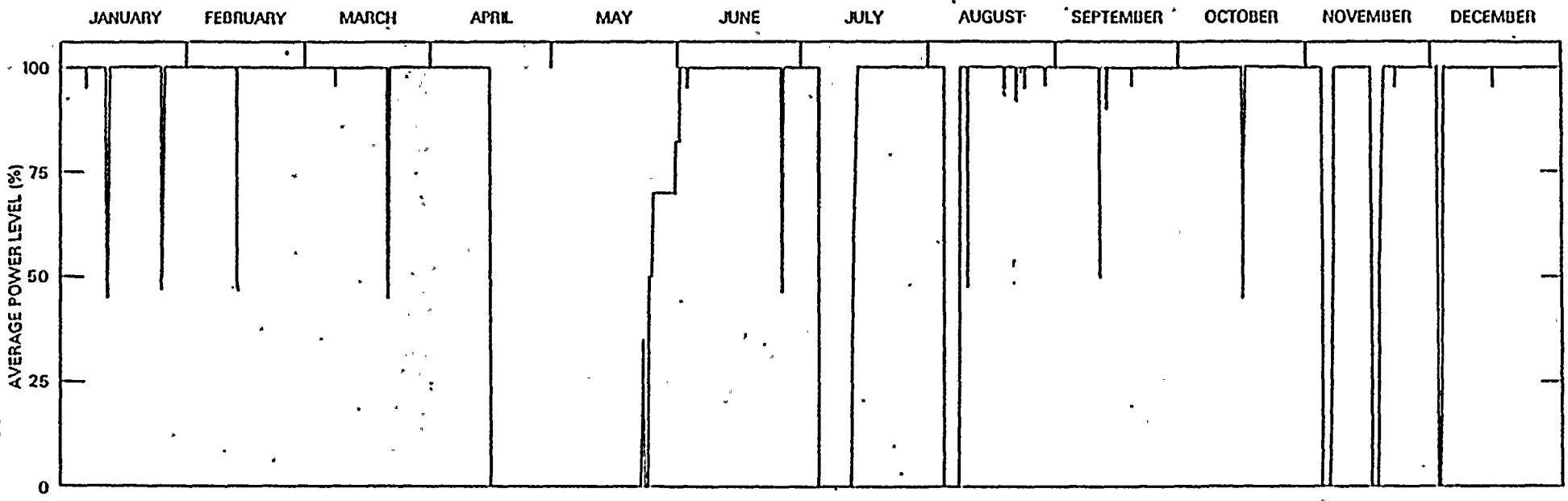
During 1977, Rochester Gas & Electric Corporation provided monitoring for 677 individuals. The table below summarizes the doses received by individuals during this year. The 2 individuals who exceeded 3.0 Rem were permanent Ginna Station employees and the maximum dose was 3.085 Rem.

<u>Dose Ranges (Rem)</u>	<u>Number of Individuals in each range</u>
No measurable Exposure	147
Measurable Exposure less than .100	112
.100 - .250	70
.250 - .500	79
.500 - .750	52
.750 - 1.000	48
1.000 - 1.500	99
1.500 - 2.000	46
2.000 - 3.000	22
3.000 - 4.000	2
4.000 +	0

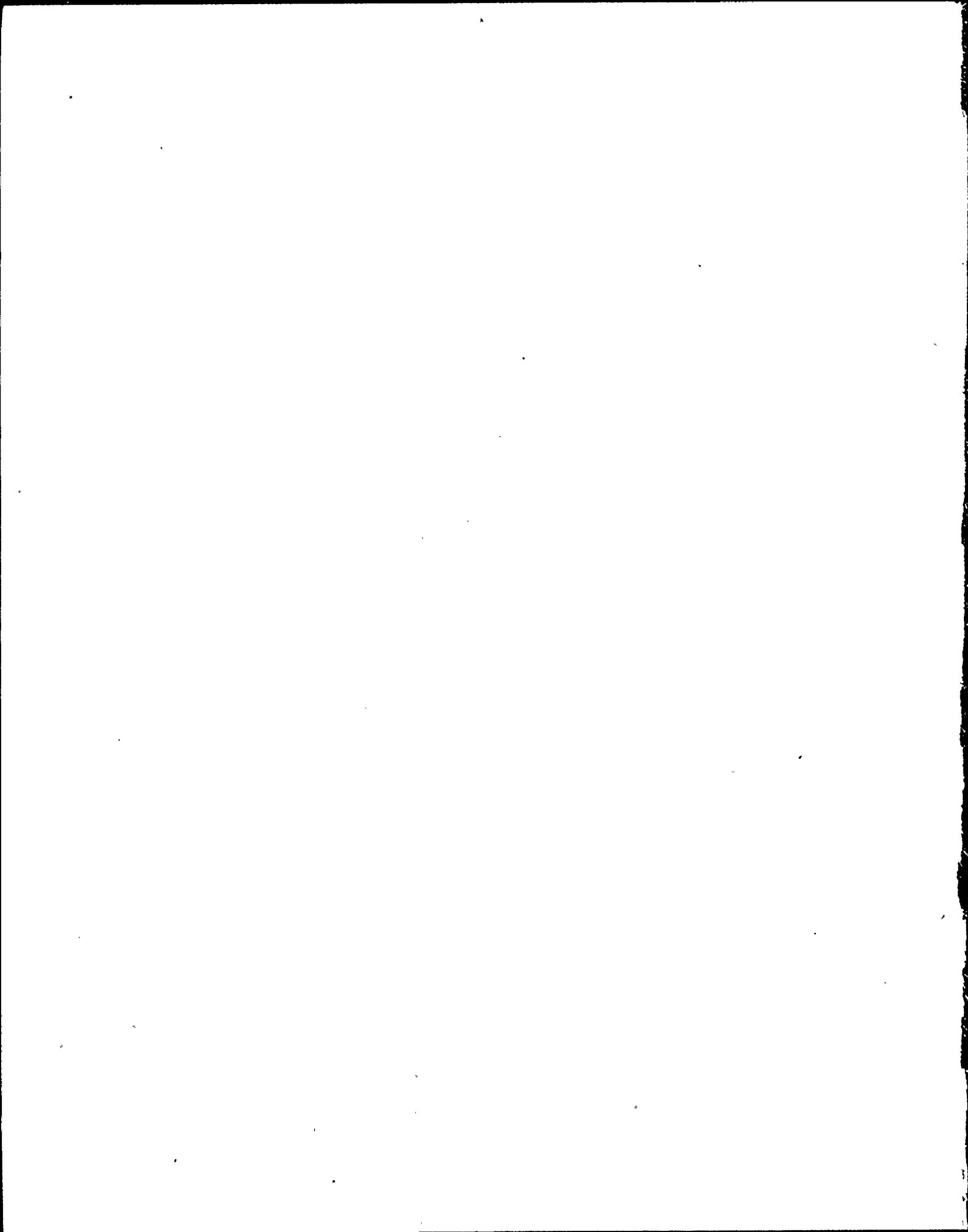
PLANT ORGANIZATION AND PERSONNEL

No changes in organization or personnel in 1977.

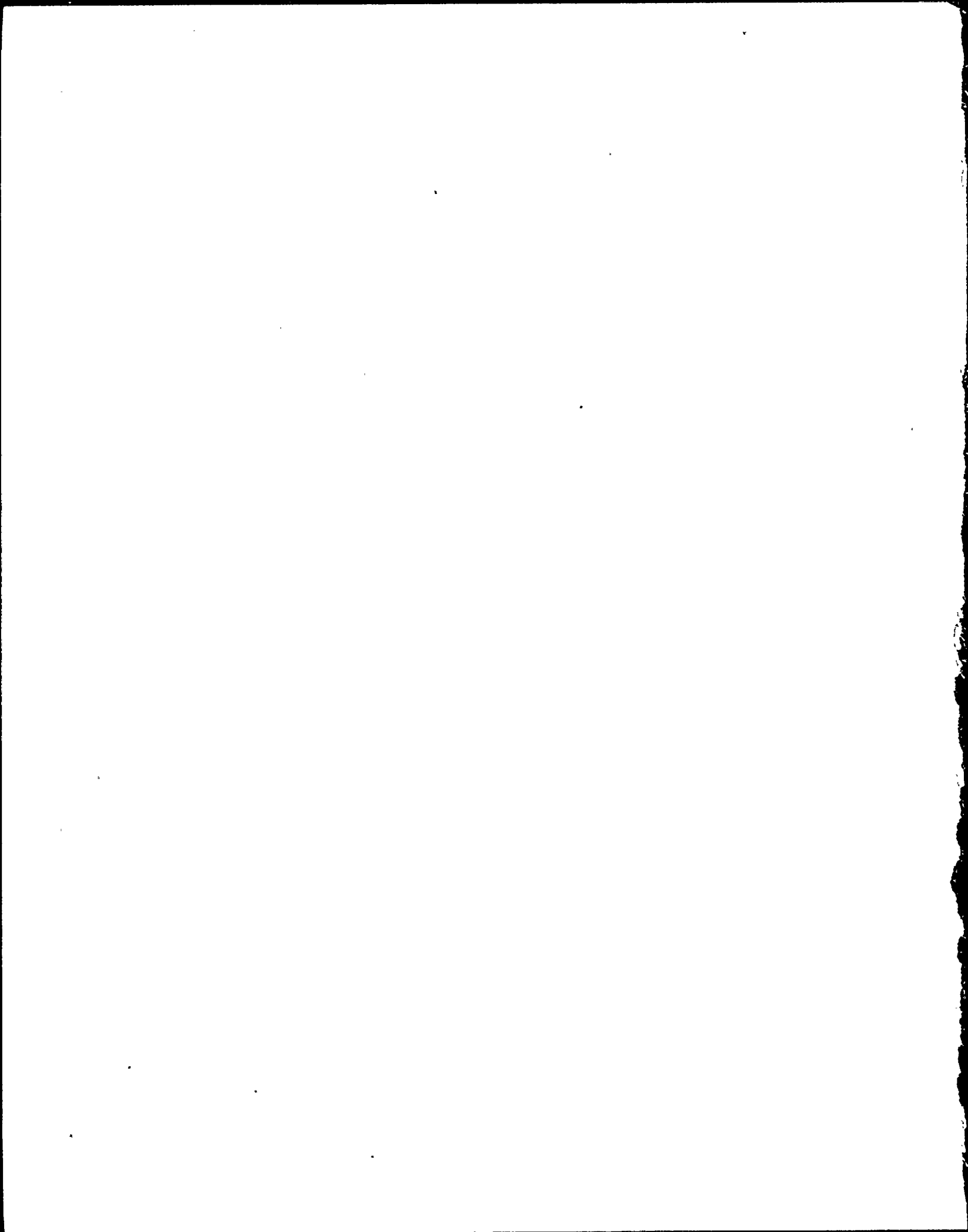
GINNA STATION
UNIT #1
Operating Histogram
1977



-88-



UNIQUE REPORTS



Review of Condenser Tube Maintenance

On January 11, 1977 load was reduced to 50% to allow inspection of 1B2 condenser as indications of leakage were present. One (1) leaking tube was located and plugged.

On January 25, 1977 load was again reduced for inspection of 1B1 condenser. One (1) tube was found to be leaking and was plugged.

On March 21, 1977 the 1B1 condenser was inspected for a leaking tube after preliminary tests gave indications of leakage in that condenser box. One (1) tube was located and plugged.

During the maintenance and refueling outage of April & May 1977 as many condenser tubes were Eddy Current tested as practical yet not to interfere with critical path items. No tubes which required plugging were found at that time. No further indications of tube leakage were noted in 1977.

Annual Report
1977

Primary Coolant

SG Blowdown

		Primary Coolant					SG Blowdown		
		Total Radioactivity $\mu\text{Ci/gm}$	Tritium $\mu\text{Ci/gm}$	Chloride ppm	Fluoride ppm	Oxygen ppb	Equiv. I-131 $\mu\text{Ci/gm}$	Boron ppm	I-131 $\mu\text{Ci/cc}$
January 1-31	Max.	29.02	.62	.10	<.01	<5	.39	480	<MDA
	Avg.	24.63	.57	<.05	<.01		.33	434	
	Min.	18.04	.53	<.05	<.01		.29	385	
February 1-28	Max.	25.19	.67	.07	<.01	<5	.46	381	<MDA
	Avg.	22.55	.65	<.05	<.01		.35	340	
	Min.	20.76	.62	<.05	<.01		.31	293	
March 1-31	Max.	25.36	.76	<.05	<.01	<5	.39	293	<MDA
	Avg.	22.76	.70	<.05	<.01		.31	248	
	Min.	18.23	.66	<.05	<.01		.26	200	
April 1-15	Max.	25.54	.78	.14	<.04	<5	2.00	2702	<MDA
	Avg.	15.32	.69	.07	<.01		0.78	1017	
	Min.	2.85	.53	<.05	<.01		0.03	121	
May 23-31	Max.	3.01	.31	.32	<.01	<5	.032	2278	<MDA
	Avg.	0.98	.30	.13	<.01		.012	1514	
	Min.	0.018	.29	<.05	<.01		.001	976	
June 1-30	Max.	2.99	.59	.11	<.01	<5	.164	952	<MDA
	Avg.	2.21	.50	<.05	<.01		.135	850	
	Min.	1.28	.40	<.05	<.01		.100	804	
July 1-5, 13-31	Max.	5.77	.49	.12	<.01	10	.150	1138	5.87 E-7
	Avg.	4.93	.47	.07	<.01	<5	.144	792	<MDA
	Min.	1.61	.44	<.05	<.01	<5	.138	729	<MDA
August 1-31	Max.	5.93	.67	.14	<.01	<5	.177	1196	7.12 E-7
	Avg.	4.62	.58	.05	<.01	<5	.150	743	<MDA
	Min.	1.17	.48	<.05	<.01	<5	.127	642	<MDA
September 1-30	Max.	5.82	.81	.09	<.02	<5	.170	640	2.69 E-7
	Avg.	5.04	.74	<.05	<.02	<5	.154	552	<MDA
	Min.	4.50	.68	<.05	<.02	<5	.141	544	<MDA
October 1-31	Max.	6.11	.88	.11	<.025	<5	.177	547	<MDA
	Avg.	5.20	.85	<.05	<.02	<5	.161	499	
	Min.	4.15	.81	<.05	<.02	<5	.153	450	
November 1-30	Max.	5.90	.99	.07	<.026	<5	.176	989	3.15 E-6
	Avg.	4.57	.77	.02	<.022	<5	.143	540	<MDA
	Min.	0.76	.68	<.01	<.022	<5	.048	349	<MDA
December 1-31	Max.	5.90	.83	<.05	<.02	<5	.260	470	<MDA
	Avg.	5.28	.76	<.05	<.02	<5	.187	326	
	Min.	4.38	.69	<.01	<.02	<5	.152	263	

MDA I¹³¹ = 2.1 E-7

LEAK TESTS ON SOURCES

Tech. Spec.
6.9.3 c

Leak tests on sources containing greater than 10 CFR 30.71 Schedule B quantities were made at 6 month intervals. No tests performed on the sources revealed the presence of .005 microcurie or more of removable contamination.

APPENDIX A
REPORT OF RADIOACTIVE EFFLUENTS
(ND - None Detected)

FACILITY Rochester Gas & Electric Corporation

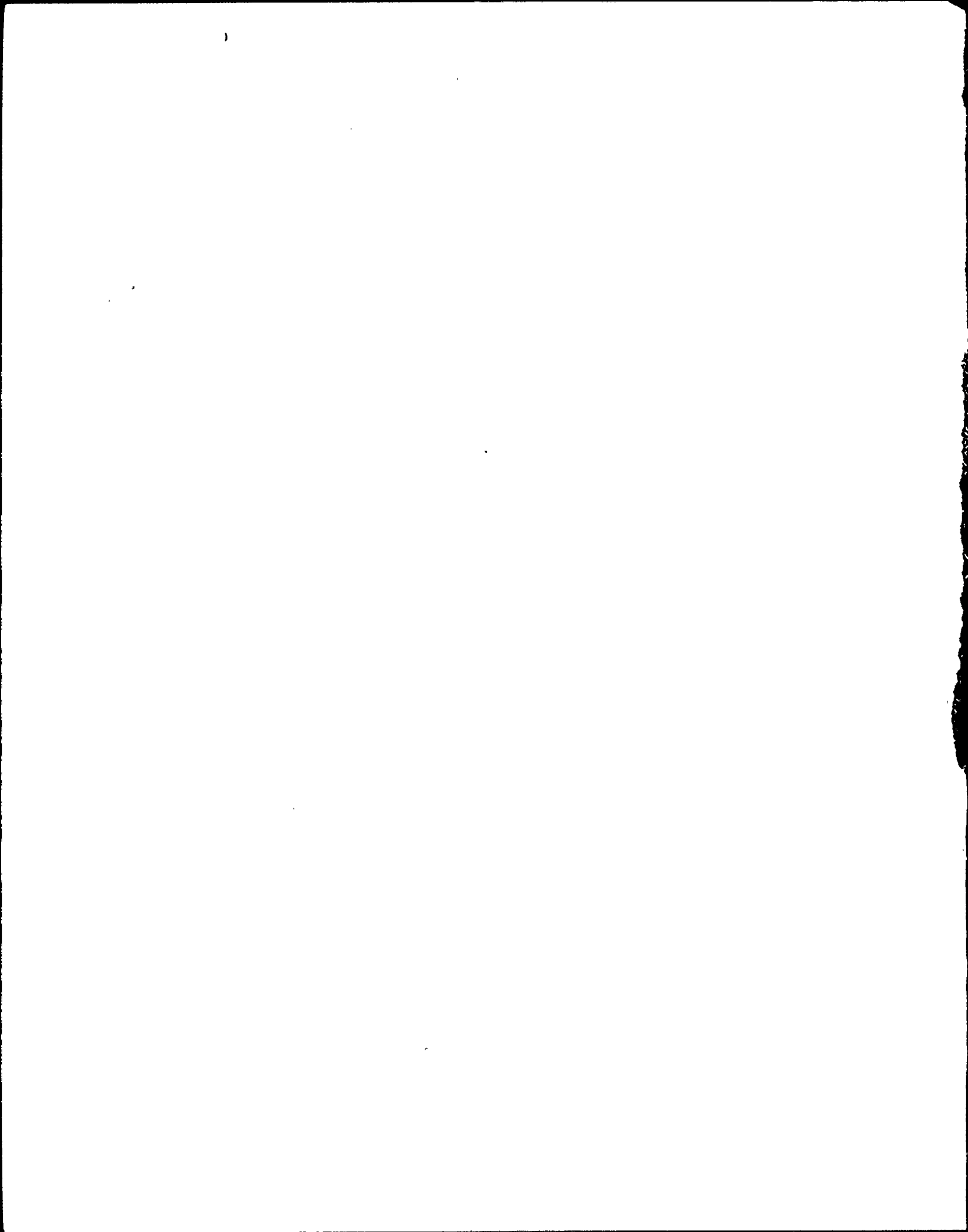
DOCKET

YEAR 1977

1. LIQUID RELEASES

	UNITS	JAN.	FEB.	MAR.	APR.	MAY	JUNE	JULY	AUG.	SEPT.	OCT.	NOV.	DEC.	TOTAL
1. GROSS RADIOACTIVITY (B.Y.)														
A) TOTAL RELEASE	CURIES	.00187	.00161	.00049	.02544	.02218	.00206	.00406	.00120	.00333	.00035	.00081	.00130	.06472
B) AVERAGE CONCENTRATION RELEASED	UCI/M.	3.44E-11	3.25E-11	1.01E-11	5.36E-10	5.06E-10	3.14E-11	6.01E-11	1.78E-11	5.09E-11	5.18E-12	1.33E-11	2.0E-11	9.00E-11
C) MAXIMUM CONCENTRATION RELEASED	UCI/M.	3.61E-9	4.51E-9	2.28E-9	9.50E-9	1.08E-8	3.21E-9	4.44E-9	3.37E-9	8.78E-9	1.24E-9	1.74E-9	2.51E-9	1.58E-8
2. ITRITIUM														
A) TOTAL RELEASE	CURIES	3.66	.0015	12.2	14.8	31.15	14.6	4.80	3.39	4.24	4.23	6.59	19.408	112.07
B) AVERAGE CONCENTRATION RELEASED	UCI/M.	6.73E-8	3.02E-11	2.03E-7	3.12E-7	7.10E-7	2.23E-7	7.10E-8	5.00E-8	6.59E-8	6.25E-8	1.01E-7	2.97E-7	1.66E-7
3. DISSOLVED MOBILE GASES														
A) TOTAL RELEASE	CURIES	N.D.	N.D.	N.D.	N.D.	N.D.	N.D.	N.D.	N.D.	ND	ND	ND	ND	ND
B) AVERAGE CONCENTRATION RELEASED	UCI/M.													
4. GROSS ALPHA RADIOACTIVITY				N.D.										
A) TOTAL RELEASE	CURIES	1.21E-6	1.2E-7		1.52E-5	1.92E-8	1.89E-5	1.00E-6	N.D.	4.12E-6	ND	ND	ND	4.057E-5
B) AVERAGE CONCENTRATION RELEASED	UCI/M.	2.22E-14	2.41E-15		3.21E-13	4.4E-13	2.89E-13	1.48E-14		6.09E-14				5.65E-14
5. VOLUME OF LIQUID WASTE TO DISCHARGE CANAL	LITERS	1.06E6	8.47E5	9.06E5	8.58E5	1.10E6	1.05E5	2.47E6	6.51E4	9.05E4	6.13E4	1.81E5	1.30E5	2.86E6
6. VOLUME OF DILUTION WATER	LITERS	5.44E10	4.98E10	6.02E10	4.75E10	4.38E10	6.54E10	6.76E10	6.76E10	6.54E10	6.76E10	6.39E10	6.53E10	7.18E11
7. ISOTOPES RELEASED	CURIES													
HA-24 (Steam Gen. carry-over test)		N.D.	.00011	N.D.	N.D.	N.D.	N.D.	N.D.	N.D.	ND	ND	ND	ND	.00011
SM 89														
I-131		N.D.	N.D.	.000004	.00108	.00138	.00001	.00008	.00002	ND	.000009	ND	.000007	.00039
CE-141		N.D.	.00002	N.D.	.00015	.00018	N.D.	N.D.	N.D.	ND	ND	ND	ND	.00035
RIP-103		N.D.	.00001	.00001	.00091	.00182	.00008	.00025	.000004	.00003	.000008	ND	ND	.00312
CS-137		.00058	.00057	.00029	.00804	.00416	.00082	.00132	.00063	.00145	.00015	.00053	.00085	.01942
CS-134		.00016	.00011	.00006	.00271	.00109	.00023	.00037	.00017	.00030	.00002	.00015	.00033	.00578
CO-60		.00094	.00074	.00008	.00757	.00625	.00048	.00109	.00027	.00072	.00008	.000096	.00008	.01840
CO-58		.00005	.00001	.00001	.00231	.00187	.00024	.00049	.00005	.00012	.00001	ND	.00002	.00228
CA-51		N.D.	N.D.	N.D.	N.D.	N.D.	N.D.	N.D.	N.D.	ND	ND	ND	ND	ND
MI-54		.00005	.00005	.000002	.00075	.00017	.00002	.00005	.00002	.00007	ND	ND	ND	.00118
NB-95		N.D.	N.D.	.00002	.00039	.00139	.00005	.00010	.00002	.00003	ND	ND	.00003	.00220
AG-110M		N.D.	N.D.	N.D.	.00004	.00022	.00001	.00005	N.D.	ND	ND	ND	.00002	.00035
Co-Pr 144		.00009	N.D.	N.D.	.00051	.00066	.00005	.00016	.00001	.00001	.00005	.000046	.00001	.00160
8. PERCENT OF TECHNICAL SPECIFICATION LIMIT FOR TOTAL ACTIVITY RELEASED	%	<.001	<.001	<.001	.009	.012	<.001	<.001	<.001	<.001	<.001	<.001	.0001	.002
ZR-95					.00069	.00098	.00002	.00011	.00001	ND	.000002	ND	ND	.00181
Ba/Li-140											.00002	7.E-7	ND	.00002

-93-



Rochester Gas & Electric Corporation

II AIRBORNE RELEASES

1977

	UNITS	JAN.	FEB.	MAR.	APR.	MAY	JUNE	JULY	AUG.	SEPT.	OCT.	NOV.	DEC.	TOTAL
1. TOTAL MOBILE GASES	CURIES	226.65	255.94	612.76	1147.82	214.32	81.74	187.17	143.91	73.29	5.28	204.31	50.37	1203.56
2. TOTAL HALOGENS	CURIES	4.57E-4	1.88E-4	3.73E-4	1.54E-2	8.27E-3	4.19E-4	2.68E-4	1.63E-4	1.67E-4	1.51E-4	1.32 E-4	6.95 E-5	2.625 E-2
3. TOTAL PARTICULATE GROSS RADIOACTIVITY (B.V)	CURIES	7.38E-6	1.48E-5	4.11E-6	2.3 E-5	5.7E-6	2.3E-6	7.2E-6	1.85E-6	4.2E-7	8.92E-7	8.7 E-7	1.76 E-6	7.028 E-5
4. TOTAL TRITIUM	CURIES	1.37	.293	2.69	3.20	21.94	3.15	5.747	3.725	2.76	2.236	2.097	1.27	50.074
5. TOTAL PARTICULATE GROSS ALPHA RADIOACTIVITY		6.0E-8	2.7E-8	1.3E-8	8.58E-8	4.98E-8	ND	ND	ND	3.5E-8	1.5E-8	2.3 E-8	5.2 E-8	3.606 E-7
6. MAXIMUM MOBILE GAS RELEASE RATE	uCi/sec	8500	251	1120	12000.	3910	1770	3000	2950.	3090	1910	3630	41.7	12,000
7. PERCENT OF APPLICABLE LIMIT FOR:														
A. MOBILE GASES	%	.160	.203	.439	.751	.160	.062	.119	.091	.048	.005	.136	.033	.181
B. HALOGENS	%	.455	.397	.366	20.28	10.62	.466	.367	.167	.122	.106	.122	.055	2.77
C. PARTICULATES	%	<.001	.002	.001	.005	<.001	<.001	<.001	<.001	<.001	<.001	.007	.025	.001
B. ISOTOPE RELEASES (I)	CURIES													
PARTICULATES														
CS-137		3.61E-6	7.08E-6	2.52E-6	2.18E-6	N.D.	3.69E-7	6.28E-8	9.05E-7	3.2E-7	ND	ND	3.45 E-7	1.782 E-5
MA-LA-140														
CO-60		7.89E-7	1.36E-6	1.06E-6	3.32E-6	8.14E-6	4.68E-7	1.94E-6	1.96E-7	ND	ND	1.24 E-5	7.54E-8	1.225 E-5
CS-134		6.89E-7	1.11E-6	5.57E-7	2.00E-6	N.D.	ND	1.76E-7	9.92E-8	9.8E-8	ND	ND	3.28E-6	8.002 E-6
SR-89														
CO-58		N.D.	5.03E-8	N.D.	2.82E-6	6.44E-7	ND	1.38E-6	ND	ND	ND	ND	1.65 E-8	4.911 E-6
HALOGENS														
I-131		3.55E-4	2.79E-4	2.84E-4	1.52E-2	8.25E-3	3.5E-4	2.53E-4	1.30E-4	9.15E-5	8.2E-5	9.21 E-5	4.26 E-5	2.541 E-2
I-133		1.02E-4	1.09E-4	8.94E-5	2.3E-4	2.2E-5	6.9E-5	1.83E-5	3.30E-5	7.54E-5	5.9E-5	3.92 E-5	2.69 E-5	8.732 E-4
I-135														
GASES														
AR-85				36.5	6.13	3.80	1.77	26.815	47.007	.089	733	.491	41.88	168.015
XE-133		211.00	237.36	531.00	1122.62	204.31	72.10	157.635	24.377	72.034	4.150	199.408	4.89	2220.784
XE-133M		.32	N.D.	N.D.	.015	N.D.	ND	.092	ND	.0004	ND	ND	ND	.427
XE-133H				.06	.157	1.13	.06	.182	1.266	.027	ND	.846	.033	3.761
AR-85M		.41	1.98	4.85	1.105	1.02	3.00	.489	ND	ND	175	.604	.747	14.379
XE-133M														
XE-133H														
XE-133M														
XE-133H														
AR-41		14.92	16.60	40.68	7.79	4.06	4.81	1.956	1.358	.339	.218	2.962	.855	96.548
OTHERS AS APPROPRIATE (SPECIFY)														
CR-51		N.D.	7.14E-8	1.00E-7	N.D.	N.D.	ND	4.6E-9	ND	ND	ND	ND	ND	1.76 E-7
CR-144		N.D.	4.15E-7	N.D.	7.09E-7	1.05E-7	4.94E-7	5.13E-7	ND	ND	3.46E-7	4.26 E-7	2.82 E-9	1.088 E-6
ZR-95				1.07E-6	1.28E-7	3.10E-7	6.42E-7	3.41E-7	ND	ND	ND	ND	ND	2.685 E-6
NB-92				2.05E-7	1.45E-6	7.38E-7	5.02E-7	8.11E-7	ND	ND	1.37E-7	ND	2.97 E-9	1.846 E-6
CR-51					3.37E-6	5.10E-7	ND	1.11E-6	6.59E-8	ND	ND	ND	ND	5.056 E-6
Ru-103					1.47E-6	3.52E-7	1.42E-7	4.77E-7	1.16E-7	ND	ND	ND	ND	2.557 E-6
Fe-59					2.09E-8	N.D.	ND	ND	ND	ND	ND	ND	ND	2.09 E-8
Hf-54								1.25E-7	8.68E-8	ND	ND	ND	ND	2.12 E-7
Np-239								7.7E-9	ND	ND	ND	ND	ND	7.7 E-9
Tc-132											4.1E-7	6.25 E-8		4.72 E-7
Ag-110a													2.08E-8	2.08 E-8

-76-

SOLID WASTE

JANUARY 1, 1977 through JUNE 30, 1977

During the six month period, 10,306 cubic feet of solid waste containing 676 curies of radioactive material was shipped offsite for burial. All the solid waste was sent to the Barnwell, South Carolina burial ground operated by Chem-Nuclear Systems, Inc.

<u>Date Shipped</u>	<u>Curies</u>	<u>Cubic Feet</u>
11 January	0.045	1904*
23 February	0.105	1904*
10 March	92.	60
15 March	0.062	1352*
24 March	92.	60
29 March	3.90	368
4 April	2.66	790
11 April	0.118	640
13 April	460.	60
2 May	2.66	556
6 May	6.61	650
12 May	1.84	612
19 May	5.18	558
26 May	4.76	360
3 June	<u>3.63</u>	<u>432</u>
January - June Total	676.	10,306

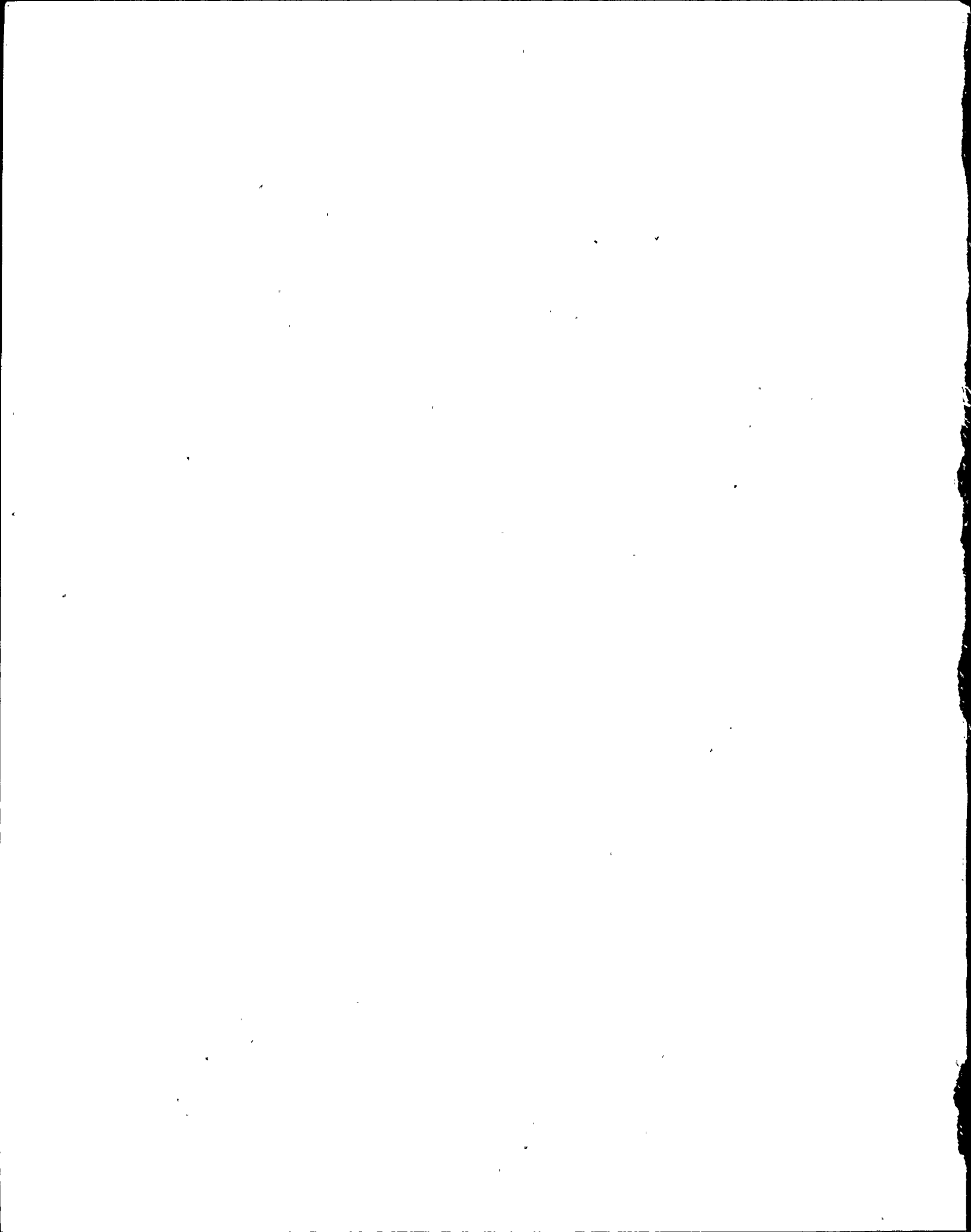
*Spent Fuel Pit Racks (See SM -75-57)

SOLID WASTE

JULY 1, 1977 THROUGH DECEMBER 31, 1977

During this six month period, 2032 cubic feet of solid waste containing 14.21 curies of radioactive material was shipped off-site for burial. All five shipments went to the Barnwell, South Carolina burial ground operated by Chem-Nuclear Systems, Incorporated.

<u>DATE SHIPPED</u>	<u>CURIES</u>	<u>CUBIC FEET</u>
5 August.	3.54	404
16 September	1.94	368
21 September	2.82	419
28 September	1.96	360
5 October	<u>3.95</u>	<u>481</u>
July - December TOTAL	14.21	2032



ANNUAL REPORT #13

JANUARY 1, 1977 TO DECEMBER 31, 1977 INCLUSIVE

6.9.3.a Environmental Monitoring 1977

A summary of the survey results is presented below:

(1) <u>TYPE</u>	NO. OF <u>LOCATION</u>	(ab) NO. OF <u>SAMPLES</u>	(d) <u>FREQUENCY</u>	(c) <u>RESULTS</u>	
				<u>AVG.</u>	<u>MAX.</u>
Atmospheric Dust	11	560	Weekly	.138	1.14 pCi/m ³ (a)
Fall Out	5	55	Monthly	80.1	451 pCi/m ² (a)
Radiation - Film	12	144	Monthly	<10	< 10 mRem
TLD	23	92	Quarterly	16.6	23.4 mRem
Lake Ontario Water	4	205	Weekly	4.8	17.2 pCi/l
Deer Creek Water	2 (b)	17	Monthly	7.0	15.7 pCi/l.
Well Water	1	10	Monthly	9.8	15.6 pCi/l
Milk	3	16	Monthly	.78	9.38 pCi/l(a)
Fruit/Vegetables	1	4	Annual	.018	<u>pCi Cs137</u>
Marine Organisms - Fish	1	23	Quarterly	3.36	5.12 pCi/gm
Algae	1	2		3.83	pCi/gm
Lake Bottom	1	1	Annual	No Sediment in Area	

6.9.3.a (2) No sample or measurement indicated statistically significant levels of radioactivity above the background, other than Chinese fallout samples.

(a) High value caused by Chinese atmospheric bomb test of Sept. 1977.

(b) One sampling location discontinued in June.

(c) Average without the one high value is 0:21.

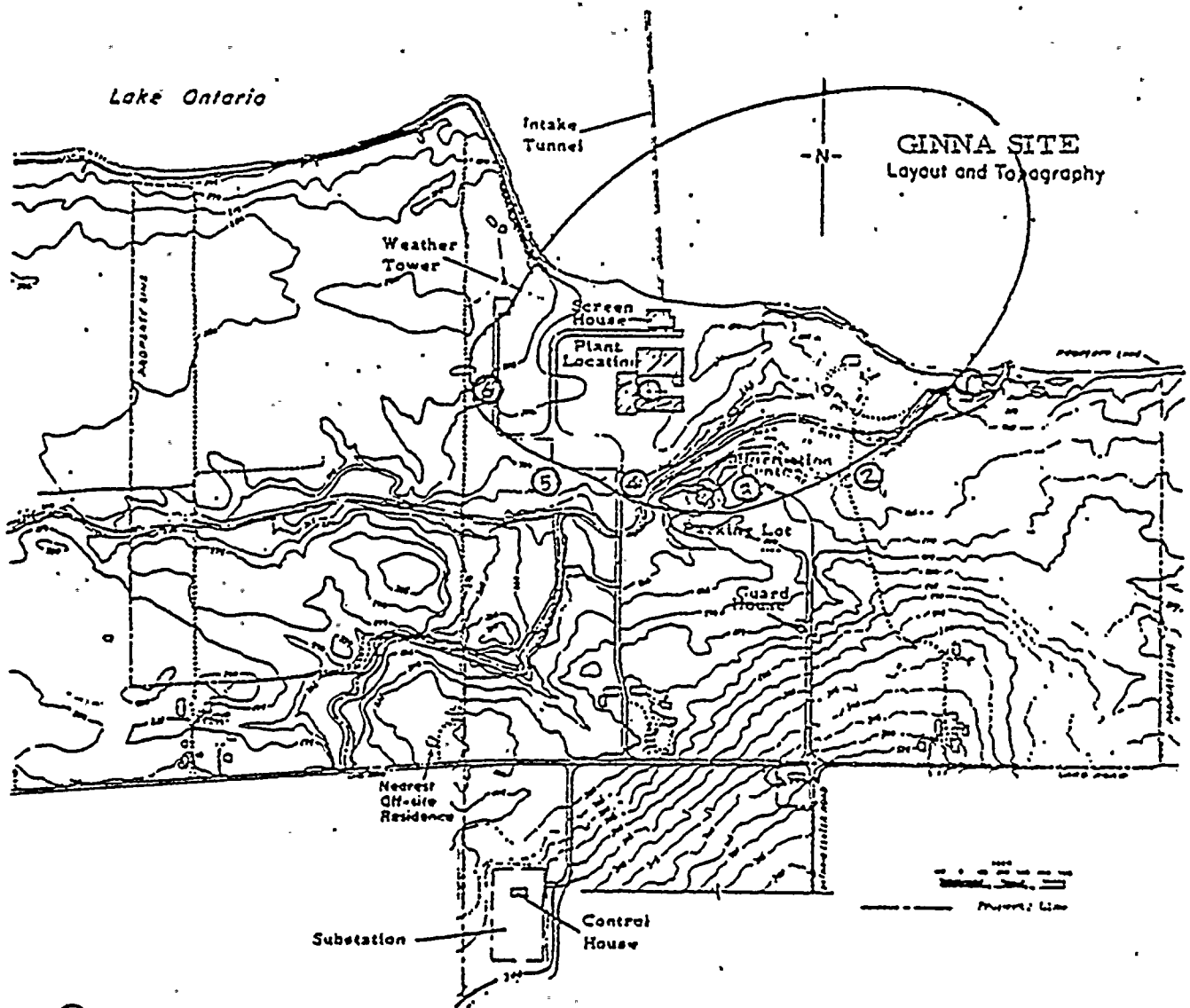
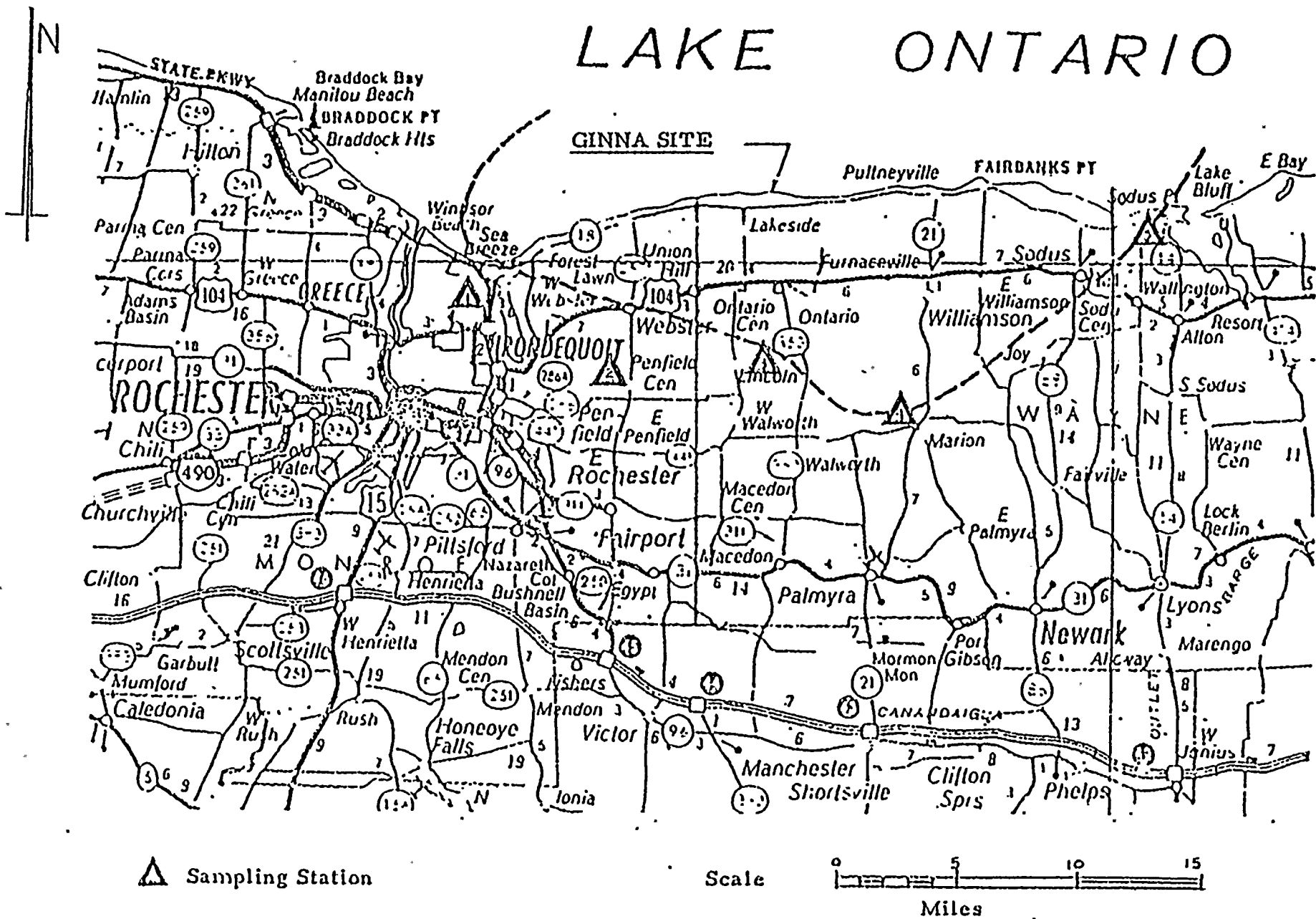


FIGURE 3.
 LOCATIONS OF RADIATION MONITORING
 AND SAMPLING STATIONS (Inner Ring)

LAKE ONTARIO



-56-

FIGURE 1
 LOCATIONS OF RADIATION MONITORING
 AND SAMPLING STATIONS (Outer Ring)

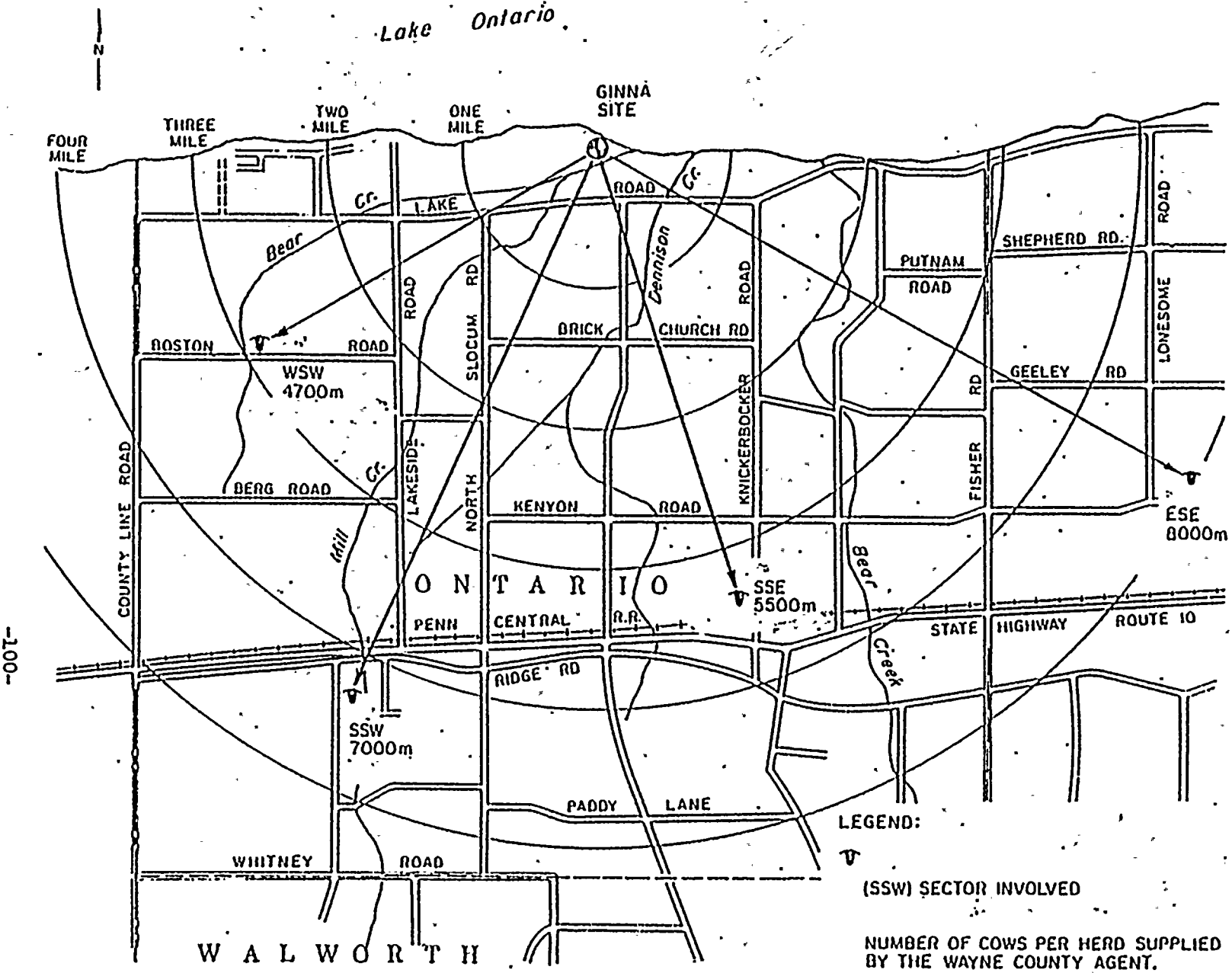
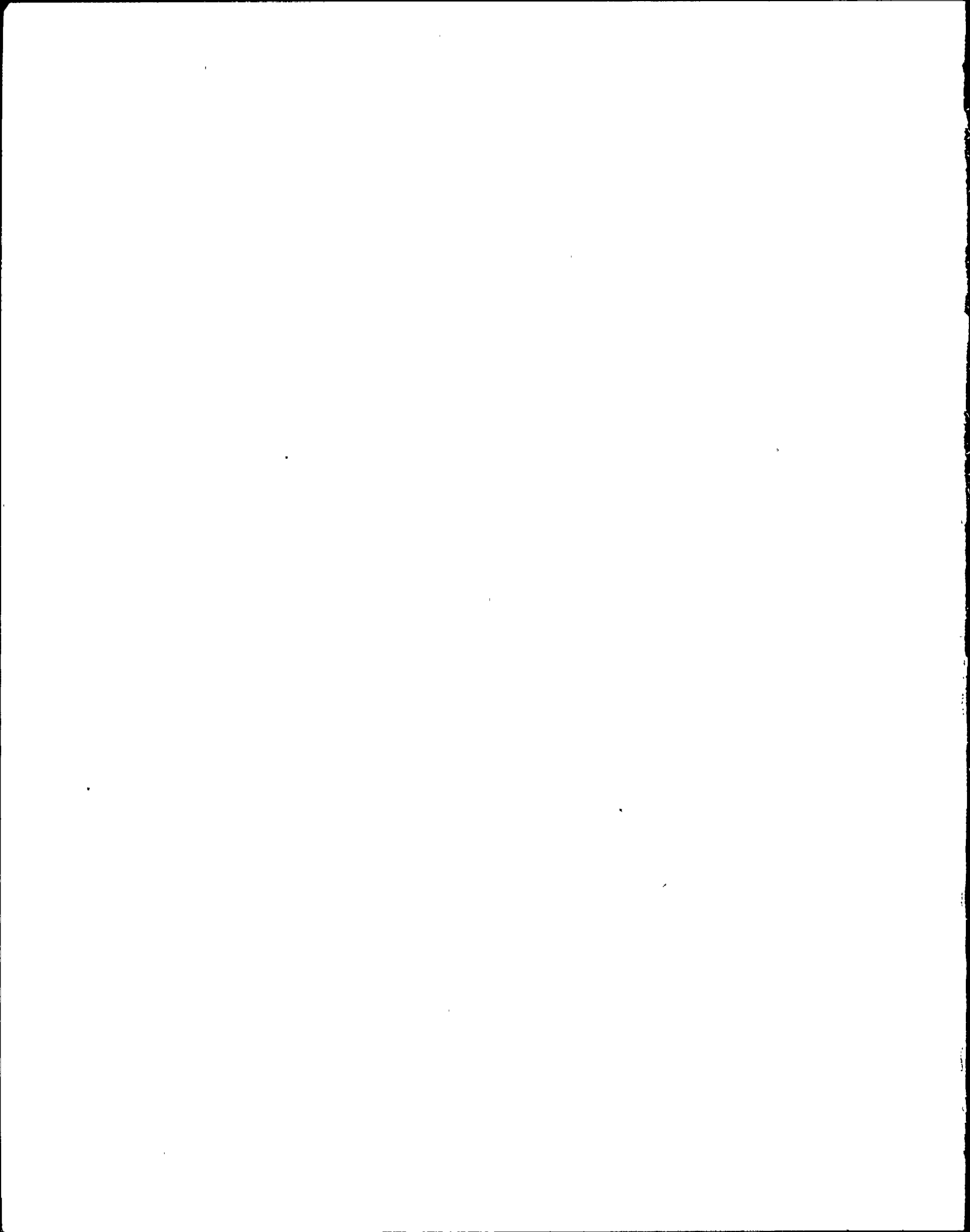
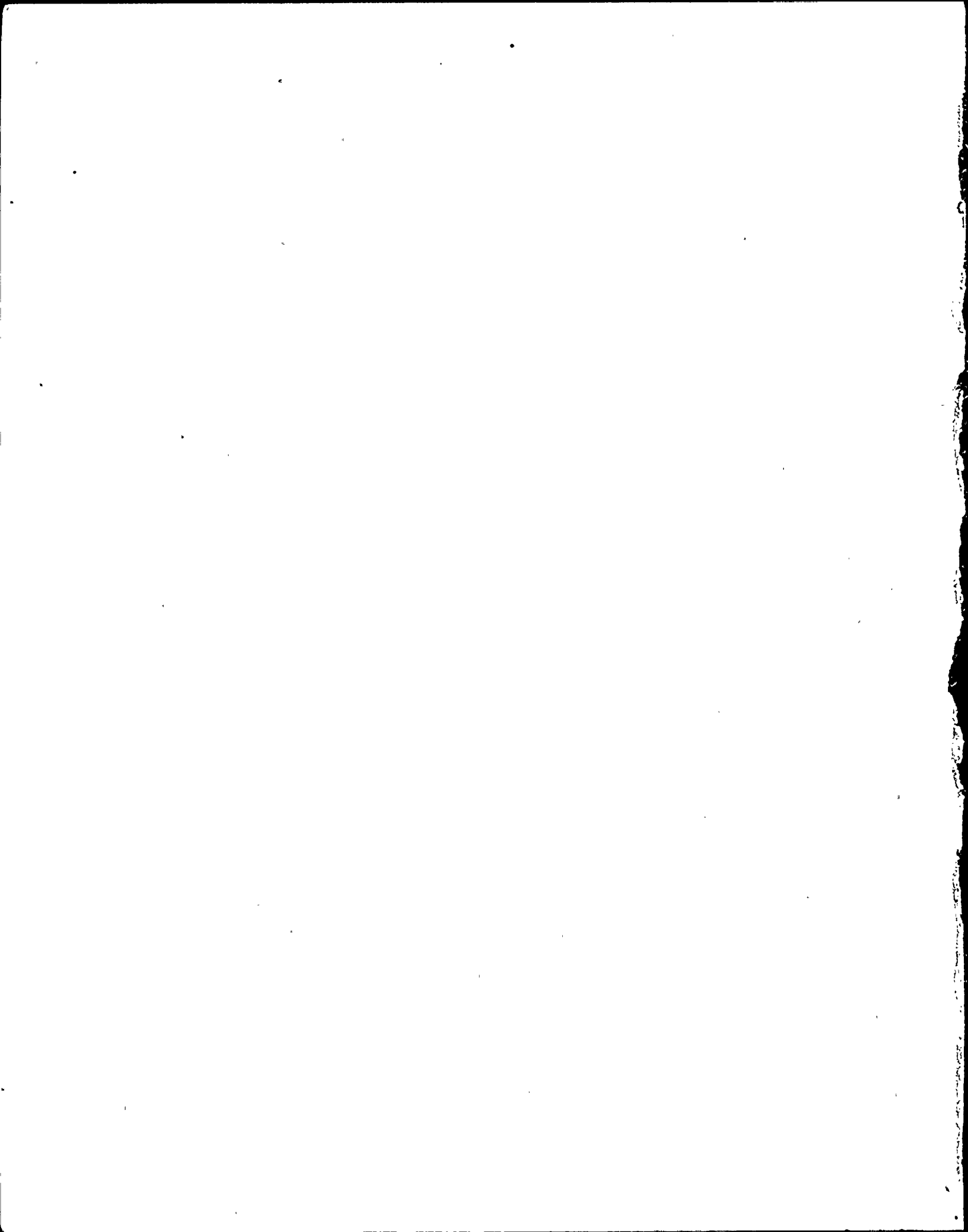


Fig. 5. Locations of dairy herds within 5 miles of the Ginna reactor.



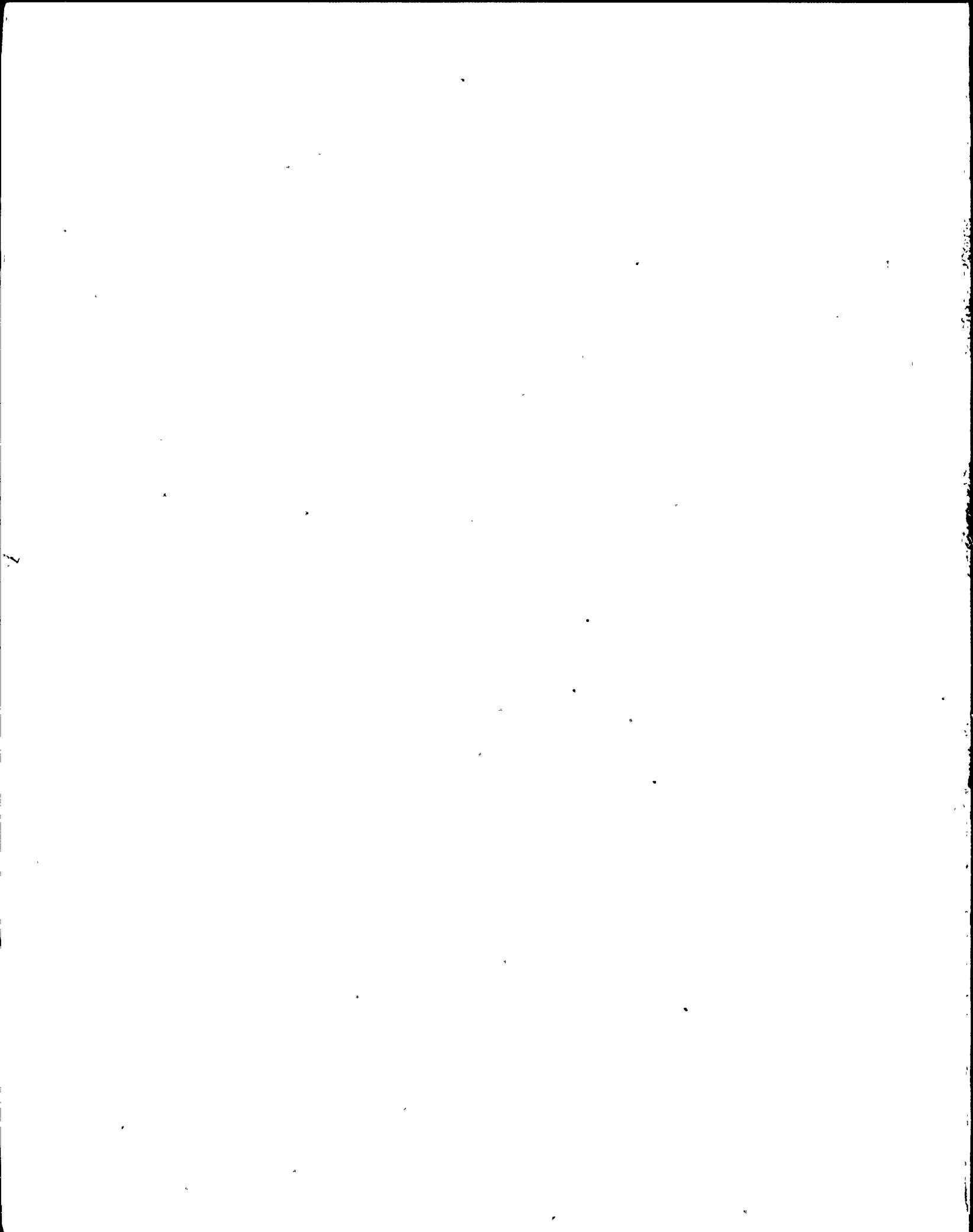
GLOSSARY

The terms, words, and phrases used in this report are commonly used throughout the nuclear industry, and no special definitions or explanations are necessary.



ANNUAL OPERATING REPORT #15
OF
R. E. GINNA NUCLEAR STATION
ROCHESTER GAS & ELECTRIC CORPORATION
1979
DOCKET NO. 50-244
LICENSE NO. DPR 18

REGULATORY DOCKET FILE COPY



FORC
ITEM #

PROCEDURE

79-01 *S-23.2.2, Containment Purge Procedure, PCN 78-2705.

79-03 *HP-10.1, Quality Control of Counting Systems, PCN 78-2683.

79-04 *GS-3.0, Security Call List, PCN 78-2679.

79-05 *M-11.4.4, Charging Pumps Varidrive Overhaul Unit # PCN 78-2730.

79-06 *PT-13.1.9, Halon System Testing (Resetting Only) System Numbers 3, 4, 5, and 6, PCN 78-2729.

79-07 *GS-8.0, DETEX Tours by Security Guards, PCN 78-2706.

79-08 *SC-3.15.14, Ginna Station Fire Doors, PCN 78-2685.

79-10 *O-6.11, Routine Operations Check Sheet, PCN 78-2694.

79-11 PT-23.13B, Containment Isolation Valve Leak Rate Testing "B" Steam Generator Sample, PCN 78-2695.

PT-23A, Containment Isolation Valve Test Connection Boundary Control PCN 78-2696.

79-12 *A-54.3, Open Flame, Welding and Grinding Permit, PCN 78-2481.

79-13 *HP-9.3, Air Ejector Gas Sampling, PCN 78-2680.

79-15 *SC-3.16.11, Operating Instruction - Self-Contained Breathing Apparatus, PCN 78-2093.

FORC
ITEM #

PROCEDURE

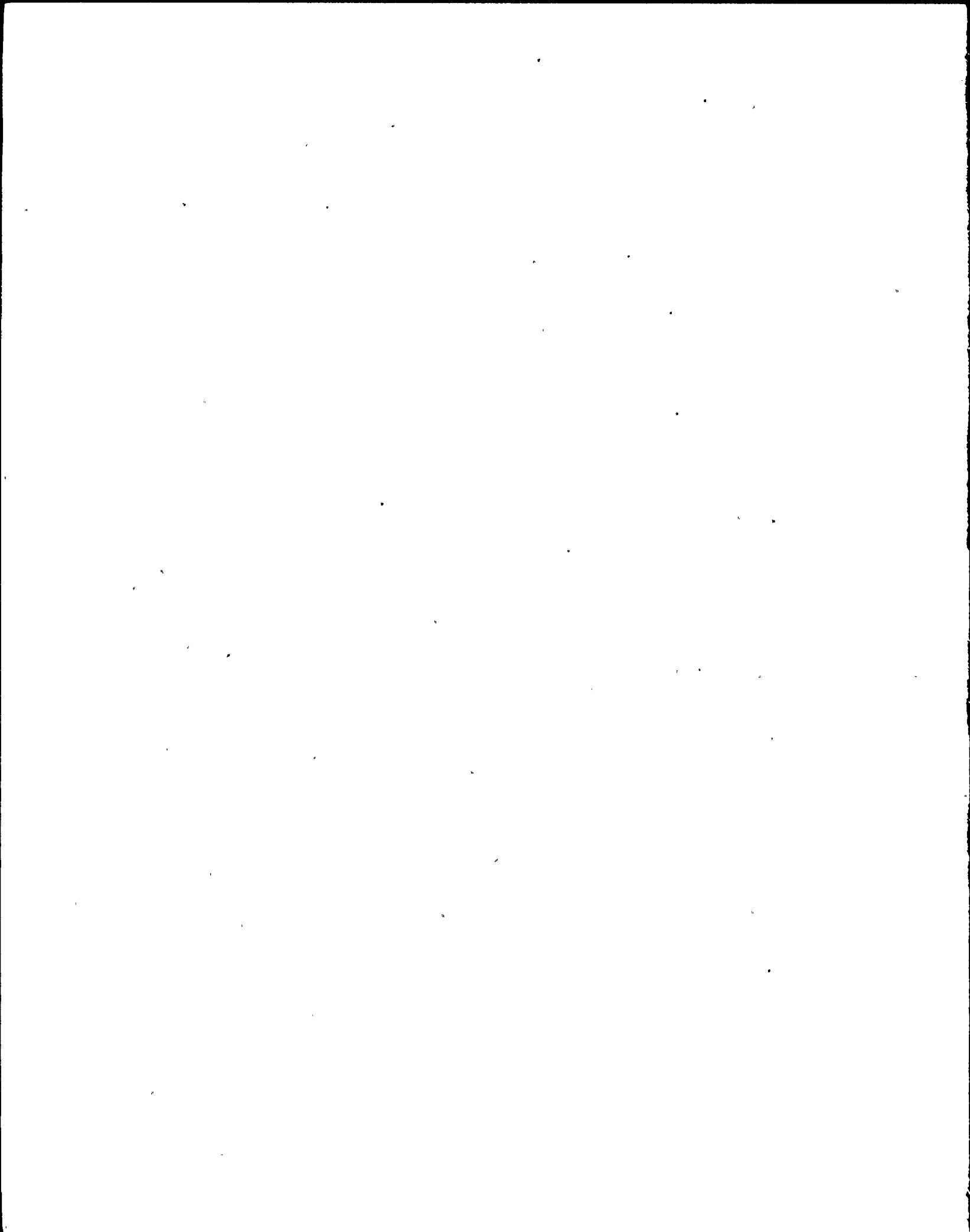
- 79-16 PT-13.4.1, Multimatic Deluge Valve System Testing - System #1 Turbine Condenser Pit, PCN 78-2650.
- PT-13.4.2, Multimatic Deluge Valve System Testing - System #2 Turbine Oil Reservoir, PCN 78-2651.
- PT-13.4.11, Multimatic Deluge Valve System Testing - System #11, Cable Tunnel, PCN 78-2652.
- RSSP-15.10, ISI Hydro Test of Class "B" Piping (RHR System Low Pressure), PCN 78-2665.
- RSSP-15.11, ISI Hydro Test of Class "B" Piping (RHR System High Pressure), PCN 78-2666.
- M-73.10, Welding, PCN 78-2480.
- 79-17 *A-30.3, Plant Procedure Content and Format Requirements, PCN 78-2699.
- 79-18 *M-67.5, Halon System Maintenance Computer Room, and Relay Room, PCN 78-2728.
- 78-19 PT-13.1.9, Halon System Testing (Resetting) Computer Room and Relay Room, PCN 78-2731.
- 79-27 *M-11.4.11, Charging Pump Cylinder Block Replacement for Charging Pump, PCN 79-26.
- 79-29 A-102.3, R. E. Ginna HP Orientation Program, PCN 78-2677.
- HP-11.3, Noble Gas Sampling and Analysis Utilizing a Marinelli Beaker, PCN 78-2689.

FORC
ITEM #

PROCEDURE

- HP-7.8; Alarming Dopsimeter Accuracy Test, PCN 78-2707.
- RSSP-13.9; ISI Hydro Test of Class "C" Piping (Component Cooling Water - Outside Containment). PCN 78-2702.
- RSSP-13.10; ISI Hydro Test of Class "C" Piping Component Cooling Water - Inside Containment, PCN 78-2703.
- A-1701; Quality Assurance Records, PCN 78-2658.
- 79-30 A-50.9, Facility Staff Training Program, Ginna Station Personnel Training Program, PCN 79-03.
- 79-31 *SM 78-1890.2; Feedwater Line Drain Valves After the Containment Check Valves, PCN 78-2664.
- 79-32 *SM 78-1594; A-1; Installation of Spent Fuel Pit Suction Strainer, PCN 79-29.
- 79-34 SC-1.7D; Emergency Radiation Monitoring Off-Site Survey Team #9D (RED), PCN 78-2735.
- SC-1.7F; Emergency Radiation Monitoring Off-Site Survey Team #9F (GREEN), PCN 78-2736.
- SC-1.7H; Emergency Radiation Monitoring Off-Site Survey Team #9H (ORANGE), PCN 78-2734.
- 79-35 *SC-1.12D; Management Call List, PCN 78-2733.
- 79-36 *SC-1.12C; Specialized Call List, PCN 78-2732.

<u>FORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-37</u>	<u>*GS-8.0, Detex Tours by Security Guards, PCN 79-01.</u>
<u>79-38</u>	<u>*HP-1.1, Issuing Personnel Dosimeters, PCN 78-2748.</u>
<u>79-40</u>	<u>PC-16, Solid Determination, PCN 78-2746.</u>
	<u>RD-1.3, Tritium in Containment Air Sampling and Analysis, PCN 78-2747.</u>
<u>79-41</u>	<u>*RF-48.2, Testing of Reactor Cavity Seal Ring, PCN 78-2738.</u>
<u>79-42</u>	<u>*RD-2, Containment Purge Release, PCN 78-2749.</u>
<u>79-43</u>	<u>*HP-9.2, Primary to Secondary Leakage Detection and Measurements, PCN 78-2750.</u>
<u>79-46</u>	<u>*HP-4.3, Work Permit Use, PCN 79-09.</u>
<u>79-48</u>	<u>*PT-13.1.10, Smoke Detector System Testing, PCN 79-17.</u>
<u>79-49</u>	<u>*A-30.2, Plant Procedure Classification, Review, Approval and Distribution Requirements, PCN 78-2698.</u>
<u>79-50</u>	<u>*HP-10.1, Quality Control of Counting Systems, PCN 78-2752.</u>
<u>79-59</u>	<u>*S-3.5.1, Deborating Demineralizer (B) Rinse, PCN 79-55.</u>
<u>79-62</u>	<u>*QC-1002, Quality Control Procedure Deviation, Revision 5, Change in Training Requirements.</u>
<u>79-64</u>	<u>*RF-10.1, Auxiliary Building Crane Interlock Checkout, PCN 79-79.</u>



FORC
ITEM #

PROCEDURE

79-65

M-48.7, Cross-Tie of Bus 17 to Bus 18, PCN 79-57.

M-51.1, Isolation of a Reactor Protection Relay, PCN 79-59.

M-58.1, Area Monitoring Maintenance PCN 79-62.

M-25.1, Replacement of RTD's on MS Safety Valve Support Stations, PCN 79-61.

M-7.5, Spent Fuel Pit Filter Replacement, PCN 79-78.

79-66

PT-23, Containment Isolation Valve Leak Rate Testing, PCN 79-19.

PT-23.18B, Containment Isolation Valve Leak Rate Testing "B" Containment Spray Header, PCN 79-20.

PT-23.18A, Containment Isolation Valve Leak Rate Testing "A" Containment Spray Header, PCN 79-21.

79-67

M 37.18B, Isolation of the Gas Decay Tank Reuse Valves.

M-43.7, S/G High Pressure Wash for Decontamination.

M-43.22, Condensate Storage Tank Warmup.

M-43.24, Inspection of S/G Secondary Side Tube Sheet.

M-46.1, Purging Gas From the "A" Accumulator.

M-46.2 Purging Gas From the "B" Accumulator.

FORC
ITEM #

PROCEDURE

- M-52.1, Installation of Detector and Cable on Miniature Detector Flux Mapping System.
- M-45.1A, Inspection of Safeguard Motor For
- 79-68 *HP-1.6, Neutron Exposure, PCN 79-22.
- 79-69 *HP-1.1, Issuing Personnel Dosimeters, PCN 79-23.
- 79-77 *A-52.4, Control of Limiting Conditions for Operating Equipment, PCN 79-34.
- 79-78 *SM 78-1845.1, Guide Stud Storage Racks, PCN 79-41.
- 79-79 *S-3.6, Auxiliary Building Heat Tracing System Operation, PCN 79-43.
- 79-80 *WC-1, A List of Sample Chemical Parameter and Sampling Schedule, PCN 79-45.
- 79-82 *WC-13.3, Phosphate Determination, PCN 79-65.
- 79-83 *S-4.5.11, "B" Letdown Deborating D. I. Resin Replacement, PCN 79-66.
- 79-84 *WC-12.0, Dissolved Oxygen Determination Indigo Carmine Method, PCN 79-67.
- 79-86 *RD-2, Containment Purge Releases, PCN 79-69.
- 79-87 GS-3.0, Security Call List, PCN 79-80.

PORC
ITEM #

PROCEDURE

PT-24.1, Spent Fuel Pit Charcoal Filter By-Pass Flow, PCN 79-15.

79-88

M-43.9, Steam Generator Permanent Ring Nozzle Cover Gasket Test, PCN 79-56.

M-75.1, Safety Injection Pump Suction Piping From Boric Acid Storage Tanks Ultrasonic Inspection of Welds, PCN 79-73.

M-73.1, Piping Repairs on Primary System Branch Piping, PCN 79-60.

QC-1701 Quality Assurance Records, PCN 79-35.

79-89

S-9A, Spent Fuel Pit Purification and/or Cooling System, PCN 79-33.

S-4.5.7, Letdown Mixed Bed D. I. (A or B) Resin Replacement, PCN 79-44.

M-52.3, Incore Thimble Cleaning, PCN 79-54.

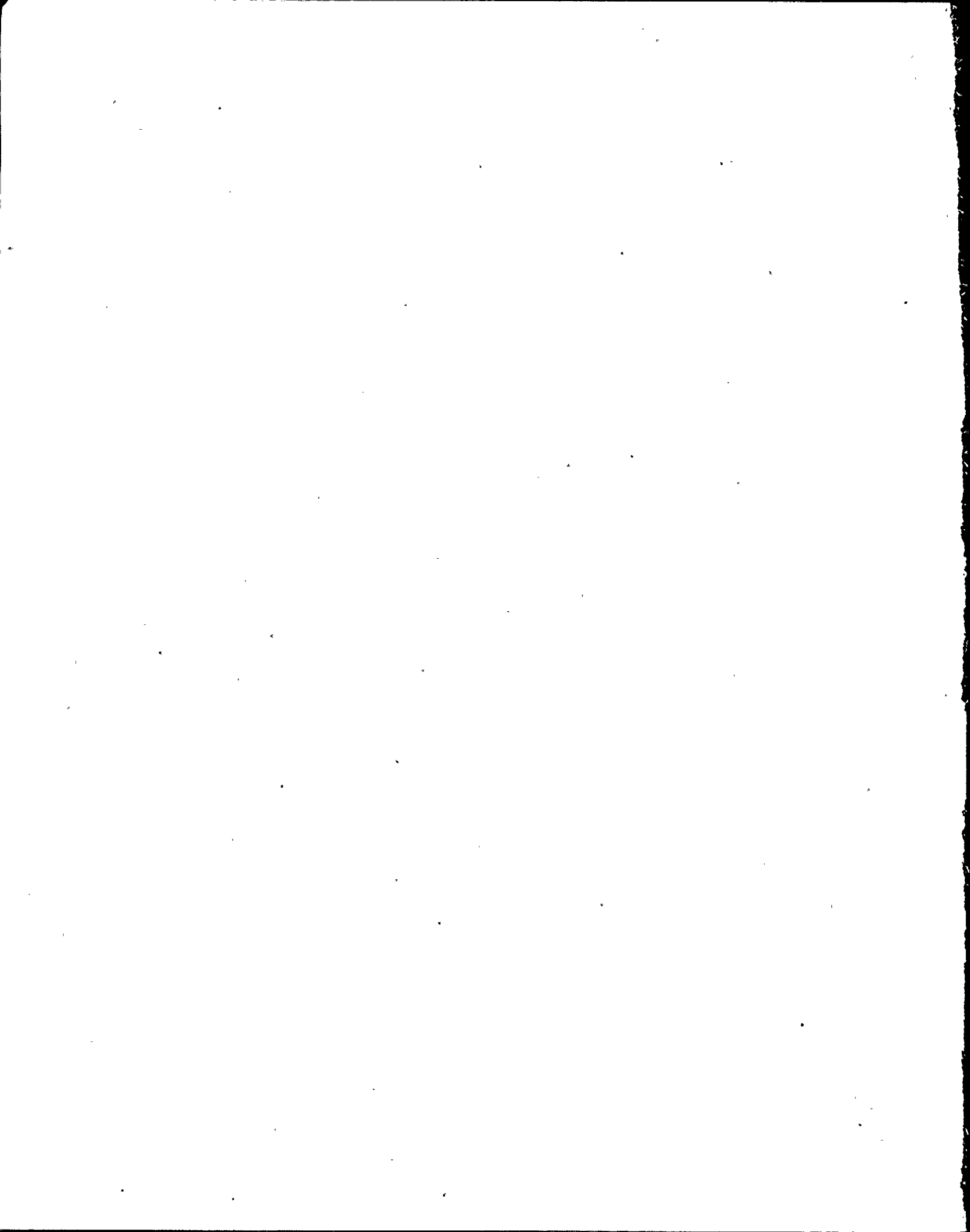
SC-3.15.15, Emergency Fire Equipment Locker Inventory and Inspection, PCN 79-76.

S-3.36, CVCS Deborating Demineralizer Bed Operation Using "B" Deborating Unit, PCN 79-84.

PT-2.7, Service Water System, PCN 79-85.

79-90

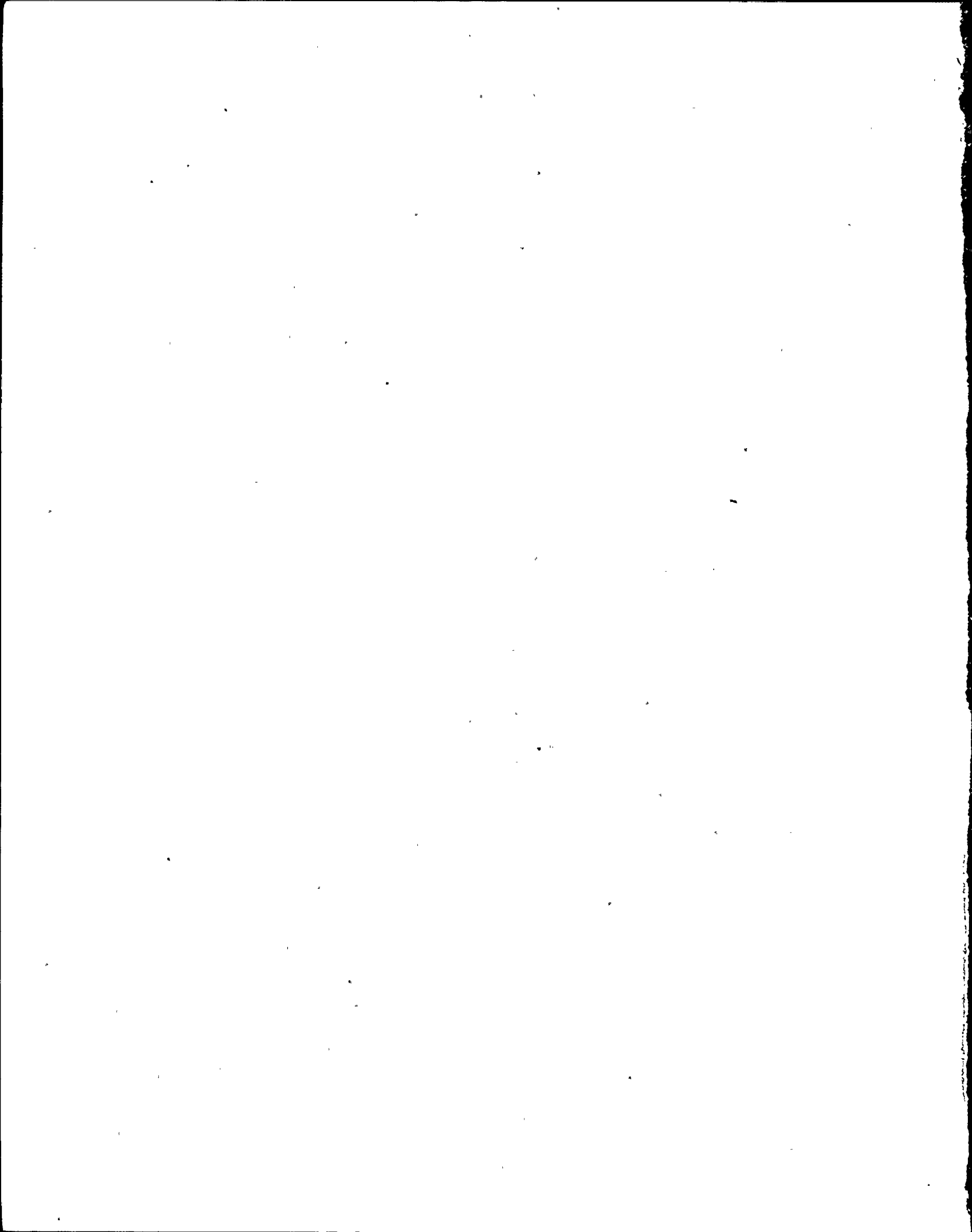
S-4.5.11, "B" Letdown Deborating DI Resin Replacement, PCN 79-77.



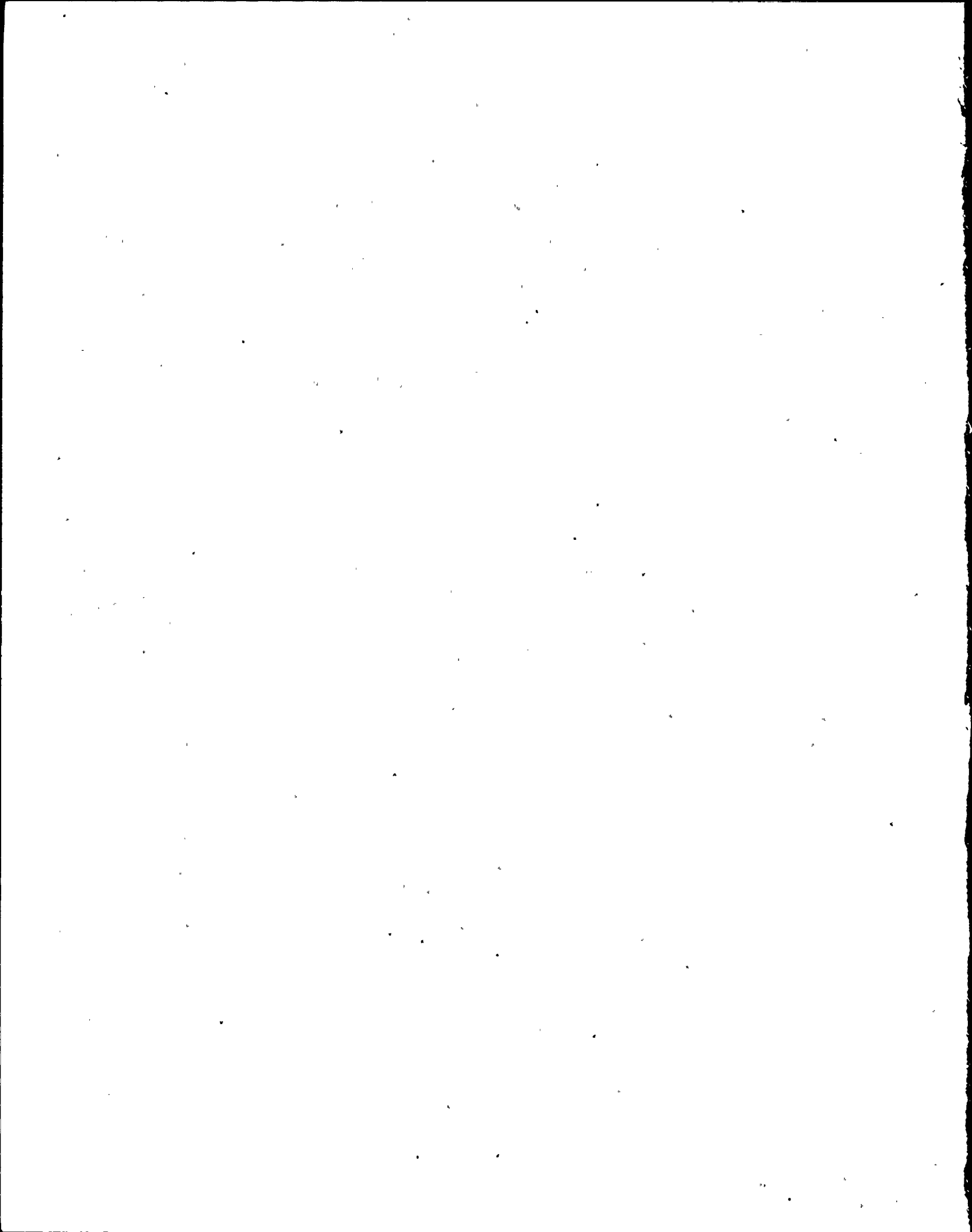
FORC
ITEM #

PROCEDURE

- S-3.2A, Charging and Volume Control System Pre-Startup Alignment, PCN 79-75.
- S-3.5.1, Deborating Demineralizer (B) Rinse, PCN 79-64.
- SM 78-1594A.1, Installation of SFP Suction Strainer, PCN 79-83.
- S-4.5.11, "B" Letdown Deborating D. I. Resin Replacement, PCN 79-82.
- 79-91 PT-24, Spent Fuel Pit Filter Bank, Mass Air Flow Check, PCN 79-16.
- O-6.12, Plant Operation During Coastdown, PCN 79-18.
- 79-92 *SM 77-1499.1, Rod Insertion Limit Circuitry, PCN 78-2744.
- 79-93 *SM 78-1872.1, Manipulator Crane Gripper System Modification, PCN 78-2668.
- 79-102 *ISI-5, Eddy Current Testing of Series 44 Steam Generator Tubes, PCN 79-109.
- 79-104 *S-13A, RHR System Lineup for Safety Injection, PCN 79-107.
- 79-105 *HP-12.2, Medical Check, Fitting and Training of Personnel Using Respirators, PCN 79-105.
- 79-106 *HP-12.1, Usage of Respirators, PCN 79-104.



<u>FORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-108</u>	<u>*M-37.14, Inspection and Maintenance of "B" Loop Accumulator Discharge Check Valve V-842A, PCN 79-102.</u>
	<u>*EM-150, Temporary Repair of Auxiliary Feedwater System Manual Isolation Valve 4012, PCN 79-99.</u>
<u>79-109</u>	<u>*CP-617.0, Component Cooling Water Pumps Discharge Pressure PIC-617, PCN 79-98.</u>
<u>79-110</u>	<u>*S-3.5.1, Deborating Demineralizer (B) Rinse, PCN 79-97.</u>
<u>79-111</u>	<u>*S-4.5.11, "B" Letdown Debroating DI Resin Replacement, PCN 79-96.</u>
<u>79-112</u>	<u>*HP-1.2, External Exposure Limits, PCN 79-88.</u>
<u>79-114</u>	<u>*GS-28.0, Inspection of Personnel and Vehicle and Package, PCN 79-81.</u>
<u>79-115</u>	<u>*A-18, Multileaf Trouble Report and Work Order, PCN 79-36.</u>
<u>79-116</u>	<u>A-54.2, Ginna Station Staff Responsibilities for Fire Protection, PCN 78-2222.</u>
	<u>M-43.1.1, Steam Generator Insert Removal Steam Generator, PCN 79-124.</u>
<u>79-127</u>	<u>A-102.2, R. E. Ginna Administrative Controls Training Program, PCN 78-2742.</u>
	<u>A-1102, Qualification of Test Personnel, PCN 79-7.</u>



FORC
ITEM #

PROCEDURE

A-1103, Test Tag Control Program, PCN 79-28.

A-603, Control of Engineering Documents by Central Records, PCN 78-2338.

A-604, Control of Engineering Documents on Modification Project, PCN 78-2337.

79-128 *EM-217, Installation of a New RCS Sample Delay Coil, PCN 79-55.

79-129 PI-17.2, Process Radiation Monitor R-11 - R-21, and Iodine Monitor R-10A and R-10B, PCN 79-147.

79-130 SM 76-21.4, Battery Room Air Conditioning - Air Conditioner Discharge Duct South End of Battery Room, PCN 79-92.

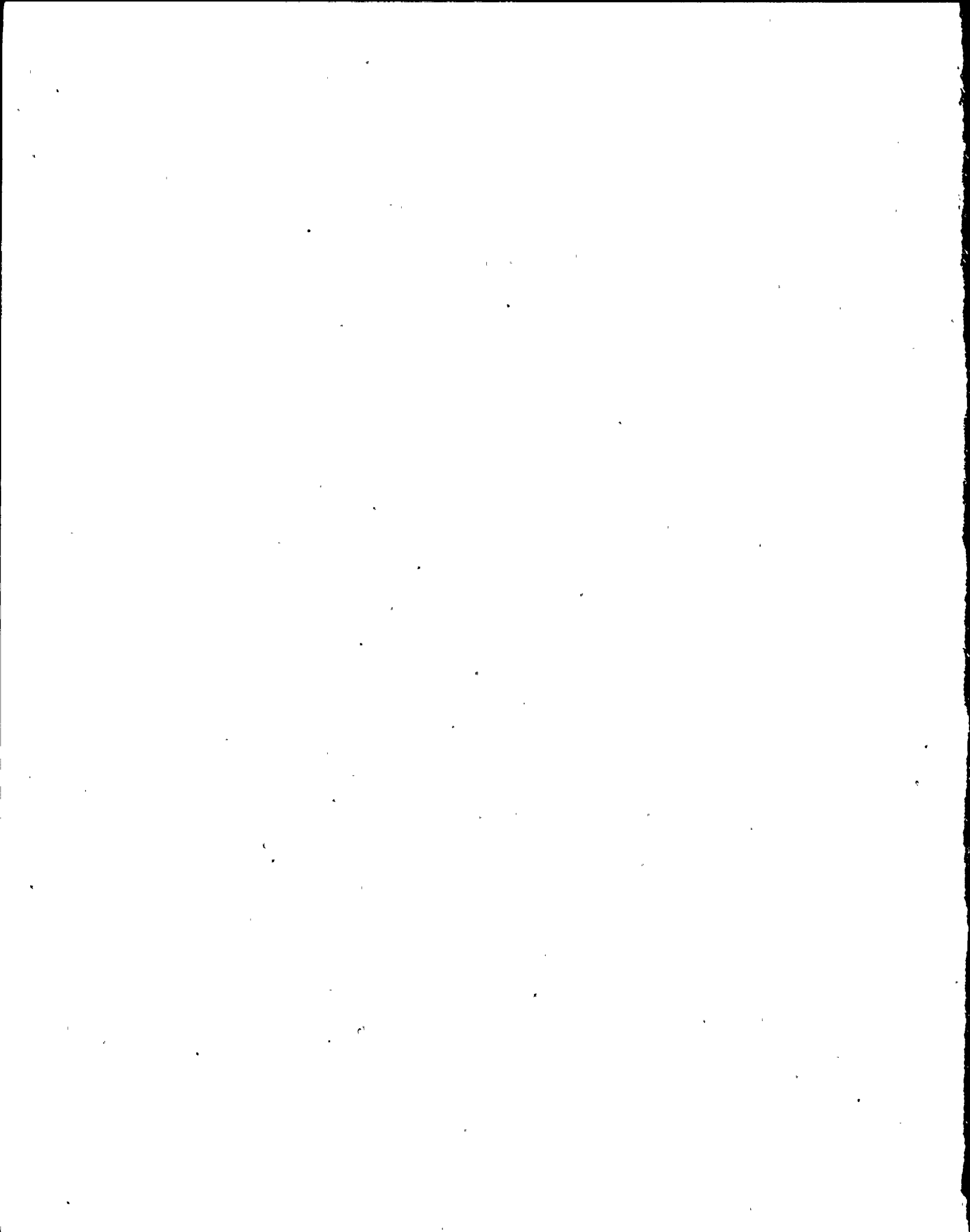
SM 76-21.6, Battery Room Air Conditioning - Air Supply Duct to the Air Conditioner, PCN 79-94.

SM 76-21.5, Battery Room Air Conditioning - Service Water Piping Installation, PCN 79-95.

SM 76-21.7, Battery Room Air Conditioning - Miscellaneous, PCN 79-93.

SM 74-28.1, Installation of Make-Up Lines to Condensate Storage Tanks, PCN 79-131.

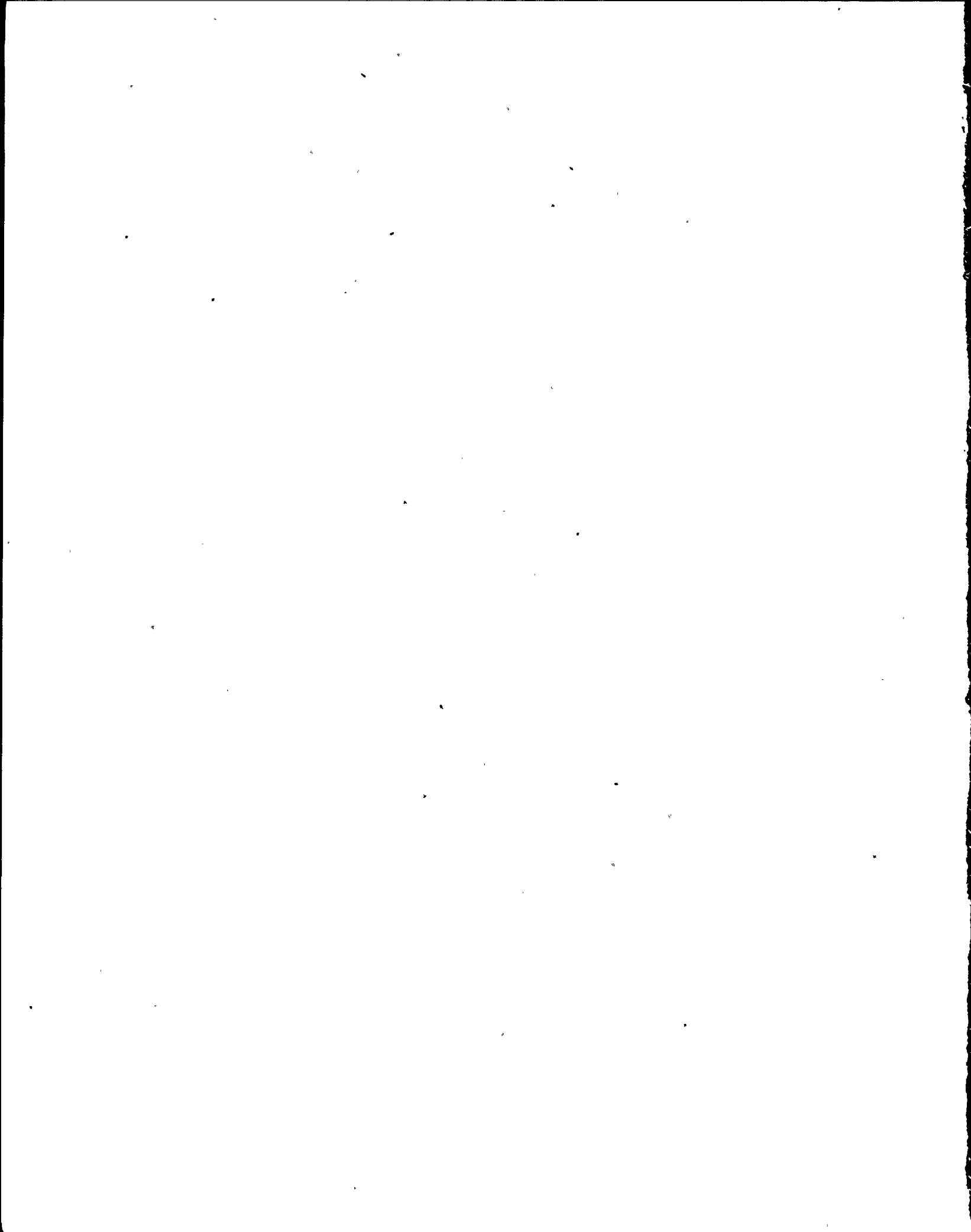
SM 75-48.30, Rework of the Auxiliary Feedwater Crossover Piping Modification, PCN 79-132.



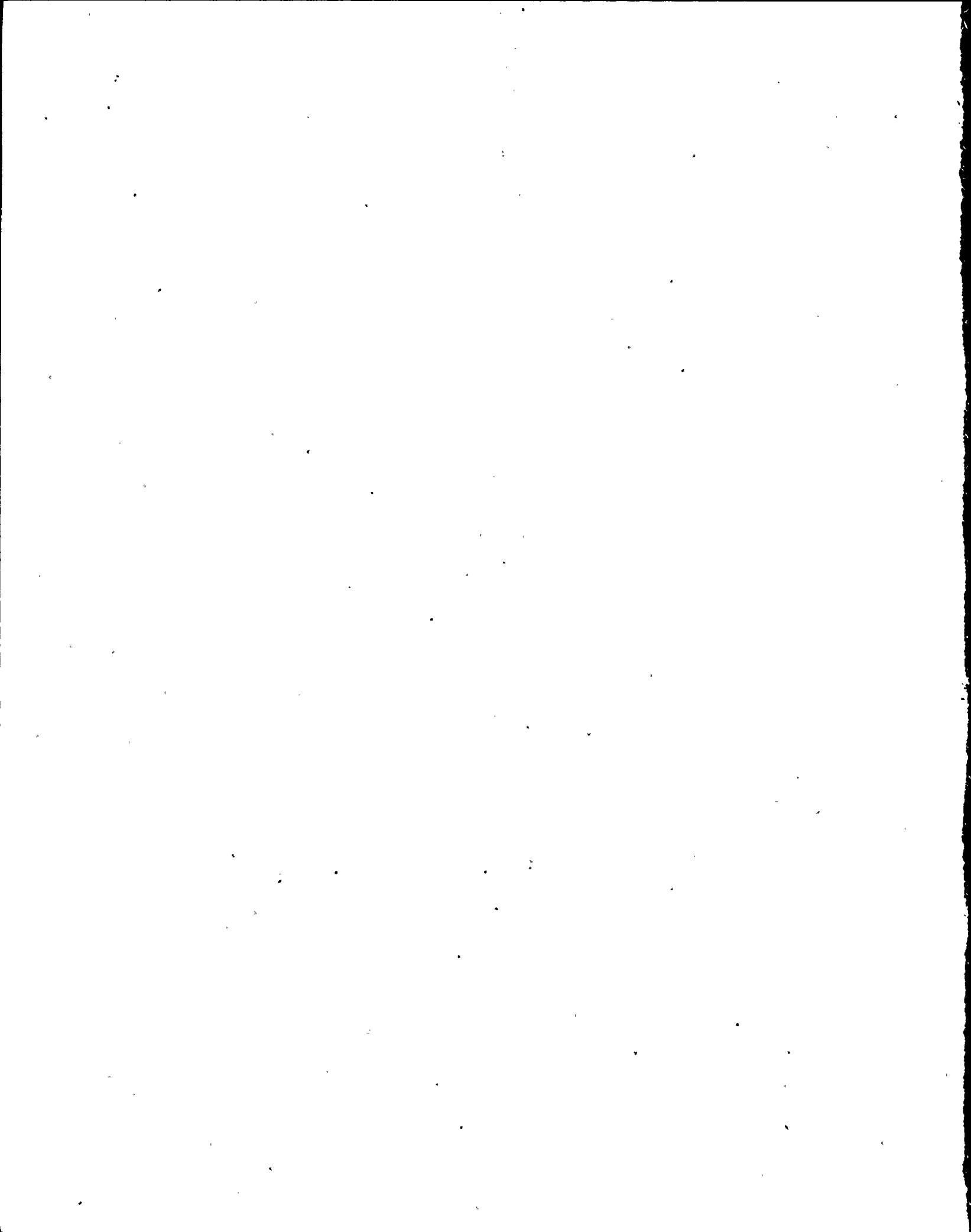
PORC
ITEM #

PROCEDURE

- SM 77-1030.1, Neutralizing Tank Logic Modification, PCN 79-133.
- SM 77-2110.1, Chem. Lab DI Storage Tank Cable Routing, PCN 79-134.
- SM 77-1449.1, Diesel Fire Pump Heater, PCN 79-135.
- A-36, Station Holding Rules, PCN 79-117.
- ST 78-1, Generator Short Circuit Test, PCN 79-87.
- 79-140 *HP-4.3, Work Permit Use, PCN 79-141.
- 79-141 *PT-17.2, Process Radiation Monitors, R-11 - R-12, and Iodine Monitors R-10A and R-10B, PCN 79-137.
- 79-142 *M-43.14, Removing Steam Generator Blast Proof Loop Nozzle Cover, Steam Generator, Leg, PCN 79-142.
- 79-143 *P-2, Reactor Coolant System Precautions and Limitations, PCN 79-169.
- 79-144 *M-7.4, Boric Acid Storage Tank Filter Replacement, PCN 79-136.
- 79-145 *SC-3.15.3, Portable Extinguisher Inspection, PCN 79-148.
- 79-146 *M-43.2, Initial Radiological Survey, PCN 79-146.
- 79-147 *M-43.15.1, Installation of Steam Generator Primary Manway Cover Closure, PCN 79-145.



<u>PORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-148</u>	<u>*M-43.16.1, Steam Generator Tube Sheet Photo-Mapping, PCN 79-144.</u>
<u>79-149</u>	<u>*M-43.21, Steam Generator Final Clean-up and Inspection, PCN 79-143.</u>
<u>79-150</u>	<u>M-36.1, 1A Waste Gas Compressor Maintenance, PCN 79-179.</u>
	<u>M-36.2, 1B Waste Gas Compressor Maintenance, PCN 79-178.</u>
	<u>GS-21.0, Site Access Control (Security) and Personnel Identification, PCN 79-175.</u>
	<u>S-3.1J, Boric Acid Storage Tank Filter Isolation, PCN 79-173.</u>
<u>79-151</u>	<u>*RD-8, Liquid Rad Waste Compositing and Analysis, PCN 79-167.</u>
<u>79-152</u>	<u>HP-2.4, TLD Readout, PCN 79-155.</u>
	<u>HP-3.2, In-Plant Reporting of Current Exposures, PCN 79-156.</u>
	<u>HP-8.2, Leak Testing of Licensed Sources, PCN 79-157.</u>
	<u>HP-8.3, Calibration of Gamma Seal Sources, PCN 79-158.</u>
	<u>HP-10.2, Operational Test of Portal and Hand and Foot Monitors, PCN 79-159.</u>



PORC
ITEM #

PROCEDURE

HP-10.5, Efficiency Calibration of Gamma Spectrometers, PCN 79-161.

RD-1.2, Containment Radiogas Sampling and Analysis, PCN 79-163.

RD-7, Liquid Waste Release, PCN 79-164.

PC-4, Hydrogen Concentration and Radiogas Activity in Primary Coolant-Sampling and Analysis, PCN 79-165.

RD-9, Preparing Waste for Shipment or Storage, PCN 79-166.

ES-2.3, Iodine - 131 Analysis of Milk, PCN 79-152.

WC-2.0, Metrohm E-415 Titrator Calibration, PCN 79-153.

79-153

PT-5.10, Process Instrumentation Reactor Protection Channel Trip Test (Channel 1), PCN 79-138.

PT-5.20, Process Instrumentation Reactor Protection Channel Trip Test (Channel 2), PCN 79-139.

PT-5.30, Process Instrumentation Reactor Protection Channel Trip Test (Channel 3), PCN 79-140.

79-154

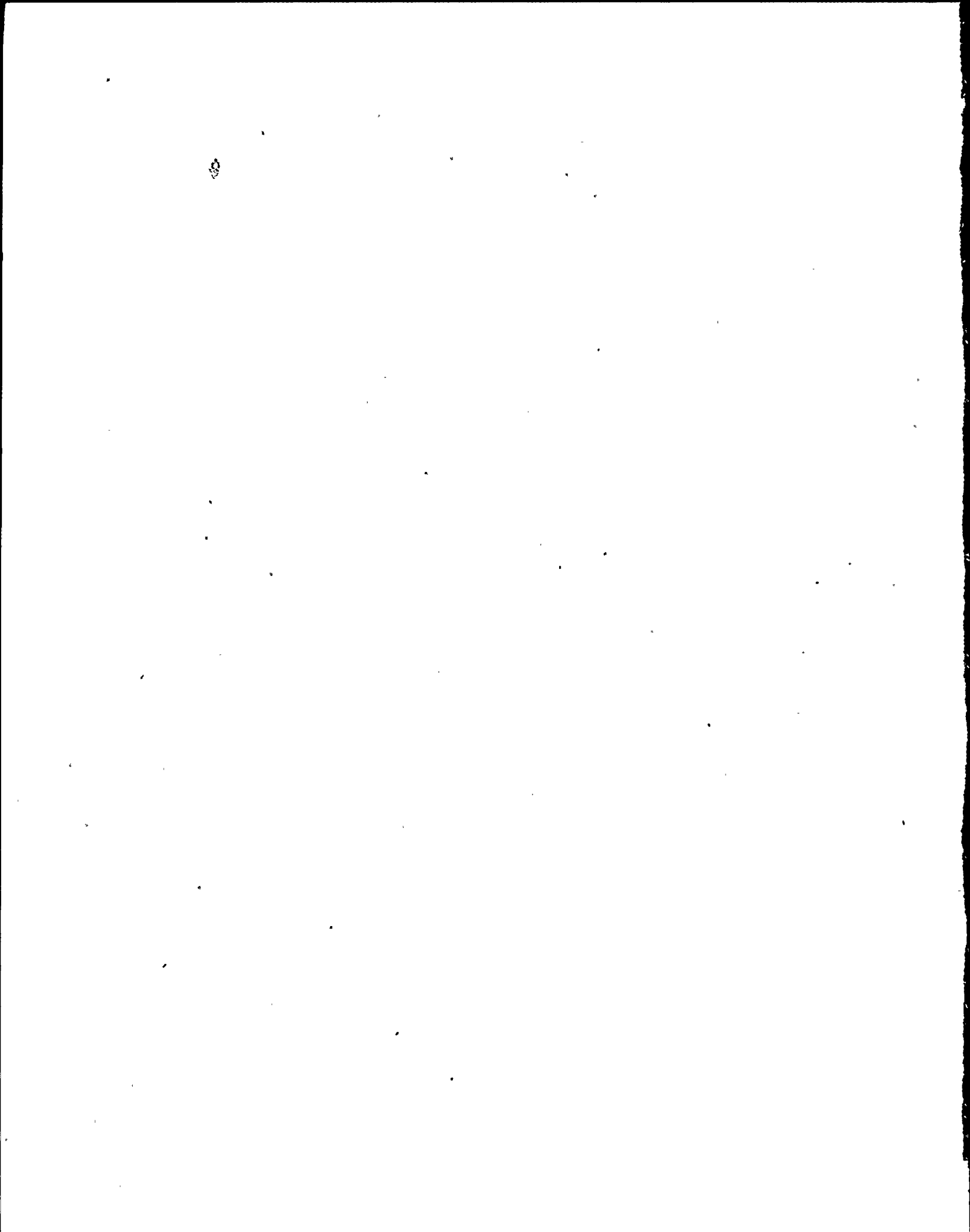
A-50.1, R. E. Ginna Administrative Control Training Program, PCN 79-02.

A-50.9, Ginna Station Personnel Training Program, PCN 79-03.

FORC
ITEM #

PROCEDURE

- 79-156 S-4.4, Spent Resin Removal to Shipping Casks, PCN 79-172.
- S-7A, RWST Purification, PCN 79-171.
- 79-157 A-601, Plant Procedure Document Control, PCN 79-48.
- CP-1012, Calibration and/or Maintenance of "A" Waste
Condensate Tank Level "1012", PCN 79-130.
- CP-1013, Calibration and/or Maintenance of "B" Waste
Condensate Tank Level "1013", PCN 79-129.
- 79-159 *SM 78-1594A.2, Standby Spent Fuel Pit Pump Cable Rout-
ing, PCN 79-180.
- 79-160 *SM 78-1594A.3, Installation of Spent Fuel Pit Cooling
Backup System, PCN 79-181.
- 79-164 *SM 75-14.1, Batch Tank Recirculation Heat Trace Installation,
PCN 79-211.
- 79-165 *HP-7.6, Tagging of Instruments Requiring Calibration
or Maintenance, PCN 79-187.
- *S-16.11, Monitoring and Enrichment of Accumulator Boron
Concentration, PCN 78-2700.
- 79-166 *HP-7.6, Tagging of Instruments Requiring Calibration
or Maintenance, PCN 79-191.
- *S-16.11, Monitoring and Enrichment of Accumulator Boron
Concentration, PCN 79-207.



PORC
ITEM #

PROCEDURE

*PT-21, Cleaning Boric Acid Tank Level Sensing Lines,
PCN 79-183.

79-167 *HP-7.3, Calibration of Alpha Survey Instruments PAC-4S,
PCN 79-186.

79-168 *HP-1.3, External Exposure Records, PCN 79-188.

79-169 *M-36.1, 1A Waste Gas Compressor Maintenance, PCN 79-
192.

*M-36.2, 1B Waste Gas Compressor Maintenance, PCN 79-
193.

79-170 *HP-7.1, Calibration of Gamma Survey Instruments -
Low Range, PCN 79-194.

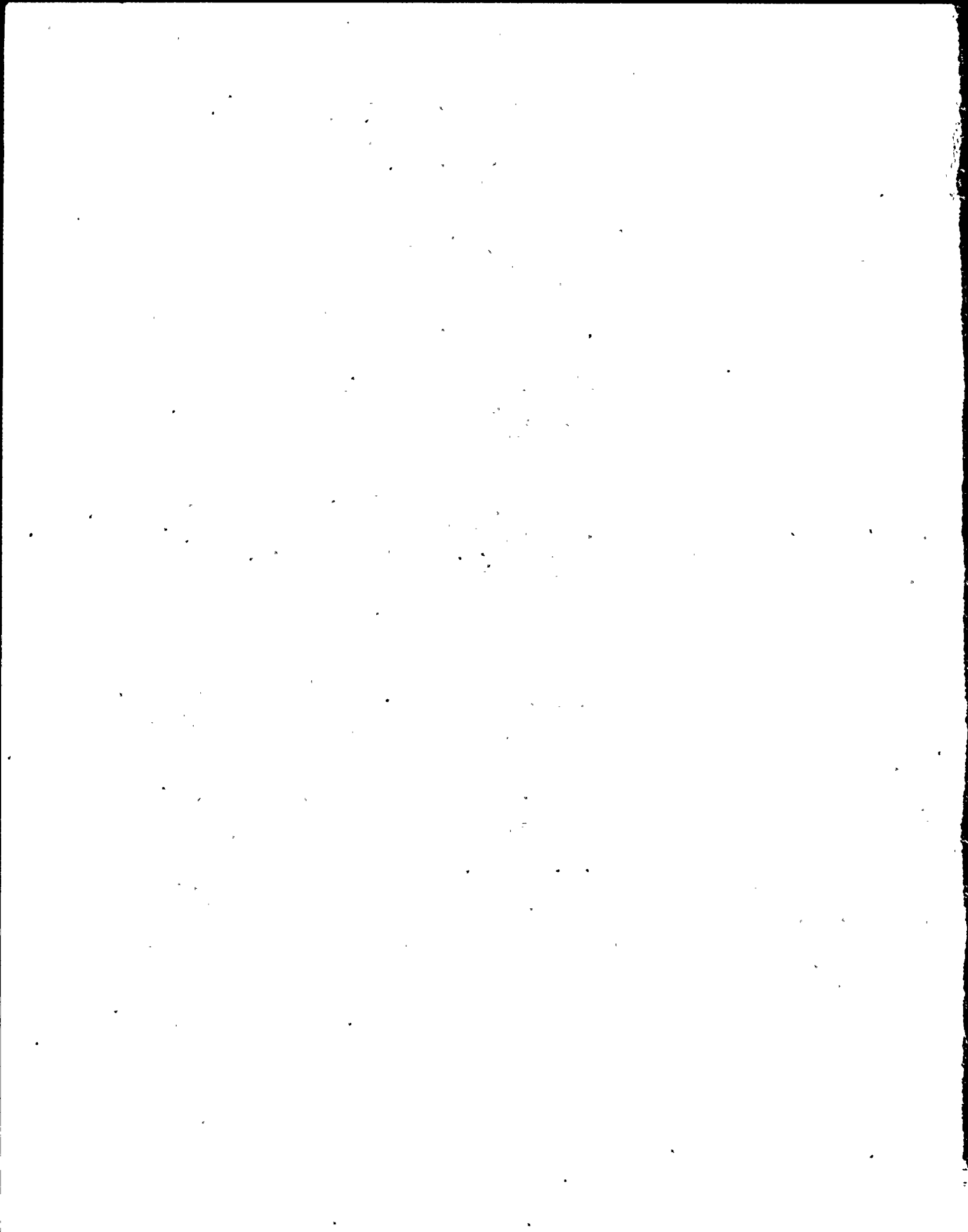
79-171 *S-3.1N, Isolation of Miscellaneous Valves in Boric
Acid Storage Tank Room, PCN 79-201.

79-172 *SM 77-1682.5, Penetration Testing Modification -
Penetration 143, PCN 79-202.

79-173 *S-3.1B, Pre-Operational Line Up of Boric Acid System,
PCN 79-195.

*S-3.1G, Isolating, Flushing and Draining the "A" Boric
Acid Storage Tank for Major Valve Tank or Piping Main-
tenance and Inspection, PCN 79-196.

*S-3.1H, Isolating, Flushing and Draining the "B" Boric
Acid Storage Tank for Major Valve Tank or Piping Main-
tenance and Inspection, PCN 79-197.



PORC
ITEM #

PROCEDURE

*S-3.1K, 1A Boric Acid Transfer Pump Isolation/Restoration, PCN 79-198.

*S-3.1L, 1B Boric Acid Transfer Pump Isolation/Restoration, PCN 79-199.

*S-3.1M, Isolation of Batching System, PCN 79-200.

79-183

A-1011, Performance of Test, PCN 79-06.

A-1401, Station Holding Rules, PCN 79-116.

M-53.3, 14' Diameter Equipment Air Lock Removal and Replacement, PCN 79-260.

CP-1039, Calibration and/or Maintenance of #4 Gas Decay Tank Pressure Channel "I039", PCN 79-112.

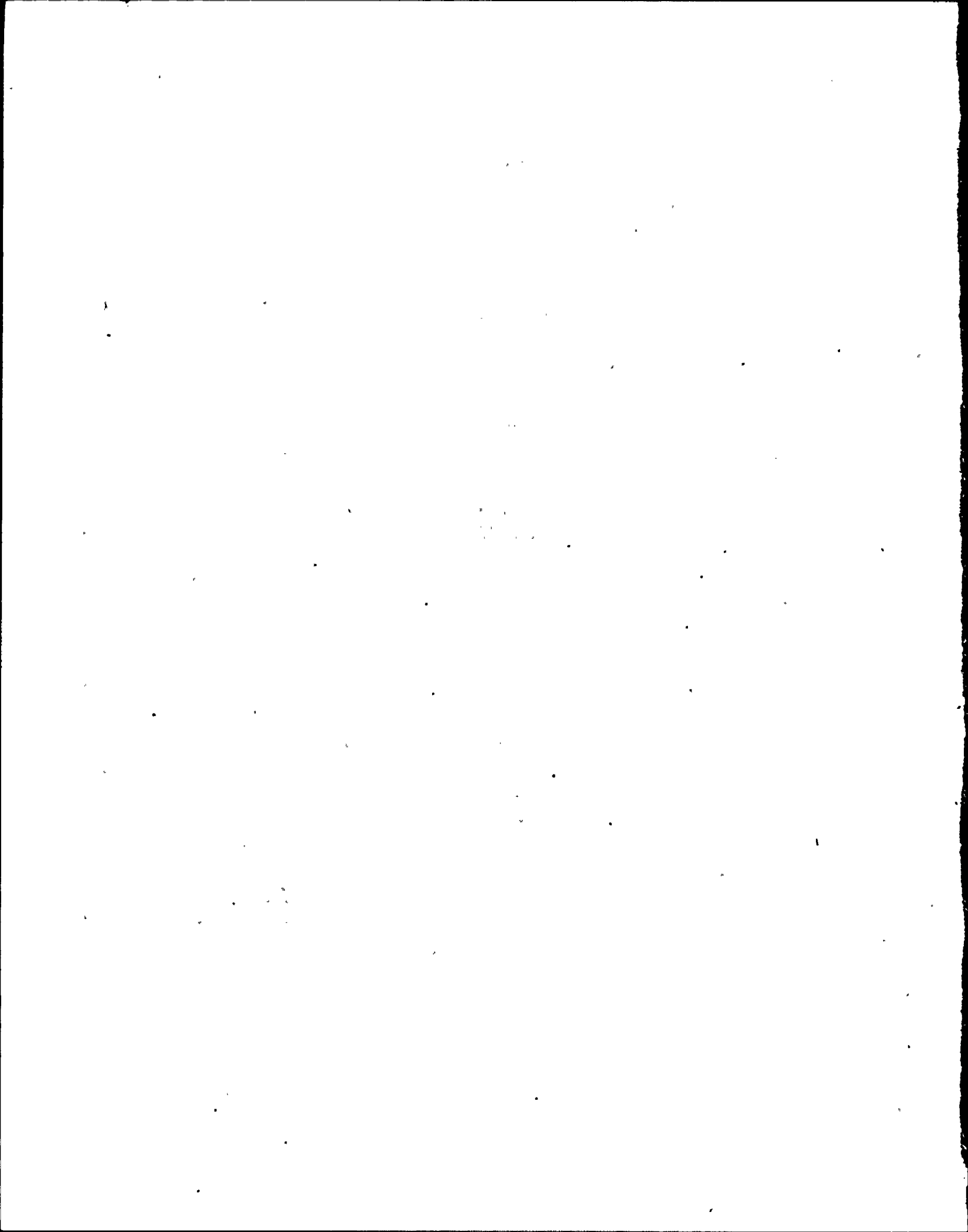
CP-1038, Calibration and/or Maintenance of #3 Gas Decay Tank Pressure Channel "I038", PCN 79-113.

CP-1037, Calibration and/or Maintenance of #2 Gas Decay Tank Pressure Channel "I037", PCN 79-114.

CP-1036, Calibration and/or Maintenance of #1 Gas Decay Tank Pressure Channel "I036", PCN 79-114.

CP-12.0, Rod Insertion Limit and Delta T Reactor Control Rack, PCN 79-150.

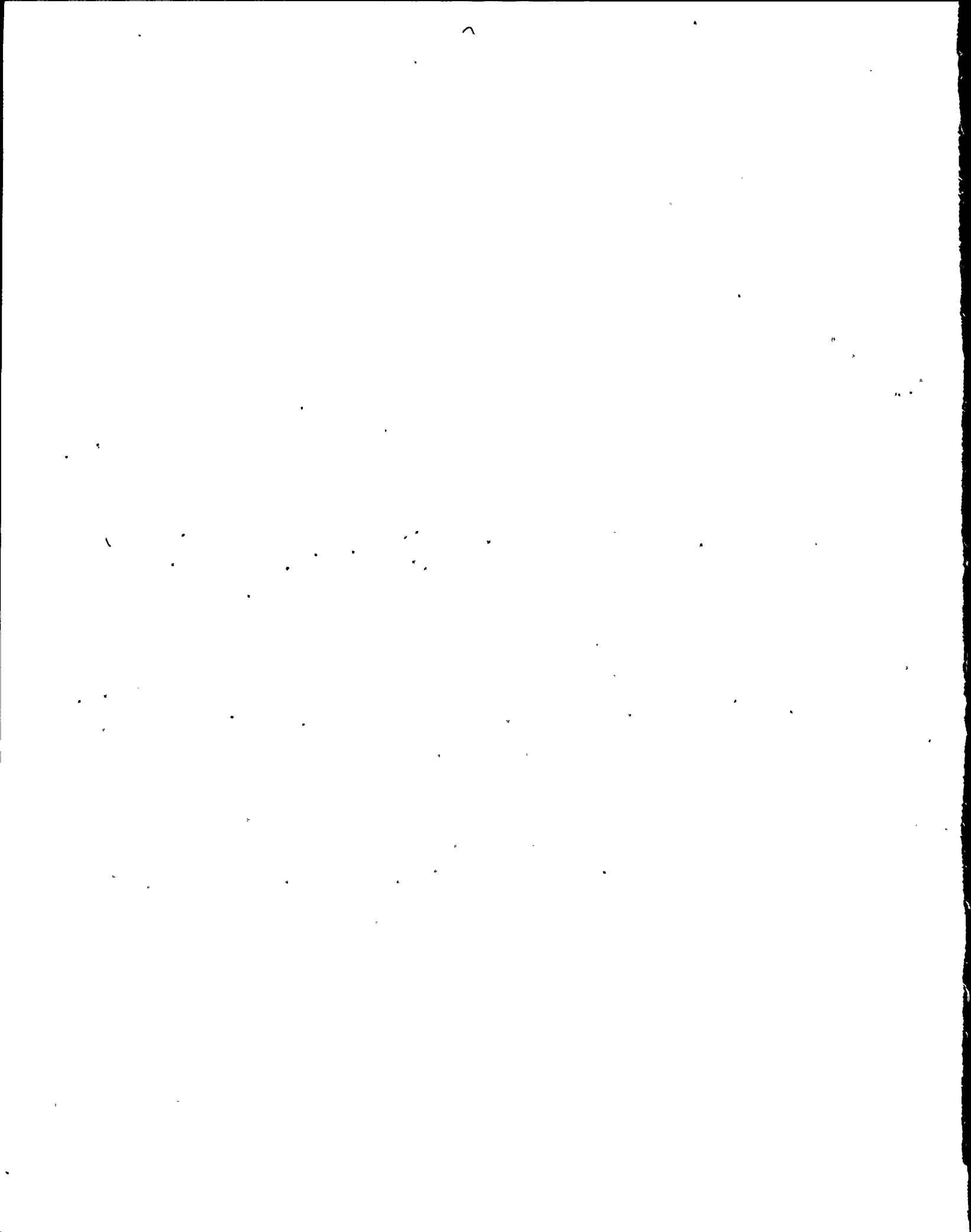
RSSP-13.8.2, Hydro Test of Loop "A" Underground Service Water Piping (Screen House to Turbine Building), PCN 79-185.



FORC
ITEM #

PROCEDURE

- HP-11.7, Drager Multi Gas Detector Use, PCN 79-205.
- HP-12.7, Constant Flow Breathing Air System Setup, PCN 79-206.
- CP-11.8, Condensed Hydrocarbon Content of Breathing Air, PCN 79-217.
- 79-184 *SM-2714.1, "A" Steam Generator J-Tube Modification, PCN 78-2693.
- 79-185 *SM-2714.2, "B" Steam Generator J-Tube Modification, PCN 78-2692.
- 79-187 CP-12.1, Delta T Signal, Summer Unit TM-405D, PCN 79-232.
- CP-12.2, Delta T Repeater Unit TM-405E, PCN 79-233.
- CP-12.3, Delta T Repeater Unit TM-405S, PCN 79-234.
- CP-12.4, Deviation Bistable TC-405F, PCN 79-235.
- CP-12.5, Deviation Bistable TC-406F, PCN 79-236.
- CP-12.6, Deviation Bistable TC-407F, PCN 79-237.
- CP-12.7, Deviation Bistable TC-408F, PCN 79-238.
- CP-12.8, RIL Constant Current Source HC-405B, PCN 79-239.
- CP-12.9, RIL Computer Unit TM-405G, PCN 79-240.
- CP-12.10, RIL Medium Current Selector TM-405V, PCN 79-241.



PORC
ITEM #

PROCEDURE

CP-12.11, RIL E/I Converter TM-405K, PCN 79-242.

CP-12.12, RIL Bistable TC-405 G/N, PCN 79-243.

CP-12.13, RIL Recorder TR-405, PCN 79-244.

CP-12.14, RIL Computer Unit TM-405A, PCN 79-245.

CP-12.15, RIL Medium Current Selector TW-405W, PCN 79-246.

CP-12.16, RIL E/I Converter TM-405L, PCN 79-247.

CP-12.17, RIL Bistable TC-405 H/M, PCN 79-248.

CP-12.18, RIL Computer Unit TM-405I, PCN 79-249.

CP-12.19, RIL Medium Current Selector TW-405X, PCN 79-250.

CP-12.20, RIL E/I Converter TM-405M, PCN 79-251.

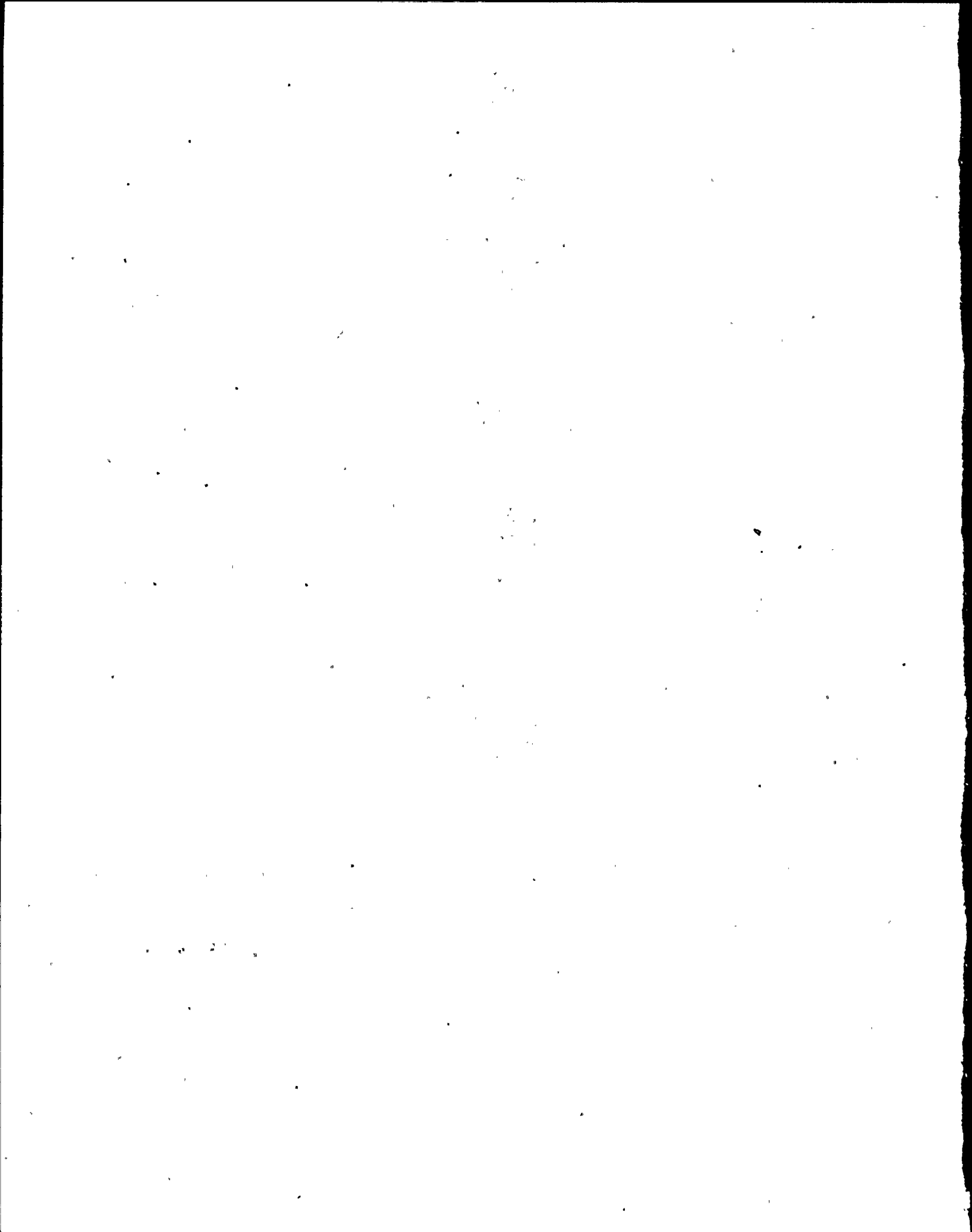
CP-12.21, RIL Bistable TC-405 I/L, PCN 79-252.

CP-12.22, RIL Computer Unit TM-405J, PCN 79-253.

CP-12.23, RIL E/I Converter TM-405N, PCN 79-254.

CP-12.24, RIL Bistable TC-405 J/O, PCN 80-255.

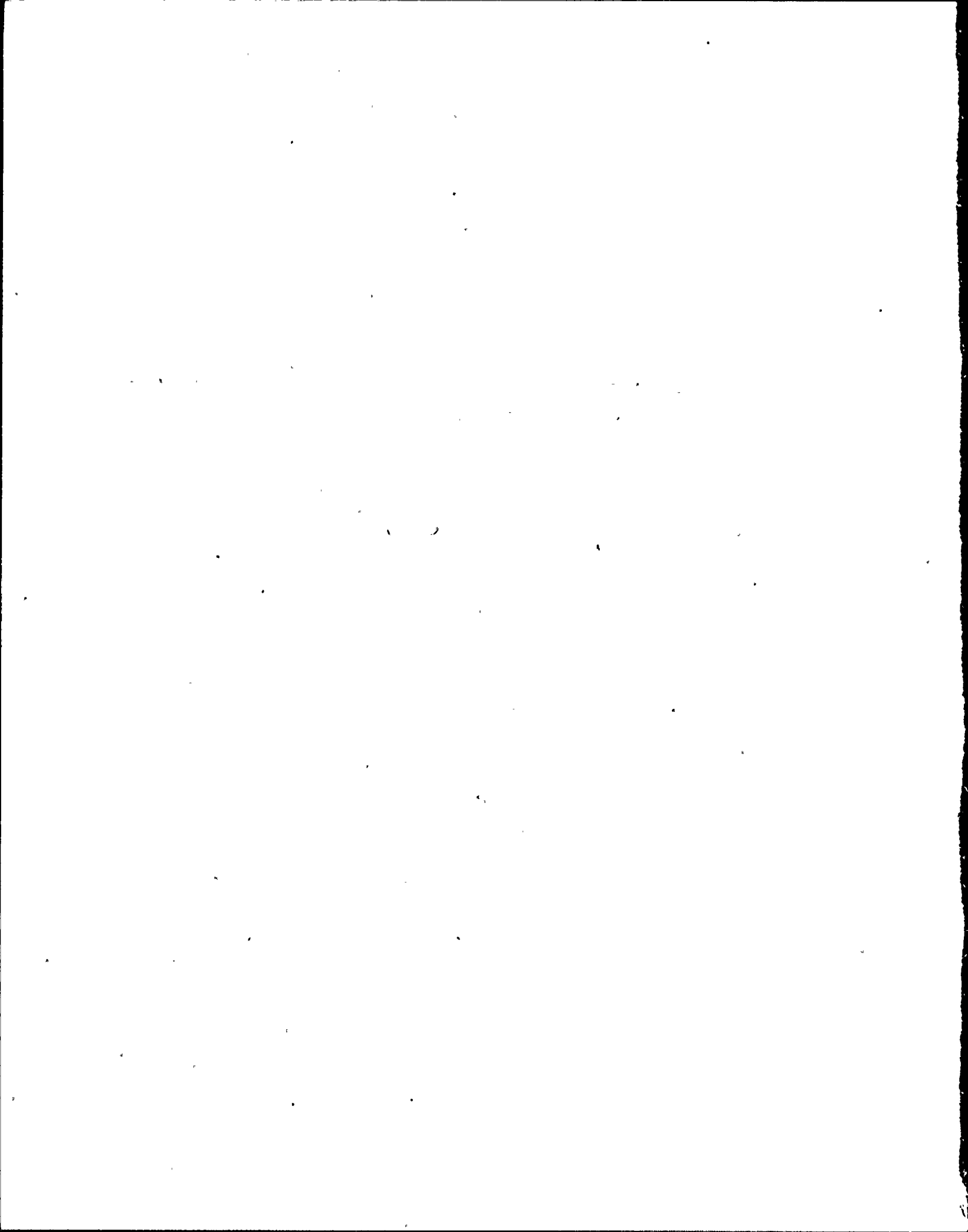
CP-12.25, RIL Bistable TC-405K, PCN 79-256.



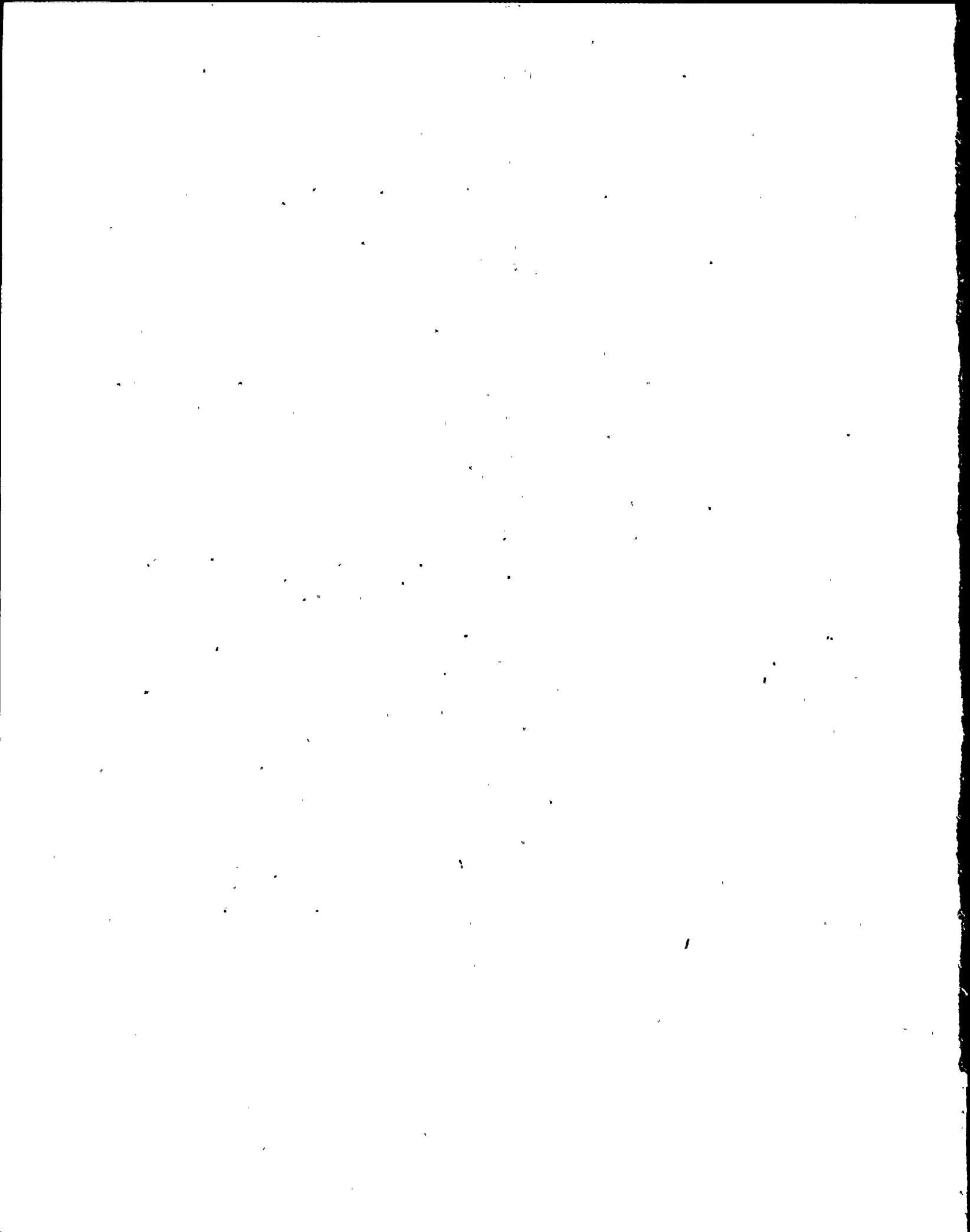
PORC
ITEM #

PROCEDURE

- 79-188 MHE-100-1, Maintenance and Inspection of 40/5 Ton Shaw Box Crane in Auxiliary Building, PCN 78-2285.
- MHE-100-2, Maintenance and Inspection of 100/20 Ton Whiting Crane in Containment, PCN 78-2286.
- MHE-200-1, Inspection and Maintenance of 5 Ton Robbins and Meyers Hoist in Auxiliary Building Drumming Station, PCN 78-2288.
- MHE-500-1, Inspection and Maintenance of the 1 Ton P&H Electric Operated Cable Hoist in the CV, PCN 78-2292.
- 79-189 *HP-12.6, Issuance, Proper Use, and Return of Respirators, PCN 79-106.
- 79-190 *SM 78-1657.1, Installation of 42" Duct for Control Room Ventilation, PCN 79-218.
- 79-191 *PT-6.2, NIS Intermediate Range Channels Channel No. , PCN 79-218.
- 79-192 *GS-35.0, Issuing and Destruction of Ginna Photo-Identification Cards, PCN 79-209.
- 79-193 *HP-11.4, High Volume Air Sampling, PCN 79-212.
- 79-194 *HP-11.2, Iodine in Air - Charcoal Cartridge Method, PCN 79-213.
- 79-195 *PC-1.1, Primary Coolant Analysis Schedule and Limits, PCN 79-214.
- 79-196 *RSSP-13.10, ISI Hydro Test - Component Cooling Water, PCN 79-216.



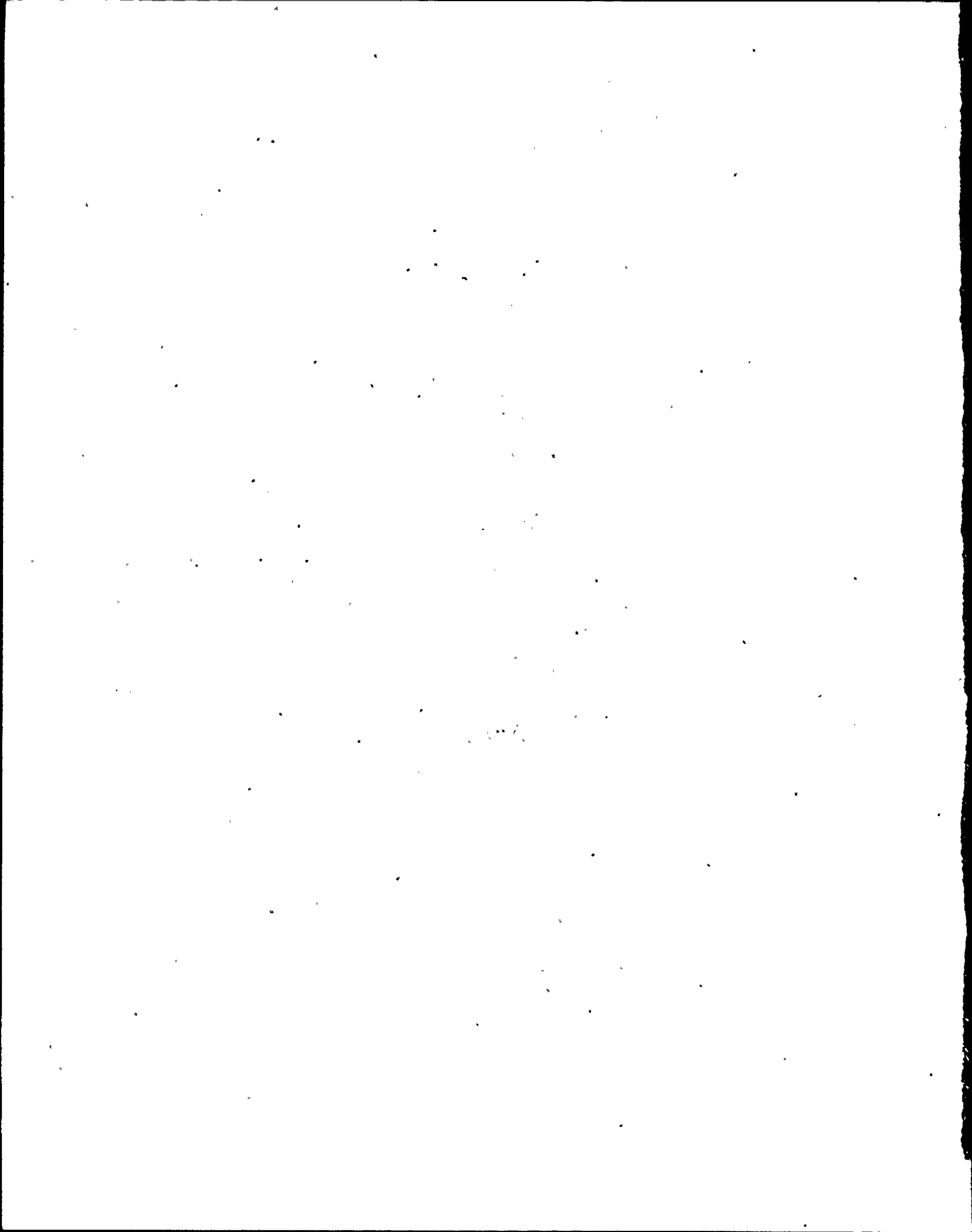
<u>PORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-202</u>	<u>*RF-53, Cycle VIII - IX Refueling, PCN 79-305.</u>
<u>79-204</u>	<u>*SM 77-1623.1, Charging Pump Monorail, PCN 79-308.</u>
<u>79-205</u>	<u>*EM-217, Installation of a New RCS Sample Delay Coil, PCN 79-310.</u>
<u>79-206</u>	<u>*A-601, Plant Procedure Document Control, PCN 79-48.</u>
<u>79-212</u>	<u>*A-1, Radiation Control Manual, PCN 79-225.</u>
<u>79-213</u>	<u>*A-17, Plant Operations Review Committee Operating Proce- dure, PCN 79-259.</u>
<u>79-214</u>	<u>*SM 79-1880.1, Part Length Control Rod Drive Shaft Retention for Operation Without Part Length Rods, PCN 79-306.</u>
<u>79-215</u>	<u>*SM 78-2133.1, Valves 866A and 866B Modification for Testing, PCN 79-309.</u>
<u>79-216</u>	<u>*SM 79-1832A.1, Circuit Separation Fuse Installation, PCN 79-333.</u>
	<u>*A-501, Plant Procedures Preparation and Classification, PCN 79-46.</u>
	<u>*A-502, Plant Procedure Content and Format Requirements, PCN 79-47.</u>
	<u>*A-602, Plant Procedure Distribution, PCN 79-49.</u>
	<u>*A-1302, Control of Material Handling and Handling Equip- ment, PCN 79-71.</u>



PORC
ITEM #

PROCEDURE

- *A-1602, Preventive Action Report, PCN 79-90.
- *CP-2039, Check and/or Maintenance of Sump "A" Level Control and Alarm Switches, PCN 79-210.
- *HP-11.9, MSA Oxygen Indicator Use, PCN 79-222.
- *CP-208.0, Calibration of Area Monitor Drawer R- , PCN 79-231.
- *HP-11.10, Air Sampling With Siersat Low Volume Air Sampler, PCN 79-258.
- 79-225 *QC-802, Identification and Marking of Material, PCN 79-301.
- *PT-13.4.13, Alarm Valve System Testing - Fire System #13, Service Building Sprinkler, PCN 79-294.
- *HP-2.2, Whole Body Counter Operation, PCN 79-273.
- 79-226 *HP-5.2, Posting of Radiation Areas and Container Labeling, PCN 79-267.
- *HP-3.2, In-Plant Reporting of Current Exposures, PCN 79-268.
- *M-7.4, Boric Acid Storage Tank Filter Replacement, PCN 79-271.
- 79-232 *M-11.4, Inspection and Maintenance of Ingersol Rand Pumps, PCN's 79-275 and 79-292.
- 79-233 *QC-304, Control of Modification Activities by the Ginna Modification Project, PCN 79-290.



<u>PORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-234</u>	<u>*HP-12.2, Medical Check, Fitting and Training of Personnel Using Respirators, PCN 79-288.</u>
<u>79-235</u>	<u>*PT-30, Containment Spray Nozzles Check of "A" and "B" Rings, PCN 79-287.</u>
<u>79-236</u>	<u>*S-4.4.1, Spent Resin Package-Site Receipt and Shipment, PCN 79-286.</u>
<u>79-237</u>	<u>*S-7B, Reactor Coolant Low Pressure Purification, PCN 79-263.</u>
<u>79-238</u>	<u>*S-7C, Transferring and Purification of RWST Water From RWST to SFP, PCN 79-262.</u>
<u>79-239</u>	<u>*M-45.1B, Residual Heat Removal Pump Motor Maintenance at Cold Shutdown, PCN 79-261.</u>
<u>79-240</u>	<u>*S-12.4, RCS Leakage Surveillance Record Instructions, PCN 79-257.</u>
<u>79-241</u>	<u>*S-4.4, Spent Resin Removal to Shipping Casks, PCN 79-227.</u>
<u>79-242</u>	<u>*GS-41.0, Employee Screening Program, PCN 79-224.</u>
<u>79-243</u>	<u>*GS-39.0, Protected Vital/Area Access, PCN 79-223.</u>
<u>79-244</u>	<u>*A-1.1, Locked Radiation Area, PCN 79-219.</u>
<u>79-245</u>	<u>*PT-13.7, Fire Hose Reel Assembly Inspection, PCN 79-284.</u>
<u>79-247</u>	<u>*SM 78-1845.1, Guide Stud Storage Racks, PCN 79-276.</u>

PORC
ITEM #

PROCEDURE

79-248 PT-12.1, Emergency Diesel Generator 1A, PCN 79-278.

PT-12.2, Emergency Diesel Generator 1B, PCN 79-285.

79-250 *SM 79-2428, RCP Lube Oil Level Indication.

79-260 *SM 77-1431.1, AFW Flow Control Valve - Mechanical Installation, PCN 79-337.

79-264 *CP-12, Rod Insertion Limit and Delta T Reactor Control Rack, PCN 79-374.

79-265 *CP-12.24, R. I. L. BISTABLE TC-405 J/O, PCN 79-373.

79-266 *CP-209.4, Nuclear Measurements Corporation Air Monitor, PCN 79-339.

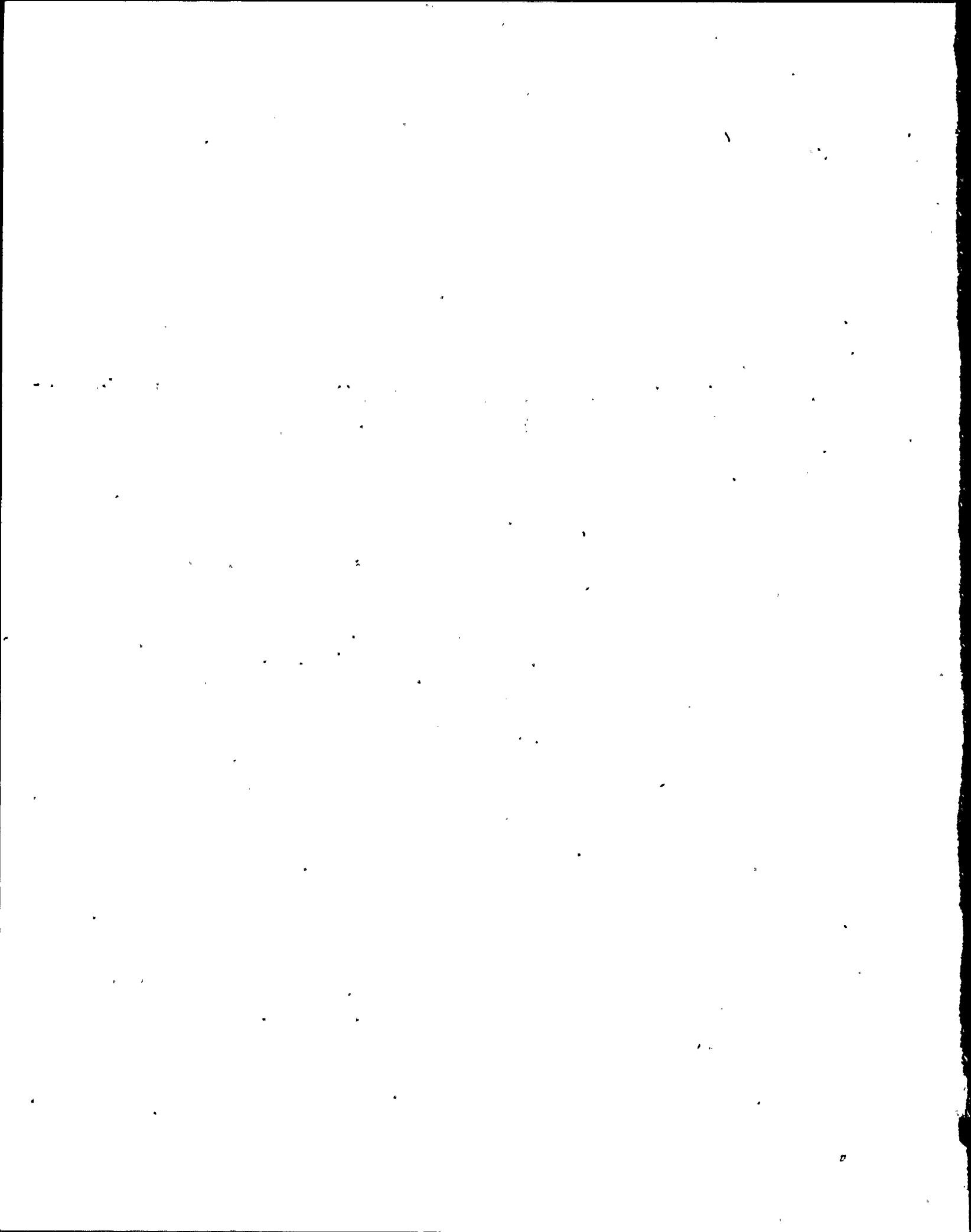
79-267 *CP-213.4, Check of R-13 and R-14 Flow Control System, PCN 79-378.

79-268 *CP-221.2, Calibration of R-21 Detector, PCN 79-359.

79-269 *CP-468.0, Calibration And/Or Maintenance of Steam Generator Pressure Channel "468", PCN 79-314.

79-270 CP-468.3, Steam Generator Pressure Transmitter PT-468, PCN 79-382.

CP-469.3, Steam Generator Pressure Transmitter PT-469, PCN 79-383.



PORC
ITEM #

PROCEDURE

CP-478.3, Steam Generator Pressure Transmitter PT-478,
PCN 79-384.

CP-479.3, Steam Generator Pressure Transmitter PT-479,
PCN 79-385.

CP-482.3, Steam Generator Pressure Transmitter PT-482,
PCN 79-386.

CP-483.3, Steam Generator Pressure Transmitter PT-483
PCN 79-387.

79-271 *HP-3.1, Exposure Reports to Individuals and the Nuclear
Regulatory Commission, PCN 79-326.

79-272 *HP-4.2, Self Reading Dosimeter Use, PCN 79-293.

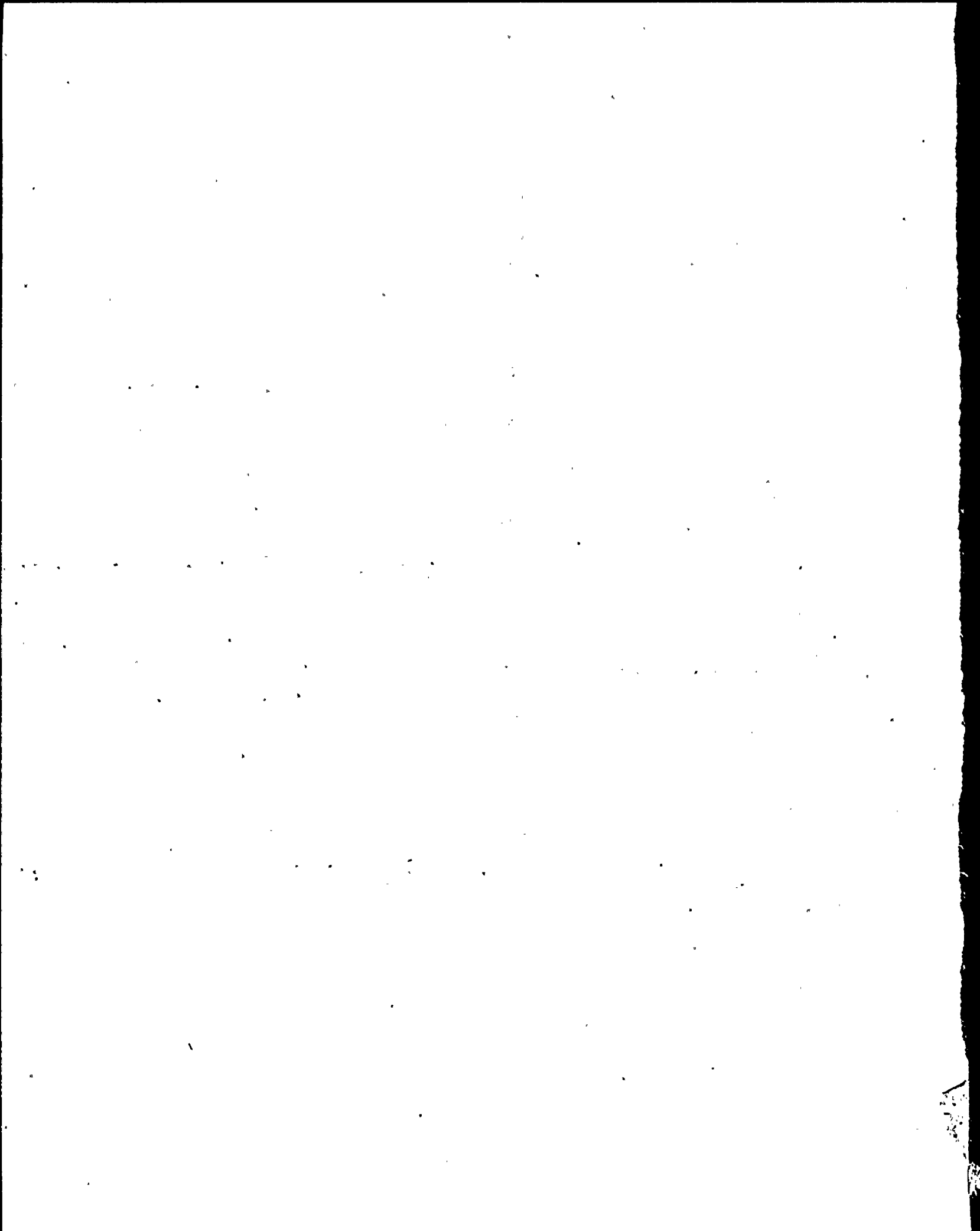
79-273 *ISI-1.1.9, Quality Group "A" Inservice Inspection,
PCN 79-365.

79-274 *M-11.4.5, Charging Pump Varidrive Low Speed and High
Speed Mechanical Stop Setting Adjustment, PCN 79-335.

79-275 M-38.6, 1A Battery Charger Maintenance or Repair, PCN
79-319.

M-38.7, 1B Battery Charger Maintenance or Repair, PCN
79-320.

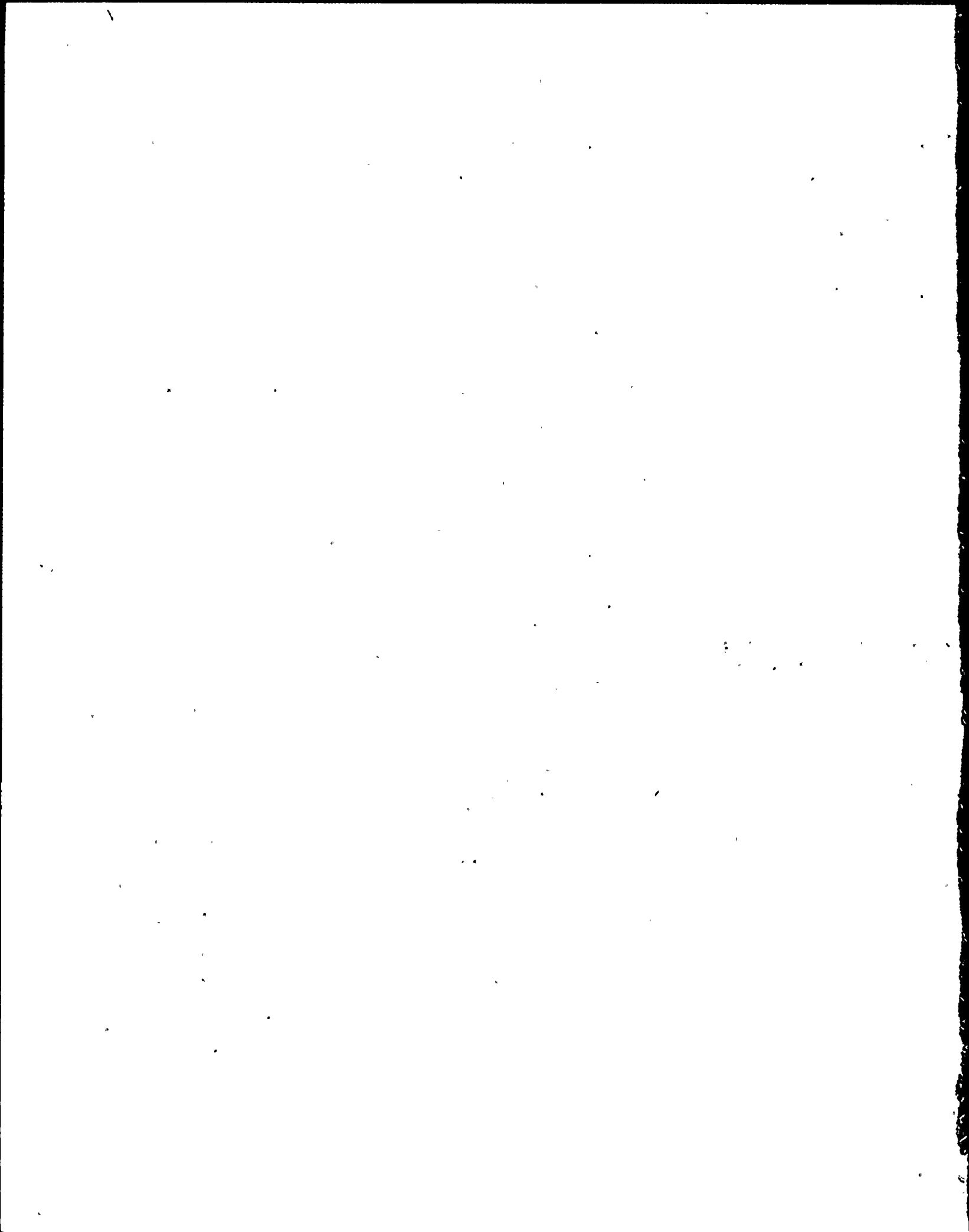
79-276 *M-37.20, Inspection and Repair of Main Steam Isolation
Valves, PCN 79-316.



PORC
ITEM #

PROCEDURE

- 79-277 *M-67.4, Alarm Valve System Maintenance, System Numbers 12, 13, Warehouse, PCN 79-317.
- 79-278 *RD-10, Shipping Radioactive Material, PCN's 79-307 and PCN 79-318.
- 79-283 S-9E, Draining Spent Fuel Pit Transfer Canal to Refueling Water Storage Tank, PCN 79-311.
- S-9F, Transferring Spent Fuel Pit Transfer Canal to the CVCS Holdup Tank, Waste Holdup Tank or Monitor Tanks, PCN 79-312.
- 79-286 *CP-12.13, R.I.L. Recorder TR-405, PCN 79-371.
- *ISI-5, Eddy Current Testing of Series 44 Steam Generator Tubes, PCN 79-399.
- *M-11.4.4, Charging Pump Vari-Drive Overhaul Unit # , PCN 79-347.
- *M-15.1, "A" or "B" Diesel Generator Removal From Service, PCN 79-364.
- *M-37.1A, Repacking of Pressurizer Spray Valves 431A and 431B, PCN 79-397.
- *M-37.57, Containment DI Check Valve 8419 Maintenance, PCN 79-380.
- *O-2.5, Plant Shutdown From Hot Shutdown to Cold Shutdown When Condenser Steam Dump is Unavailable, PCN's 79-327 and 79-346.



FORC
ITEM #

PROCEDURE

*O-7, Alignment and Operation of the Reactor Vessel Over-pressure Protection System, PCN 79-315.

*PR-2, Protective Relay Calibration and Trip Test 1A Emergency Diesel, PCN 79-381.

*PT-23.15, Containment Isolation Valve Leak Rate Testing Containment Air Sample Out, PCN 79-336.

*PT-29.3, Auxiliary Building Ventilation Main HEPA Filter Bank Testing, PCN 79-360.

*RF-53, Cycle VIII - IX Refueling, PCN's 79-343, 79-344, 79-345 and 79-391.

*CP-11.8, Pressurizer Bistable PC-431B, PCN 79-390.

*CP-11.0, Calibration and/or Maintenance of P.L.P. Rack Modules Not Covered by Other Calibration Procedures, PCN's 79-392 and 79-393.

79-287

CP-12.7, Deviation Bistable TC-408F, PCN 79-366.

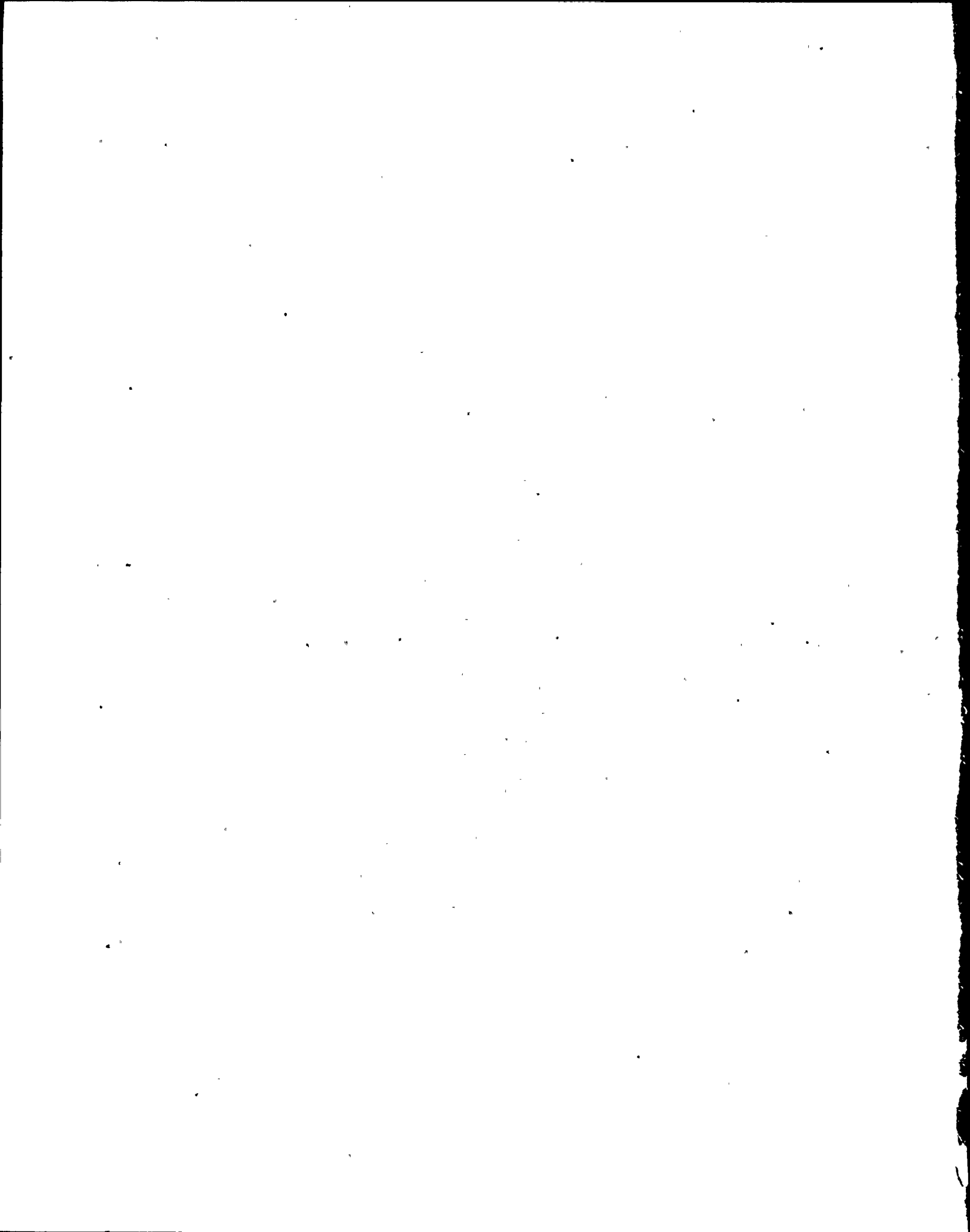
CP-12.6, Deviation Bistable TC-407F, PCN 79-367.

CP-12.5, Deviation Bistable TC-406F, PCN 79-368.

CP-12.4, Deviation Bistable TC-405F, PCN 79-369.

79-288

*SM 77-1682.10, Hydrostatic Testing of Penetration 143 Piping Modification, PCN 79-297.



PORC
ITEM #

PROCEDURE

79-289 A-102.9, Maintenance Training Program, PCN 79-174.

A-102.11, Emergency Plan Training Program, PCN 79-221.

A-102.12, Firewatch Training, PCN 79-220.

A-1104, Ginna Station Technical Specification Surveillance Program, PCN 79-29.

A-1601, Corrective Action Report, PCN 79-89.

GS-43.0, Off Loading Portal Procedure, PCN 79-208.

PT-17.4, Liquid Waste Flow Indicator and Rotameter Check, PCN 79-274.

RF-54, Baffle Repair, PCN 79-304.

79-290 *SM 79-1832A.2, Circuit Separation Installation, PCN 79-351.

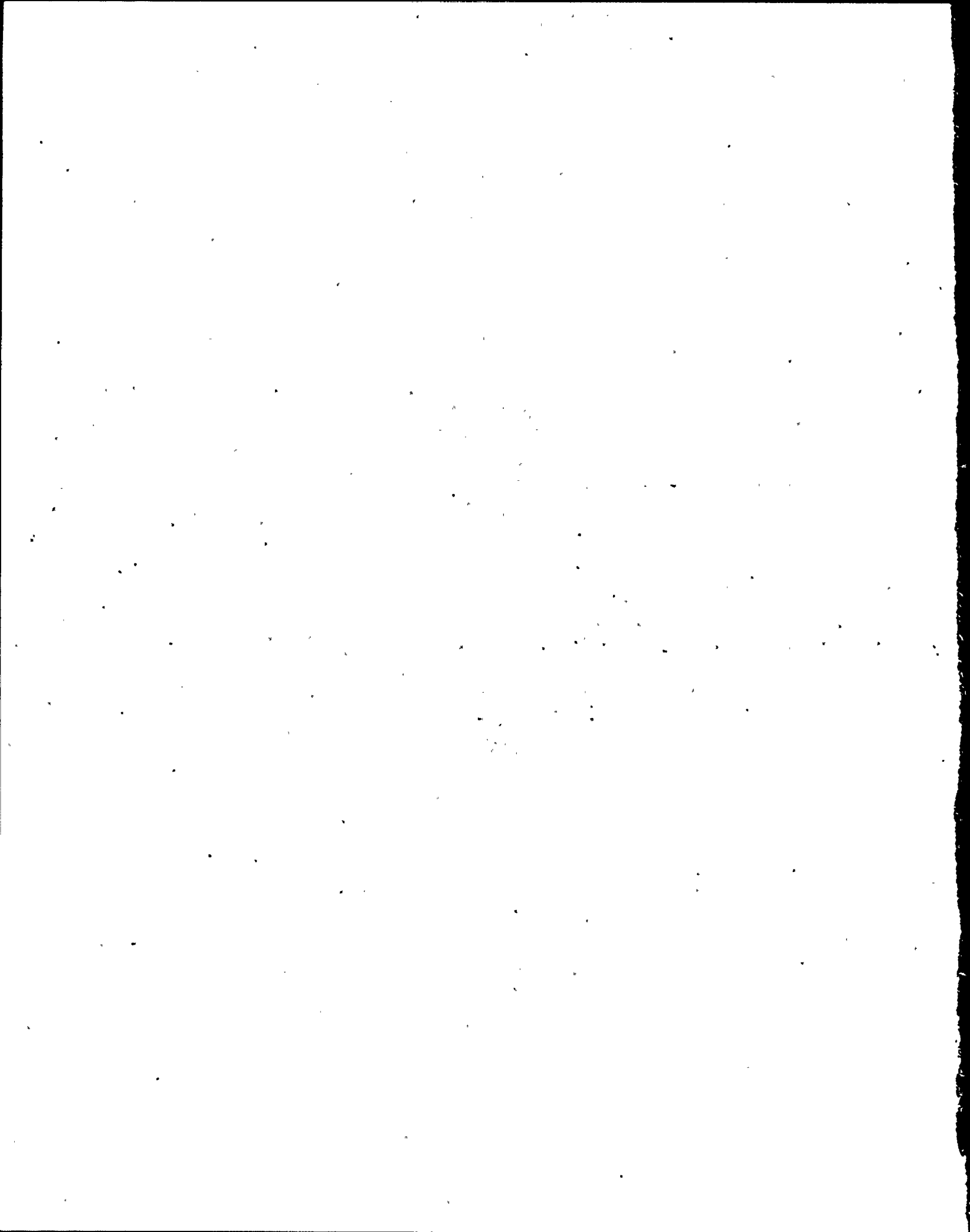
79-302 *CP-401.7, Tavg Repeater, TM-401C, PCN 79-418.

79-306 *A-102.2, R. E. Ginna Administrative Controls Training Program, PCN 79-396.

79-307 *A-1701, Quality Assurance Records, PCN 79-395.

79-308 *SM 77-1891.1, Electric Heat Tracing for Valve 825 A & B, PCN 79-443.

79-309 PT-2.8, Component Cooling Water Pump System, PCN 79-448.



PORC
ITEM #

PROCEDURE

RF-2D, Filling the Refueling Canal, PCN 79-247.

RF-8.0, Fuel Assembly and Core Component Movement Prerequisites and Precautions, PCN 79-430.

RF-8.2, Fuel Handling Instruction Pre-Loading & Periodic Valve Test, PCN 79-429.

RF-8.3, Core Loading Periodic Equipment Status Check, PCN 79-248.

SM 1594A.4, Qualification and Leak Testing of Standby SFP Cooling Components, PCN 79-269.

SM 78-1872.1, Manipulator Crane Gripper System Modification, PCN 79-407.

SM 79-2417.1, "A" Steam Generator J-Tube Modification, PCN 79-415.

SM 79-2417.1, "A" Steam Generator J-Tube Modification, PCN 79-437.

SM 79-2417.2, "B" Steam Generator J-Tube Modification, PCN 79-414.

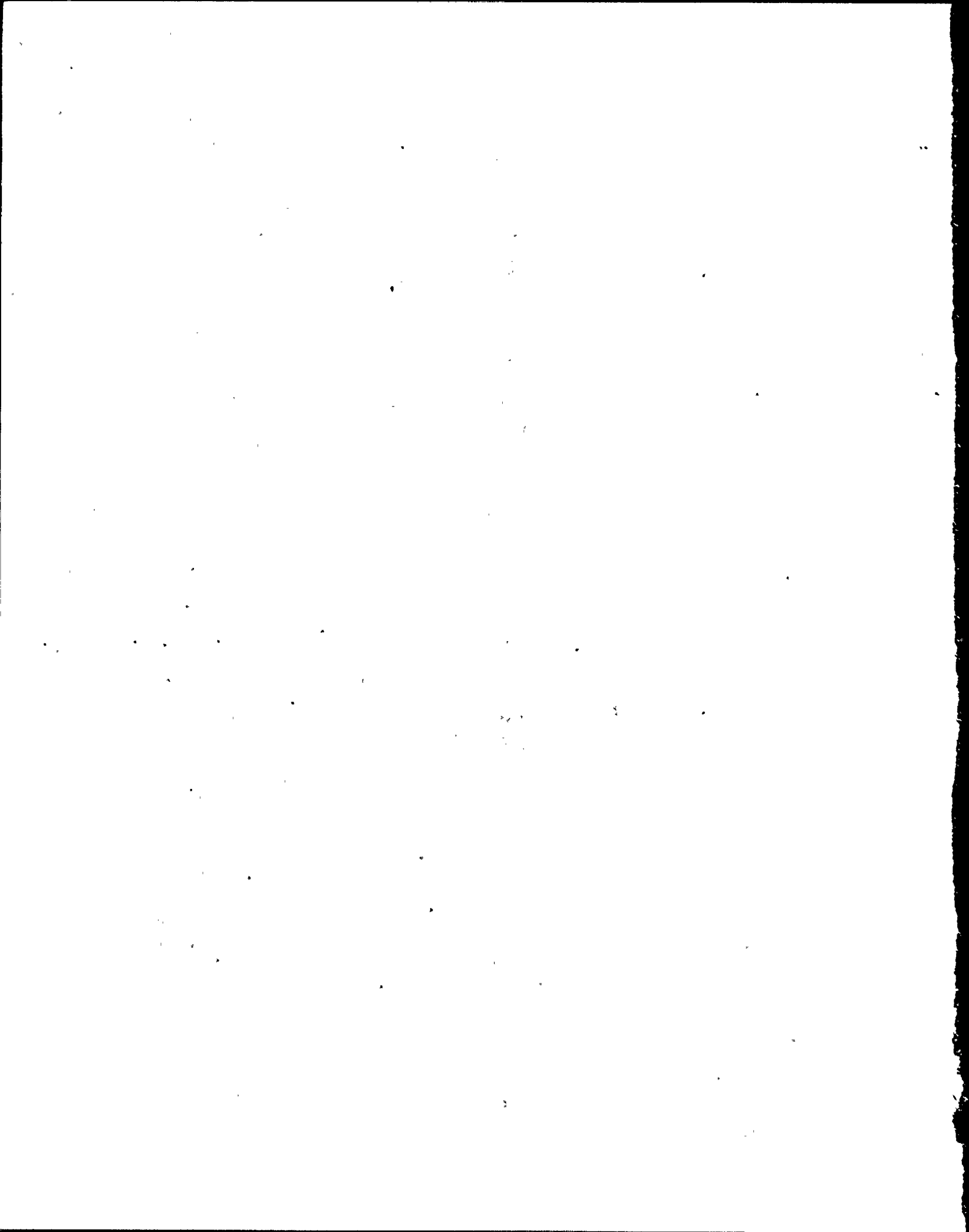
SM 79-2417.2, "B" Steam Generator J-Tube Modification, PCN 79-438.

79-310

*A-48, Test Tag Control Program, PCN 79-246.

79-311

CP-31.3, Calibration of Source Range N-31, PCN's 79-403 and 79-404.



FORC
ITEM #

PROCEDURE

CP-32.3, Calibration of Source Range N-32, PCN's 79-419 and 79-420.

CP-35.4, Intermediate Range N-35 Bistable Relay Drivers Calibration, PCN 79-423.

CP-36.4, Intermediate Range N-36 Bistable Relay Drivers Calibration, PCN 79-424.

ISI-5, Eddy Current Testing of Series 44 Steam Generator Tubes, 79-405.

M-43.15.1, Isolation of Steam Generator Primary Manway Cover Closure, PCN 79-406.

PI-23.19, Containment Isolation Valve Leak Rate Testing Safety Injection System, PCN 79-425.

PI-23.22, Containment Isolation Valve Leak Rate Testing R.C.D.T. Discharge, PCN 79-416.

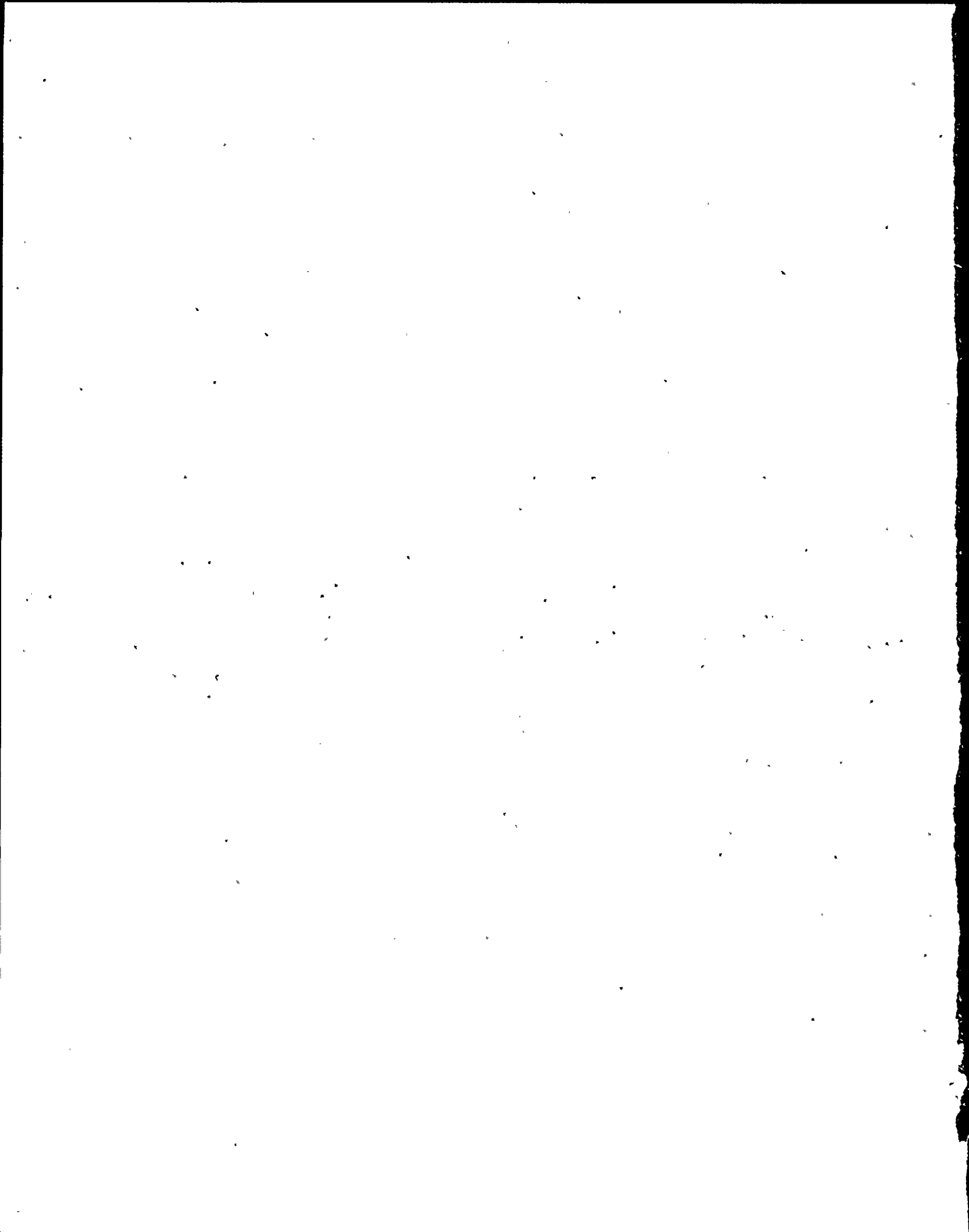
PI-23.23, Containment Isolation Valve Leak Rate Testing Sump "A" Discharge, PCN 79-417.

RF-2E, Draining of Refueling Canal, PCN 79-442.

RF-9.1, Step by Step Fuel Loading and Maps, PCN 79-434.

RF-53, Cycle VIII - IX Refueling, PCN's 79-431, 79-432, 79-433 and 79-436.

S-9S, Standby (Skid-Mounted) SFP Heat Exchanger Operation, PCN 79-444.



PORC
ITEM #

PROCEDURE

S-9T, Standby (Skid-Mounted) SFP Heat Exchanger Removal, PCN 79-445.

S-27.209, Isolation of CVCS Relief Valve 209, PCN 79-440.

79-312

A-102.4, Indoctrination and Training of Quality Control Personnel, PCN 79-342.

A-102.5, Training of Modification Project Personnel, PCN 79-341.

A-102.6, R. E. Ginna Systems Familiarization Program, PCN 79-350.

A-102.7, R. E. Ginna Auxiliary Operator Qualification Program, PCN 79-349.

A-102.10, Health Physics Technician Training & Responsibility Limits, PCN 79-335.

A-102.13, R. E. Ginna NRC Licensing Training Program, PCN 79-379.

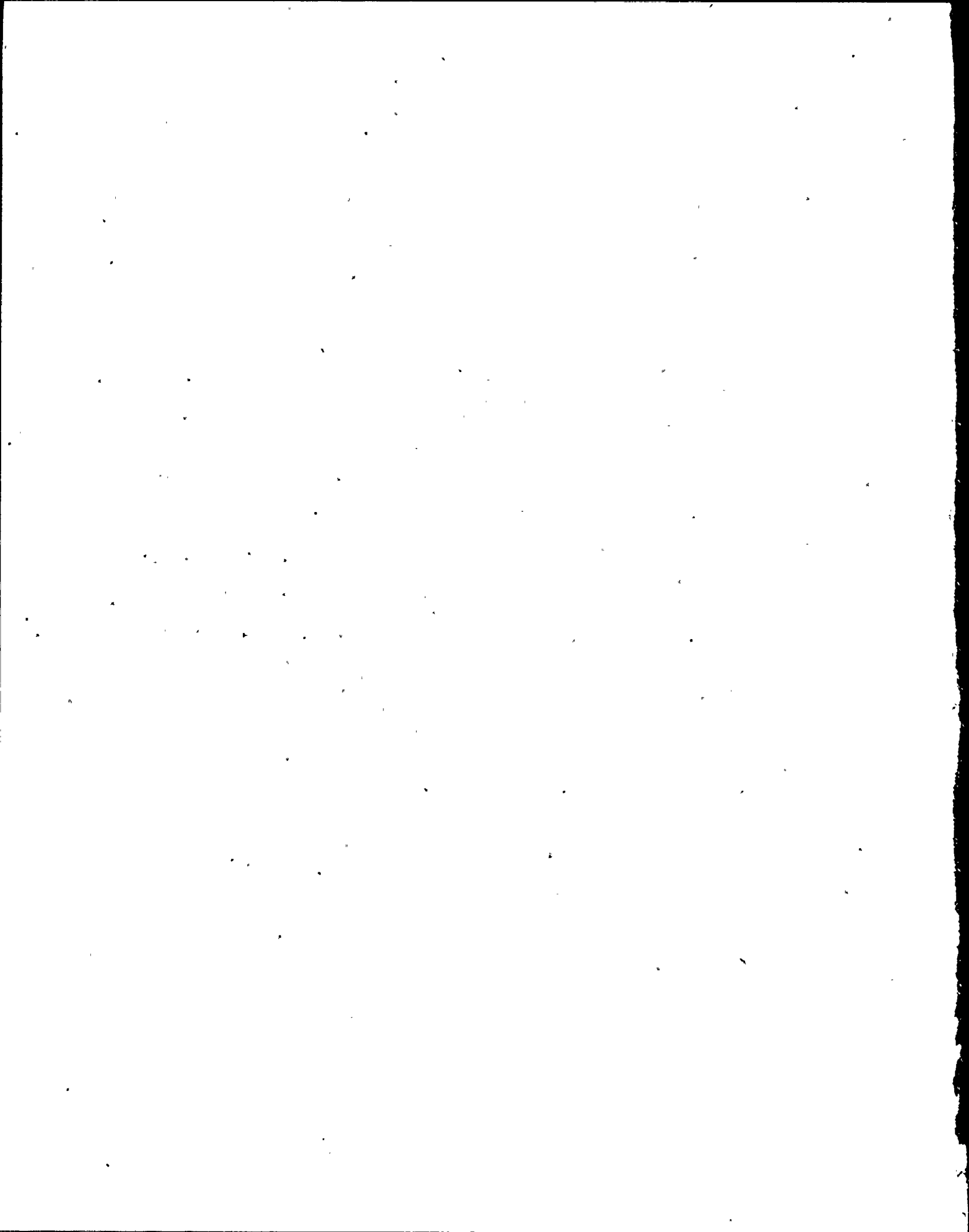
A-102.14, R. E. Ginna Operator Requalification Program, PCN 79-372.

79-312A

CP-221.0, Calibration and/or Maintenance of RMS Channel R-21, PCN 79-357.

CP-221.1, Calibration of Ratemeter Drawer R-21, PCN 79-358.

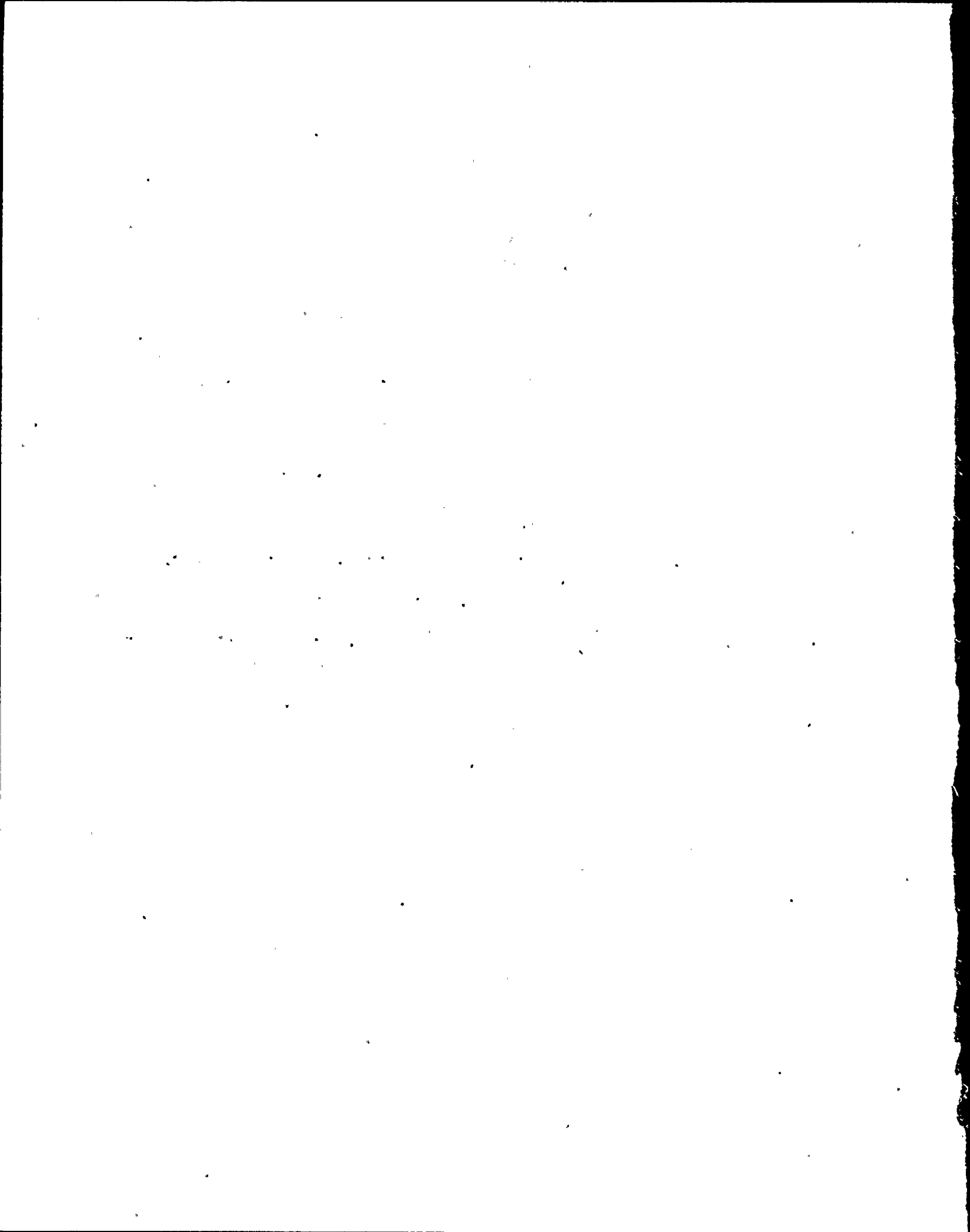
CP-221.3, Replacement of Detector for RMS Channel R-21, PCN 79-362.



PORC
ITEM #

PROCEDURE

- *S-16.17, IC Safety Injection Pump Alternate Suction Relief Valve 1817 Isolation for Maintenance, PCN 79-435.
- 79-317 *A-102.14, R. E. Ginna Operator Requalification Program, PCN 79-372.
- 79-318 *M-43.33, Examination and/or Removal of Objects from the Steam Generator Through Inspection Holes Steam Generator, PCN 79-514.
- 79-320 *CP-12, Rod Insertion Limit and Delta T Reactor Control Rack, PCN 79-411.
- 79-321 *S-3.3E, Burping VCT for Gas Removal, PCN 79-447.
- 79-322 *PT-2.4, Cold/Refueling Motor Operated Valve Surveillance, PCN's 79-476 and 79-480.
- 79-323 *RSSP-1.1, Interlock Verification Residual Heat Removal System, PCN 79-478.
- 79-324 *RF-2D, Filling the Refueling Canal, PCN 79-481.
- 79-325 *S-7C, Transferring and Purification of RWST Water From RWST to SFP, PCN 79-482.
- 79-326 *QCIP-3, Receiving Inspection of New Fuel, PCN 79-489.
- 79-327 *GS-43.0 Off-Loading Portal Procedure, PCN 79-487.
- 79-328 *GS-40.0, Loss of Perimeter Lighting, PCN 79-488.



PORC
ITEM #

PROCEDURE

79-329 *S-3.2D, Transferring Water From CVCS HUT's to RWST or SFP, PCN 79-495.

79-330 *CP-449.0, Pressurizer Pressure Channel "449", PCN 79-500.

79-331 *A-17, Plant Operations Review Committee Operating Procedure, PCN 79-502.

79-332 *HP-7.1, Calibration of Gamma Survey Instruments - Low Range, PCN 79-504.

79-332 HP-4.3, Work Permit Use, PCN 79-441.

HP-2.2, Whole Body Count Operation, PCN 79-497.

CP-408.5, Delta T Repair TM-408P, PCN 79-499.

79-334 PR-2, Protective Relay Calibration and Trip Test 1A Emergency Diesel, PCN 79-477.

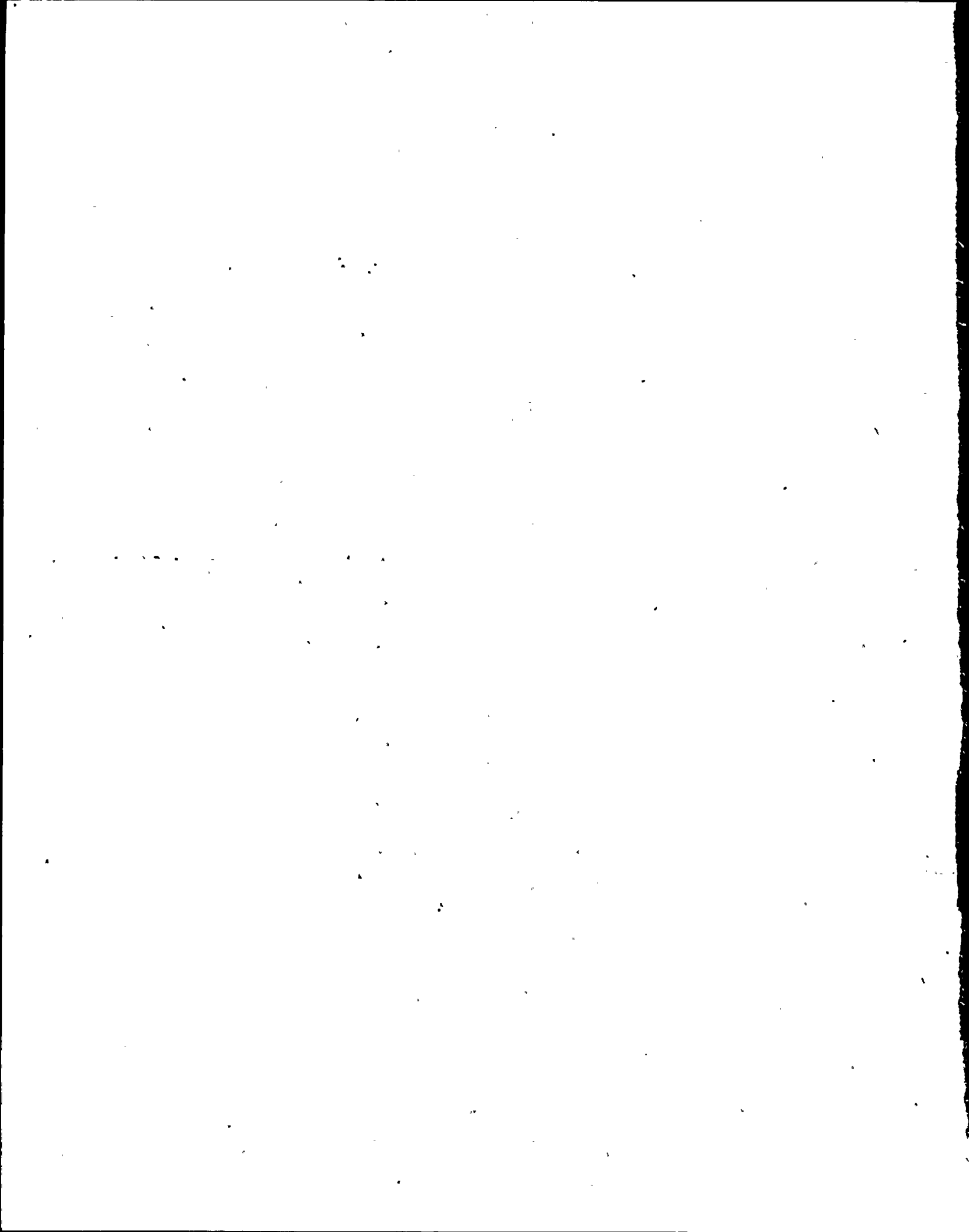
PR-3, Protective Relay Calibration and Trip Test 1B Emergency Diesel, PCN 79-475.

79-335 AR L-24, Holdup Tank 1B High Level, PCN 79-471.

AR L-32, Holdup Tank 1C High Level, PCN 79-472.

AR L-16, Holdup Tank 1A High Level, PCN 79-473.

79-336 HP-9.1, Steam Generator Blowdown Activity, PCN 79-461.



FORC
ITEM #

PROCEDURE

PC-7, Iodine 131 Equivalent Calculation, PCN 79-462.

79-337

RSSP-1.1, Interlock Verification Residual Heat Removal System, PCN 79-483.

SM 78-1464.1, Replacement of Volume Control Tank Level Transmitters, PCN 79-493.

SM 78-1464.2, Replacement of Accumulator Level Transmitters, LT-935, LT-934, LT-938 and LT-939, PCN 79-494.

SM 77-1623.1, Charging Pump Monorails, PCN 79-470.

SM 78-1890.2, Feedwater Line Drain Valves After the Containment Check Valves, PCN 79-503.

SM 78-2133.1, Valves 866A and 866B Modification for Testing, PCN 79-509.

SM 79-2417.2, "B" Steam Generator J-Tube Modification, PCN 79-501.

79-338

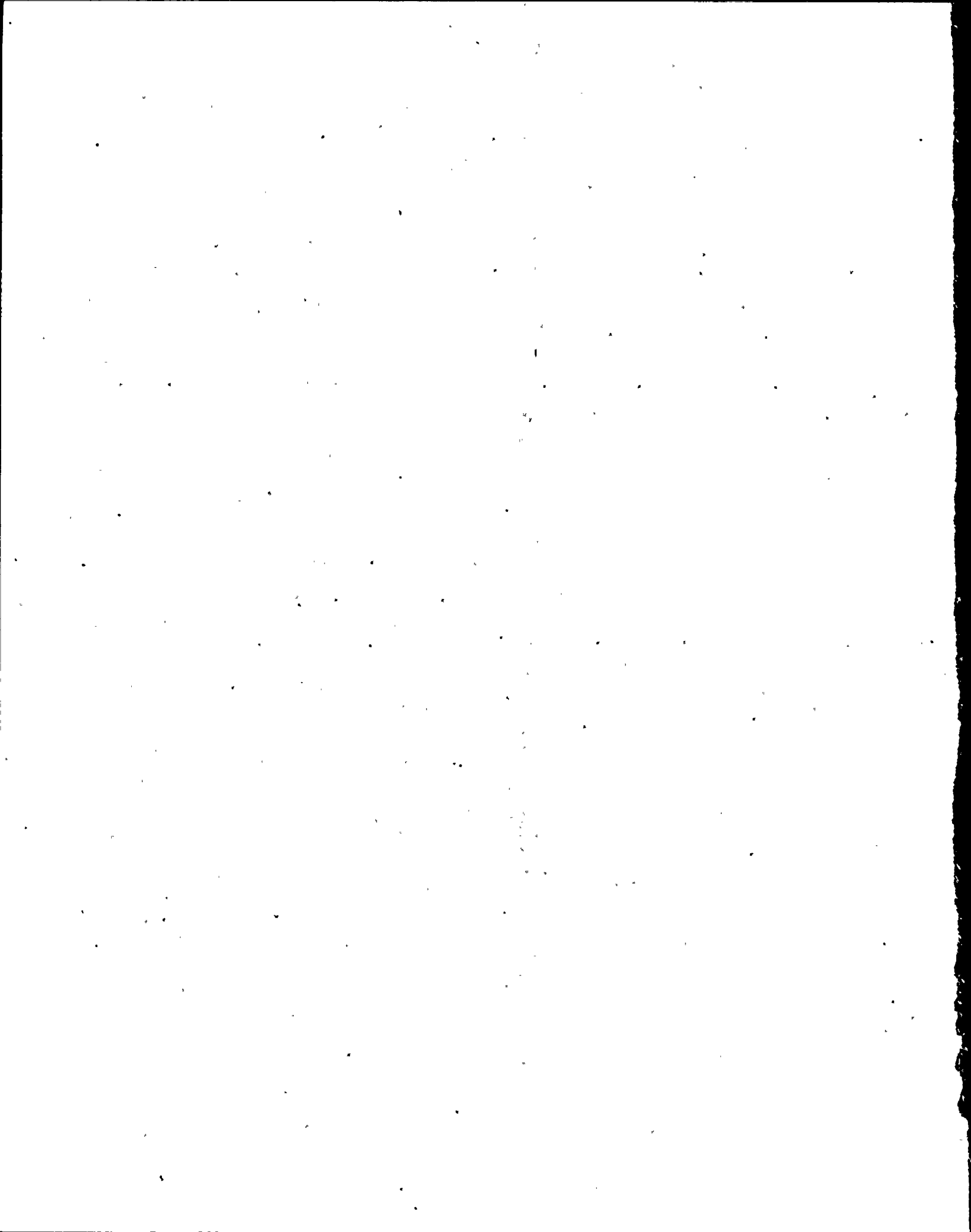
S-13B, RHR Pump Isolation, PCN 79-484.

79-340

A-1105, Calibration Surveillance Program for Instrumentation/Equipment of Safety Related Components, PCN 79-30.

A-901, Control of Welding, PCN 79-50.

A-902, Heat Treating Control, PCN 79-51.



PORC
ITEM #

PROCEDURE

A-904, Welding Equipment Performance Verifications, PCN 79-53.

A-503, Plant Procedure Adherence Requirements, PCN 79-329.

A-605, Control of NRC Correspondence, PCN 79-458.

A-1703, System and Equipment Histories, PCN 79-469.

PT-2.4.1, Cold/Refueling Motor Operated Valve Surveillance, PCN 79-479.

A-1003, Control of Inservice Inspection Activities, PCN 79-490.

A-1002, Qualification of Surveillance and Inspection Personnel, PCN 79-491.

PT-2.9.3, Check Valve Exercising Quarterly Requirement, PCN 79-505.

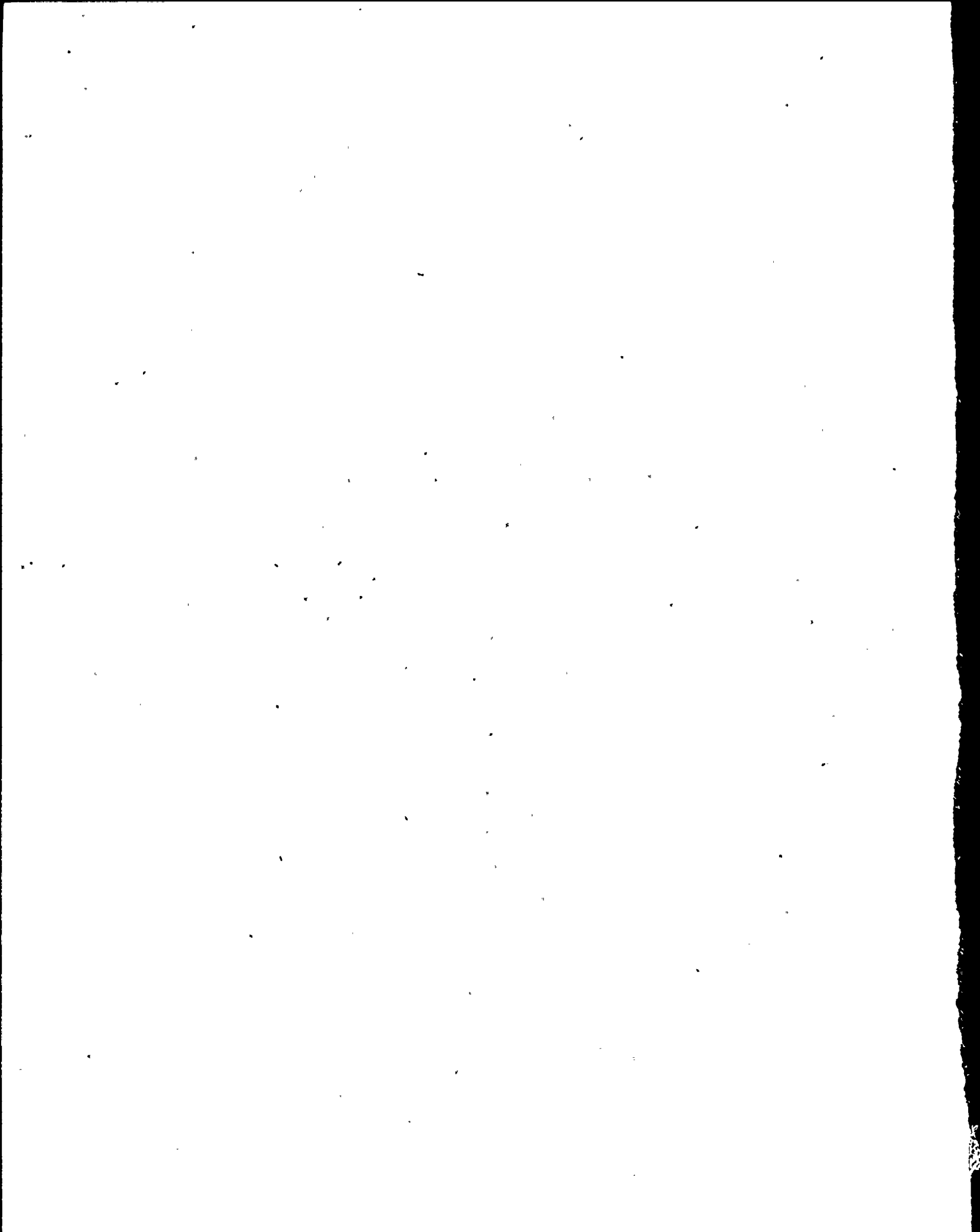
M-37.58, Repair of Aloyco Motor Operated Gate Valve for Valve Number , PCN 79-518.

S-27.823, Isolation of RMW to Component Cooling Valve 823, PCN 79-519.

79-353 *SM 77-1431.2, Hydrostatic Testing of Auxiliary Feedwater Modification, PCN 79-597.

79-356 *A-1601, Corrective Action Report, PCN 79-89.

79-357 *A-19, Changes in Written Procedures, PCN 79-535.



FORC
ITEM #

PROCEDURE

79-358 *A-30.3, Plant Procedure Content and Format Requirements, PCN 79-553.

79-360 *PI-32.0, Reactor Trip Logic Test "A" or "B" Trains, PCN 79-552.

*QC-303, Control of System Modification Activities, PCN-79-545.

*RF-42, Television Visual Fuel Assembly Inspection, PCN-79-538.

*RF-53, Cycle VIII - IX Refueling, PCN 79-551.

*RF-8.3, Core Loading Periodic Equipment Status Check, PCN 79-593.

*RSSP-13.1, ISI Hydro Test of Class "C" Piping (Component Cooling Water - Inside Containment) PCN 79-592.

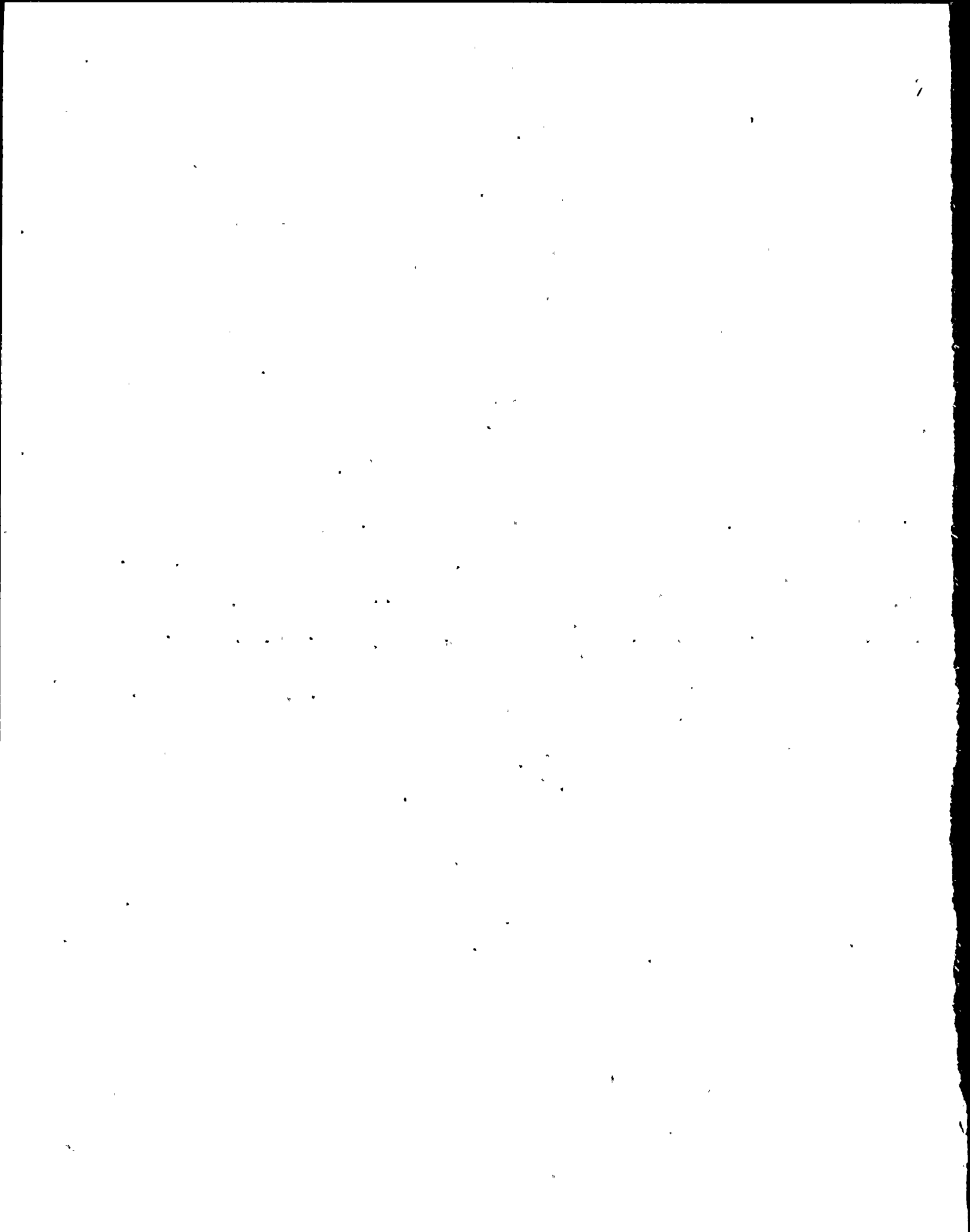
*RSSP-13.9, ISI Hydro Test of Class "C" Piping (Component Cooling Water - Outside Containment) PCN 79-578.

*S-13B, RHR Pump Isolation, PCN 79-605.

79-361 *AR-K-29, Spent Fuel Pit Hi Temp, PCN 79-517.

79-362 *A-601, Plant Procedure Document Control, PCN 79-614.

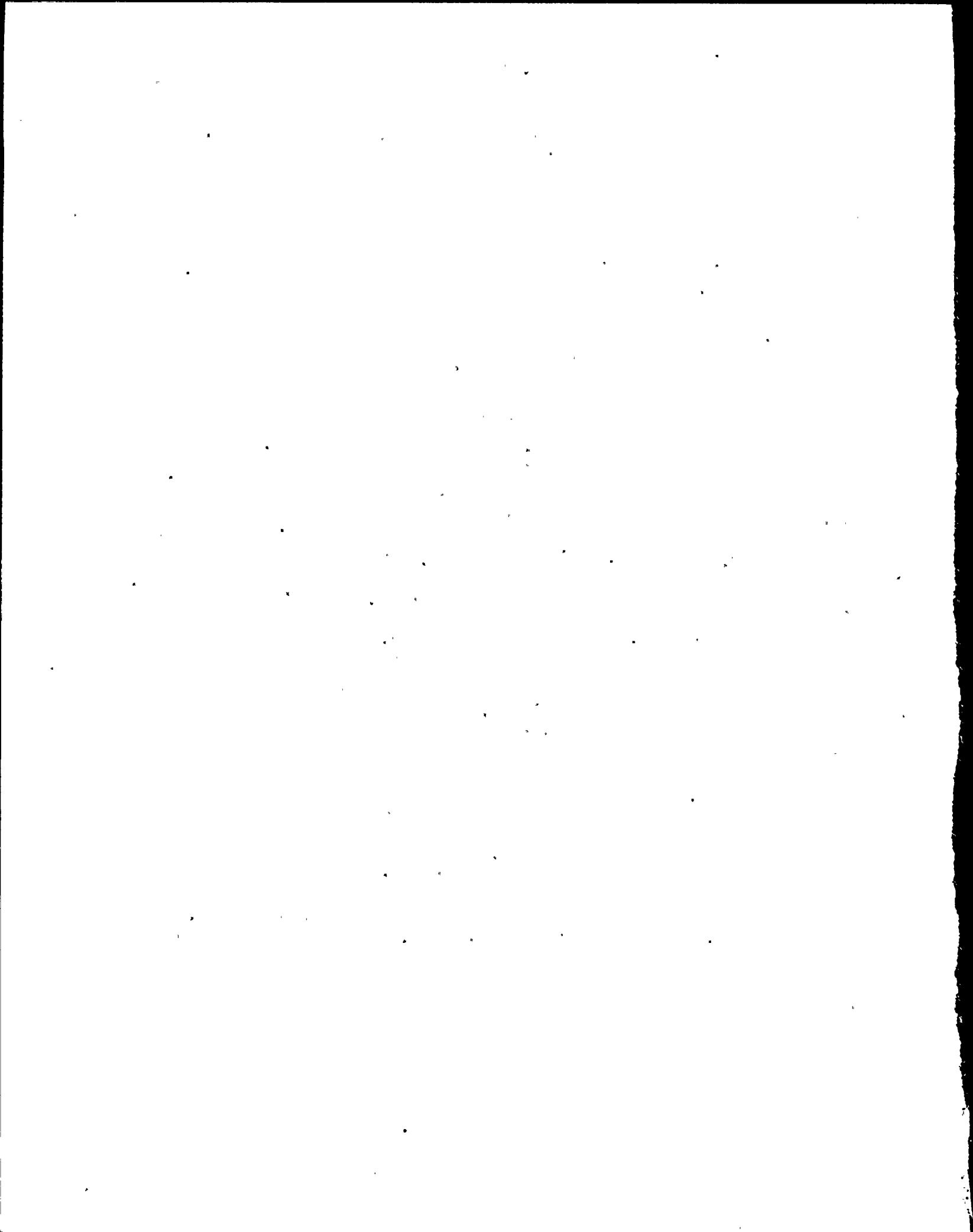
79-364 CP-112.0, Calibration and/or Maintenance of Volume Control Tank Level Channel "112", PCN 79-542.



PORC
ITEM #

PROCEDURE

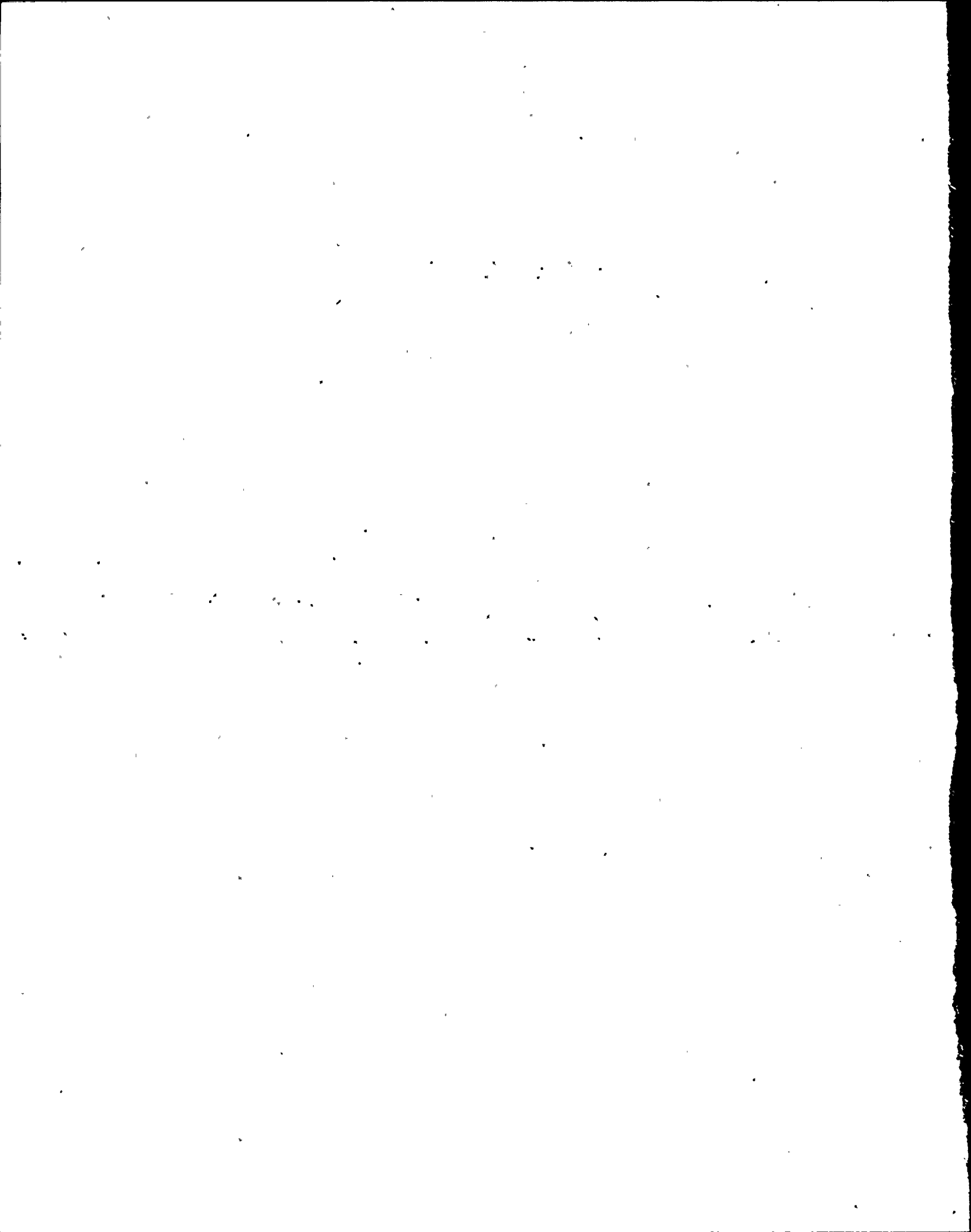
- CP-139.0, Calibration and/or Maintenance of Volume Control Tank Level Channel "139", PCN 79-540.
- 79-365 *CP-412.0, Reactor Coolant Flow Channel "412", PCN 79-537.
- 79-366 *GS-18.0, Training and Qualifying of Guards, PCN 79-549.
- 79-367 *GS-35.0, Issuing and Destruction of Ginna Photo-Identification Cards, PCN 79-516.
- 79-368 *GS-39.0, Protected/Vital Area Access, PCN 79-515.
- 79-369 *HP-5.1, Area Radiation Surveys, PCN's 79-508 and 79-550.
- 79-370 *M-43.15.1, Installation of Steam Generator Primary Manway, PCN 79-531.
- 79-371 *M-43.26, Installation of Steam Generator Secondary Manway Cover S/G, PCN 79-530.
- 79-372 *M-43.24.1, Inspection of Steam Generator Secondary Side, Upper Internals, PCN 79-529.
- 79-373 0-1.2, Plant From Hot Shutdown to Steady Load, PCN 79-532.
- 0-2.2, Plant Shutdown From Hot Shutdown to Cold Condition, PCN 79-533.
- 79-374 *PT-12.1, Emergency Diesel Generator IA, PCN 79-563.
- 79-376 *RSSP-1.5, Valve Interlock Verification - Feedwater Isolation, PCN 79-564.



PORC
ITEM #

PROCEDURE

- 79-378 *T-41A, Alignment of Auxiliary Feedwater System Prior to Power Operation, PCN 79-560.
- 79-379 *T-41D, Motor Driven Auxiliary Feedwater Pump Isolation, PCN 79-559.
- 79-380 A-102.8, Ginna Station Engineers (Cadet, Assistant Station and Station) Training Program, PCN 79-356.
- A-1201, Calibration and Control of Test Equipment, PCN 79-507.
- 79-388 *SM 77-1431.4, Installation of Electrical Equipment for Auxiliary Feedwater Flow Control Bypass Valves, PCN 79-631.
- 79-390 *EM 218, Grout Pier for MK-RH14, Pipe Support, PCN 79-659.
- 79-392 *A-1001, Inspection and Surveillance Activities, PCN 79-536.
- 79-393 *ISI-7, Visual Leakage Inspection of High Energy Piping PCN 79-626.
- 79-394 *ISI-1.1.9, Quality Group A Inservice Inspection, PCN 79-624.
- 79-395 *ISI-4.1.6, High Energy Inservice Inspection, PCN 79-625.
- 79-396 *SM 78-1838.1, Containment Wall-Mounted Jib Crane, PCN 79-671.
- 79-397 *M-3.2, Reactor Cavity Clean-Up and Decontamination, PCN 79-615.



PORC
ITEM #

PROCEDURE

79-399 *A-102.2, R. E. Ginna Administrative Controls Training Program, PCN 79-599.

79-400 *A-102.12, Fire Watch Training, PCN 79-657.

79-401 *A-1701, Quality Assurance Records, PCN's 79-467 and 79-474.

79-402 CP-934.0, Calibration and/or Maintenance Procedure of Accumulator Level Channel "934", PCN 79-591.

CP-935.0, Calibration and/or Maintenance of Accumulator Level Channel "935", PCN 79-589.

CP-938.0, Calibration and/or Maintenance of Accumulator Level Channel "938", PCN 79-587.

CP-939.0, Calibration and/or Maintenance of Accumulator Level Channel "939", PCN 79-585.

79-403 *E-33, Quadrant Power Tilt, PCN 79-598.

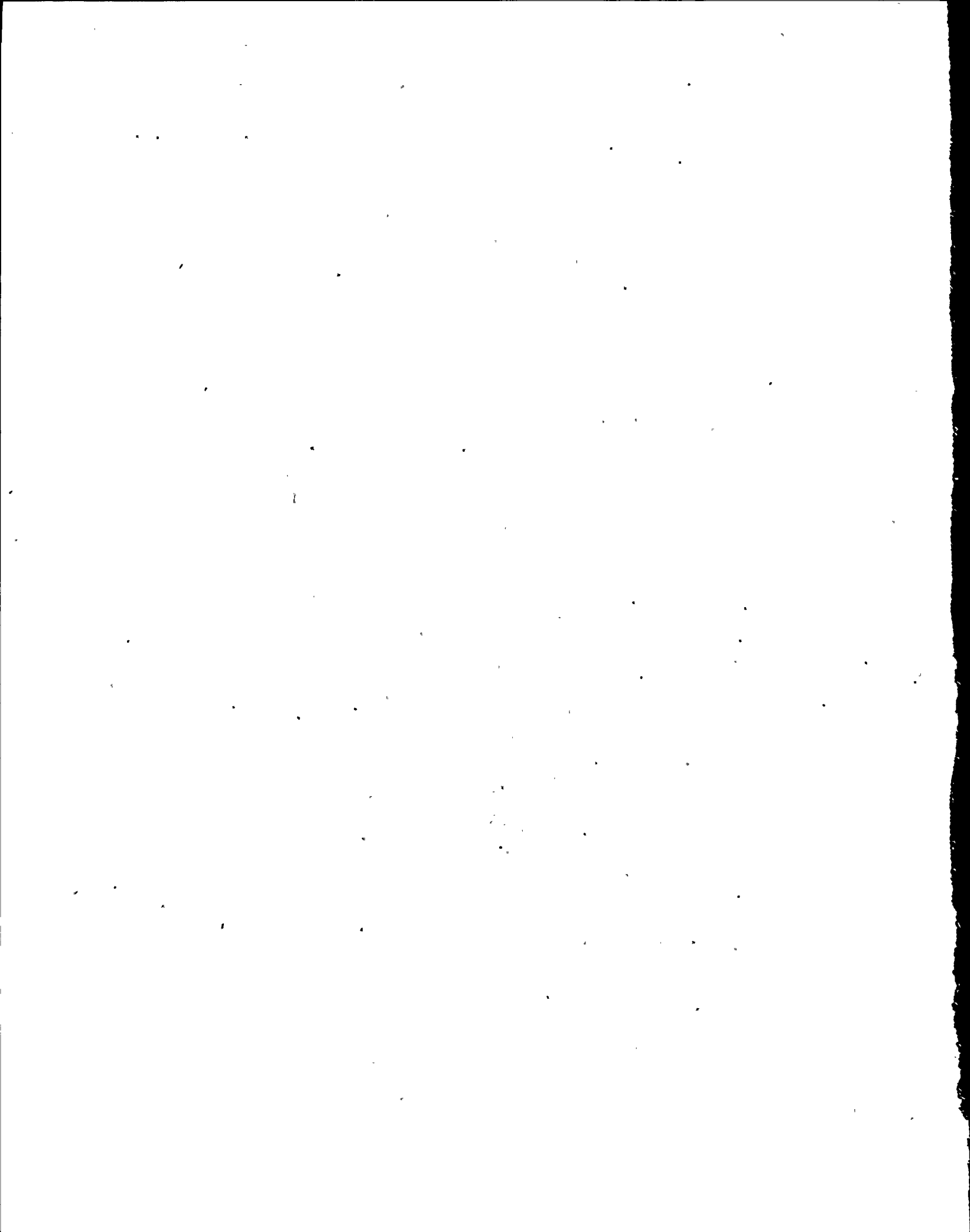
79-404 *ISI-1.1.9, Quality Group A Inservice Inspection, PCN 79-624.

79-405 *ISI-4.1.6, High Energy Inservice Inspection, PCN 79-625.

79-407 *M-72.1, Replacement Primary Loop RTD, PCN 79-619.

79-408 PT-34.0, Startup Physics Test Program, PCN 79-634.

PT-34.1, Initial Criticality, and ARO Boron Concentration, PCN 79-635.



PORC
ITEM #

PROCEDURE

- PT-34.3, RCC Bank Worth Measurement, PCN 79-633.
- 79-409 QCIP-1, Steam Generator (Primary Side) Preclosure Inspection, PCN 79-574.
- QCIP-2, Pre-Closure Core Verification, PCN 79-571.
- QCIP-5, Cleanliness Inspection of Components, PCN 79-573.
- QCIP-9, Pressurizer Pre-Closure Cleanliness Inspection, PCN 79-575.
- QCIP-15, Visual Inspection During Hydrostatic and Steam Pressure Testing, PCN 79-572.
- QCIP-21, Shipping Package LL-50-100 Inspection, PCN 79-570.
- QCIP-22, Shipping Package HN-100 Series 1 Inspection, PCN 79-576.
- 79-410 *RF-2E, Draining of Refueling Canal, PCN 79-568.
- 79-412 *S-3.2A, Charging and Volume Control System Pre-Startup Alignment, PCN 79-622.
- 79-413 *S-4.1U, Velocity Flush of R-18, PCN 79-623.
- 79-414 *S-7C, Transferring and Purification of RWST Water From RWST to SFP, PCN 79-616.
- 79-420 A-203, Ginna Modification Project Organization, PCN 79-388.

312

1925

1925

1925

1925

1925

1925

1925

PORC
ITEM #

PROCEDURE

A-401, Control of Procurement Documents Prepared at
Ginna, PCN 79-510.

A-402, Control of Procurement Documents Prepared by the
Ginna Station Modification Project, PCN 79-513.

A-403, Requisitioning Services from General Maintenance,
PCN 79-511.

M-702, Supplier Qualification Technical Evaluations,
PCN 79-512.

A-1203, Calibration and Control of Mechanical Measur-
ing Tools and Torque Wrenches, PCN 79-506.

A-1702, Records Storage Facilities and Equipment, PCN
79-463.

A-1801, Ginna Station Response to Internal Audits, PCN
79-459.

CP-80.4, Calibration Procedure for Category #1 Mechani-
cal Measuring Tools, PCN 79-567.

CP-81.0, Calibration TQTP 650 Dual Bench Tester for
Torque Wrenches, PCN 79-569.

CP-81.1, Calibration Procedure for Torque Wrenches, PCN
79-568.

CP-934.4, Calibration and/or Maintenance of Accumulator
Level Power Supply, LQ-934, PCN 79-590.

FORC
ITEM #

PROCEDURE

CP-935.4, Calibration and/or Maintenance of Accumulator Level Power Supply, LQ-935, PCN 79-588.

CP-938.4, Calibration and/or Maintenance of Accumulator Level Power Supply, LQ-938, PCN 79-586.

CP-939.4, Calibration and/or Maintenance of Accumulator Level Power Supply LQ-939, PCN 79-584.

ISI-7, Visual Leakage Inspection of High Energy Piping, PCN 79-626.

S-7L, Mixed-Bed DI Saturation with Boron, PCN 79-664.

79-427 *EM-219, Repair of Safety Injection Pipe Support, PCN 79-691.

79-428 *EM-220, Repair of Pressurizer Surge Line Pipe Restraint MK-SP-1, PCN 79-692.

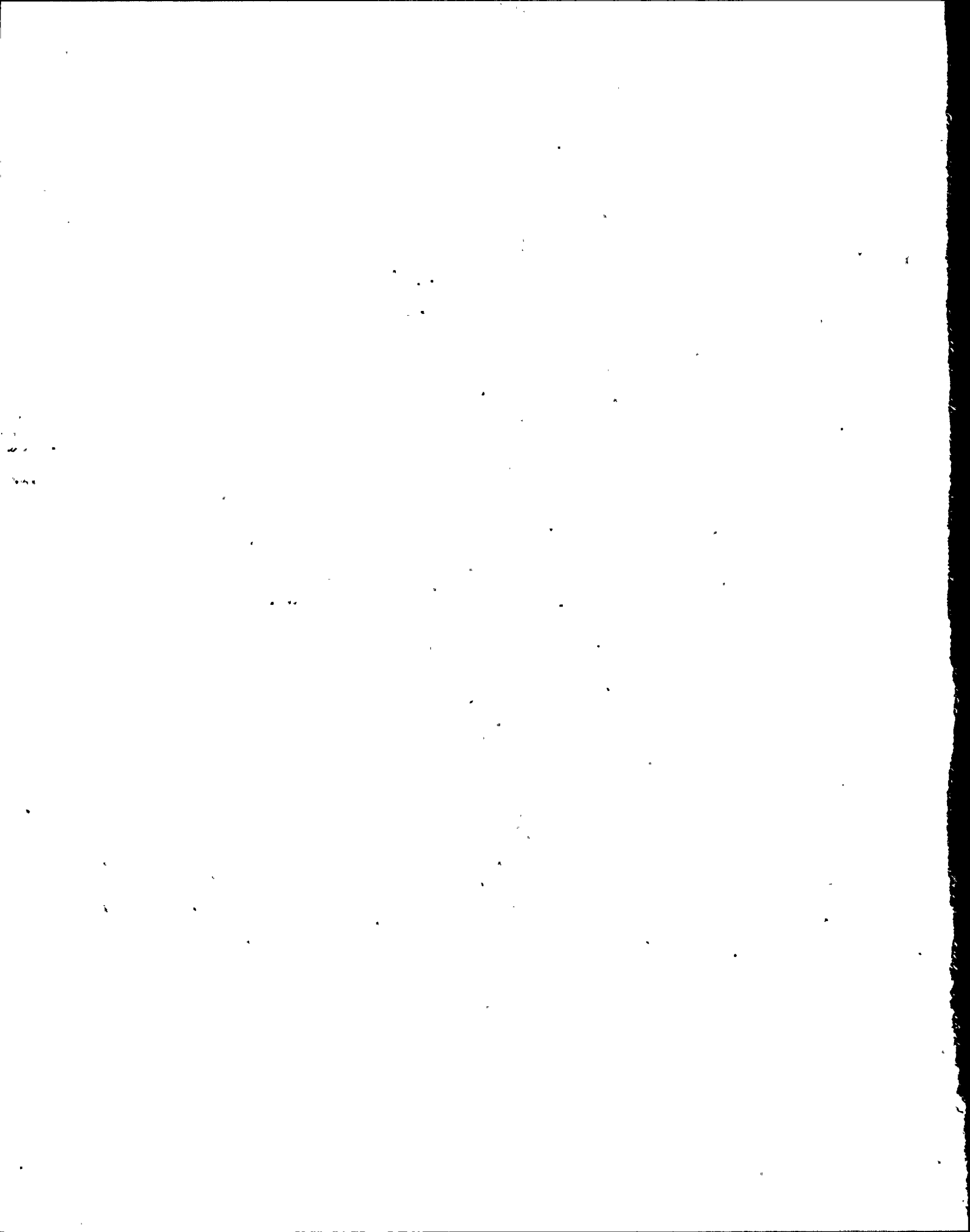
79-429 *EM-221, Repair of RHR Pipe Support MK-RH-9, PCN 79-695.

79-430 *ISI-5, Eddy Current Testing of Series 44 Steam Generator Tubes, PCN's 79-399 and 79-405.

79-431 *EM-224, Repair of RCS Drain Line Pipe Hanger MK-D1, PCN 79-718.

79-432 *EM-225, Repair of Auxiliary Spray Line Pipe Restraint, ASL-PCN 79-719.

79-433 *EM-223, Repair of Letdown Line Pipe Support Near AOV-227, PCN 79-720.



PORC
ITEM #

PROCEDURE

- 79-434 *EM-222, Repair of Safety Injection Pipe Support MK-SI-5, PCN 79-721.
- 79-440 *EM-224, Repair of RCS Drain Line Pipe Hanger MK-D1, PCN 79-737.
- 79-441 *EM-226, Grout Placement on Pipe Support ACH-41A, PCN 79-732.
- 79-442 *EM-227, Repair of RHR Pipe Restraints Between RHR Heat Exchanger an 857 Valves, PCN 79-733.
- 79-443 *EM-228, Repair of Clips on Safety Injection Pipe Support MK-SI-10, PCN 79-749.
- 79-446 *A-52.2, Control of Locked Valve Operation, PCN 79-696.
- 79-447 *A-102.8, Ginna Station Engineers (Cadet, Assistant Station, and Station Training Program, PCN 79-715.
- 79-448 *HP-6.1, Contamination Surveys, PCN 79-713.
- 79-449 *O-2.3.2, Filling and Venting the Reactor Coolant System, PCN 79-693.
- 79-450 *PC-1.1, Primary Coolant Analysis Schedule and Limits, PCN 79-727.
- 79-451 *RD-8, Liquid Radwaste Compositing and Analysis, PCN 79-665.
- 79-452 *S-3.2A, Charging and Volume Control System Pre-Startup Alignment, PCN 79-697.

PORC
ITEM #

PROCEDURE

- 79-453 *S-8A, Component Cooling Water System Startup and Normal Operation Valve Alignment, PCN 79-685.
- 79-454 *S-17.1, Containment Spray System, PCN 79-694.
- 79-455 *M-11.4.5, Charging Pump Varidrive Low Speed and High Speed Mechanical Stop Setting Adjustment and Actuator Maintenance, PCN 79-717.
- *M-48.2, Isolation of Bus 14, PCN 79-686.
- *M-50.1, Isolation of 12A Transformer, PCN 79-714.
- *O-2.3.2, Filling and Venting the Reactor Coolant System, PCN 79-684.
- *O-7, Alignment and Operation of the Reactor Vessel Over-Pressure Protection System, PCN 79-683.
- *S-8A, Component Cooling Water System Startup and Normal Operation Valve Alignment, PCN's 79-680 and 79-682.
- 79-457 A-701, Receipt and Acceptance of Materials/Parts, PCN 79-600.
- A-705, Control of Purchased Services, PCN 79-601.
- A-1202, Calibration of Process System Instrumentation, PCN 79-636.
- SC-15.8, Fire System Locked Valve Cycling, PCN 79-650.

FORC
ITEM #

PROCEDURE

79-464 *EM-229, Repair of RHR Pipe Support MK-RH-
PCN 79-767.

79-469 *A-54.3, Open Flame, Welding and Grinding Permit, PCN
79-632.

79-470 *M-73.10, Welding, PCN 79-728.

79-471 *A-1104, Ginna Station Technical Specification Surveillance
Program, PCN 79-782.

79-472 *GS-35.0, Issuing and Destruction of Ginna Photo-Identifica-
tion Cards, PCN 79-759.

79-473 *HP-1.1, Issuing Personnel Dosimeter, PCN 79-739.

79-474 *ISI-5, Eddy Current Testing of Series 44 Steam Generator
Tubes, PCN 79-741.

79-475 *M-40, Surveillance and Maintenance of Hydraulic Snubbers,
PCN 79-791.

79-476 *M-43.0, Steam Generator Maintenance Test and Repair Proce-
dures, PCN 79-740.

79-477 *M-43.1, Steam Generator Manway Cover Removal Steam Genera-
tor - , PCN 79-747.

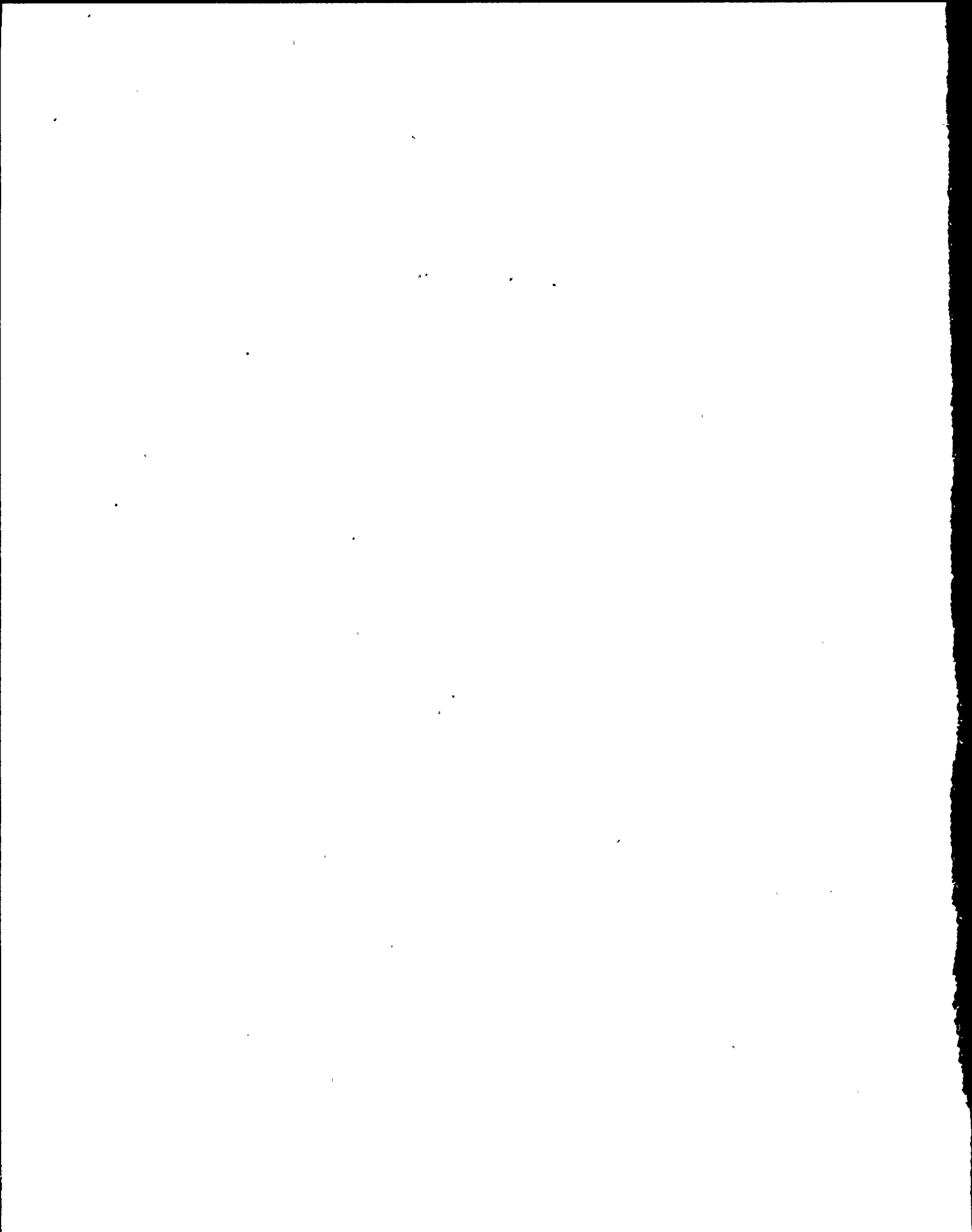
79-478 *Steam Generator Insert Removal - , Steam Generator,
PCN 79-746.

79-479 *M-43.5, Isolation of Steam Generator Blast Proof Loop
Nozzle Covers, S/G , PCN 79-745.

FORC
ITEM #

PROCEDURE

- 79-480 *M-43.14, Removing Steam Generator Blast Proof Loop Nozzle Cover Steam Generator Leg, PCN 79-743.
- 79-481 *M-43.15, Installation of Steam Generator Manway Insert, PCN 79-742.
- 79-482 *M-43.21, Steam Generator Final Clean-Up and Inspection, PCN 79-744.
- 79-483 *O-2.5, Plant Shutdown From Hot Shutdown to Cold Shutdown When Condenser Steam Dump is Unavailable, PCN 79-766.
- 79-484 *O-6, Operation and Process Monitoring, PCN 79-760.
- 79-485 *PI-2.1, Safety Injection System Pumps, PCN 79-781.
- 79-486 *PI-2.2, Residual Heat Removal System, PCN 79-769.
- 79-487 *PI-2.3, Safeguard Valve Operation, PCN 79-783.
- 79-488 PI-13.7, Fire Hose Reel Assembly Inspection, PCN 79-780.
- PI-13.4.7, Protomatic Deluge Valve System Testing System #7 (Oil Storage Room), PCN 79-786.
- PI-13.4.8, Protomatic Deluge Valve System Testing, System #8, (Main Transformer), PCN 79-787.
- PI-13.4.9, Protomatic Deluge Valve System Testing, System #9, (#11 Transformer), PCN 79-788.



PORC
ITEM #

PROCEDURE

PT-13.4.10A, Protomatic Deluge Valve System Testing, System #10A, (#12-A Transformer), PCN 79-789.

PT-13.4.10B, Protomatic Deluge Valve System Testing, System #10B, (#12-B Transformer), PCN 79-790.

79-489 *S-3.6, Auxiliary Building Heat Tracing System Operation, PCN 79-729.

79-490 *SC-1.12B, Station Call List, PCN 79-784.

79-492 *A-52.2, Control of Locked Valve Operation, PCN 79-751 and 79-792.

*E-1.1, Safety Injection System Actuation, PCN 79-752.

*O-1.1, Plant Heatup from Cold Shutdown to Hot Shutdown, PCN's 79-735 and 79-750.

*O-1.1B, Establishing Containment Integrity, PCN 79-753.

*O-1.2, Plant From Hot Shutdown to Steady Load, PCN 79-807.

*PT-7, Hydro Test of Reactor Coolant System, PCN 79-734.

79-495 A-801, Control of Accepted Material, Parts and Components at Ginna Station, PCN 79-698.

A-802, Identification and Marking of Accepted Material, Parts, and Components, PCN 79-699.

FORC
ITEM #

PROCEDURE

- A-1501, Control of Nonconforming Items, PCN 79-700.
- WC-11.0, Operating Procedure for IL-351 AA/AE Spectrophotometer, PCN 79-730.
- 79-508 *O-1.2, Plant From Hot Shutdown to Steady Load, PCN 79-807.
- 79-509 *O-1.2, Plant From Hot Shutdown to Steady Load, PCN 79-836.
- 79-511 *A-102.2, R. E. Ginna Administrative Controls Training Program, PCN 79-794.
- 79-512 *A-102.11, Emergency Plan Training Program, PCN 79-833.
- 79-513 *A-601, Plant Procedure Document Control, PCN 79-817.
- 79-514 *A-602, Plant Procedure Distribution, PCN 79-805.
- 79-515 E-1.2, Loss of Coolant Accident, PCN 79-825.
- E-1.3, Steam Line Break Accident, PCN 79-826.
- E-1.4, Steam Generator Tube Rupture Accident, PCN 79-827.
- 79-516 *ISI-2.1.7, Quality Group B Inservice Inspection, PCN 79-828.
- 79-517 *O-1.2, Plant From Hot Shutdown to Steady Load, PCN 79-834.
- 79-518 O-1.2.2, Critical Rod Position Calculation, PCN 79-831.

PORC
ITEM #

PROCEDURE

O-3.1, Boron Concentration for the Xenon Free All Rods In - Most Reactive Rod Stuck Out Shutdown Margin, PCN 79-829.

O-6.6, Correcting Measured Boron Concentration to Reference Full Power Expected Boron Concentration, PCN 79-830.

79-519 *SC-1, Radiation Emergency Plan, PCN 79-779.

79-520 *PI-2.10.1, RCS and SI Header Check Valves (Loop A & B Cold Legs, PCN 79-800.

79-521 RSSP-1.2, Valve Interlock Verification - Safety Injection System, PCN 79-799.

RSSP-2.1, Safety Injection Functional Test, PCN 79-802.

RSSP-2.1A, Safety Injection Functional Test Alignment, PCN 79-801.

79-522 SC-1.3B, Site Radiation Emergency (Emergency Coordinator and Survey Center Assignees), PCN-79-832.

SC-1.3C, Site Radiation Emergency (Non-Operating Personnel), PCN's 79-777 and 79-778.

79-523 TICP-4, Test Instrument Calibration Procedure, Category II, Digital Multifunction Meter, PCN 79-853.

TICP-6, Test Instrument Calibration Procedure, Category II, Millivolt Potentiometer, PCN 79-852.

79-524 O-2.1, Normal Shutdown to Hot Shutdown, PCN 79-835.

FORC
ITEM #

PROCEDURE

- 0-1.3, Reactor Startup for Training, PCN 79-820.
- PT-6.1, Source Range Nuclear Instrumentation System, PCN 79-824.
- PT-16, Auxiliary Feedwater System, PCN 79-819.
- S-27.955/966C, AOV-955 and AOV-966C, RCS Loop Sample Valves Isolation, PCN 79-821.
- T-18C, Turbine Overspeed Trip Test, PCN 79-839.
- 79-525 A-52.2, Control of Locked Valve Operation, PCN 79-837.
- S-16A, Safety Injection System Alignment, PCN 79-838.
- T-41A, Alignment of Auxiliary Feedwater System Prior to Power Operation, PCN 79-823.
- 79-527 A-903, Non-Destructive Examination Control, PCN 79-52.
- HP-2.5, Whole Body Count Evaluation, PCN 79-804.
- 79-538 *S-30.1, Safety Injection System Valve Position Verification, PCN 79-851.
- 79-539 *S-30.2, RHR System Valve Position Verification, PCN 79-856.
- 79-540 *S-30.3, Containment Spray System Valve Position Verification, PCN 79-857.

<u>PORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-541</u>	<u>*EM-230, Pipe Repair and/or Replacement Adjacent to Blend System FCV-110A, PCN 79-878.</u>
<u>79-544</u>	<u>*A-1, Radiation Control Manual; PCN 79-840.</u>
<u>79-545</u>	<u>*M-64.2, Limitorque Valve Operator Maintenance and Inspection, PCN 79-864.</u>
<u>79-546</u>	<u>*O-1.1C, Precritical Technical Specification Check Sheet, PCN 79-849.</u>
<u>79-547</u>	<u>*O-6.11, Routine Operations Check Sheet; PCN 79-887.</u>
<u>79-548</u>	<u>PT-13.4.2, Multimatic Deluge Valve System Testing - System #2, (Turbine Oil Reservoir) PCN 79-842.</u>
	<u>PT-13.4.11, Multimatic Deluge Valve System Testing - System #11, (Cable Tunnel), PCN 79-843.</u>
	<u>PT-13.4.12, Alarm Valve System Testing - Fire System #12, (Turbine Island Sprinkler), PCN 79-844.</u>
<u>79-549</u>	<u>*RF-2, Reactor Refueling Outage Operations and Activities, PCN 79-850.</u>
<u>79-550</u>	<u>*SC-1.3A, Site Radiation Emergency (Shift Foreman and Control Room), PCN 79-822.</u>
<u>79-551</u>	<u>*S-3.1B, Pre-Operational Line Up of Boric Acid System, PCN 79-858.</u>
<u>79-552</u>	<u>*T-44.2, Standby Auxiliary Feedwater (SAFW) System Alignment for Normal Operation, PCN 79-854.</u>

PORC
ITEM #

PROCEDURE

79-553 *O-6.11, Routine Operations Check Sheet, PCN 79-888.

79-554 O-6.4, Core Quadrant Power Tilt Calculation, PCN 79-866.

CP-6.10, T avg and Delta T Alignment at 70% Power or
 Greater Loop A, Unit 2, PCN 79-867.

79-555 QC-401, Control of Procurement Documents for Purchased
 Material, Parts, Components and Services, PCN 79-860.

QC-701, Receipt and Acceptance of Materials, PCN 79-
 859.

QC-705, Control of Purchased Services, PCN 79-861.

QC-1403, Inspection Status Control, PCN 79-862.

QC-1801, Ginna Station Response to Internal Audits,
 PCN 79-863.

79-556 A-1502, Nonconformance Reports, PCN 79-762.

PT-13.4.3, Flood Valve System Testing Fire System #3,
 PCN 79-845.

PT-13.4.4, Floor Valve System Testing Fire System #4,
 PCN 79-846.

PT-13.4.5, Flood Valve System Testing Fire System #5,
 PCN 79-847.

PT-13.4.6, Flood Valve System Testing Fire System #6,
 PCN 79-848.

PORC
ITEM #

PROCEDURE

79-566 *S-30.2, RHR System Valve Position Verification, PCN 79-899.

79-568 *A-64, Pressurizer Level Safety Injection Bistables, PCN 79-900.

79-569 *EM-231, Verification and/or Maintenance of Valve 1598, PCN 79-911.

79-574 *M-4.4, Charging Pump Vari-Drive Overhaul Unit # -; PCN 79-880.

79-575 *M-40, Surveillance and Maintenance of Hydraulic Snubbers, PCN 79-896.

79-576 *O-1.1B, Establishing Containment Integrity, PCN 79-897.

79-576 *O-1.1B, Establishing Containment Integrity, PCN 79-897.

79-577 M-67.1, Protomatic Deluge Fire Systems #'s 7, 8, 9 and 10, PCN 79-869.

M-67.2, Deluge Valve Fire System #'s 1, 2 and 11, PCN 79-870.

M-67.3, Flood Valve System Maintenance System Number 3, 4, 5, and 6, PCN 79-890.

M-67.4, Alarm Valve System Maintenance, System #'s 12, 13, and 15, PCN 79-871.

M-67.5, Halon System Maintenance, Computer Room and Relay Room, PCN 79-835.

FORC
ITEM #

PROCEDURE

M-67.6, Smoke Detector Maintenance to Provide Instructions for Personnel to Maintain the Smoke Detector, PCN 79-872.

M-67.7, Containment Post-Accident (Recirc) and/or Auxiliary Filter R. T. D. Maintenance, PCN 79-873.

M-67.8, Star Corporation Heat Detector System Maintenance, PCN 79-874.

M-67.9, Fire System - Hose Reel Isolation Valve Replacement Valve No _____, PCN 79-882.

PT-13.1.8, Alarm Valve System (Resetting Only) System Numbers 12, 13, and 15, PCN 79-891.

PT-2.9.1, Check Valve Exercising Quarterly Requirement (RCDT Pump Discharge), PCN 79-875.

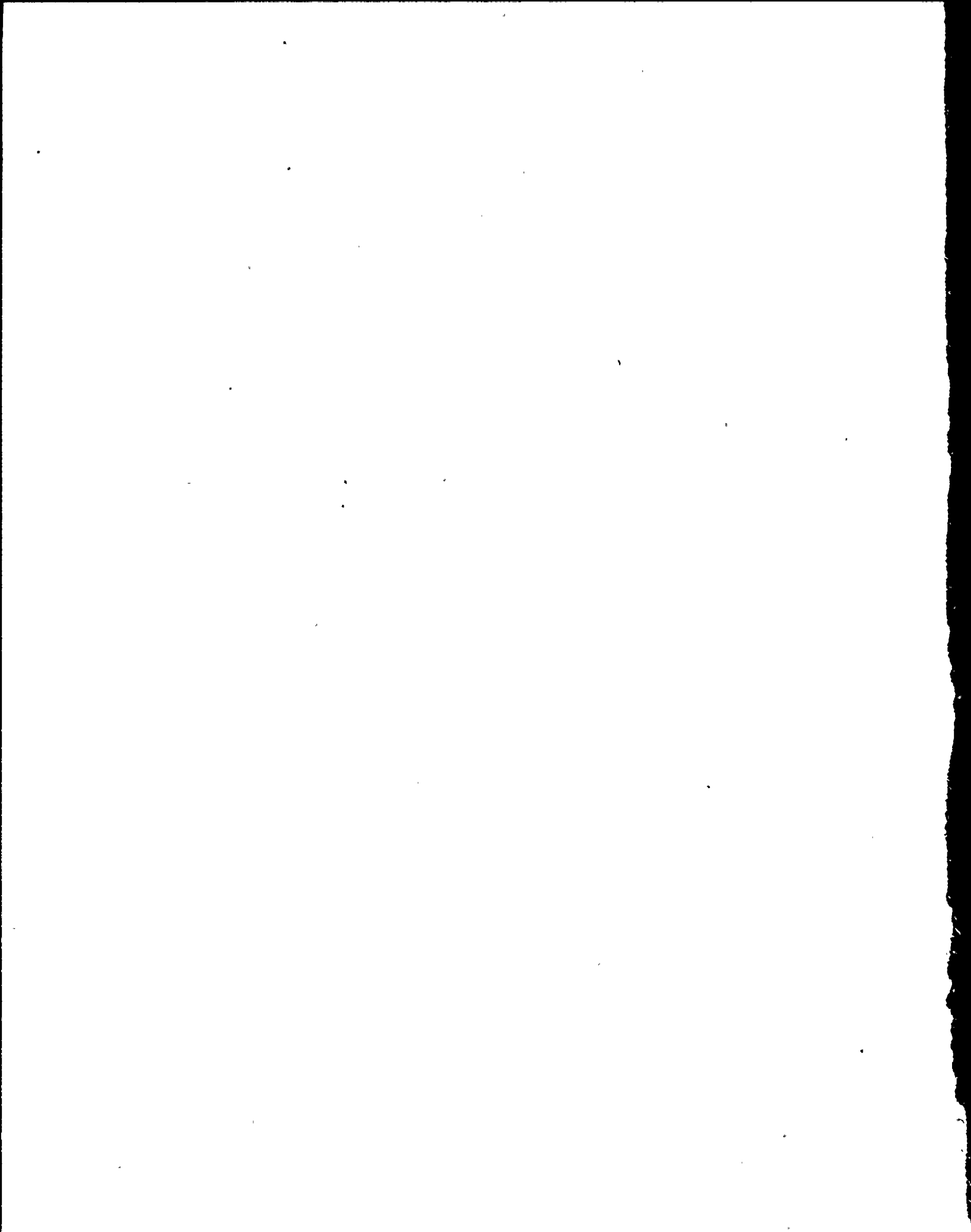
79-578 E-15.1, Malfunction of Pressurizer Power Relief or Safety Valves, PCN 79-901.

79-579 *SC-1.3A, Site Radiation Emergency (Shift Foreman and Control Room), PCN 79-905.

79-580 *S-3.2A, Charging and Volume Control System Pre-Startup Alignment, PCN 79-697.

79-581 *S-15.2, Flux Mapping Emergency & Calibration Procedure, PCN 79-881.

79-583 *T-41A, Alignment of Auxiliary Feedwater System Prior to Power Operation, PCN 79-883.



PORC
ITEM #

PROCEDURE

79-584 *T-44.2, Standby Auxiliary Feedwater (SAFW) System Alignment for Normal Operation, PCN 79-884.

79-586 *A-52.2, Control of Locked Valve Operation, PCN 79-898.

79-587 *RF-53, Cycle VIII-IX Refueling, PCN 79-879.

79-588 PT-36, Standby Auxiliary Feedwater System Flow Check, PCN 79-877.

S-4.5.5A, "A" Boron Recycle Cation DI Resin Replacement, PCN 79-808.

S-4.5.5B, "B" Boron Recycle Cation DI Resin Replacement, PCN 79-811.

S-4.5.4A, "A" Boron Recycle Base Removal DI Resin Replacement, PCN 79-809.

S-4.5.4B, "B" Boron Recycle Base Removal DI Resin Replacement, PCN 79-810.

S-4.5.3A, "A" Boric Acid Evaporator Condensate Demineralizer Resin Replacement, PCN 79-812.

S-4.5.3B, "B" Boric Acid Evaporator Condensate Demineralizer Resin Replacement, PCN 79-813.

79-599 *EM-230, Pipe Replacement Adjacent to Blend System FCV-110A, PCN 79-998.

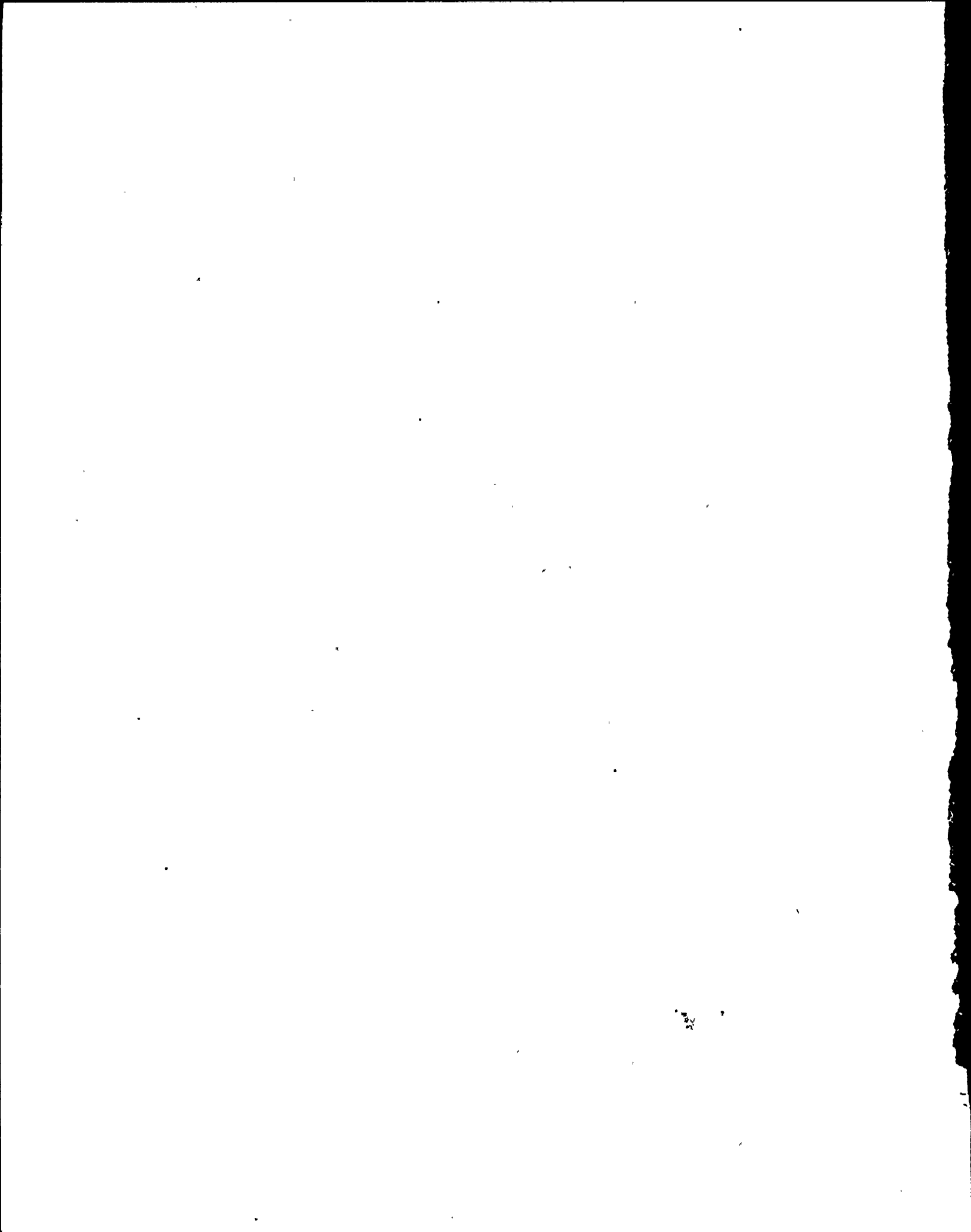
79-600 *SC-1.2, Local Radiation Emergency Procedure, PCN 79-906.

<u>PORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-601</u>	<u>*A-1401, Station Holding Rules, PCN 79-904.</u>
<u>79-602</u>	<u>*A-53.2, Three Month Lubrication and Maintenance Inspection, PCN 79-917.</u>
<u>79-604</u>	<u>*SC-1.3B, Site Radiation Emergency (Emergency Coordinator and Survey Center Assignees), PCN 79-932.</u>
<u>79-605</u>	<u>*FC-1.3, Daily Chemistry Analysis Results, PCN 79-937.</u>
<u>79-608</u>	<u>A-53.0, Preventive Maintenance Program, PCN 79-915.</u>
	<u>A-53.1, Equipment Inspection Period and Lubricant List, PCN 79-916.</u>
<u>79-609</u>	<u>*A-502, Plant Procedure Content and Format Requirements, PCN 79-940.</u>
<u>79-610</u>	<u>*E-15.1, Malfunction of Pressurizer Power Relief or Safety Valves, PCN 79-918.</u>
<u>79-611</u>	<u>*HP-2.5, Whole Body Count Evaluation, PCN 79-947.</u>
<u>79-612</u>	<u>*O-1.3, Reactor Startup for Training, PCN 79-939.</u>
<u>79-613</u>	<u>P-3, Precautions, Limitations and Setpoints P-3 Chemical and Volume Control System, PCN 79-941.</u>
	<u>P-8, Waste Disposal System, PCN 79-942.</u>
<u>79-614</u>	<u>*PT-2.2, Residual Heat Removal System, PCN 79-936.</u>

PORC
ITEM #

PROCEDURE

- 79-615 *PT-2.3, Safeguard Valve Operation, PCN 79-951.
- 79-616 *PT-4, Residual Heat Removal Loop, Annual Hydro Test of Low Pressure Piping, PCN 79-950.
- 79-617 *RF-14.4.2, Removing Reactor Vessel Guide Studs and Stud Hole Plugs, Installing and Tensioning Head Studs, PCN 79-935.
- 79-618 *SC-1.7E, Emergency Radiation Monitoring On-Site Survey Team #9H (Orange), PCN's 79-933 and 79-934.
- 79-619 S-3.1E, 1A Boric Acid Storage Tank Isolation/Restoration, PCN 79-921.
- S-3.1F, 1B Boric Acid Storage Tank Isolation/Restoration, PCN 79-922.
- 79-621. *S-12.4, RCS Leakage Surveillance Record Instructions, PCN 79-924.
- 79-623 O-6.2, Main Control Board System Status Verification, PCN 79-912.
- PT-2.2, Residual Heat Removal System, PCN 79-902.
- S-30.1, Safety Injection System Valve Position Verification, PCN 79-908.
- S-30.3, Containment Spray System Valve Position Verification, PCN 79-909.
- 79-624 QC-701, Receipt and Acceptance of Materials, PCN 79-926.



FORC
ITEM #

PROCEDURE

QC-705, Control of Purchased Services, PCN 79-927.

QC-1403, Inspection Status Control, PCN 79-928.

QC-801, Control of Accepted Materials, Parts and Components, PCN 79-930.

QC-802, Identification and Marking of Material, PCN 79-929.

QC-1501, Control and Disposition of Deficient Materials, PCN 79-931.

79-625

A-803, Control of Items to be Reconditioned, PCN 79-876.

PT-36.2, Standby Auxiliary Feedwater System Recirculation Flow Test, PCN 79-895.

S-4.5.7A, "A" Letdown Mixed Bed DI Resin Replacement, PCN 79-797.

S-4.5.7B, Letdown Mixed Bed DI Resin Replacement, PCN 79-798.

S-11.1, Transfer of "A" and/or "B" Boric Acid Storage Tanks to Batch Tank for Boron Enrichment and the Return from Batch Tank to Boric Acid Tanks After Enrichment, PCN 79-923.

79-632

*PT-13, Fire Pump Operation and System Alignment, PCN 79-954.

79-633

S-30.1, Safety Injection System Valve Position Verification, PCN 79-985.

FORC
ITEM #

PROCEDURE

S-30.2, RHR System Valve Position Verification, PCN 79-986.

S-30.3, Containment Spray System Valve Position Verification, PCN 79-987.

79-634 *GS-31.0, Door Alarm System, PCN 79-953.

79-635 *A-52.2, Control of Locked Valve Operation, PCN's 79-979 and 79-996.

79-636 *S-16A, Safety Injection System Alignment, PCN's 79-978 and 79-997.

79-637 *A-52.4, Control of Limiting Conditions for Operating Equipment, PCN 79-956.

79-638 *PT-23, Containment Isolation Valve Leak Rate Testing, PCN 79-957.

79-639 *S-11.1, Boric Acid Storage Tank Transfer to Batch Tank Transfer to Batch Tank, PCN's 79-959 and 79-995.

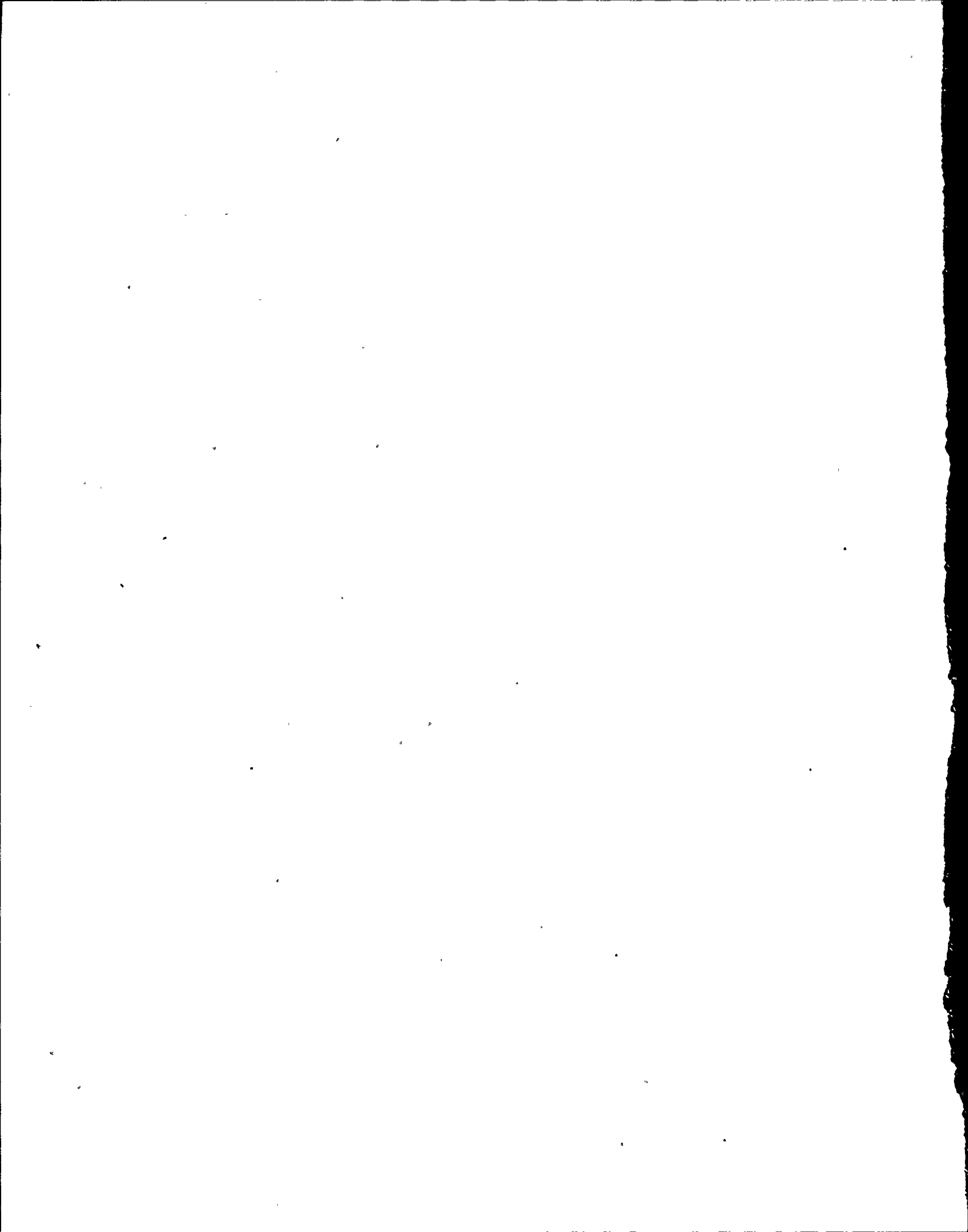
79-640 A-17, Plant Operations Review Committee Operating Procedure, PCN 79-960.

A-58, Installation of Power Control or Instrumentation Cables at Ginna, PCN 9-961.

A-501, Plant Procedures Preparation and Classification, PCN 79-962.

A-1001, Inspection and Surveillance Activities, PCN 79-964.

A-1101, Performance of Tests, PCN 79-965.



FORC
ITEM #

PROCEDURE

A-1102, Qualification of Test Personnel, PCN 79-966.

A-1703, System and Equipment Histories, PCN 79-967.

A-903, Nondestructive Examination Control, PCN 79-963.

79-643 *GS-43.0, Off-Loading Portal Procedure, PCN 79-952.

79-644 *GS-16.0, Property Removal, PCN 79-325.

79-645 S-4.5.3, Boric Acid Evaporator Condensate Demineralizer Resin Replacement, PCN 79-814.

S-4.5.4, Boron Recycle Base Removal DI Replacement, PCN 79-816.

S-4.5.5, Boron Recycle Cation DI Resin Replacement, PCN 79-815.

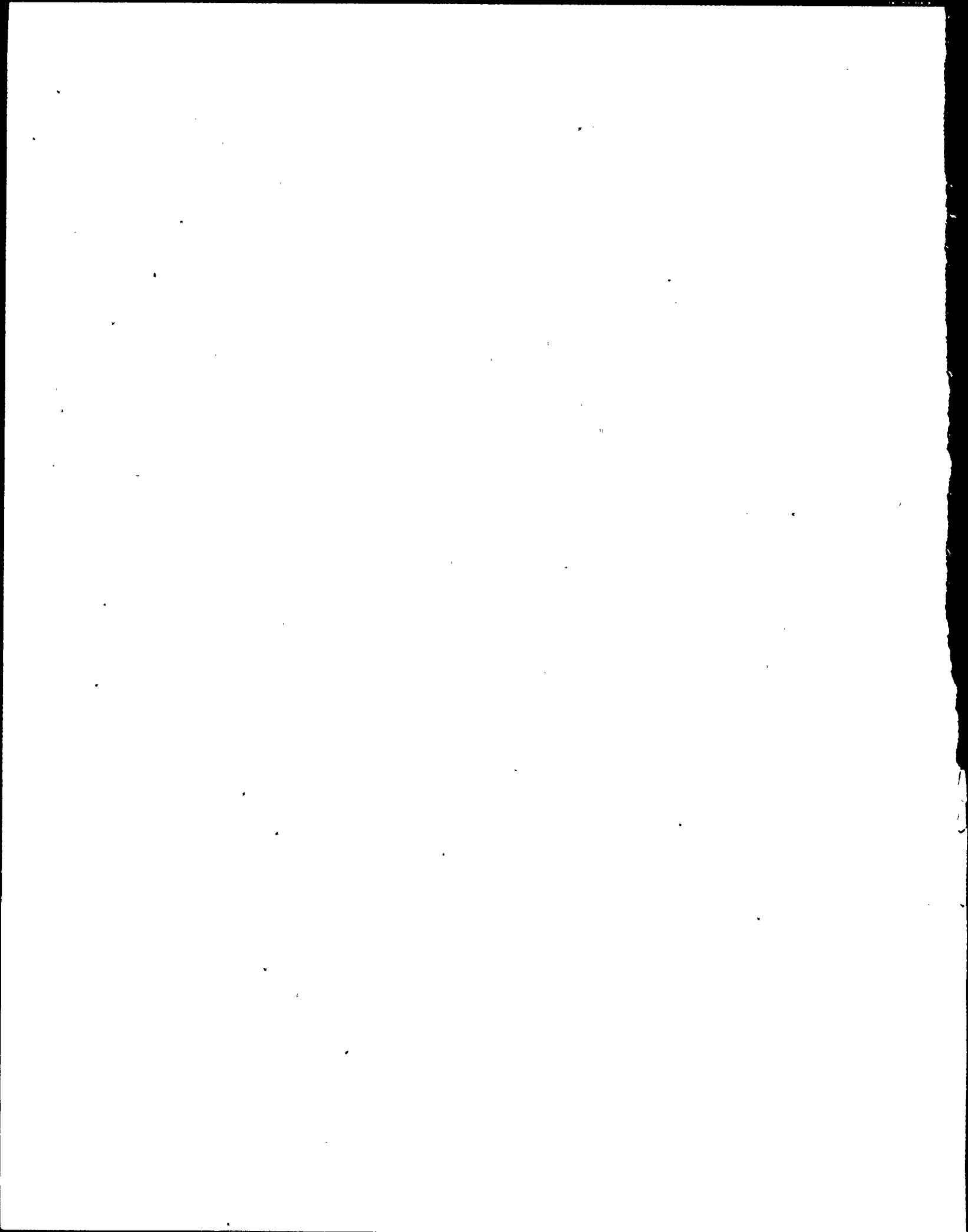
S-4.5.7, Letdown Mixed Bed DI (A or B) Resin Replacement, PCN 79-796.

79-646 *AR/C-13, Containment Ventilation Isolation Reset in Reset Position, PCN 79-976.

79-648 *S-30.5, Standby Auxiliary Feedwater Pump Valve Position Verification PCN 79-955.

*S-30.4, Auxiliary Feedwater System Valve Position Verification, PCN 79-910.

*A-1303, Storage and Preservation of Materials at Ginna Station, PCN 79-855.



PORC
ITEM #

PROCEDURE

*A-1301, Control of Material Handling and Handling Equipment, PCN 79-865.

79-650 *A-37.1, System Modification Requirements, PCN 79-1002.

79-660 *A-55, Requests for Engineering Design Services, PCN 79-972.

79-661 *A-203, Ginna Modification Project Organization, PCN 79-989.

79-662 *A-1701, Quality Assurance Records, PCN 79-1016.

79-663 *CP-211.4, Check of Flow Control System for R-11 and R-12, PCN 79-1005.

79-664 *HP-1.6, Neutron Exposure, PCN 79-1015.

79-665 *HP-11.10, Air Sampling with Siersat Low Volume Air Samples, PCN 79-1003.

79-666 *M-11.8J, Reactor Coolant Pump Motor Major Inspection - Mechanical, PCN 79-998.

79-667 *O-1.1, Plant Heat Up From Cold Shutdown to Hot Shutdown, PCN 79-992.

79-668 *O-1.1C, Precritical Technical Specification Check Sheet, PCN 79-988.

79-669 *PC-5, Gamma Isotopic Analysis of Degassed Primary Coolant, PCN 79-991.

79-670 *PC-15.2, Specific Conductance, PCN 79-990.

<u>PORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-671</u>	<u>*SC-1.15, Inspection of Emergency Equipment, PCN 79-1001.</u>
<u>79-672</u>	<u>*PT-5.20, Process Instrumentation Reactor Purification Protection Channel Trip Test, PCN 79-1008.</u>
<u>79-673</u>	<u>PT-13.4.3, Flood Valve System Testing Fire System #3, H₂ Seal Oil System, PCN 79-1010.</u>
	<u>PT-13.4.4, Flood Valve System Testing Fire System #4, I-A Diesel Generator, PCN 79-1013.</u>
	<u>PT-13.4.5, Flood Valve System Testing Fire System #5, I-B Diesel Generator, PCN 79-1012.</u>
	<u>PT-13.4.6, Flood Valve System Testing Fire System #6, Auxiliary Feedwater Pump Lube Oil Reservoir, PCN 79- 1011.</u>
<u>79-675</u>	<u>A-60.1, Ginna Station Technical Specification Surveillance Program, PCN 79-994.</u>
	<u>A-60.3, Calibration Surveillance Program for Instrumentation/ Equipment of Safety Related Components, PCN 79-993.</u>
<u>79-676</u>	<u>*S-30.6, Safeguard Valve Position Verification (Inside C. V.), PCN 79-938.</u>
<u>79-684</u>	<u>*A-601, Plant Procedure Document Control, PCN 79-925.</u>
<u>79-685</u>	<u>*PT-2.2, Residual Heat Removal System, PCN 79-1017.</u>
<u>79-686</u>	<u>*P-9, Precautions, Limitations, and Setpoints P-9 Radiation Monitoring System, PCN 79-1082.</u>

PORC
ITEM #

PROCEDURE

79-687

A-301, Control of Station Modifications, PCN 79-968.

QCIP-5, Cleanliness Inspection of Systems and Components, PCN 79-1006.

79-689

A-52.4, Control of Limiting Conditions for Operating Equipment, PCN 79-1041.

O-1.1, Plant Heat Up From Cold Shutdown to Hot Shutdown, PCN 79-1046.

O-2.2, Plant Shutdown from Hot Shutdown to Cold Shutdown, PCN 79-1043.

O-2.3, Plant at Cold Shutdown, PCN 79-1047.

O-2.3.1, Draining the Reactor Coolant System, PCN 79-1044.

O-2.3.2, Filling and Venting the Reactor Coolant System, PCN 79-1045.

O-2.4, Plant Shutdown from Hot Shutdown to Cold Shutdown During Blackout, PCN 79-1050.

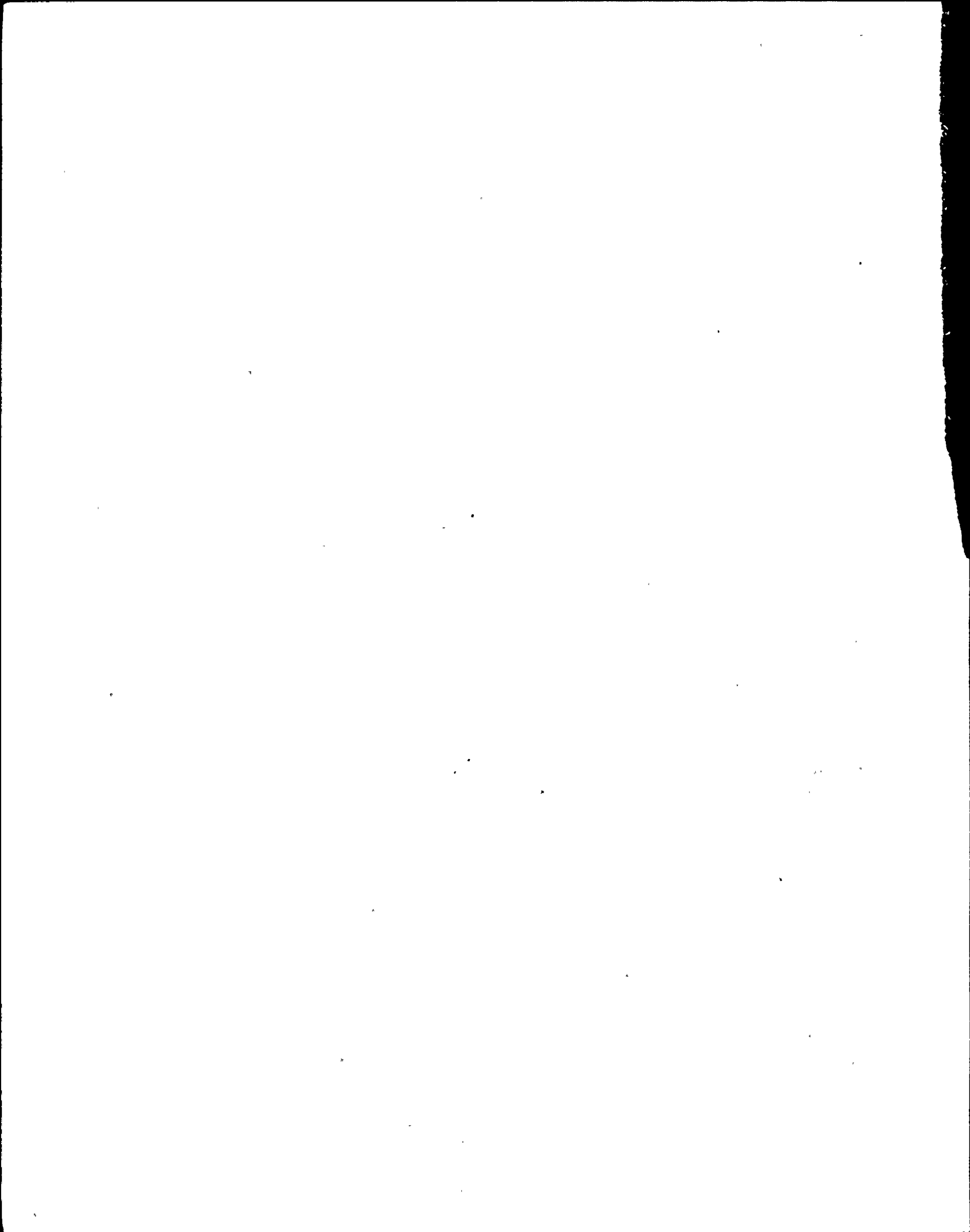
O-2.5, Plant Shutdown from Hot Shutdown to Cold Shutdown When Condenser Steam Dump is Unavailable, PCN 79-1049.

O-7, Alignment and Operation of the Reactor Vessel Overpressure Protection System, PCN 79-1042.

S-2.1, Reactor Coolant Pump Operation, PCN 79-1048.

79-690

*A-52.2, Control of Locked Valve Operation, PCN 79-1051.



PORC
ITEM #

PROCEDURE

79-691 *A-52.4, Control of Limiting Conditions for Operating Equipment, PCN 79-1052.

79-692 *A-602, Plant Procedure Distribution, PCN's 79-1060 and 79-1062.

79-693 *A-1301, Control of Material Handling and Handling Equipment, PCN 79-1053.

79-694 *CP-6.10, T avg. and Delta T Alignment at 70% Power or Greater, Loop A, Unit 1, PCN 79-1059.

79-695 *CP-11.0, Calibration and/or Maintenance of P.L.P. Rack Modules Not Covered by Other Calibration Procedures, PCN 79-1055.

79-696 *HP-9.1, Steam Generator Blowdown Activity, PCN 79-1067.

79-697 *P-1, Precautions, Limitations and Setpoints P-1, Reactor Control and Protection System, PCN 79-1022.

79-698 *PT-12.1, Emergency Diesel Generator "1A", PCN 79-1058.

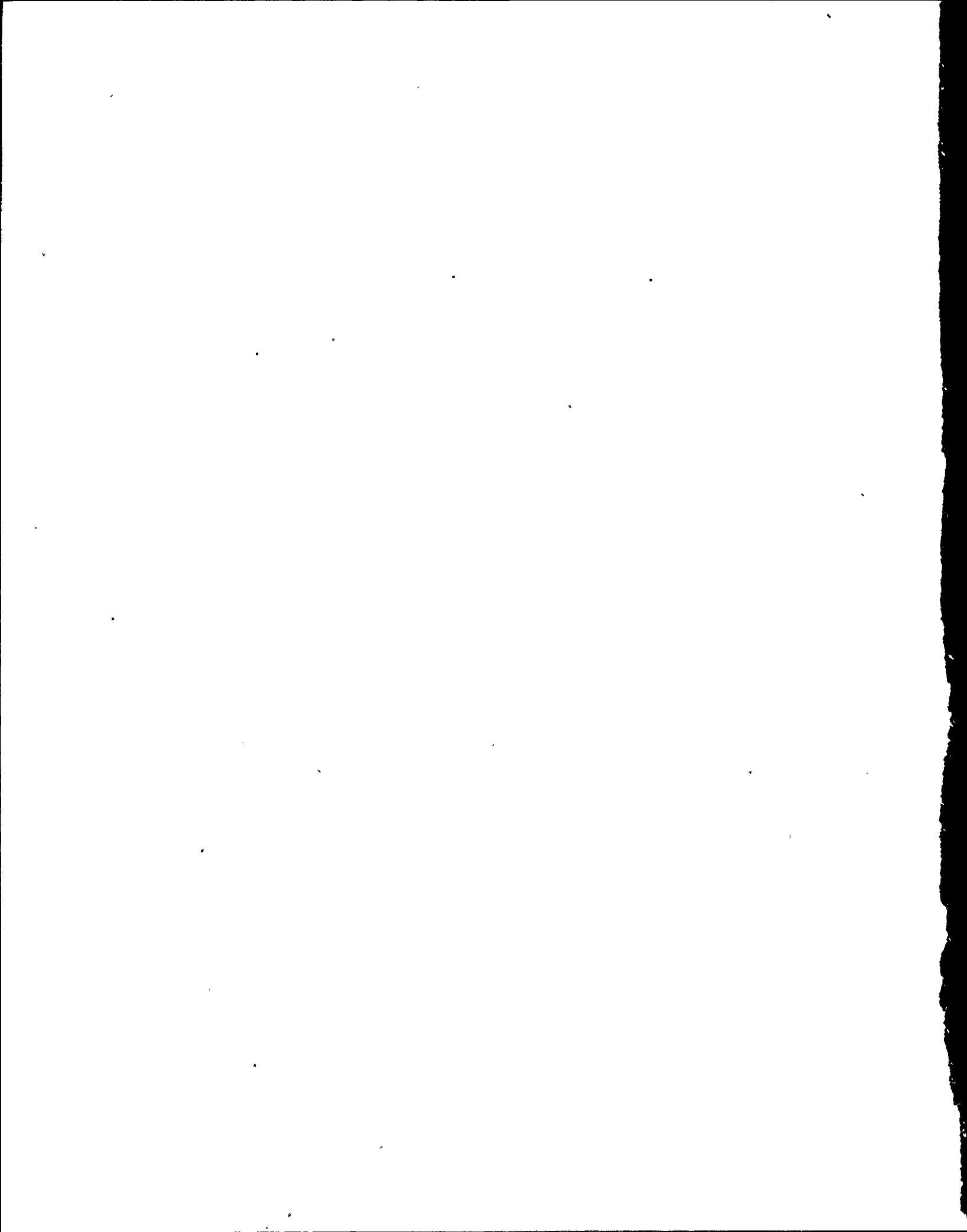
79-699 *PT-16, Auxiliary Feedwater System, PCN 79-1024.

79-700 *RF-46, Special Nuclear Material Physical Inventory, PCN 79-1054.

79-701 *S-4.1V, Transfer of Waste Evaporator Feed Tank to Waste Holdup Tank, PCN 79-1069.

79-702 *S-26.4, Operating Instruction Computer Rod Position Alarm Program, PCN 79-1019.

79-703 *S-30.4, Auxiliary Feedwater System Valve Position Verification, PCN 79-1065 and 79-1071.



<u>FORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-704</u>	<u>*S-30.5, Standby Auxiliary Feedwater Pump Valve Position, Verification PCN 79-1074.</u>
<u>79-705</u>	<u>*SC-1.12C, Specialized Call List, PCN 79-1064.</u>
<u>79-706</u>	<u>*T-27.1, 1A Emergency Diesel Generator Pre-Startup Alignment, PCN 79-1057.</u>
<u>79-707</u>	<u>*T-27.2, 1B Emergency Diesel Generator Pre-Startup Alignment, PCN 79-1056.</u>
<u>79-709</u>	<u>*T-44.2, Standby Auxiliary Feedwater (SAFW) System Alignment for Normal Operation, PCN 79-1073.</u>
<u>79-728</u>	<u>*SM 75-9.24, RWST Jet Shielding Installation Rework, PCN 79-1020.</u>
<u>79-740</u>	<u>*A-502, Plant Procedure Content and Format Requirements, PCN 79-973.</u>
<u>79-741</u>	<u>*M-37.13, Inspection and Maintenance of "A" Loop Accumulator Discharge Check Valve V-842B, PCN 79-1086.</u>
<u>79-742</u>	<u>*M-37.14, Inspection and Maintenance of "B" Loop Accumulator Discharge Check Valve V-842A, PCN 79-1087.</u>
<u>79-743</u>	<u>*S-3.3D, CVCS Cation Demineralizer Bed Operations Using "A" Deborating Unit, PCN 79-1127.</u>
<u>79-744</u>	<u>*S-23.5, Post Accident Charcoal Filter Operation, PCN 79-1084.</u>
<u>79-746</u>	<u>*A-52.2, Control of Locked Valve Operation, PCN's 79-1051 and 79-1124.</u>
<u>79-747</u>	<u>*A-603, Control of Engineering and Design Documents By Central Records, PCN 79-974.</u>

PORC
ITEM #

PROCEDURE

79-748 *A-604, Control of Construction Documents, PCN 79-975.

79-749 *A-803, Control of Items to be Reconditioned, PCN 79-1085.

79-750 CP-450.0, Overpressure Protection Loop 450, PCN 79-1078.
CP-451.0, Overpressure Protection Loop 451, PCN 79-1079.
CP-452.0, Overpressure Protection Loop 452, PCN 79-1080.

79-751 *GS-3.0, Security Call List, PCN 79-1091.

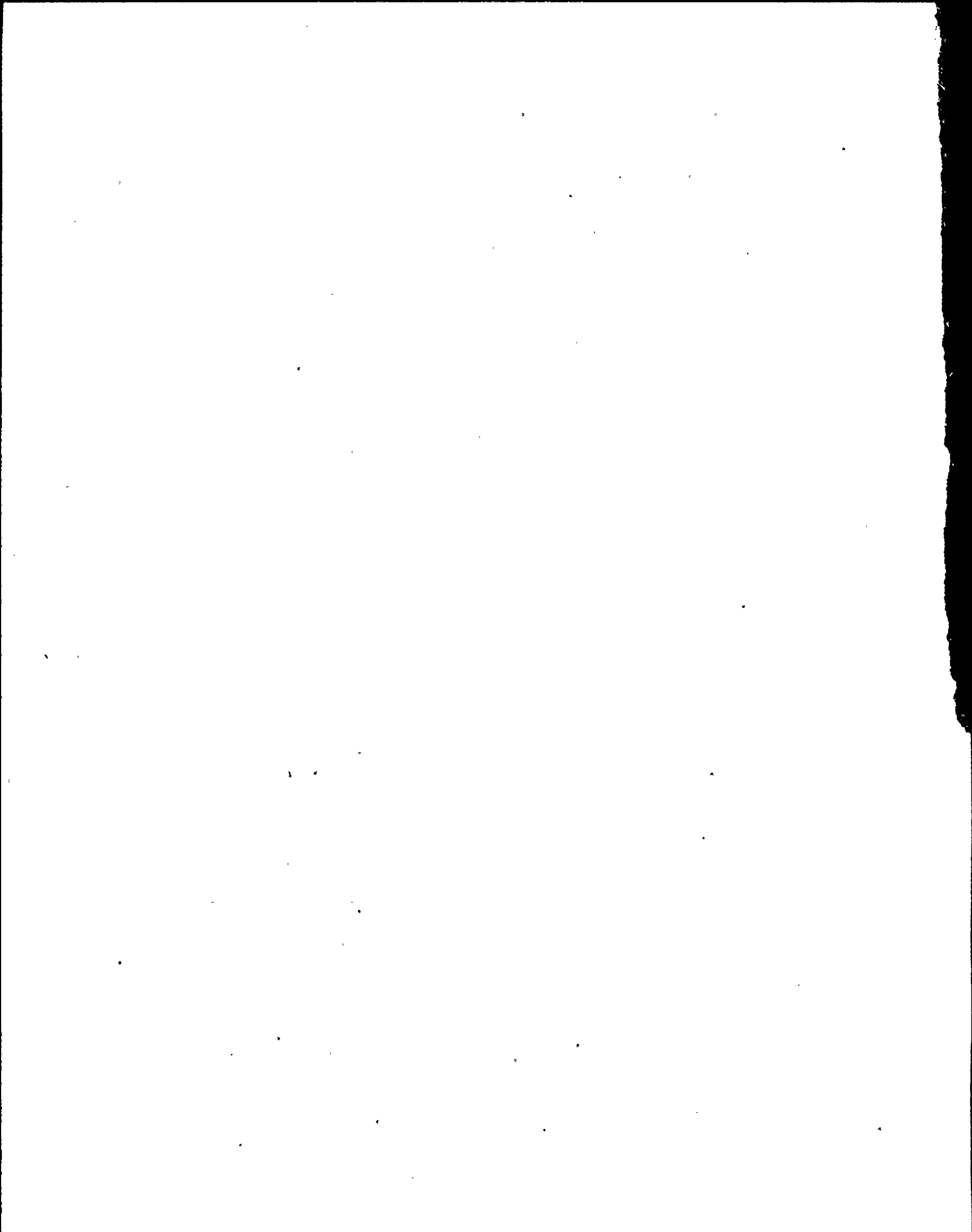
79-752 *M-11.8B, Reactor Coolant Pump - Seal Inspection and Service - Mechanical, PCN 79-1092.

79-753 *P-1, Precautions, Limitations and Setpoints, P-1, Reactor Control and Protection System, PCN 79-1072.

79-754 PT-5.10, Process Instrumentation Reactor Protection Channel Trip Test (Channel 1), PCN 79-1075.
PT-5.20, Process Instrumentation Reactor Purification Protection Channel Trip Test, PCN 79-1076.
PT-5.30, Process Instrumentation Reactor Protection Channel Trip Test (Channel 3), PCN 79-1077.

79-755 *S-8A, Component Cooling Water System Startup and Normal Operation Valve Alignment, PCN 79-1123.

79-756 *S-30.4, Auxiliary Feedwater System Valve Position Verification, PCN 79-1133.



PORC
ITEM #

PROCEDURE

- 79-757 *S-30.5, Standby Auxiliary Feedwater Pump Valve Position Verification PCN 79-1088.
- 79-758 *S-30.6, Safeguard Valve Position Verification (Inside CV), PCN 79-1089.
- 79-759 *SC-3.15.8, Fire System Locked Valve Cycling, PCN 79-1126.
- 79-762 A-101, Ginna Quality Assurance Program Implementation, PCN 79-980.
- A-201, Ginna Station Administrative and Engineering Staff Responsibilities, PCN 79-981.
- A-202, Ginna Station Staff Responsibilities for Fire Prevention, PCN 79-982.
- A-302, Preparation, Review and Approval of Design Input Documents for Minor Modifications, PCN 79-969.
- A-303, Preparation, Review and Approval of Safety Analysis for Minor Modifications or Special Tests, PCN 79-970.
- QCIP-31, Shipping Package #6144 Inspection, PCN 79-1081.
- TICP-12, Test Instrument Calibration Procedure, Category II, DC Voltage Calibrator, PCN 79-1068.
- 79-763 *SM 75-6.23, Service Water Redundant Return Line Support Assemblies Installation, PCN 79-1093.
- *SM 75-50.31, Jet Shielding Rework, Main Steam and Feedwater, PCN 79-1118.

PORC
ITEM #

PROCEDURE

*SM 76-01.1, Safeguard Buses DC Control Voltage Monitor, PCN 79-1117.

*SM 76-21.8, Reroute of Service Water Supply to Battery Room Air Conditioner, PCN 79-1116.

*SM 77-1474.1, Reactor Coolant Pump Seal Bypass Modification, PCN 79-1115.

*SM 77-1499.1, Rod Insertion Limit Circuitry, PCN 79-1114.

*SM 77-1623.1, Charging Pump Monorails, PCN 79-1113.

*SM 77-1623.2, Wedge Washer Installation - Charging Pump Monorail, PCN 79-1112.

*SM 77-1891.1, Electric Heat Tracing for Valves 825A and 825B, PCN 79-1110.

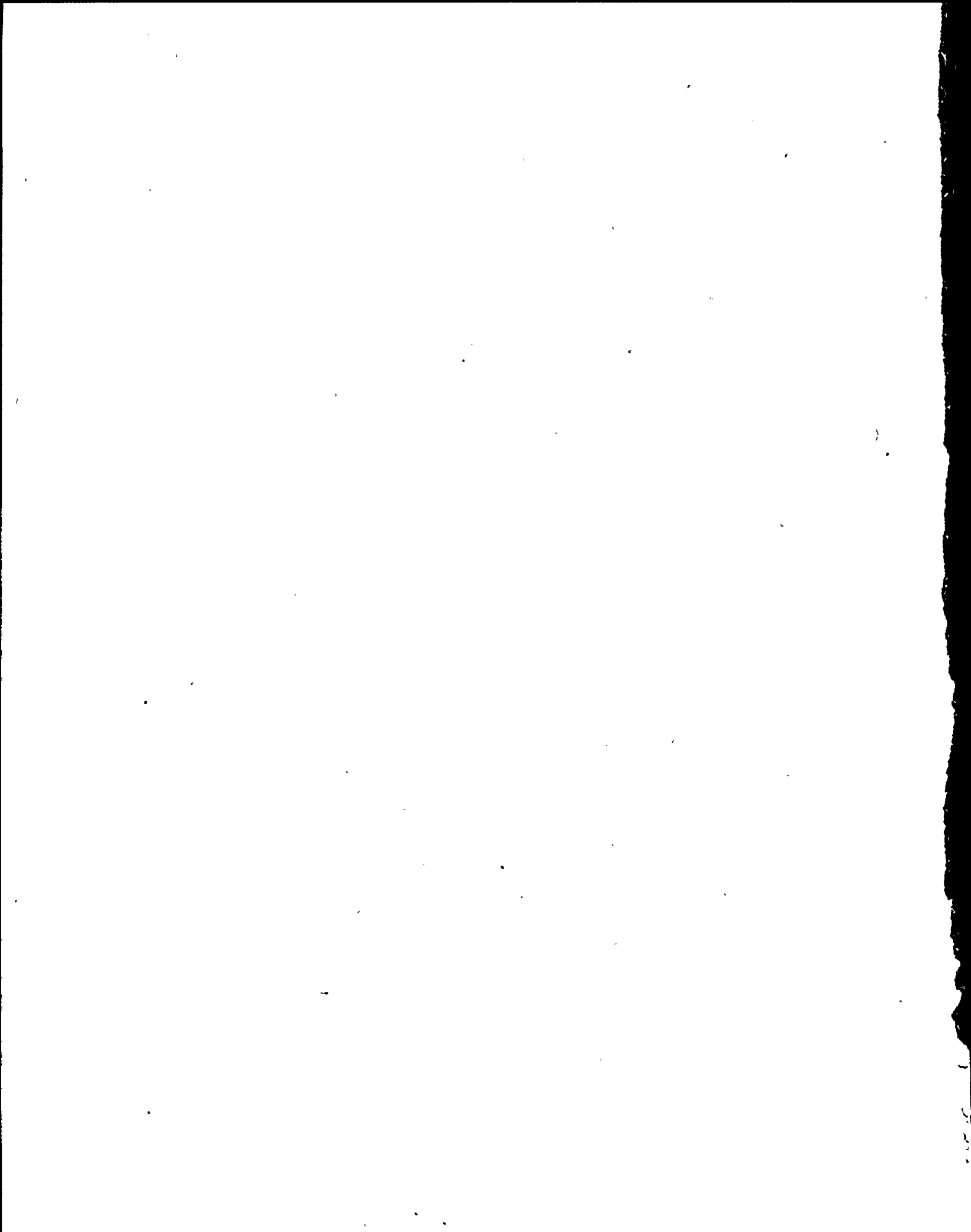
*SM 78-1464.1, Replacement of Volume Control Tank Level Transmitters LT-139 and LT-112, PCN 79-1108.

*SM 78-1464.2, Replacement of Accumulator Level Transmitters LT-935, LT-934, LT-938, and LT-939, PCN 79-1107.

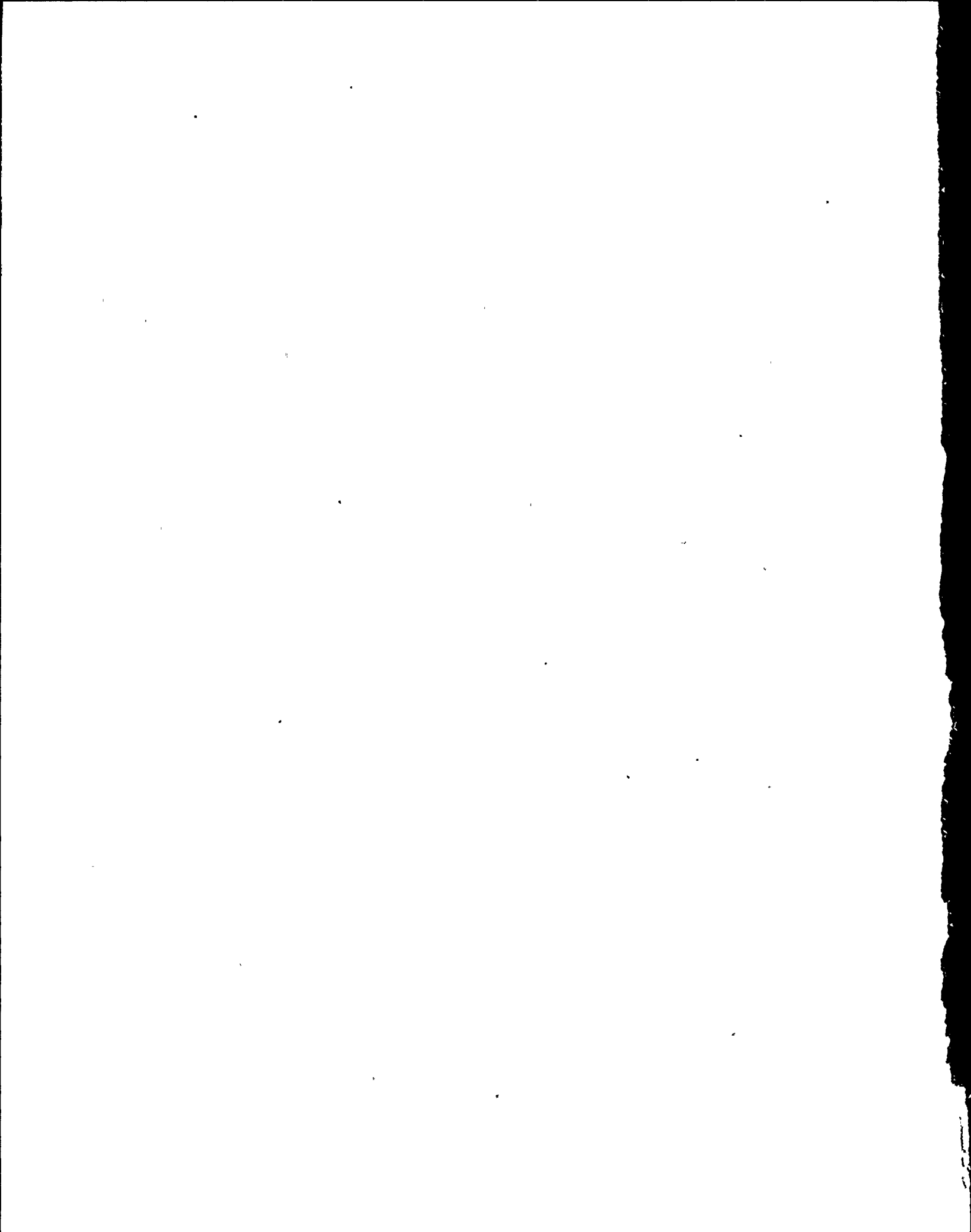
*SM 78-1890.2, Feedwater Line Drain Valves after the Containment Check Valves, PCN 79-1101.

*SM 78-2133.1, Valves 866A and 866B Modification for Testing, PCN 79-1100.

*SM 78-2273.1, Armor Plate on the East Wall of the Control Room.



<u>FORC</u> <u>ITEM #</u>	<u>PROCEDURE</u>
<u>79-778</u>	<u>*A-2.1, Receiving, Inventory and Shipment of Fuel, PCN 79-1121.</u>
<u>79-779</u>	<u>*A-2.8, Special Nuclear Material Inventory and Record Requirements, PCN 79-1122.</u>
<u>79-780</u>	<u>*M-37.18K, Repair of the "A" Safety Injection Pump Discharge Check Valve 889A, PCN 79-1131.</u>
<u>79-783</u>	<u>QC-1301, Control of Material Handling and Handling Equipment, PCN 79-1130.</u>
	<u>QC-1302, Storage and Preservation of Materials, PCN 79-1129.</u>
<u>79-786</u>	<u>*P-9, Precautions, Limitations, and Setpoints, P-9, Radiation Monitoring System, PCN 79-1151.</u>
<u>79-787</u>	<u>PT-2.1, Safety Injection System Pumps, PCN 79-1145.</u>
	<u>PT-2.2, Residual Heat Removal System, PCN 79-1147.</u>
	<u>PT-2.7, Service Water System, PCN 79-1146.</u>
	<u>PT-2.8, Component Cooling Water Pump System, PCN 79-1143.</u>
	<u>PT-2.9.1, Check Valve Exercising Quarterly Requirements (RCDT Pump Discharge), PCN 79-1144.</u>
	<u>PT-3, Containment Spray Pumps and NaOH Additive System, PCN 79-1148.</u>
	<u>PT-16, Auxiliary Feedwater System, PCN 79-1142.</u>



PORC
ITEM #

PROCEDURE

79-788 *S-7A, RWST Purification, PCN 79-1141.

79-789 *SC-3.15.8, Fire System Locked Valve Cycling, PCN 79-1128.

79-795 *A-1.6, ALARA Committee Operating Procedure, PCN 79-1184.

79-796 M-11.5B, Major Mechanical Inspection of AFWP, PCN 79-1155.
M-11.5C, Minor Mechanical Inspection of AFWP, PCN 79-1154.
M-11.5K, Turbine Driven Auxiliary Feedwater Pump Major or
Mechanical Inspection, PCN 79-1161.
M-11.10, Major Inspection of Service Water Pump, PCN 79-1162.
M-11.12.1, Safety Injection Pump, Mechanical Inspection, PCN
79-1157.
M-11.15, RHR Pumps Inspection/Maintenance, PCN 79-1156.

79-797 *M-11.5E, Pipefitters Inspection of Motor Driven Auxiliary Feed Pump
For Pump, PCN 79-1153.

79-798 *SC-1.2, Local Radiation Emergency Procedure, PCN 79-1185.

79-799 *PT-5.20, Process Instrumentation Reactor Protection Channel Trip
Test (Channel 2), PCN 79-1200.

79-800 S-4.2.12, Gas Analyzer Operation, PCN 79-1172.
S-5, Primary System Sampling, PCN 79-1173.

PORC
ITEM #PROCEDURE

79-801 *A-52.2, Control of Locked Valve Operation, PCN 79-1152.

79-802 *A-53.0, Preventive Maintenance Program, PCN 79-1198.

79-803 *A-54.1, Licensed Personnel Authority, PCN 79-1168.

79-804 *A-102.11, Emergency Plan Training Program, PCN 79-1196.

79-805 *A-502, Plant Procedure and Format Requirements, PCN 79-1203.

79-806 *A-1401, Station Holding Rules, PCN 79-1194.

79-807 *A-1701, Quality Assurance Records, PCN's 79-1135 and 79-1136.

79-808 E-1.1, Safety Injection System Actuation, PCN 79-1204.

E-1.2, Loss of Coolant Accident, PCN 79-1205.

E-1.3, Steam Line Break Accident, PCN 79-1206.

E-1.4, Steam Generator Tube Rupture Accident, PCN 79-1207.

79-809 *HP-1.3, External Exposure Records, PCN 79-1174.

79-810 *HP-5.2, Posting of Radiation Areas and Container Labeling, PCN
79-1171.

79-811 *P-9, Radiation Monitoring System, PCN 79-1209.

79-812 PC-1.1, Primary Coolant Analysis Schedule and Limits, PCN 79-1192.

FORC
ITEM #

PROCEDURE

PC-1.2, Non-Primary Coolant Analysis Schedule and Limits, PCN 79-1191.

PC-1.3, Daily Chemistry Analysis Results, PCN 79-1193.

79-813 *PT-21, Cleaning Boric Acid Tank Level Sensing Lines, PCN 79-1197.

79-814 *S-3.1F, 1B Boric Acid Storage Tank Isolation/Restoration, PCN 79-1176.

79-816 *S-13A, RHR System Lineup for Safety Injection, PCN 79-1182.

79-817 *S-13B, RHR Pump Isolation, PCN 79-1179.

79-818 *S-16A, Safety Injection System Alignment, PCN 79-1180.

79-819 *S-16.6, Draining the "B" Accumulator, PCN 79-1181.

79-820 *S-16.11, Monitoring and Enrichment of Accumulator Boron Concentration, PCN 79-1178.

79-821 *S-16.14, Isolation and Restoration of SI Pumps Suction Line from Boric Acid Tanks, PCN 79-1177.

79-822 *S-16.16, Safety Injection Pump Isolation, PCN 79-1158.

79-823 *S-30.2, RHR System Valve Position Verification, PCN 79-1183.

79-824 *T-41B, Turbine Driven Auxiliary Feed Pump - Removal from Service, PCN 79-1160.

<u>PORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-825</u>	<u>*T-41D, Motor Driven Auxiliary Feedwater Pump Isolation, PCN 79-1159.</u>
<u>79-826</u>	<u>*WC-10.1, Specific Conductance, PCN 79-1170.</u>
<u>79-827</u>	<u>A-37.0, Conduct of Operations for Safety Evaluation as Related to Facility Modifications, Special Tests or Experiments, PCN 79-1167.</u>
	<u>A-37.1, System Modification Requirements, PCN 79-1166.</u>
	<u>QC-301, Work Start Authorization, PCN 79-1165.</u>
	<u>QC-303, Control of System Modification Documentation, PCN 79-1164.</u>
	<u>QC-304, Control of Modification Activities by the Ginna Modification Project, PCN 79-1163.</u>
<u>79-828</u>	<u>CP-221.0, Calibration and/or Maintenance of RMS Channel, PCN 79-1186.</u>
	<u>CP-221.1, Calibration of Ratemeter Drawer R-2, PCN 79-1187.</u>
	<u>CP-221.2, Calibration of R-21 Detector, PCN 79-1188.</u>
	<u>CP-221.3, Replacement of Detector for RMS Channel R-21, PCN 79-1189.</u>
<u>79-838</u>	<u>*SC-1, Radiation Emergency Plan, PCN 79-1218.</u>
<u>79-839</u>	<u>*SM 77-1431.3, Auxiliary Feedwater Flow Control Bypass Modification Termination and Checkout, PCN 79-1210.</u>

FORC
ITEM #

PROCEDURE

- 79-840 *A-52.4, Control of Limiting Conditions for Operating Equipment, PCN 79-1134.
- 79-841 *A-54.3, Open Flame, Welding and Grinding Permit, PCN 79-1224.
- 79-842 *A-701, Receipt and Acceptance of Materials/Parts, PCN 79-1223.
- 79-843 *HP-8.4, Radioactive Source Inventory, PCN 79-1219.
- 79-844 *M-67.3, Flood Valve System Maintenance System Number 3, 4, 5, 6, PCN 79-1199.
- 79-845 *S-3.3C, H² or O² Removal From Primary System By Burping Volume Control Tank With N², PCN 79-1217.
- 79-846 *S-4.1A, Liquid Waste Process Startup, PCN 79-1201.
- 79-847 *S-13A, RHR System Lineup for Safety Injection, PCN 79-1211.
- 79-849 M-73.2, Welding on Stainless Steel Piping or Components, PCN 79-32.
- M-73.3, Welding on Carbon Steel Piping or Components, PCN 79-31.
- QC-101, Ginna QA Program Implementation, PCN 79-1215.
- QC-201, Ginna Quality Control Organization, PCN 79-1214.
- QC-702, Receipt of New Fuel, PCN 79-1227.
- QC-904, Nondestructive Examination, PCN 79-1213.

FORC
ITEM #

PROCEDURE

- QC-1204, Calibration of Process System Instrumentation, PCN 79-1226.
- QC-1502, Nonconformance Reports, PCN 79-1212.
- QC-1703, Record Storage Facility and Equipment, PCN 79-1225.
- 79-860 *A-17, Plant Operation Review Committee Operating Procedure, PCN 79-1271.
- 79-861 *A-54.4, Duty Engineer's Responsibilities, PCN 79-1232.
- 79-862 *AR/E-12, NIS Power Range Lower Detector High Flux Dev. or Auto Defeat, PCN 79-1243.
- 79-863 *AR/E-26, NIS Power Range Channel Deviation, PCN 79-1244.
- 79-866 *CP-81.0, Calibration Procedure for TQTP 650 Dual Bench Tester for Torque Wrenches, PCN 79-1228.
- 79-867 *CP-81.1, Calibration Procedure for Torque Wrenches, PCN 79-1229.
- 79-868 *E-1.1, Safety Injection System Actuation, PCN 79-1270.
- 79-869 *E-1.4, Steam Generator Tube Rupture Accident, PCN's 79-1153 and 79-1154.
- 79-870 *GS-3.0, Security Call List, PCN 79-1236.
- 79-871 *GS-26.1, Unoccupied Vital Area Key Control and Accountability, PCN 79-1251.

FORC
ITEM #PROCEDURE

79-872 *M-15.1, A or B Diesel Generator Removal From Service, PCN 79-1262.

79-873 *M-73.10, Welding, PCN 79-1255.

79-874 *PT-17.2, Process Radiation Monitors R-11 - R-21 and Iodine Monitors R-10A and R-10B, PCN 79-1272.

79-875 *PT-36.2, Standby Auxiliary Feedwater System Recirculation Flow Test PCN 79-1261.

79-877 *S-30.5, Standby Auxiliary Feedwater Pump Valve Position Verification PCN's 79-1231 and 79-1234.

79-878 *SC-1.7C, Monitoring Site Radiation Level By TLD, PCN 79-1266.

79-879 *SC-1.7D, Emergency Radiation Monitoring Off-Site Survey Team #9D (Red), PCN 79-1264.

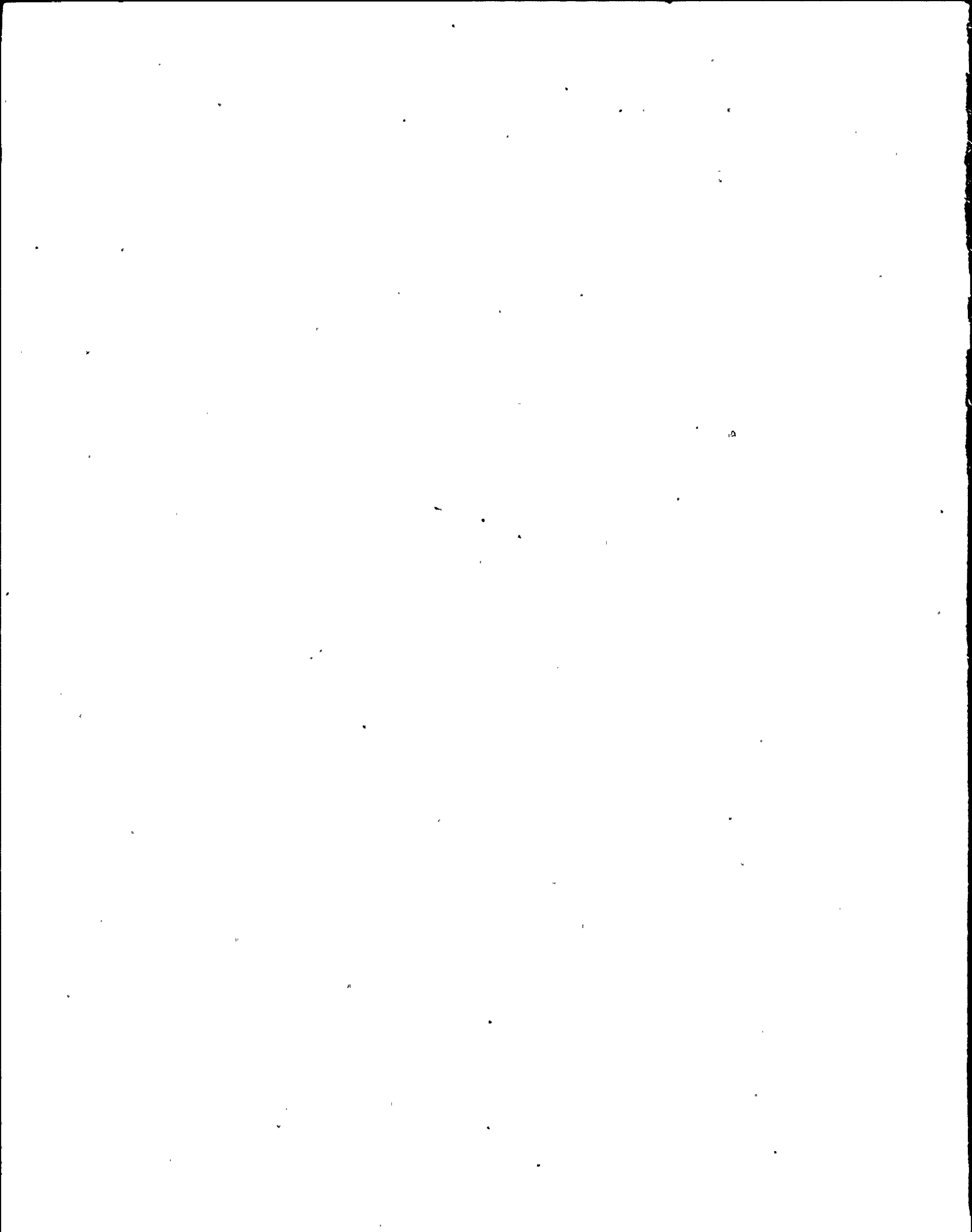
79-880 *SC-1.7F, Emergency Radiation Monitoring Off-Site Survey Team #9F (Green), PCN 79-1268.

79-881 *SC-1.7H, Emergency Radiation Monitoring Off-Site Survey Team #9H (Orange), PCN 79-1267.

79-882 *SM 77-1431.3, Auxiliary Feedwater Flow Control Bypass Modification Termination and Checkout, PCN 79-1260.

79-885 A-54, Ginna Station Administrative and Engineering Staff Responsibilities, PCN 79-1251.

A-54.2, Ginna Station Staff Responsibilities for Fire Prevention, PCN 79-1256.



PORC
ITEM #

PROCEDURE

- SC-3.15.6, Fire Hose Reel Inspection, PCN 79-1269.
- 79-886 E-1.5, Void Formation in the Reactor Coolant System, PCN 79-1208.
- HP-7.9, 1000B Gamma Calibrator Operation, PCN 79-1230.
- 79-893 *SM 79-2449.1, Safety Injection Logic Modification, PCN 79-1278.
- 79-894 *O-5.1, Load Reductions, PCN 79-1242.
- 79-895 *PT-5.10, Process Instrumentation Reactor Protection Channel Trip Test (Channel 1), PCN 79-1237.
- 79-896 *PT-5.20, Process Instrumentation Reactor Protection Channel Trip Test (Channel 2), PCN 79-1238.
- 79-897 *PT-5.30, Process Instrumentation Reactor Protection Channel Trip Test (Channel 3), PCN's 79-1239, 79-1241, 79-1248.
- 79-898 *PT-5.40, Process Instrumentation Reactor Protection Channel Trip Test (Channel 4), PCN 79-1240.
- 79-899 *A-304, Preparation, Review and Approval of Design Output Documents for Minor Modifications, PCN 79-971.
- 79-906 *O-8, Natural Circulation in the RCS, PCN 79-1247.
- *SC-3.19, Maintaining Access to Hydrants During Severe Snow Storms, PCN 79-1265.
- 79-907 *E-22.1, Malfunction of the Charging System, PCN 79-1312.

PORC
ITEM #

PROCEDURE

*E-22.2, Complete Loss of Charging Flow Due to Rupture of System, PCN 79-1311.

*E-22.3, Malfunction of the Letdown System, PCN 79-1310.

79-908

*HP-1.4, Noble Gas Exposure, PCN 79-1315.

*HP-2.1, Whole Body Counting Guide, PCN 79-1316.

*HP-2.3, TLD Reader Calibration, PCN 79-1314.

*HP-6.3, Personnel Decontamination, PCN 79-1317.

*HP-6.4, Contaminated Clothing Report Form Use, PCN 79-1318.

*HP-8.1, Radioactive Source Use, PCN 79-1319.

*RD-4, Vent Radiogas Background and Factor Determination, PCN 79-1320.

*RD-11, Receipt of Radioactive Materials, PCN 79-1321.

79-909

*A-1, Radiation Control Manual, PCN 79-1276.

79-910

*A-201, Ginna Station Administrative and Engineering Staff Responsibilities, PCN 79-1258.

79-911

*A-1702, Record Storage Facilities and Equipment, PCN 79-1297.

79-912

*AR/D-18, Intermediate Range Reactor Temp., PCN 79-1300.

FORC
ITEM #

PROCEDURE

- 79-913 E-12.1, Malfunctioning Rod Cluster Control Full Length Rods, PCN 79-1287.
- E-13.2, Intermediate Range NIS Malfunction (IR-35 or -36), PCN 79-1288.
- E-13.3, Source Range NIS Malfunction (SR-31 or -32), PCN 79-1289.
- E-13.4, Malfunction of Start-up Rate, Comparator or Flux Deviation Channel (N-37, N-46 or Upper and Lower Ion Chamber), PCN 79-1290.
- E-18.1, Loss of Spent Fuel Pit Cooling Due to the Failure of the Spent Fuel Pit Pump, PCN 79-1291.
- E-19, Continuous Insertion of RCCA Control Bank, PCN 79-1292.
- E-37.4, Sticking of a Fuel Assembly Inside the Reactor Vessel or Inside the Transfer Basket, PCN 79-1294.
- E-38, Loss of Service Water, PCN 79-1295.
- 79-914 *HP-2.4, TLD Readout, PCN 79-1313.
- 79-915 *HP-6.1, Neutron Exposure, PCN 79-1275.
- 79-916 *HP-9.1, Steam Generator Blowdown Activity, PCN 79-1277.
- 79-917 *M-52.3, Incore Thimble Cleaning, PCN 79-1309.
- 79-918 *O-6.4.1, Reference Equilibrium Indicated Axial Flux Difference Determination, PCN 79-1327.
- 79-919 *P-9, Radiation Monitoring System, PCN 79-1274.

PORC
ITEM #

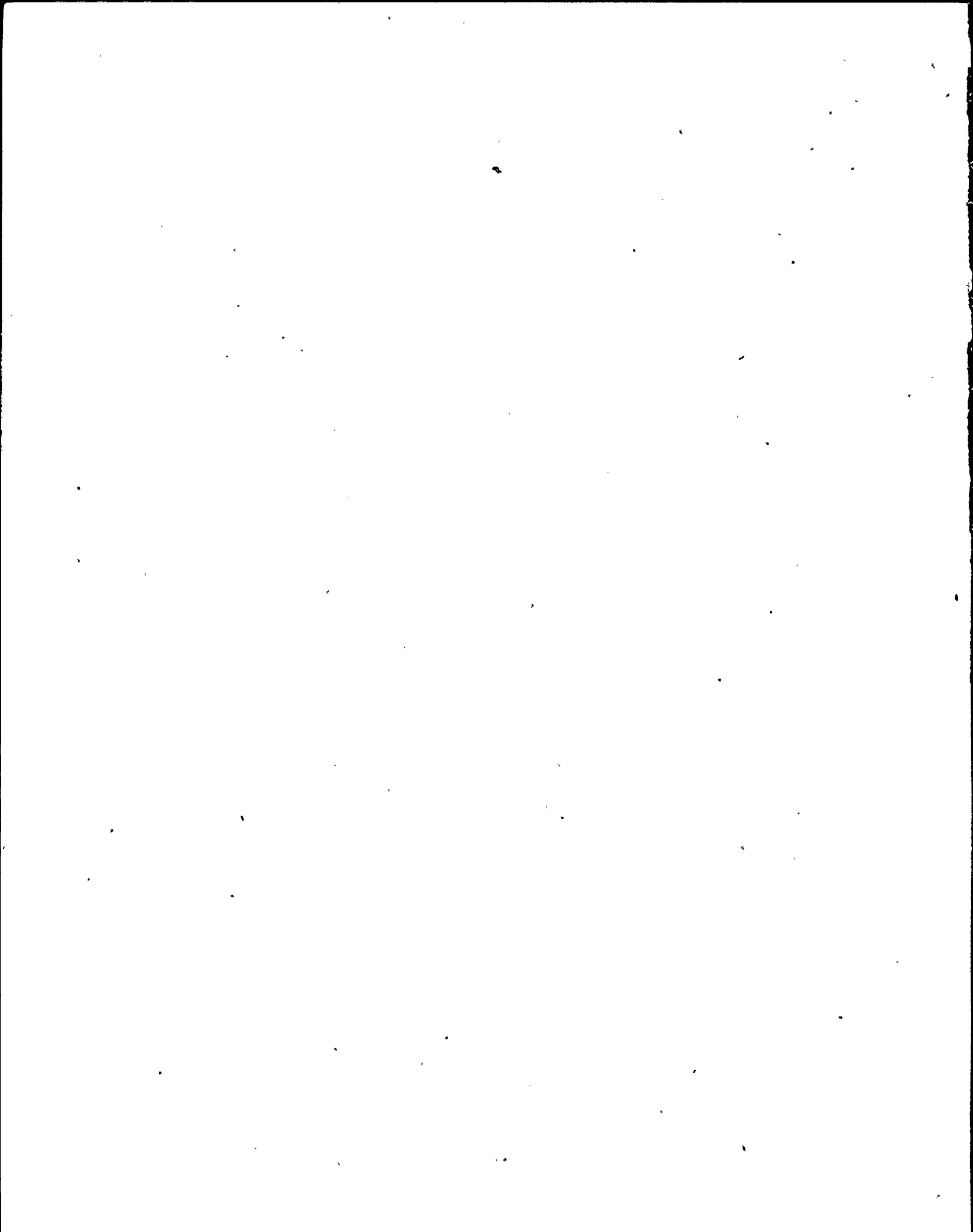
PROCEDURE

- 79-920 *PT-2.5, Air Operated Valves, Quarterly Surveillance, PCN 79-1283.
- 79-921 *PT-2.9.1, Check Valve Exercising Quarterly Requirement (RCDT Pump Discharge), PCN 79-1328.
- 79-922 *PT-36.2, Standby Auxiliary Feedwater System Recirculation Flow Test PCN 79-1273.
- 79-924 *SM 77-1431.3, Auxiliary Feedwater Flow Control Bypass Modification Termination and Checkout, PCN 79-1279.
- 79-927 *T-27.2, 1B Emergency Diesel Generator Pre-Startup Alignment, PCN 79-1280.
- 79-929 *T-36.1, Station Service Water Header Valve Alignment for Two Loop Operation, PCN 79-1298.
- 79-931 *T-41A, Alignment of Auxiliary Feedwater System Prior to Power Operation, PCN 79-1284.
- 79-941 *EM-233, Rework of Pipe Supports, Baseplates, and Anchor Bolts, PCN 79-1390.
- 79-942 *ISI-9, Inservice Inspection of Wedge Type Anchor Bolts, PCN 79-1392.
- 79-943 A-102.1, Facility Staff Training Program Ginna Station Personnel Training Program, PCN 79-1354.
- A-102.11, Emergency Plan Training Program, PCN 79-1355.
- A-102.13, R. E. Ginna NRC Licensing Training Program, PCN 79-1356.

PORC
ITEM #

PROCEDURE

- 79-944 E-29.1, Loss of Normal Feedwater (Not Recoverable), PCN 79-1351.
- E-29.2, Loss of Normal Feedwater (Recoverable), PCN 79-1352.
- 79-945 *M-37.11, Pressurizer Power Operated Relief Valves 430 and 431C
Repair and Inspection, PCN 79-1394.
- 79-946 *O-1.2, Plant From Hot Shutdown to Steady Load, PCN 79-1353.
- 79-947 *PT-2.1, Safety Injection System Pumps, PCN 79-1341.
- 79-948 PT-2.5.1, Air Operated Valves, Quarterly Surveillance (Containment)
PCN 79-1338.
- PT-2.5.2, Air Operated Valves, Quarterly Surveillance (Valves 112B
and 112C), PCN 79-1339.
- PT-2.6, Cold/Refueling Shutdown Air Operated Valve Surveillance,
PCN 79-1340.
- 79-949 *PT-2.9, Check Valve Exercising Quarterly Requirement, PCN 79-1337.
- 79-950 *PT-13.4.8, Protomatic Deluge Valve System Testing System #8, (Main
Transformer), PCN 79-1367.
- 79-951 *S-23.2.2, Containment Purge Procedure, PCN 79-1343.
- 79-952 *SC-3.3.2, Off-site Notification of Fire, PCN 79-1329.
- 79-953 *SM 77-1431.3, Auxiliary Feedwater Flow Control Bypass Modification
Termination and Checkout, PCN 79-1342.



PORC
ITEM #

PROCEDURE

79-954 *T-27.1, 1A Emergency Diesel Generator Pre-Start Alignment, PCN 79-1360.

79-955 *T-27.2, 1B Emergency Diesel Generator Pre-Startup Alignment, PCN 79-1361.

79-956 *T-41E, Alternate Water Supply for Auxiliary Feedwater Pumps, PCN 79-1299.

79-956A *ISI-8, Inservice Inspection of Feedwater Nozzle Welds, PCN 79-1324.

79-956B *EM-232, Repair of Feedwater Piping Inside Containment, PCN 79-1358.

79-957 *SM 79-01.1, Feedwater Drain Lines Inside Containment, PCN 79-1359.

79-958 AR/H-31, Vacuum Priming System, PCN 79-1345.

AR/L-12, Condenser Pit, Screen House, High Level, PCN 79-1344.

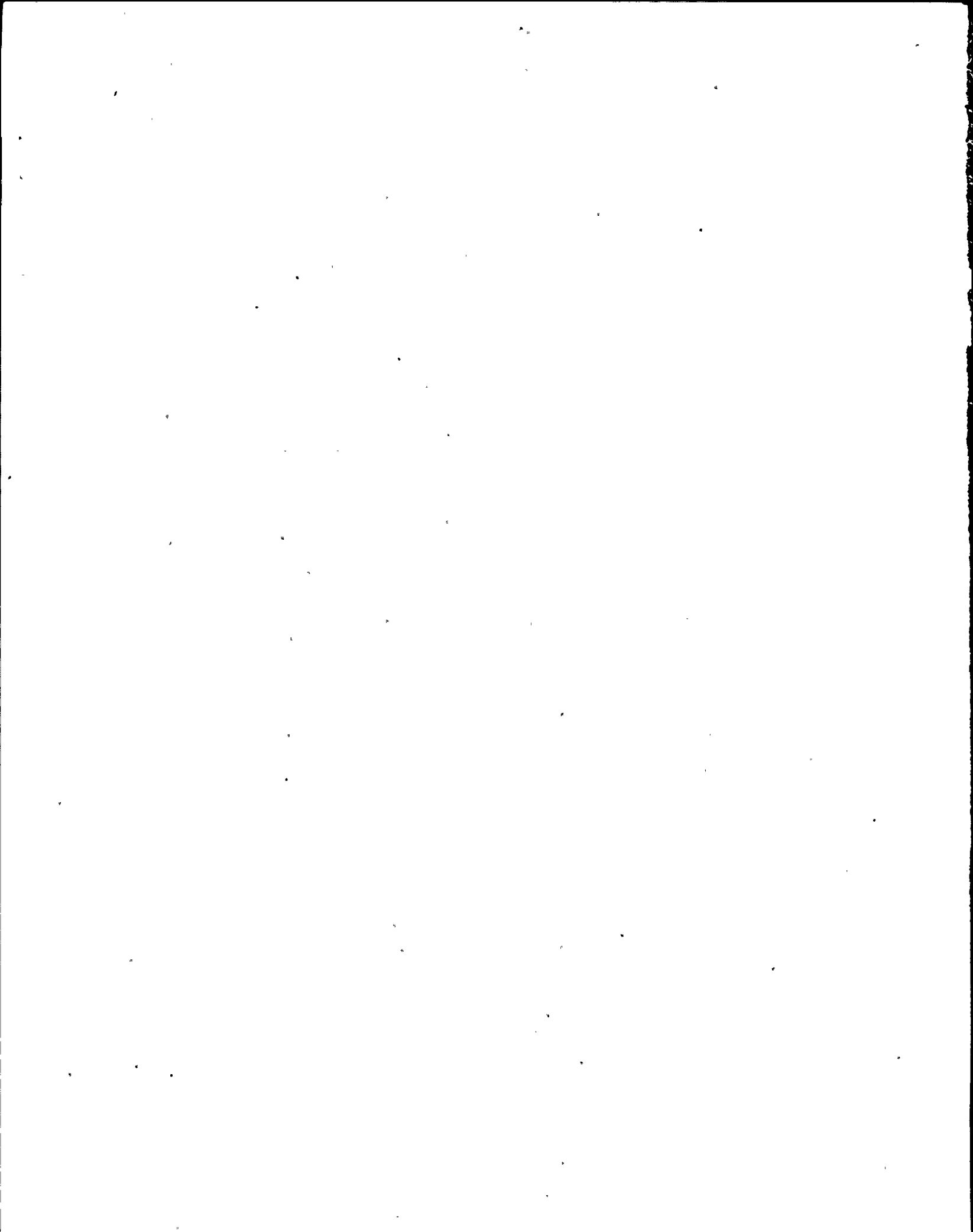
E-7, Drop of a Full Length Rod Cluster Control Assembly, PCN 79-1346.

E-21, Malfunction of Reactor Make-up Control, PCN 79-1349.

O-1.1D, Pre-Heatup Plant Requirement Check List, PCN 79-1357.

PR-9A, Protective Relay Calibration and Trip Test Reactor Trip Underfrequency Protection Bus 11A, PCN 79-1330.

PR-9B, Protective Relay Calibration and Trip Test Reactor Trip Underfrequency Protection Bus 11B, PCN 79-1331.



FORC
ITEM #

PROCEDURE

PT-5.50, Auto Stop Oil Pressure Switch and Relay Test, PCN 79-1332.

PT-13.2, Diesel Fire Pump Standard Protection Test, PCN 79-1333.

RSSP-13.3, Hydro on WDS Class "C" Safety Related Piping - Part 1
Waste Holdup Tank Area (WHUT), PCN 79-1334.

RSSP-13.4, Hydro on WDS Class "C" Safety Related Piping - Part 2
Sump Tank Pump Area, PCN 79-1335.

S-3.2F, Draining the A, B, or C CVCS Holdup Tank to the Waste
Holdup Tank Via the Sump Tank Pump, PCN 79-1336.

79-958A

E-13.4, Malfunction of Start-up Rate, Comparator or Flux Deviation
Channel (N-37, N-46 or Upper and Lower Ion Chamber), PCN 79-1347.

E-22.1, Malfunction of the Charging System, PCN 79-1350.

79-962

*ISI-9, Inservice Inspection of Wedge Type Anchor Bolts, PCN
79-1403.

79-965

*ISI-8, Inservice Inspection of Feedwater Nozzle Welds, PCN 79-1413.

79-966

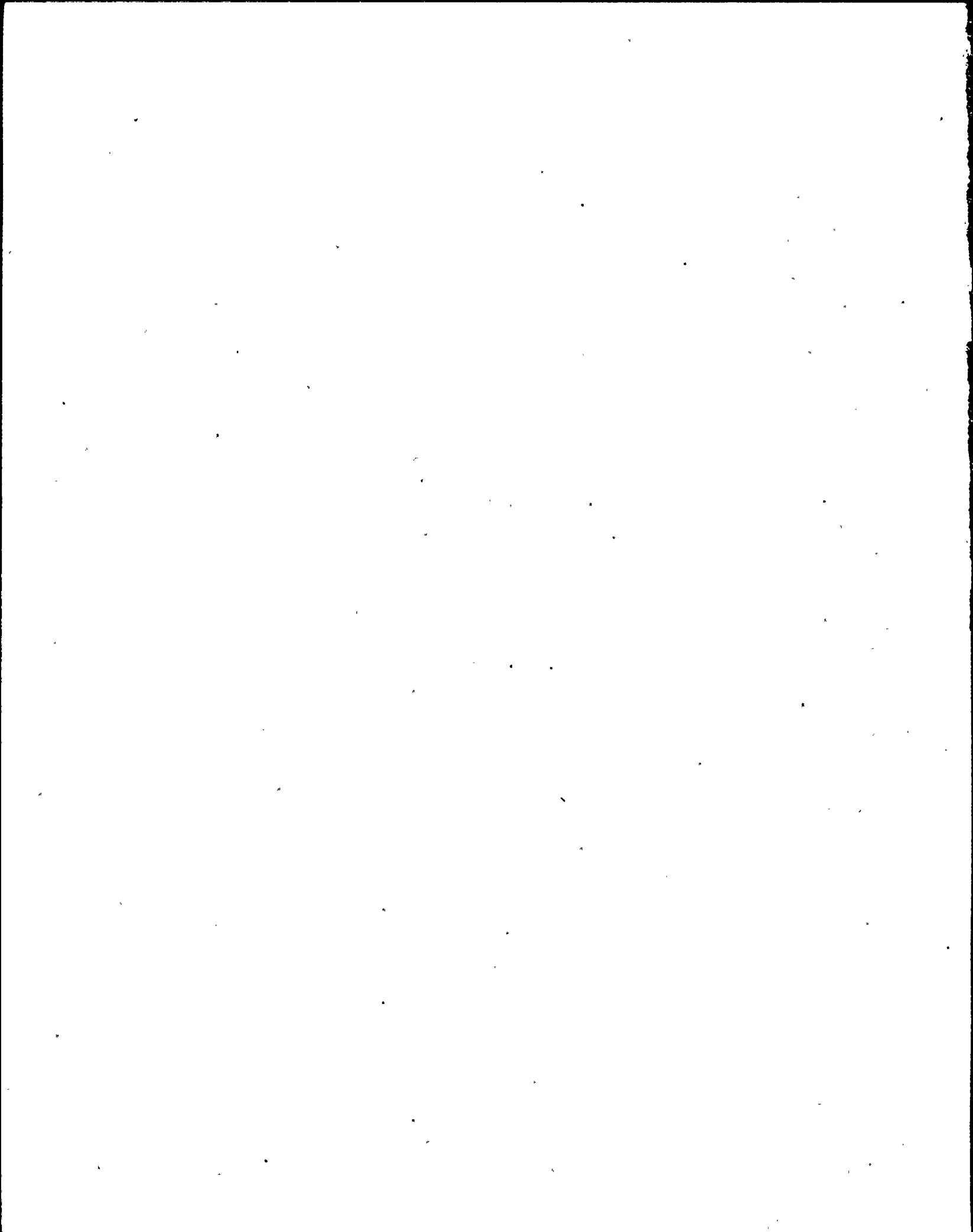
*PT-36.2, Standby Auxiliary Feedwater System Recirculation Flow
Test, PCN 79-1431.

79-967

E-1.6, CVCS Leak, PCN 79-1302.

M-37.55.1, Spray Additive Valve Controller Repair (836A or 836B).

S-3.3H, Non-Condensable Gas Bubble in the RCS, PCN 79-1296.



PORC
ITEM #

PROCEDURE

79-967A

*EM-234, Repair of Feedwater - Steam Generator Nozzles, PCN 79-1460.

79-968

EM-163, Repair of 10" Fire Main Between Turbine Building and Auxiliary Building, PCN 79-1384.

EM-165, Boric Acid Pipe Line Repairs Between Valve 827B and 1B Boric Acid Storage Tank, PCN 79-1383.

EM-166, Boric Acid Pipe Line Weld Repairs Between Valves 827A and 827B and Valves 825A and 825B Weld Joint No. , PCN 79-1382.

EM-167, Safety Injection Pump Suction Piping Hydro, PCN 79-1381.

EM-168, Safety Injection Pump Suction Piping Hydro for Repaired Piping, PCN 79-1380.

EM-171, Divert Line Leak Repair Between 112A Letdown Diversion Valve and 1103D Divert to CVCS HUT, PCN 79-1379.

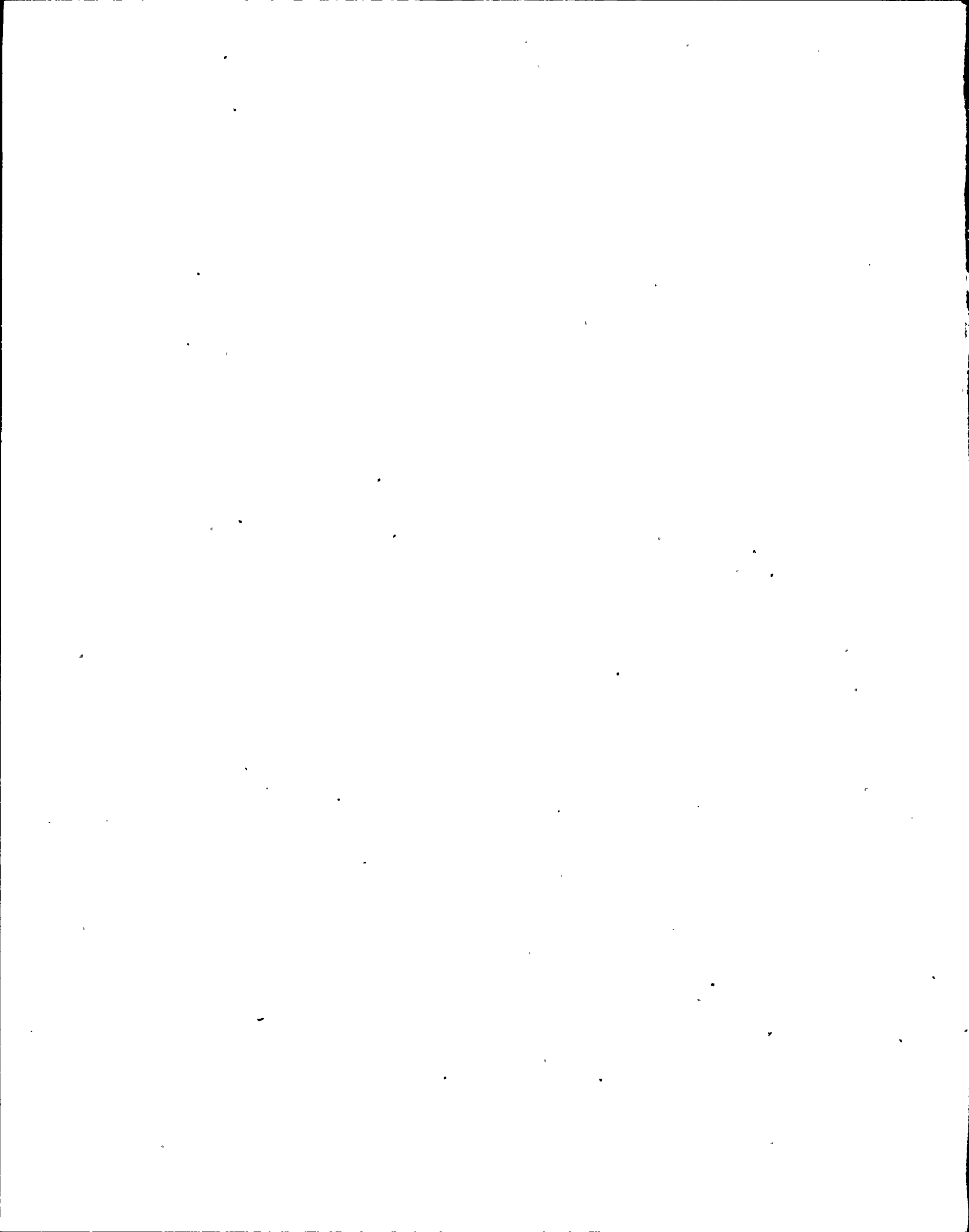
EM-172, "A" Mixed Bed Demineralizer Piping Leak Valve Repair, PCN 79-1378.

EM-176, Liquid Penetrant Indication Removal on Letdown Line to Regen-erative Heat Exchanger, PCN 79-1377.

EM-177, Repair of Pipe to Valve Body Weld on Downstream Side of Valve 392B, PCN 79-1376.

EM-178, Replacement of Alternate Charging Check Valve 383B, PCN 79-1375.

EM-181, Replacement of Manipulator Crane Ceramic 300 Watts 3 Ohm Resistor, PCN 79-1374.



FORC
ITEM #

PROCEDURE

EM-182, Liquid Penetrant Indication Removal on Primary Coolant Piping, PCN 79-1373.

EM-189, Auxiliary Building Crane - Repair of Misalignment Rail Ends, PCN 79-1372.

EM-191, Repair of Weld Downstream of Non-Regenerative Heat Exchanger, PCN 79-1371.

EM-192, Repair of Leak on RHR System Near Valve 718A, PCN 79-1370.

EM-193, Enlarge Two Holes in the Instrument Shield Protection at Valves 4274 and 4273, PCN 79-1369.

GS-22.0, Computer Guard Check, PCN 79-1395.

79-969

HP-9.4, Actions for an R-21 Alarm, PCN 79-1362.

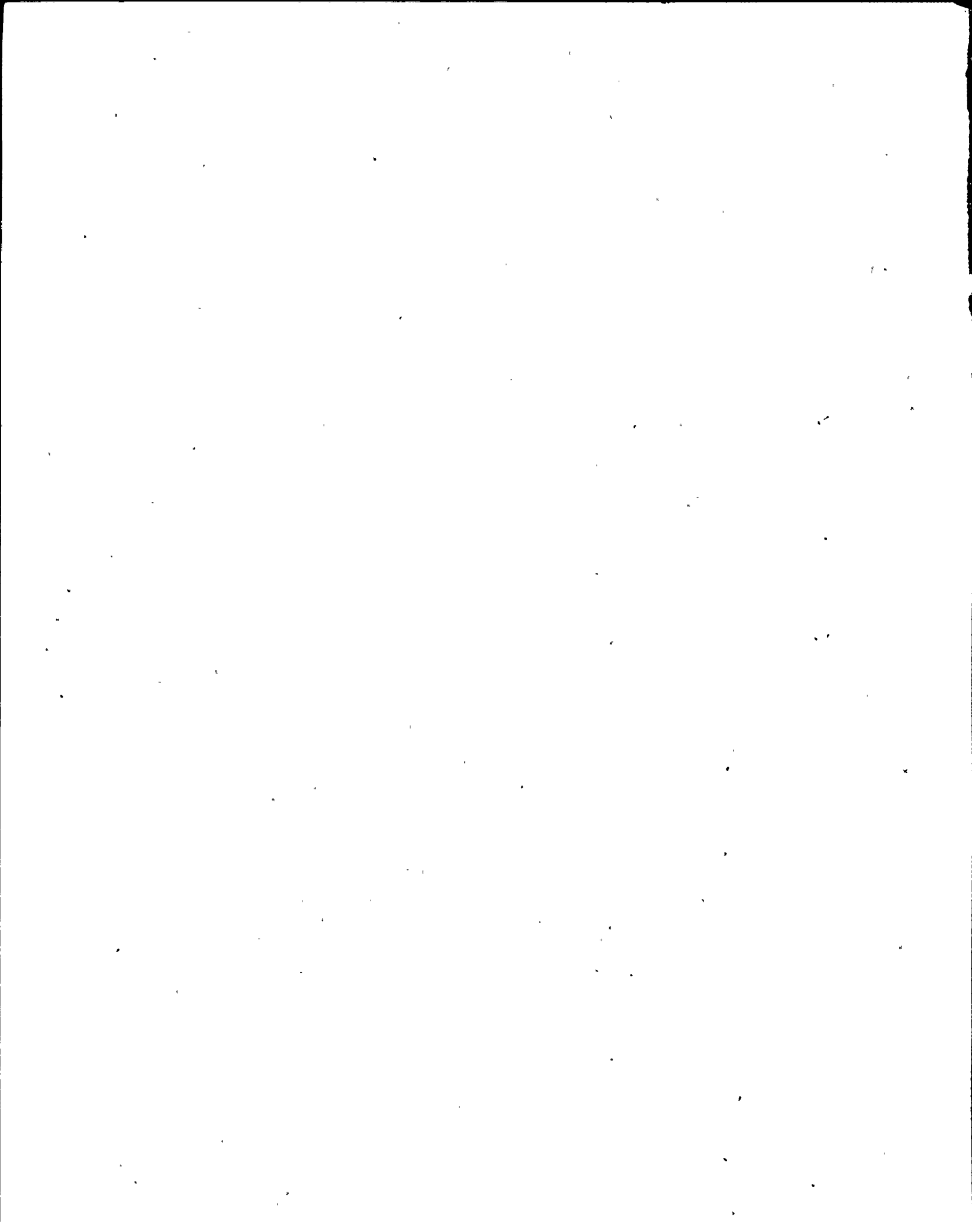
PT-13.1.13, Star Corporation Heat Detector System Test, PCN 79-1430.

PT-15.1, Reactor Coolant System Power Operated Relief Valve Timing, PCN 79-1429.

PT-32.1, Plant Safeguard Logic Test "A" or "B" Train, PCN 79-1428.

RSSP-2.5, Service Water Pump Discharge Check Valves, PCN 79-1427.

RSSP-10.1, Conditions for Main Steam Safety Valve Test, PCN 79-1426.



PORC
ITEM #

PROCEDURE

RSSP-13.5, Hydro on Class "C" Safety Related Piping (Spent Resin Storage Tanks and Resin Discharge Lines from CVCS DI's), PCN 79-1436.

RSSP-15.1, Hydro Class "B" Safety Related Piping (Safety Injection and Containment Spray Suction Piping, to Include NaOH Piping), PCN 79-1418.

RSSP-15.2, Hydro Class "B" Safety Related Piping (Reactor Coolant Letdown - 600 psig Pressure Rated Section), PCN 79-1417.

79-971 CP-429.0, Pressurizer Pressure Channel "429", PCN 79-1410.

CP-429.7, Pressurizer Pressure Bistable PC-429B, PCN 79-1404.

CP-449.0, Pressurizer Pressure Channel "449", PCN 79-1411.

CP-449.7, Pressurizer Pressure Bistable PC-449B, PCN 79-1408.

79-972 *ISI-8, Inservice Inspection of Feedwater Nozzle Welds, PCN 79-1433.

79-973 *M-37.18K, Repair of "A" SI Pump Discharge Check Valve 889A, PCN 79-1401.

79-974 *PT-2.1, Safety Injection System Pumps, PCN 79-1414.

79-975 *PT-2.3.1, Post Accident Charcoal Filter Damage, PCN 79-1366.

79-976 PT-2.4, Cold/Refueling Motor Operated Valve Surveillance, PCN 79-1415.

PORC
ITEM #

PROCEDURE

PT-2.4.1, Cold/Refueling Motor Operated Valve Surveillance (RHR System - 700 Valves), PCN 79-1416.

PT-2.7, Service Water System, PCN 79-1434.

PT-3, Containment Spray Pumps and NaOH Additive System, PCN 79-1424.

79-977

PT-5.10, Process Instrumentation Reactor Protection Channel Trip Test (Channel 1), PCN 79-1457.

PT-5.20, Process Instrumentation Reactor Protection Channel Trip Test (Channel 2), PCN 79-1458.

PT-5.30, Process Instrumentation Reactor Protection Channel Trip Test (Channel 3), PCN 79-1456.

PT-5.40, Process Instrumentation Reactor Protection Channel Trip Test (Channel 4), PCN 79-1455.

79-978

*PT-13, Fire Pump Operation and System Alignment, PCN 79-1365.

79-980

*PT-17.2, Process Radiation Monitors R-11 to R-21 and Iodine Monitors R-10A and R-10B, PCN 79-1435.

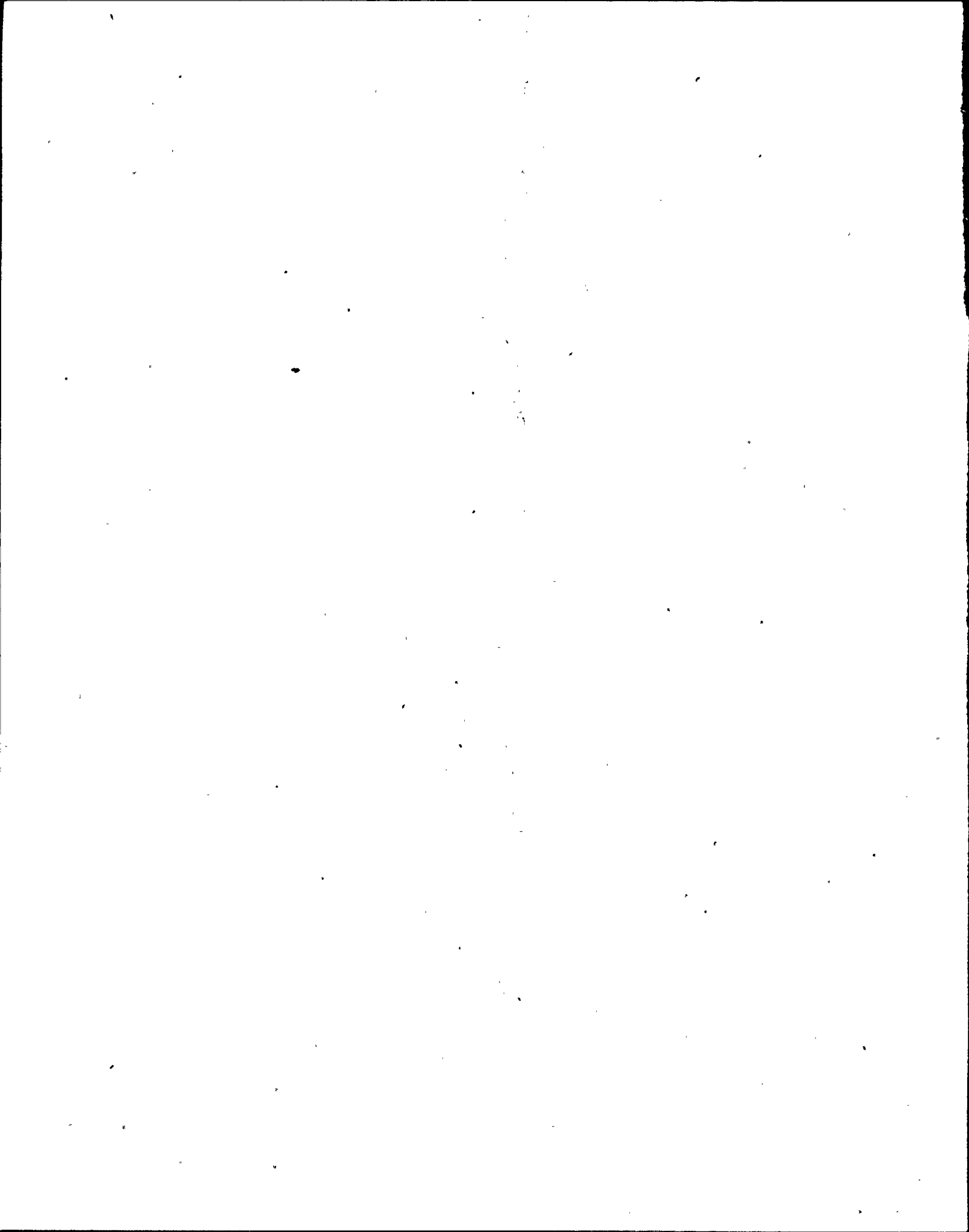
79-981

*PT-20, Main Steam Isolation Valve Solenoids Trip Test, PCN 79-1425.

79-982

PT-23.1, Containment Isolation Valve Leak Rate Testing Pressurizer Relief Tank Gas Analyzer Line, PCN 79-1400.

PT-23.2, Containment Isolation Valve Leak Rate Testing Nitrogen Supply to Pressurizer Relief Tank, PCN 79-1399.



FORC
ITEM #

PROCEDURE

PT-23.3, Containment Isolation Valve Leak Rate Testing Make-up Water to Pressurizer Relief Tank, PCN 79-1398.

PT-23.7, Containment Isolation Valve Leak Rate Testing Letdown from Reactor Coolant System, PCN 79-1397.

PT-23.8, Containment Isolation Valve Leak Rate Testing RCS Charging Line, PCN 79-1396.

PT-23.10, Containment Isolation Valve Leak Rate Testing Alternate Charging Line, PCN 79-1423.

PT-23.11, Containment Isolation Valve Leak Rate Testing RCP Seal Water Return and Ecess Letdown, PCN 79-1422.

PT-23.12A, Containment Isolation Valve Leak Rate Testing Steam Space Sample, PCN 79-1421.

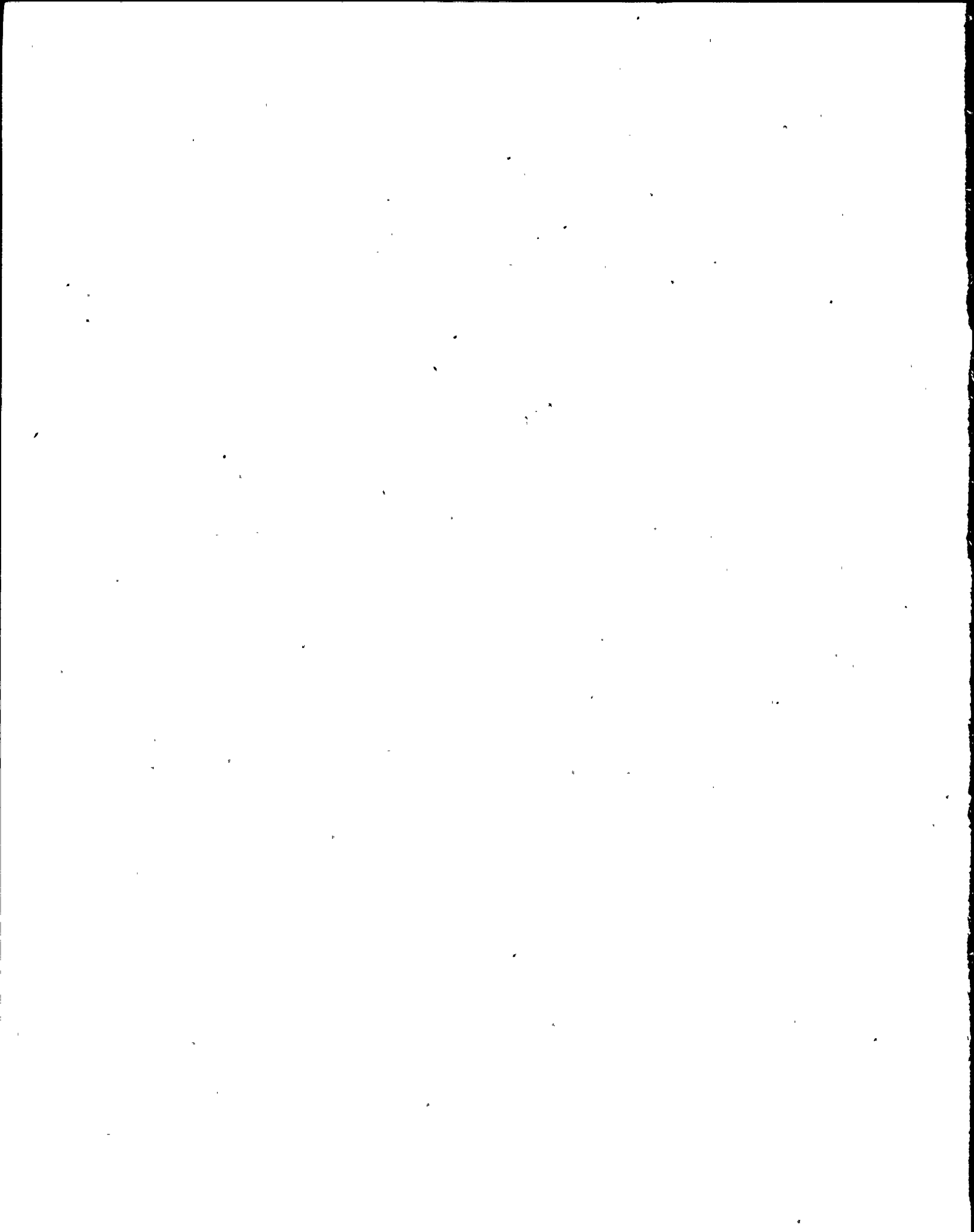
PT-23.12B, Containment Isolation Valve Leak Rate Testing Pressurizer Liquid Space Sample, PCN 79-1420.

PT-23.12C, Containment Isolation Valve Leak Rate Testing RCS Sample Loop "B", PCN 79-1419.

79-992 *Proposed Technical Specification Change - Over-pressurization Protection.

79-994 *RSSP-15.12, ISI Hydro Test of Class "B" Piping (A and B Steam Generator Feedwater Piping), PCN 79-1512.

79-996 *EM-170, Containment Spray Additive Valve Controller Repair (836A or 836B), PCN 79-1516.



FORC
ITEM #

PROCEDURE

79-997 *EM-235, Furmaniting MOV-700 RHR From "A" Loop, PCN 79-1516.

79-997A *EM-236, Furmaniting Valve 296 Auxiliary Spray to Pressurizer, PCN 79-1517.

79-997B *EM-237, Weld Repair Connection at "A" Feedwater Line from MDAFWP, PCN 79-1514.

79-997C *EM-238, Weld Repair Connection at "A" Feedwater Line from TDAFWP, PCN 79-1513.

79-998 *SM 77-1660.50, RCS Overpressurization Protection System Annunciation PCN 79-1459.

79-1001 *A-52.2, Control of Locked Valve Operation, PCN 79-1447.

79-1002 *A-52.4, Control of Limiting Conditions for Operating Equipment, PCN 79-1487.

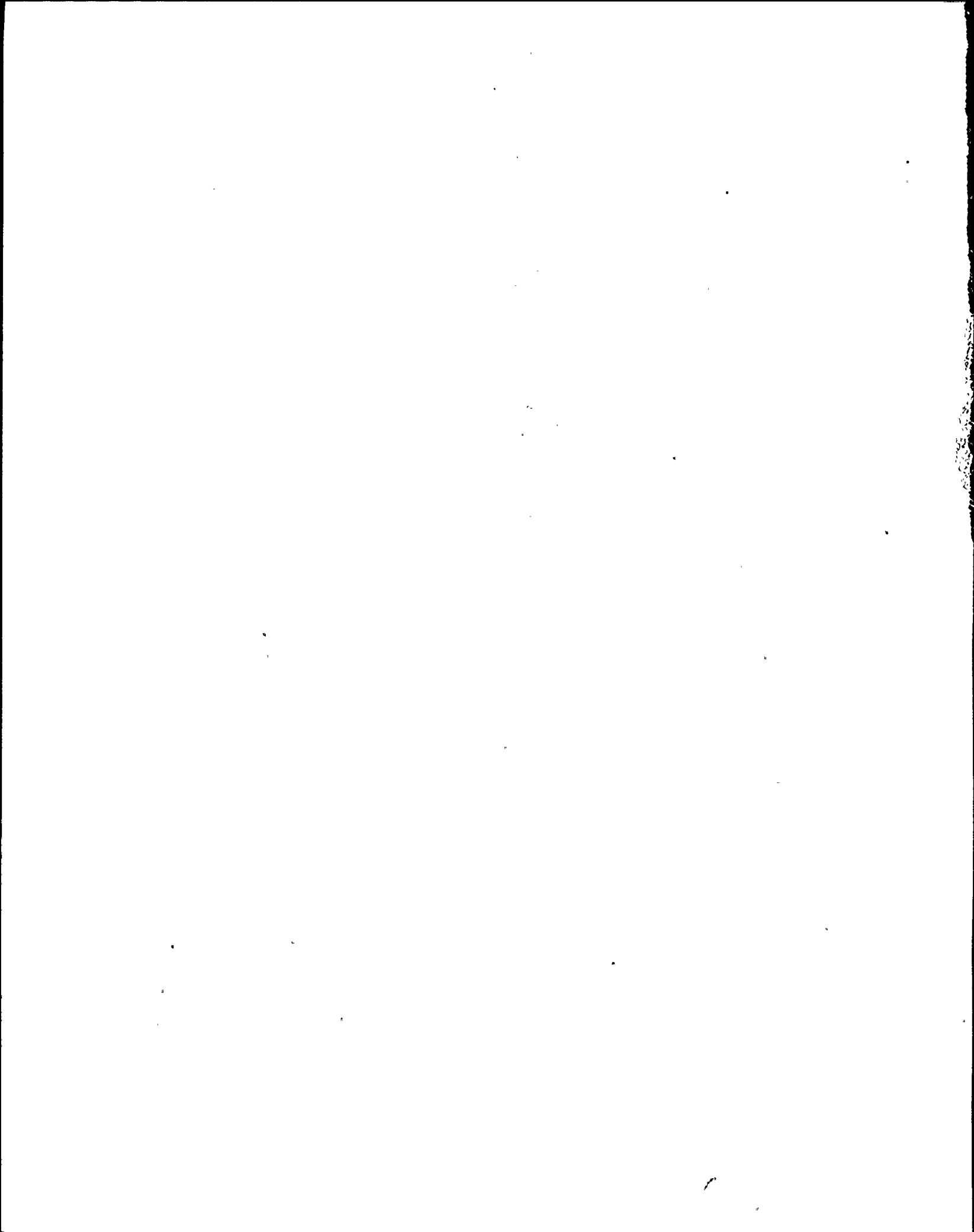
79-1003 *A-1105, Calibration Surveillance Program for Instrumentation/Equipment of Safety Related Components, PCN 79-1443.

79-1004 CP-11.1, Pressurizer Pressure Controller PC-431K, PCN 79-1478.

CP-426.0, Calibration and/or Maintenance "426" Pressurizer Level, PCN's 79-1464 and 79-1479.

CP-426.1, General Defeat Procedure "426" Pressurizer Level, PCN 79-1465.

CP-426.2, General Reinstate "426" Pressurizer Level, PCN 79-1466.



PORC
ITEM #

PROCEDURE

CP-426.5, Pressurizer Level Bistable LC-426 A/B, PCN 79-1467.

CP-427.0, Pressurizer Level Channel "427", PCN 79-1471.

CP-427.1, Defeat of "427" Pressurizer Level, PCN 79-1472.

CP-427.2, Reinstate "427" Pressurizer Level, PCN 79-1473.

CP-427.5, Pressurizer Level Bistable LC-427 A/C, PCN 79-1470.

CP-428.0, Pressurizer Level Channel "428", PCN 79-1469.

CP-428.1, Defeat of "428" Pressurizer Level, PCN 79-1468.

CP-428.2, Reinstate "428" Pressurizer Level, PCN 79-1474.

CP-428.5, Pressurizer Level Bistable LC-428 A/E, PCN 79-1475.

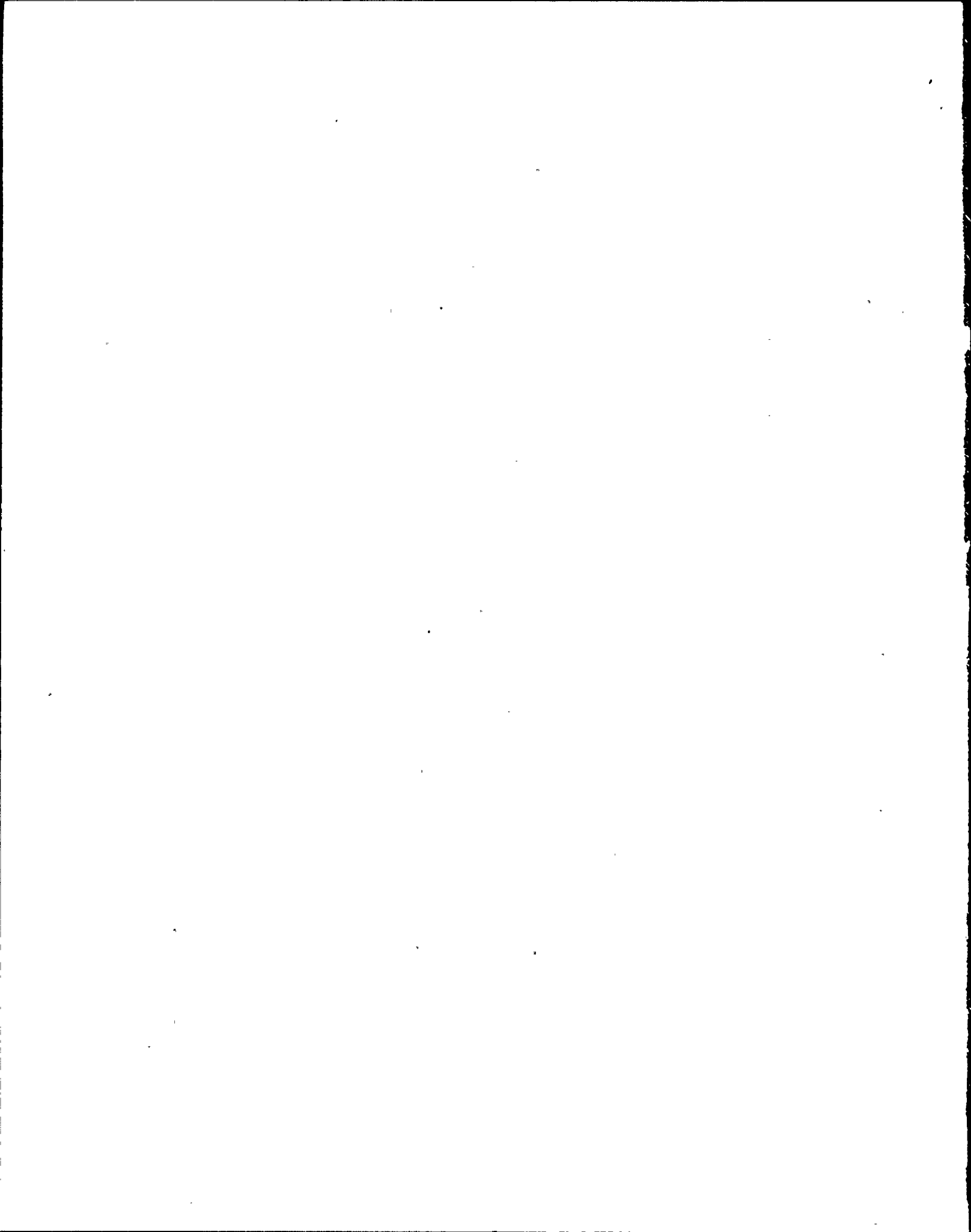
CP-429.1, General Defeat Procedure Pressurizer Pressure Channel "429", PCN 79-1476.

CP-449.1, General Defeat Procedure for Pressurizer Pressure Channel P-449, PCN 79-1477.

79-1005 *EM-175, Replacement of Main Steam Vent Valve 3503, PCN 79-1515.

79-1006 *M-37.11, Pressurizer Power Operated Relief Valves 430 and 431C Repair and Inspection, PCN 79-1496.

79-1007 *M-52.3, Incore Thimble Cleaning, PCN 79-1368.



FORC
ITEM #

PROCEDURE

79-1008

*O-2.2, Plant Shutdown from Hot Shutdown to Cold Shutdown, PCN's 79-1444 and 79-1486.

79-1009

PT-23.13A, Containment Isolation Valve Leak Rate Testing "A" Steam Generator Sample, PCN 79-1450.

PT-23.13B, Containment Isolation Valve Leak Rate Testing "B" Steam Generator Sample, PCN 79-1449.

PT-23.14, Containment Isolation Valve Leak Rate Testing Air Sample Inlet, PCN 79-1480.

PT-23.15, Containment Isolation Valve Leak Rate Testing Containment Air Sample Outlet, PCN 79-1451.

PT-23.16A, Containment Isolation Valve Leak Rate Testing "A" Steam Generator Blowdown, PCN 79-1452.

PT-23.16B, Containment Isolation Valve Leak Rate Testing "B" Steam Generator Blowdown, PCN 79-1453.

PT-23.17A, Containment Isolation Valve Leak Rate Testing Containment Pressure Sensing Transmitter PT-945 and PT-946, PCN 79-1481.

PT-23.17B, Containment Isolation Valve Leak Rate Testing Containment Pressure Sensing Transmitter PT-947 and PT-948, PCN 79-1482.

PT-23.17C, Containment Isolation Valve Leak Rate Testing Containment Pressure Sensing Transmitter PT-949 and PT-950, PCN 79-1483.

PT-23.18A, Containment Isolation Valve Leak Rate Testing "A" Containment Spray Header, PCN 79-1484.

FORC
ITEM #

PROCEDURE

PT-23.18B, Containment Isolation Valve Leak Rate Testing "B" Con-
tainment Spray Header, PCN 79-1485.

PT-23.19, Containment Isolation Valve Leak Rate Testing Safety
Injection System, PCN 79-1491.

PT-23.20, Containment Isolation Valve Leak Rate Testing R.C.D.T.
Gas Header, PCN 79-1492.

PT-23.21, Containment Isolation Valve Leak Rate Testing R.C.D.T.
to Gas Analyzer, PCN 79-1493.

PT-23.22, Containment Isolation Valve Leak Rate Testing R.C.D.T.
Discharge, PCN 79-1494.

PT-23.23, Containment Isolation Valve Leak Rate Testing "Sump "A"
Discharge, PCN 79-1495.

- 79-1010 *PT-32.1, Plant Safeguard Logic Test A or B Train, PCN 79-1489.
- 79-1011 *RSSP-2.2, Diesel Generator Load and Safeguard Sequence Test, PCN
79-1454.
- 79-1019 *EM-239, CV Spray Ring Header Pipe Clamp Replacement, PCN 79-1540.
- 79-1020 *EM-240, Rework of Pipe Supports, Pipe Support, ECN-2512, PCN
79-1542.
- 79-1022 *EM-241, Isolate and Drain the Standby Auxiliary Feedwater
Lines for Rework of Pipe Supports AFW-87 and AFW-116,
PCN 79-1572.

PORC
ITEM #

PROCEDURE

79-1023

*EM-242, Isolate and Drain Service Water Line for Rework of Pipe Supports SW-116, SW-117, and SW-133, PCN 79-1573.

79-1024

*EM-243, Filling, Venting and Hydrostatic Testing of Portions of the SAFW and Portion of the Service Water System, PCN 79-1591.

79-1026

*T-36.1, Station Service Water - Header Valve Alignment for Two Loop Operation, PCN 79-1544.

79-1029

*SM 79-1662.1, Installation of Snubber Platforms, PCN 79-1510.

79-1031

*EM-200, Replacement of Rework of Solenoid Valves for CV Purge Exhaust Valve 5879, PCN 79-1546.

79-1033

AR/G-28, Steam Generator "A" Lo Lo Level Single Channel Alert, PCN 79-1575.

AR/G-30, Steam Generator "A" Lo Lo Level Single Channel Alert, PCN 79-1574.

79-1034

CP-10.1, Defeat of Delta Flux Signal Channel 1, PCN 79-1518.

CP-10.2, Reinstate of Delta Flux Signal Channel 1, PCN 79-1522.

CP-20.1, Defeat of Delta Flux Signal Channel 2, PCN 79-1519.

CP-20.2, Reinstate of Delta Flux Signal Channel 2, PCN 79-1524.

CP-30.1, Defeat of Delta Flux Signal Channel 3, PCN 79-1520.

CP-30.2, Reinstate of Delta Flux Signal Channel 3, PCN 79-1525.

FORC
ITEM #

PROCEDURE

- CP-40.1, Defeat of Delta Flux Signal Channel 4, PCN 79-1521.
- CP-40.2, Reinstate of Delta Flux Signal Channel 4, PCN 79-1523.
- CP-41.2, General Reinstate Procedure for N-41, PCN 79-1526.
- CP-42.2, Reinstate Power Range Channel N-42, PCN 79-1529.
- CP-43.2, General Reinstate Procedure for N-43, PCN 79-1528.
- CP-44.2, General Reinstate Procedure for N-44, PCN 79-1527.
- 79-1035 *EM-236, Furmaniting Valve 296 - Auxiliary Spray to Pressurizer, PCN 79-1555.
- 79-1036 *ISI-9, Inservice Inspection of Wedge Type Anchor Bolts, PCN 79-1538.
- 79-1037 *M-37.11, Pressurizer Power Operated Relief Valves 430 and 431C Repair and Inspection, PCN 79-1593.
- 79-1038 *M-43.26, Installation of Steam Generator Secondary Manway Cover S/G, PCN 79-1545.
- 79-1039 *O-7, Alignment and Operation of the Reactor Vessel Overpressure Protection System, PCN 79-1589.
- 79-1040 *PT-2.8, Component Cooling Water Pump System, PCN 79-1511.
- 79-1041 *PT-3, Containment Spray Pumps and NaOH Additive System, PCN 79-1554.

PORC
ITEM #

PROCEDURE

79-1042 *PT-13.4.11, Multimatic Deluge Valve System Testing System #11 Cable Tunnel, PCN 79-1532.

79-1043 *PT-16.1, Auxiliary Feedwater System Flow Balance, PCN 79-1542.

79-1044 PT-23.24, Containment Isolation Valve Leak Rate Testing Reactor Support Coolers (Inlet and Outlet), PCN 79-1506.

PT-23.26, Containment Isolation Valve Leak Rate Testing Auxiliary Coolant System to "A" Reactor Coolant Pump, PCN 79-1505.

PT-23.27, Containment Isolation Valve Leak Rate Testing Auxiliary Coolant System to "B" Reactor Coolant Pump, PCN 79-1504.

PT-23.28, Containment Isolation Valve Leak Rate Testing Auxiliary Coolant System from "A" Reactor Coolant Pump, PCN 79-1503.

PT-23.29, Containment Isolation Valve Leak Rate Testing Auxiliary Coolant System from "B" Reactor Coolant Pump, PCN 79-1502.

PT-23.30, Containment Isolation Valve Leak Rate Testing Auxiliary Coolant System Excess Letdown (Supply and Return), PCN 79-1507.

PT-23.32, Containment Isolation Valve Leak Rate Testing Instrument Air, PCN 79-1501.

PT-23.33, Containment Isolation Valve Leak Rate Testing Service Air, PCN 79-1500.

PT-23.34, Containment Isolation Valve Leak Rate Testing Depressurization at Power, PCN 79-1499.

FORC
ITEM #

PROCEDURE

79-1058

A-1.6.1, Documentation of "As Low As Reasonably Achievable" (ALARA) Program, PCN 79-1233.

M-37.61, Valve HCV-105 Isolation, Inspection and Maintenance, PCN 79-1587.

79-1062

*A-52.4, Control of Limiting Conditions for Operating Equipment, PCN 79-1605.

79-1063

*A-701, Receipt and Acceptance of Material/Parts, PCN 79-1632.

79-1064

*A-1001, Inspection and Surveillance Activities, PCN 79-1611.

79-1065

AR/AA-6, RCS Overpressure Arm/Inhibit Loop A Select, PCN 79-1625.

AR/AA-7, RCS Overpressure Arm/Inhibit Loop B Select, PCN 79-1626.

79-1066

*CP-431.0, Calibration and/or Maintenance of Pressurizer Pressure Channel 431, PCN 79-1640.

79-1067

*CP-431.2, General Reinstate Procedure Pressurizer Pressure Channel 431, PCN 79-1642.

79-1068

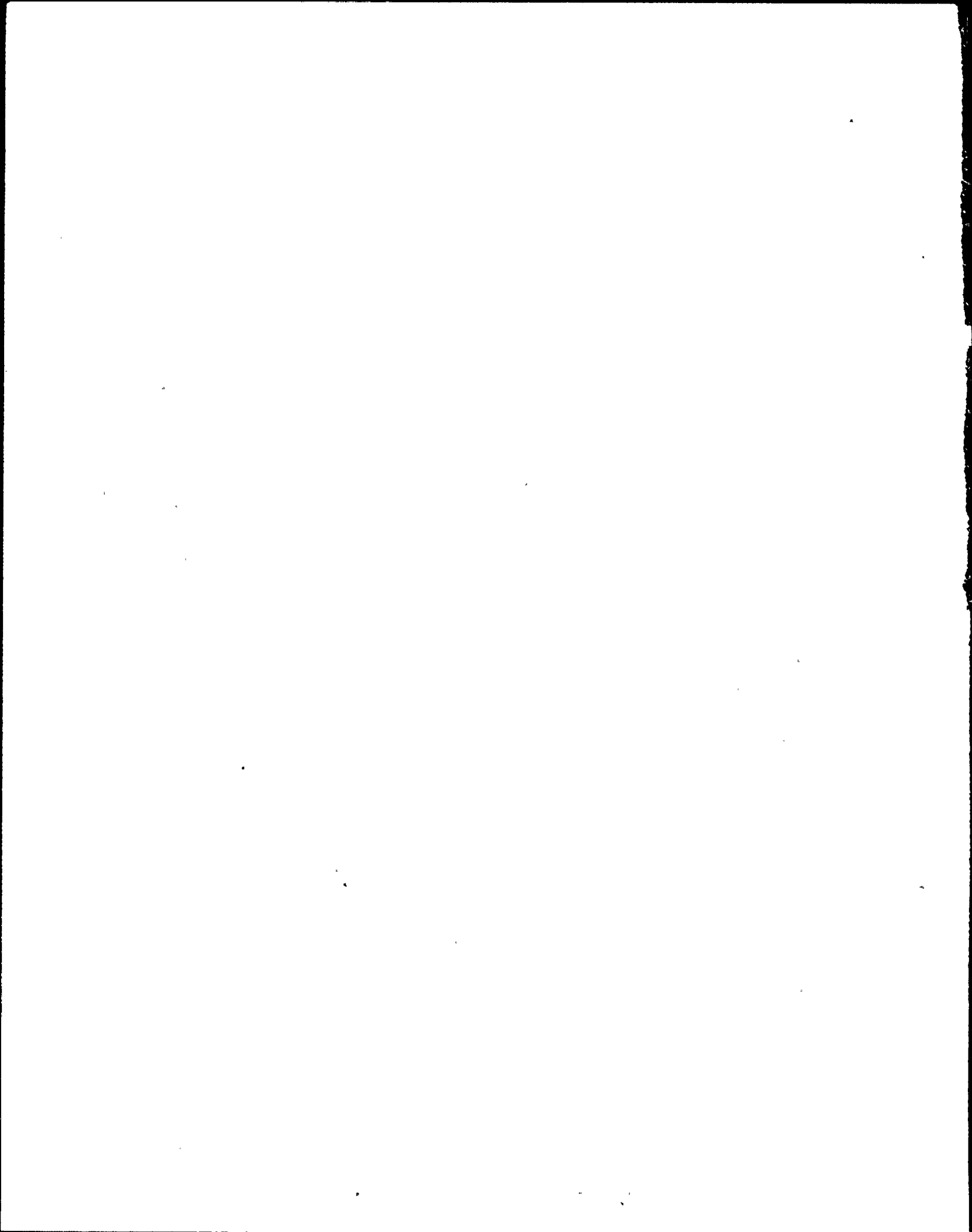
*CP-449.1, General Defeat Procedure for Pressurizer Pressure Channel 449, PCN 79-1643.

79-1069

*CP-461.0, Calibration and/or Maintenance of Steam Generator Level Channel 461, PCN 79-1577.

79-1070

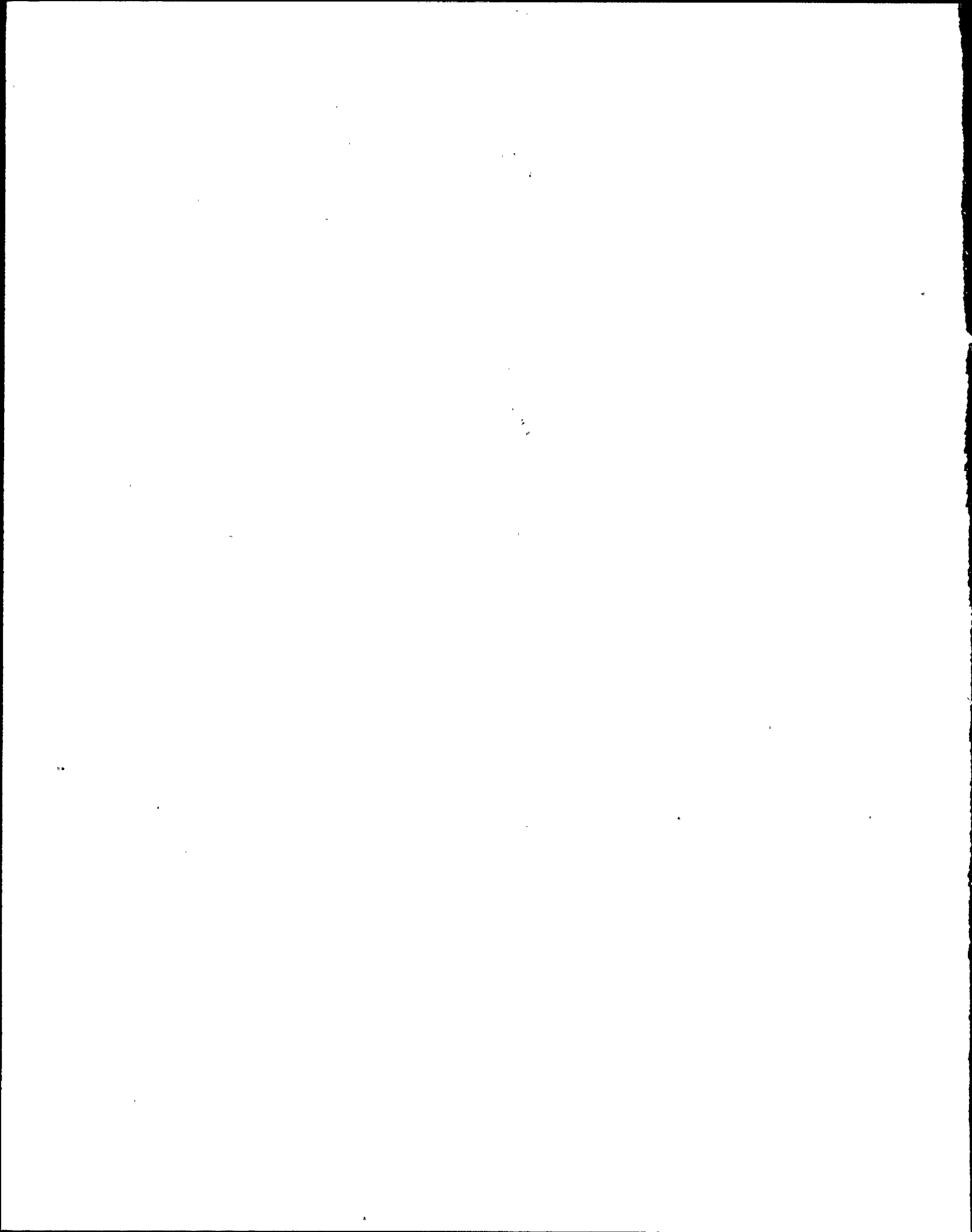
*CP-462.0, Calibration and/or Maintenance of Steam Generator Level Channel 462, PCN 79-1578.



PORC
ITEM #

PROCEDURE

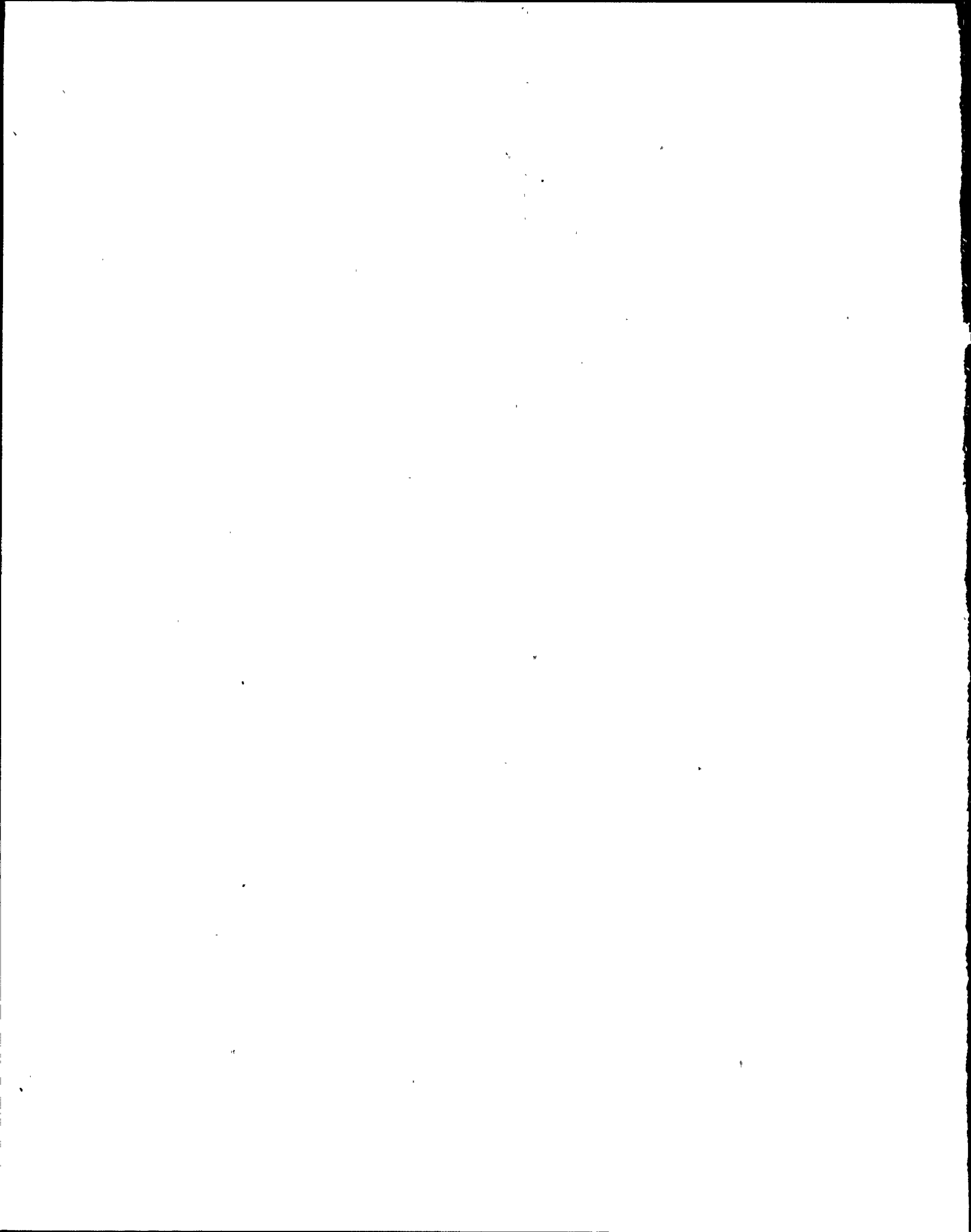
<u>79-1071</u>	<u>CP-463.0, Calibration and/or Maintenance of Steam Generator Level Channel 463, PCN 79-1580.</u>
	<u>CP-471.0, Calibration and/or Maintenance of Steam Generator Level Channel 471, PCN 79-1581.</u>
	<u>CP-472.0, Calibration and/or Maintenance of Steam Generator Level Channel 472, PCN 79-1582.</u>
<u>79-1072</u>	<u>*CP-473.0, Calibration and/or Maintenance of Steam Generator Level Channel 473, PCN 79-1579.</u>
<u>79-1072</u>	<u>*CP-473.0, Calibration and/or Maintenance of Steam Generator Level Channel 473, PCN 79-1579.</u>
<u>79-1073</u>	<u>CP-626.1, Defeat of Residual Heat Removal Flow Channel 626, PCN 79-1634.</u>
	<u>CP-626.2, Reinstate of Residual Heat Removal Flow Channel 626, PCN 79-1635.</u>
<u>79-1074</u>	<u>*EM-173, Temporary Repair of Main Steam Vent Valve 3503 Using "Furmanite" Process, PCN 79-1623.</u>
<u>79-1075</u>	<u>*EM-232, Repair of Feedwater Piping Inside Containment, PCN 79-1590.</u>
<u>79-1076</u>	<u>*EM-246, Furmanite Repair - Pressurizer Spray Valves 431A and B, PCN 79-1668.</u>
<u>79-1077</u>	<u>*IP-2.2, Whole Body Count Operations, PCN 79-1617.</u>
<u>79-1078</u>	<u>*ISI-8, Inservice Inspection Feedwater Nozzle Welds, PCN 79-1602.</u>



FORC
ITEM #

PROCEDURE

- 79-1079 *M-3.2, Reactor Cavity Cleanup and Decontamination, PCN 79-1612.
- 79-1081 *O-1.1, Plant Heatup From Cold Shutdown to Hot Shutdown, PCN's 79-1613, 79-1615, and 79-1622.
- 79-1082 *O-1.1D, Pre-Heatup Plant Requirement Check List, PCN 79-1614.
- 79-1083 *O-1.2, Plant From Hot Shutdown to Steady Load, PCN 79-1609.
- 79-1084 *P-1, Reactor Control and Protection System, PCN 79-1576.
- 79-1085 *PT-2.2, Residual Heat Removal System, PCN 79-1636.
- 79-1086 *PT-2.9, Check Valve Exercising Quarterly Requirement, PCN 79-1627.
- 79-1087 PT-5.10, Process Instrumentation Reactor Protection Channel Trip Test (Channel 1), PCN 79-1583.
- PT-5.20, Process Instrumentation Reactor Protection Channel Trip Test (Channel 2), PCN 79-1584.
- PT-5.30, Process Instrumentation Reactor Protection Channel Trip Test (Channel 3), PCN 79-1585.
- PT-5.40, Process Instrumentation Reactor Protection Channel Trip Test (Channel 4), PCN 79-1586.
- 79-1088 *PT-7, Hydro Test of Reactor Coolant System, PCN 79-1628.
- 79-1089 PT-23.48, Containment Isolation Valve Leak Rate Testing Deadweight Tester, PCN 79-1621.



FORC
ITEM #

PROCEDURE

PT-23.49, Containment Isolation Valve Leak Rate Testing Construction Fire Service Water, PCN 79-1596.

PT-23.50A, Containment Isolation Valve Leak Rate Testing Containment Post Accident Air Sample (Clean Intermediate Building), PCN 79-1597.

PT-23.50B, Containment Isolation Valve Leak Rate Testing Containment Post Accident Air Sample (Containment Intermediate Building), PCN 79-1598.

PT-23.50C, Containment Isolation Valve Leak Rate Testing Containment Post Accident Air Sample (Auxiliary Building), PCN 79-1599.

PT-23.51A, Containment Isolation Valve Leak Rate Testing "A" Hydrogen Recombiner (Pilot and Main), PCN 79-1620.

PT-23.51B, Containment Isolation Valve Leak Rate Testing "B" Hydrogen Recombiner (Pilot and Main), PCN 79-1619.

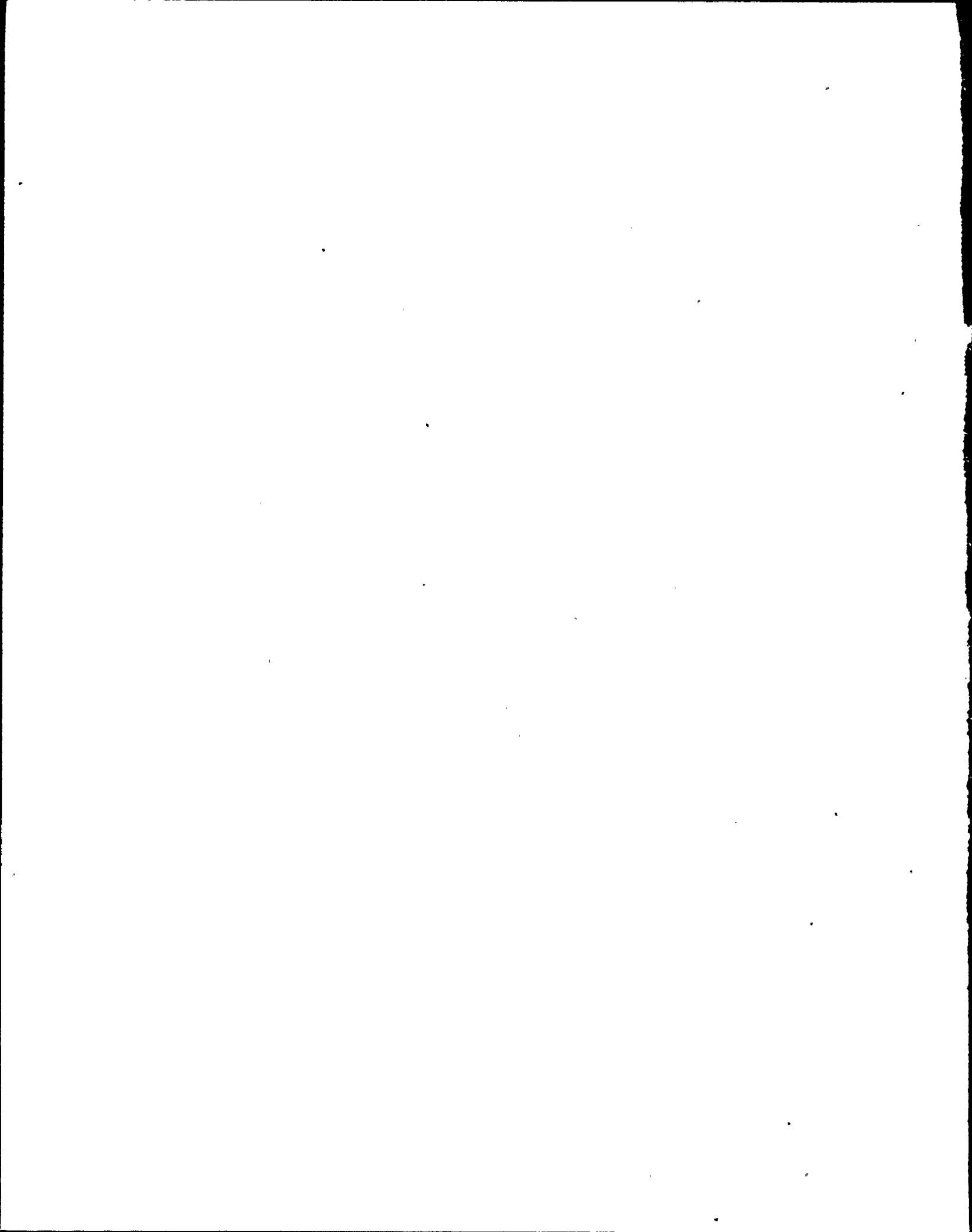
PT-23.51C, Containment Isolation Valve Leak Rate Testing "A" and "B" Hydrogen Recombiners Oxygen Makeup, PCN 79-1618.

79-1090 *RSSP-15.12, Hydro Test of Class "B" Piping ("A" and "B" Steam Generator Feedwater Piping), PCN's 79-1588, 79-1594, and 79-1595.

79-1091 *S-13A, RHR System Lineup for Safety Injection, PCN 79-1631.

79-1092 *SC-3.3.2, Offsite Notification of Fire, PCN 79-1601.

79-1093 *SM 77-1660.50, RCS Overpressurization Protection System Annunciation PCN 79-1624.

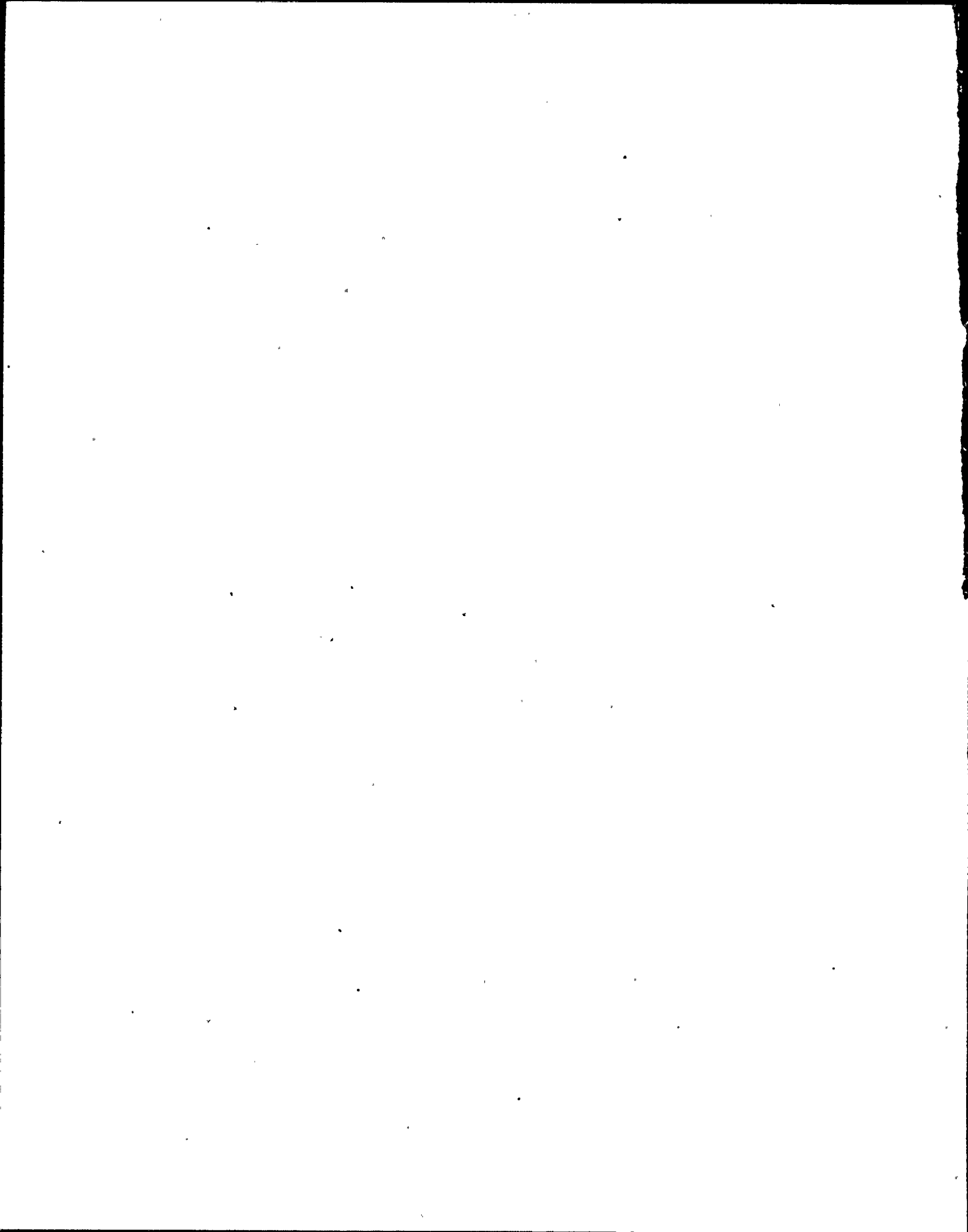


FORC
ITEM #

PROCEDURE

- 79-1097 *T-41A, Alignment of Auxiliary Feedwater System Prior to Power Operations, PCN's 79-1607 and 79-1608.
- 79-1098 *T-44.2, Standby Auxiliary Feedwater (SAFW) System Alignment for Normal Operation, PCN's 79-1592 and 79-1629.
- 79-1107 *EM-247, Rework of Pipe Supports for System, PCN 79-1684.
- 79-1108 *PT-16.2, Turbine Driven Auxiliary Feed Pump Lube Oil Temperature, PCN 79-1699.
- 79-1110 *CP-431.12, Pressurizer Pressure Bistable PC-431F, PCN 79-1645.
- *RD-13.1, Effluent Monitor Setpoints, PCN 79-1569.
- *S-27.2238, Valve 2238 Isolation, PCN 79-1558.
- 79-1112 *A-20, Control Room Logs, PCN 79-1692.
- 79-1113 *A-602, Plant Procedure Distribution, PCN 79-1323.
- 79-1114 *A-1702, Record Storage Facilities and Equipment, PCN 79-1698.
- 79-1115 *AR/C-27, Pressurizer Lo Pressure, Safety Injection Single Channel Alert, PCN 79-1651.
- 79-1116 AR/C-28, Pressurizr Lo Pressure, Safety Injection Single Channel Alert, PCN 79-1652.
- AR/D-19, Pressurizer Lo Pressure, PCN 79-1653.
- 79-1117 *AR/F-10, Pressurizer Lo Pressure, PCN 79-1650.

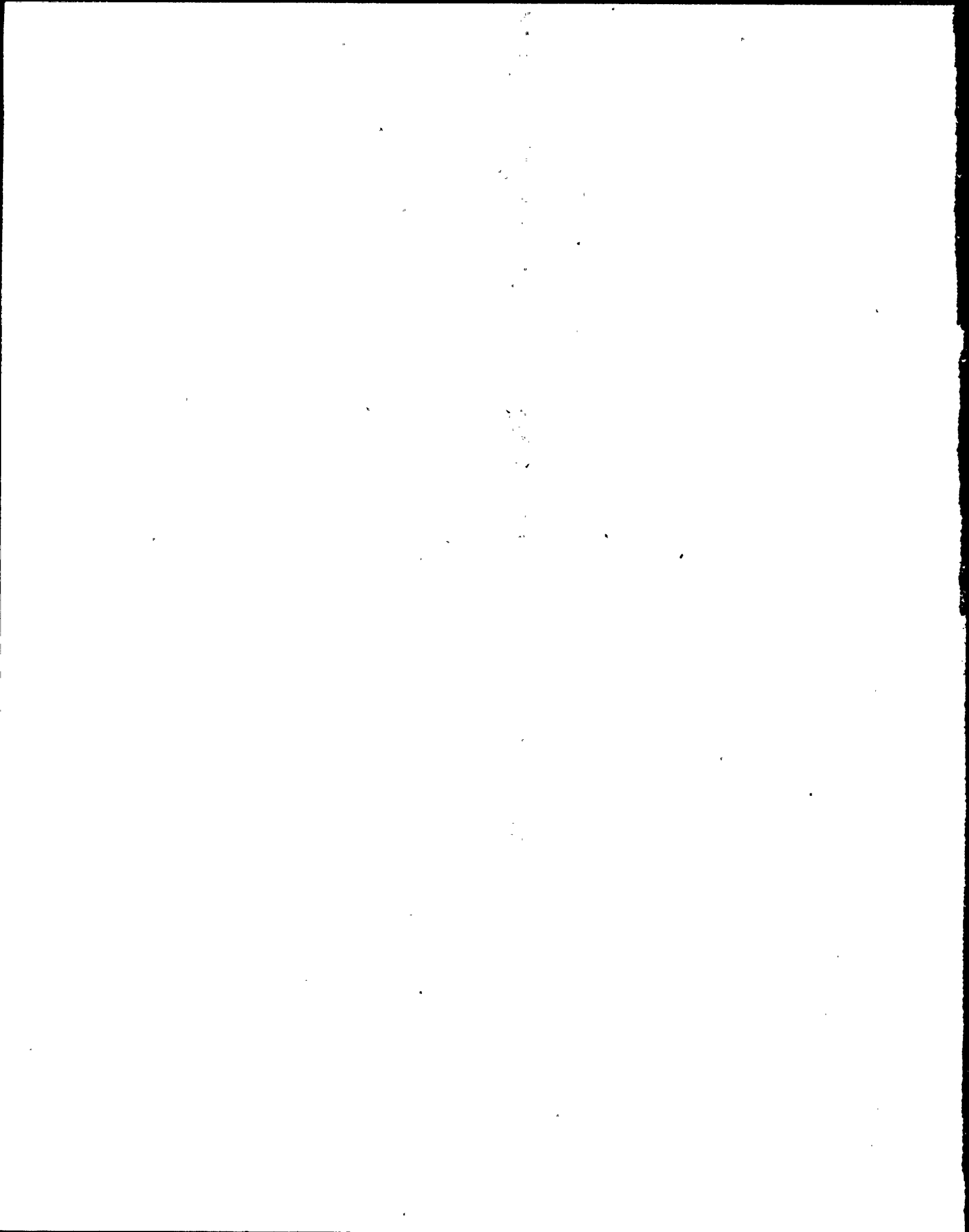
<u>PORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-1118</u>	<u>*E-15.1, Malfunction of Pressurizer Power Relief or Safety Valves, PCN 79-1703.</u>
<u>79-1119</u>	<u>M-43.20A, "A" Steam Generator Secondary Side Pressure Test, PCN 79-1696.</u>
	<u>M-43.20B, "B" Steam Generator Secondary Side Pressure Test, PCN 79-1697.</u>
<u>79-1120</u>	<u>*M-76.1, Installation and Test Sampling of Grouting, PCN 79-1682.</u>
<u>79-1121</u>	<u>*O-1.1C, Precritical Technical Specification Check Sheet, PCN 79-1678.</u>
<u>79-1122</u>	<u>*O-1.2, Plant From Hot Shutdown to Steady Load, PCN 79-1681.</u>
<u>79-1123</u>	<u>*O-7, Alignment and Operation of the Reactor Vessel Overpressure Protection System, PCN 79-1691.</u>
<u>79-1124</u>	<u>*P-1, Reactor Control and Protection System, PCN 79-1701.</u>
<u>79-1125</u>	<u>*PT-5.10, Process Instrumentation Reactor Protection Channel Trip Test (Channel 1), PCN 79-1654.</u>
<u>79-1126</u>	<u>*RD-8, Liquid Radwaste Compositing and Analysis, PCN 79-1694.</u>
<u>79-1127</u>	<u>*RSSP-1.4, Valve Interlock Verification - Reactor Coolant System, PCN 79-1700.</u>
<u>79-1128</u>	<u>*RSSP-15.12, Hydro Test of Class "B" Piping (A and B Steam Generator Feedwater Piping), PCN 79-1695.</u>



PORC
ITEM #

PROCEDURE

- 79-1129 S-13A, RHR System Lineup for Safety Injection, PCN 79-1686.
- S-16A, Safety Injection System Alignment, PCN 79-1690.
- S-30.1, Safety Injection System Valve Position Verification, PCN 79-1679.
- S-30.2, RHR System Valve Position Verification, PCN 79-1680.
- S-30.3, Containment Spray System Valve Position Verification, PCN 79-1687.
- S-30.4, Auxiliary Feedwater System Valve Position Verification, PCN 79-1688.
- S-30.5, Standby Auxiliary Feedwater Pump Valve Position Verification PCN 79-1689.
- 79-1130 *SC-3.3.2, Offsite Notification of Fire, PCN 79-1655.
- 79-1140 *EM-248, Pressurizer Pressure Controller (PC-431K), PCN 79-1719.
- 79-1141 *SM 79-1837.1, Computer Room Ceiling, PCN 79-1712.
- 79-1144 *S-30.4, Auxiliary Feedwater System Valve Position Verification, PCN 79-1766.
- 79-1145 *SM 78-1875.2, "A" Feedwater Venturi Monorail Installation of Hoist, PCN 79-1711.
- 79-1146 *CP-2.4.0, Rod Position Indication System "Part Length" Bank Hot Shutdown Alignment, PCN 79-1707.



FORC
ITEM #

PROCEDURE

79-1147 *CP-2.4.1, Rod Position Indication System "Part Length" Bank at Power Alignment, PCN 79-1735.

79-1148 SM 75-5.38, Standby Auxiliary Feedwater System Operational Testing, PCN 79-1730.

SM 75-5.39, Standby Auxiliary Feedwater System Rework, PCN 79-1729.

SM 75-5.41, Standby Auxiliary Feedwater Cable and Conduit Installation Rework, PCN 79-1728.

SM 75-5.42, Electrical Installation and Cable Termination Rework for Air Handling Units, PCN 79-1727.

SM 75-5.43, Rework of Standby Auxiliary Feedwater System, PCN 79-1726.

SM 75-5.44, Rework of SAFWP Seal Piping, PCN 79-1725.

SM 75-10.28, Service Water Piping to Air Handling Units for Auxiliary Building Addition Rework, PCN 79-1724.

SM 75-34.23, Pipe Restraints and Jet Shields Rework for Steam Generator Blowdown Lines, PCN 79-1723.

SM 77-1660.50, RCS Overpressurization Protection System Annunciation PCN 79-1722.

79-1152 *A-52.1, Shift Organization Relief and Turnover, PCN 79-1713.

79-1153 *A-102.14, Ginna Operator Requalification Program, PCN 79-1757.

79-1154 *A-1401, Ginna Station Holding Rules, PCN 79-1235.

PORC
ITEM #

PROCEDURE

79-1156

CP-2.3, Rod Position Indication System Shutdown Bank Hot Shutdown Alignment, PCN-1706.

CP-2.3.1, Rod Position Indication System Shutdown Bank at Power Alignment, PCN 79-1705.

CP-2.5.0, Rod Position Indication System A Bank Hot Shutdown Alignment, PCN 79-1736.

CP-2.5.1, Rod Position Indication System A Bank at Power Alignment, PCN 79-1737.

CP-2.6.0, Rod Position Indication System B Bank Hot Shutdown Alignment, PCN 79-1738.

CP-2.6.1, Rod Position Indication System B Bank at Power Alignment, PCN 79-1739.

CP-2.7.0, Rod Position Indication System C Bank Hot Shutdown Alignment, PCN 79-1740.

CP-2.7.1, Rod Position Indication System C Bank at Power Alignment, PCN 79-1741.

CP-2.8.0, Rod Position Indication System D Bank Hot Shutdown Alignment, PCN 79-1742.

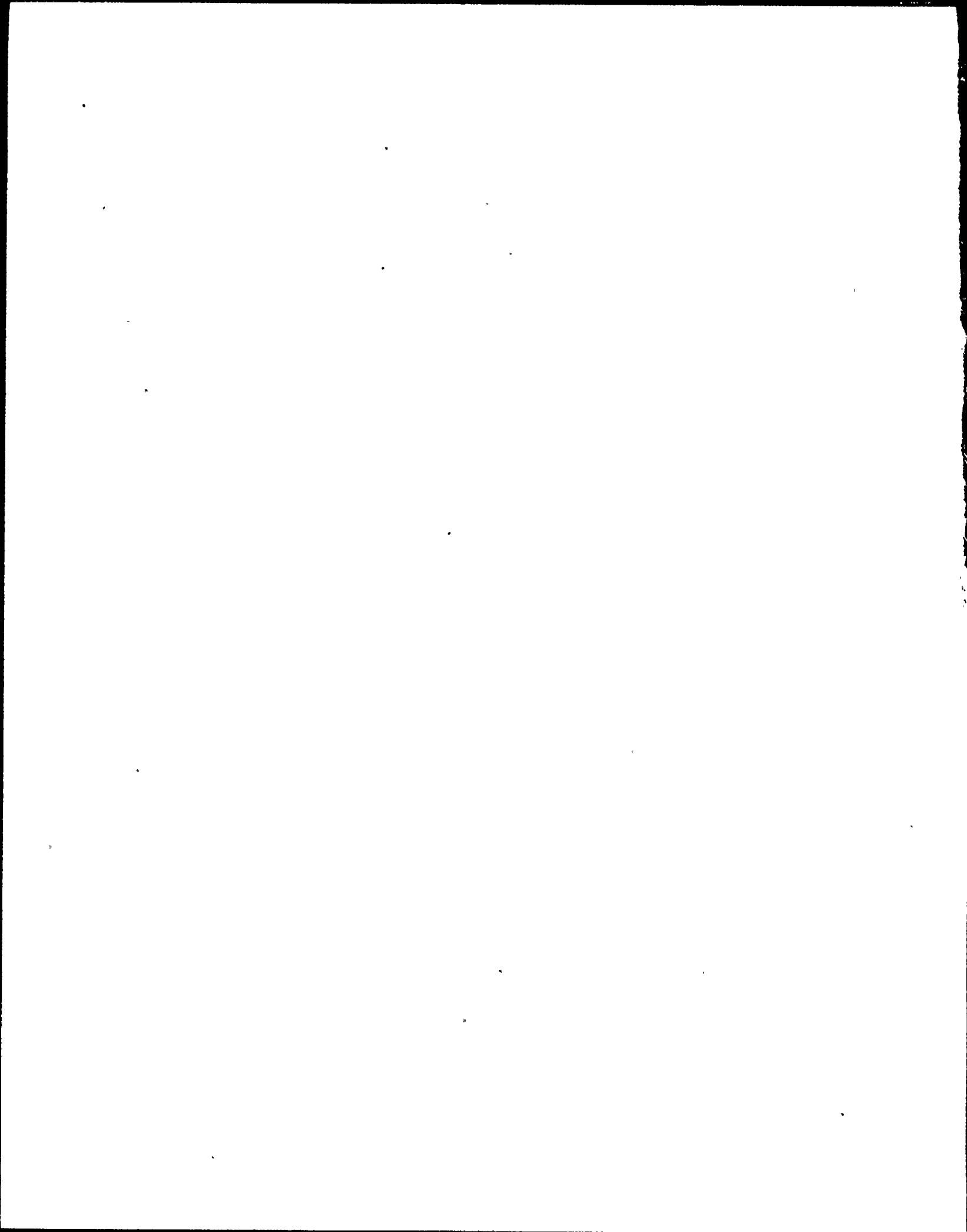
CP-2.8.1, Rod Position Indication System D Bank at Power Alignment, PCN 79-1743.

CP-2.9, Rod Position Indication System AC to DC Correlation Check, PCN 79-1747.

PORC
ITEM #

PROCEDURE

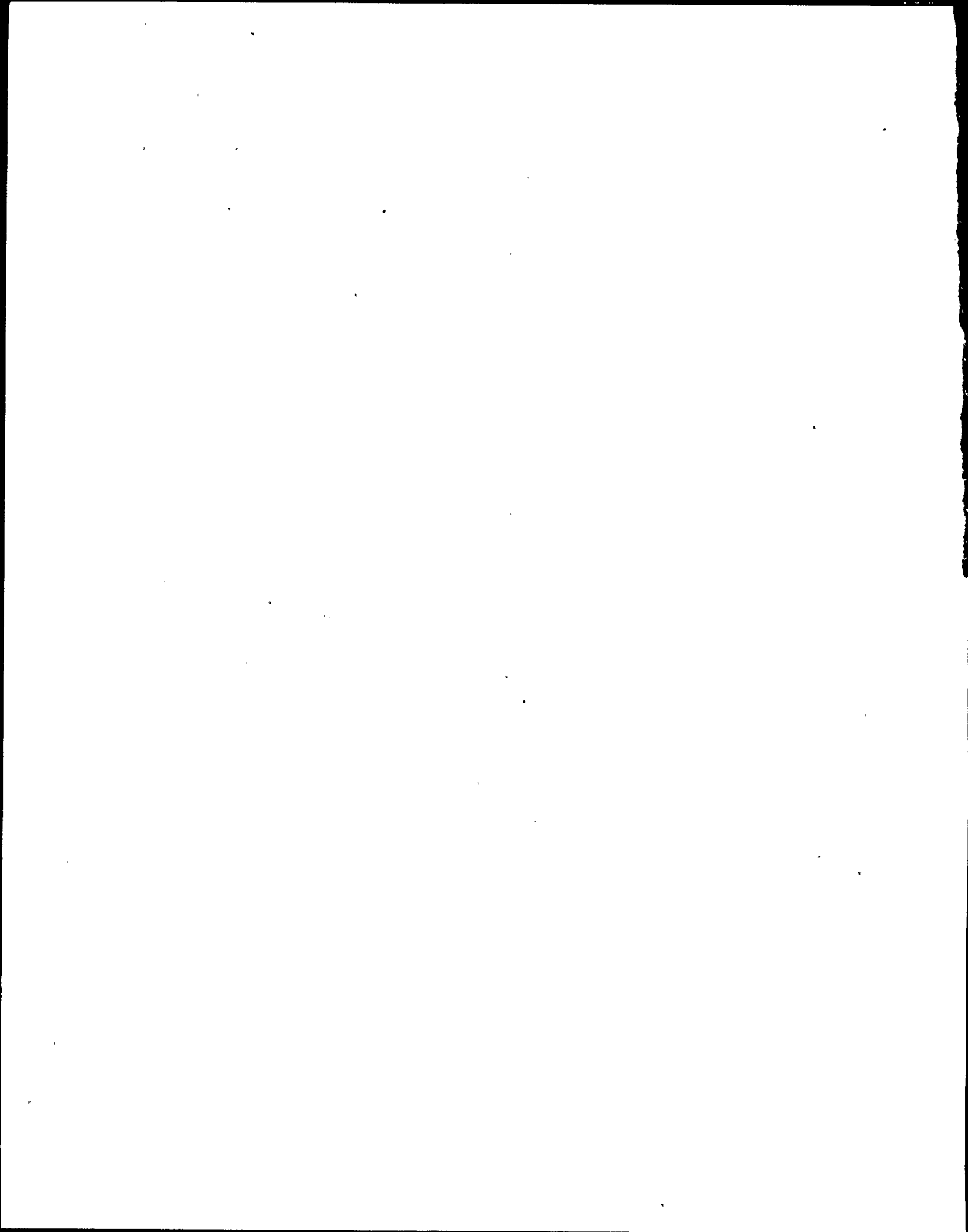
- 79-1157 E-1.1, Safety Injection System Actuation, PCN 79-1702.
- E-1.3, Steam Line Break Accident, PCN 79-1733.
- 79-1158 HP-1.1, Issuing Personnel Dosimeters, PCN 79-1710.
- HP-1.2, External Exposure Limits, PCN 79-1709.
- 79-1159 *HP-9.3, Air Ejector Gas Sampling, PCN 79-1752.
- 79-1160 *O-1.2, Plant from Hot Shutdown to Steady Load, PCN 79-1768.
- 79-1161 *O-2.2, Plant Shutdown from Hot Shutdown to Cold Shutdown, PCN 79-1714.
- 79-1162 *PT-2.3, Safeguard Valve Operations, PCN 79-1731.
- 79-1163 *PT-2.4, Cold/Refueling Motor Operated Valve Surveillance, PCN 79-1734.
- 79-1164 *PT-2.7, Service Water System, PCN 79-1715.
- 79-1165 *PT-4, RHR Loop Annual Hydro Test of Low Pressure Piping, PCN 79-1718.
- 79-1166 PT-5.10, Process Instrumentation Reactor Protection Channel Trip Test (Channel 1), PCN 79-1753.
- PT-5.20, Process Instrumentation Reactor Protection Channel Trip Test (Channel 2), PCN 79-1754.



FORC
ITEM #

PROCEDURE

- PT-5.30, Process Instrumentation Reactor Protection Channel Trip Test (Channel 3), PCN 79-1755.
- PT-5.40, Process Instrumentation Reactor Protection Channel Trip Test (Channel 4), PCN's 79-1748 and 79-1756.
- 79-1167 PT-23.9A, Containment Isolation Valve Leak Rate Testing "A" RCP Seal Water Line, PCN 79-1716.
- PT-23.9B, Containment Isolation Valve Leak Rate Testing "B" RCP Seal Water Line, PCN 79-1717.
- 79-1168 *RD-7, Liquid Waste Release, PCN 79-1708.
- 79-1169 *T-36.1, Station Service Water - Header Valve Alignment for Two Loop Operation, PCN 79-1720.
- 79-1181 A-47, Record and Document Retention, PCN 79-1770.
- M-37.32, Repacking of Accumulator Motor Operated Discharge Valves, PCN 79-1557.
- 79-1182 *AR/G-30, Steam Generator "A" Lo Lo Level Single Channel Alert, PCN 79-1790.
- 79-1183 CP-2.10, RPI System Maintenance Calibration Check, PCN 79-1765.
- CP-2.20.1, Calibration of Pulse to Analog Convertor Power Supply, PCN 79-1764.
- CP-2.20.2, Calibration Alignment of Pulse to Analog Convertor Bank PCN 79-1763.



PORC
ITEM #

PROCEDURE

CP-2.20.4, Calibration Alignment of Pulse to Analog Convertor Bank
PCN 79-1762.

CP-2.20.5, Calibration Alignment of Pulse to Analog Convertor Bank
PCN 79-1761.

CP-11.1, Pressurizer Pressurer Controller PC-431K, PCN 79-1772.

CP-11.2, Pressurizer Repeater PM-431G, PCN 79-1773.

CP-11.3, Pressurizer Spray Controller PC-431C, PCN 79-1775.

CP-11.4, Pressurizer Spray Repeater PM-431A, PCN 79-1774.

CP-11.5, Pressurizer Spray Controller PC-431H, PCN 79-1776.

CP-11.6, Pressurizer Spray Repeater PM-431B, PCN 79-1777.

CP-11.7, Bistable PC-431D/E, PCN 79-1778.

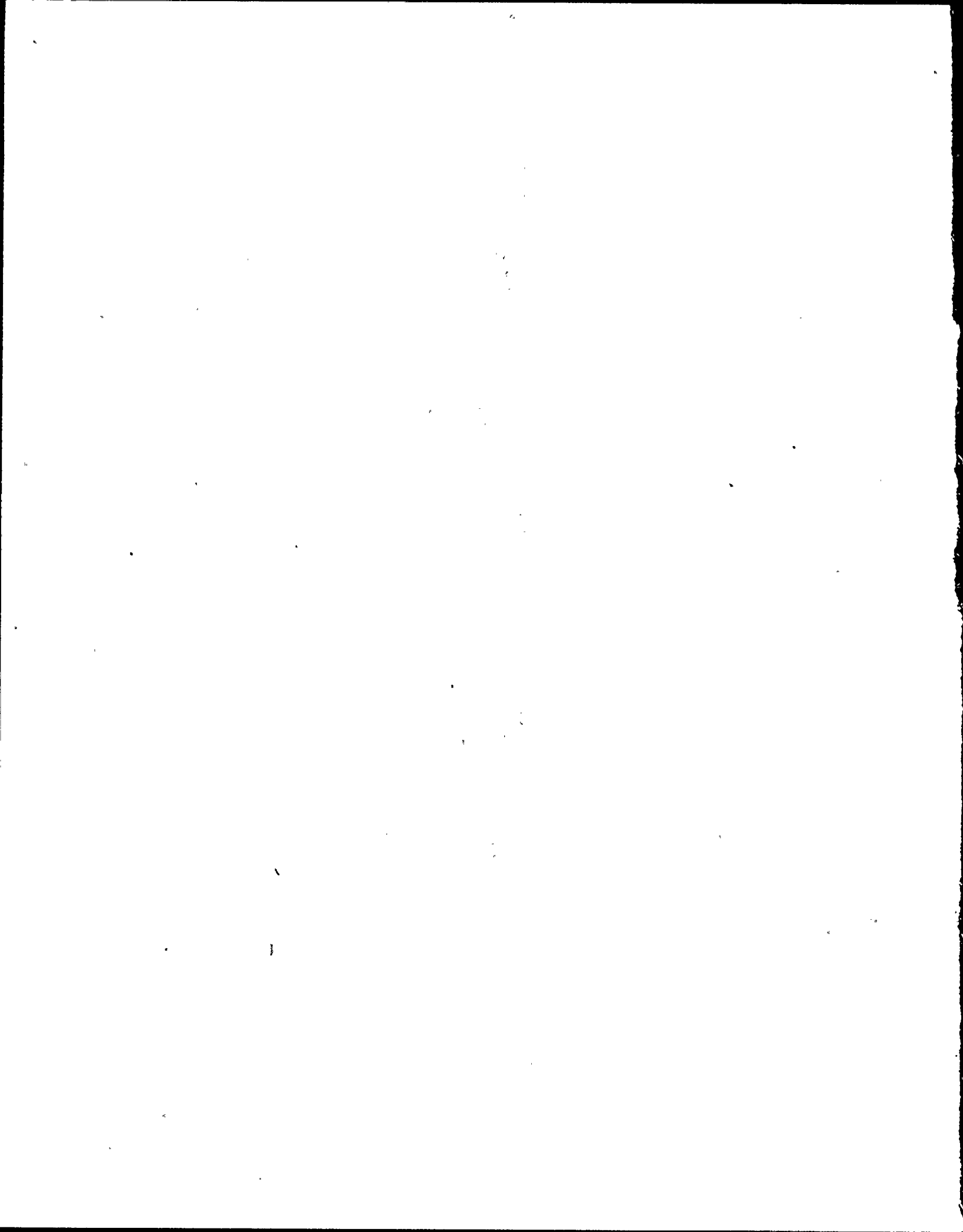
CP-11.8, Pressurizer Bistable PC-431B, PCN 79-1779.

CP-11.9, Bistable PC-430B, PCN 79-1780.

CP-11.10, Bistable PC-430C, PCN 79-1781.

CP-11.11, Bistable LC-428B/C, PCN 79-1782.

CP-11.12, Bistable LC-428D, PCN 79-1783.



PORC
ITEM #

PROCEDURE

CP-11.13, Bistable LC-427B/D, PCN 79-1784.

79-1184 *CP-10.5, Delta Flux Controller QL (TC-401R), PCN 79-1759.

79-1185 *E-16.1, High Activity - Radiation Monitoring System, PCN 79-1788.

79-1186 *HP-3.1, Exposure Reports to Individuals and the NRC, PCN 79-1771.

79-1187 *HP-4.3, Work Permit Use, PCN's 79-1798 and 79-1810.

79-1188 *O-6.2, Main Control Board System Status Verification, PCN 79-1769.

79-1189 *PI-2.9.2, Accumulator Discharge Check Valves Quarterly Exercising, PCN 79-1786.

79-1190 *PT-3, Containment Spray Pumps and NaOH Additive System, PCN 79-1789.

79-1191 *PT-17.2, Process Radiation Monitors R-11 thru R-21 and Iodine Monitors R-10A and R-10B, PCN 79-1787.

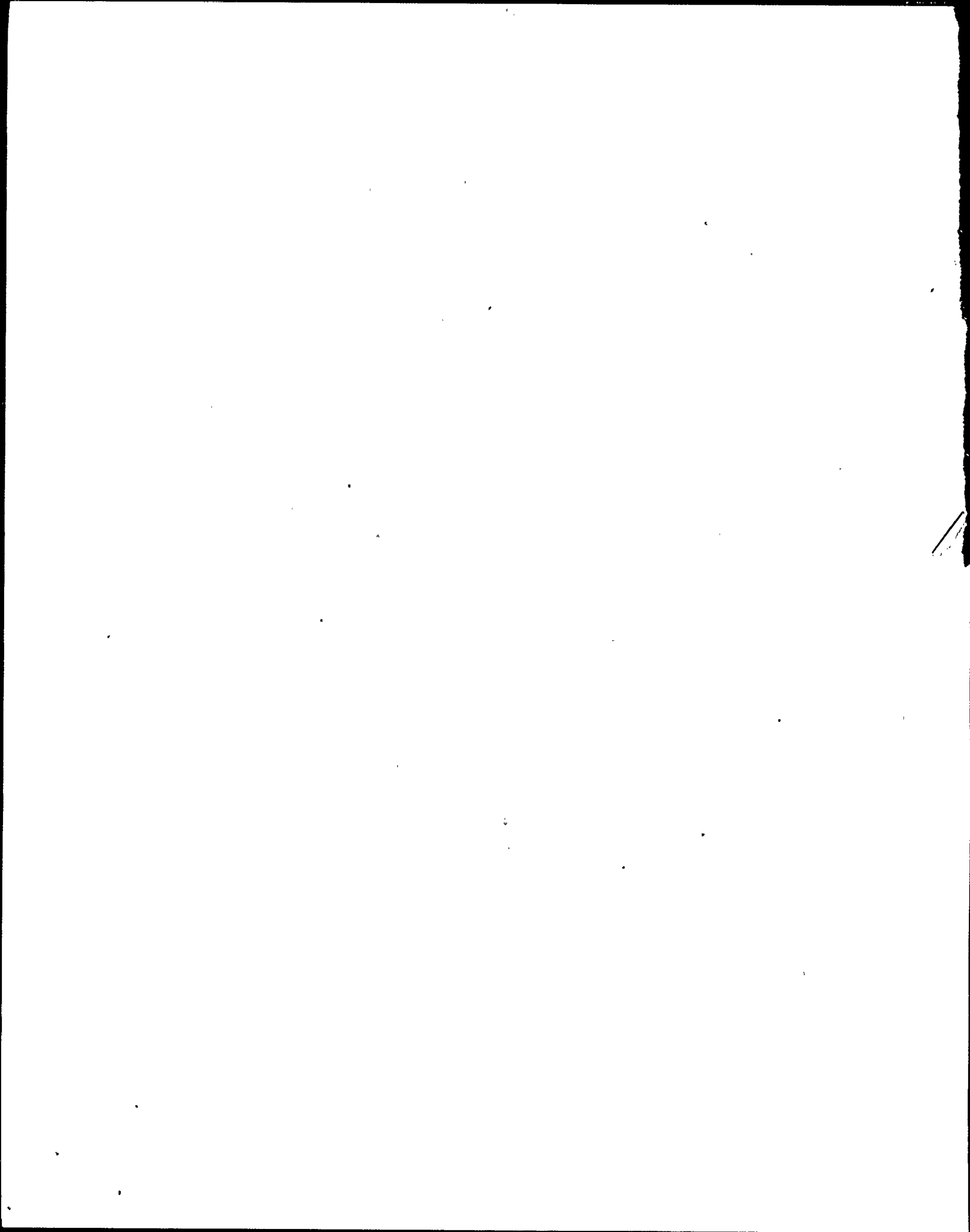
79-1193 *S-30.1, Safety Injection System Valve Position Verification, PCN 79-1749.

79-1194 *SC-1.12A, Immediate Call List, PCN 79-1797.

79-1201 *A-17, Plant Operations Review Committee Operating Procedure, PCN 79-1816.

79-1202 *E-13.1, Power Range NIS Malfunction, PCN 79-1813.

79-1203 *GS-18.0, Training and Qualifying of Guards, PCN 79-1807.



FORC
ITEM #

PROCEDURE

79-1204 *EM-169, Valve HCV-105, Isolation, Inspection and Maintenance, PCN 79-1563.

79-1205 *A-7.2, Inventory of the First Aid Emergency Kit Hospital Radiation Kit Hospital Radiation Equipment, PCN 79-1827.

79-1206 *A-705, Control of Purchased Services, PCN 79-1815.

79-1207 CP-20.1, Defeat of Delta Flux Signal Channel 2, PCN 79-1791.

CP-30.1, Defeat of Delta Flux Signal Channel 3, PCN 79-1793.

CP-63.3.1, General Defeat Procedure "63-3" Auto Stop Pressure, PCN 79-1803.

CP-63.4.1, General Defeat Procedure "63-4" Auto Stop Pressure, PCN 79-1802.

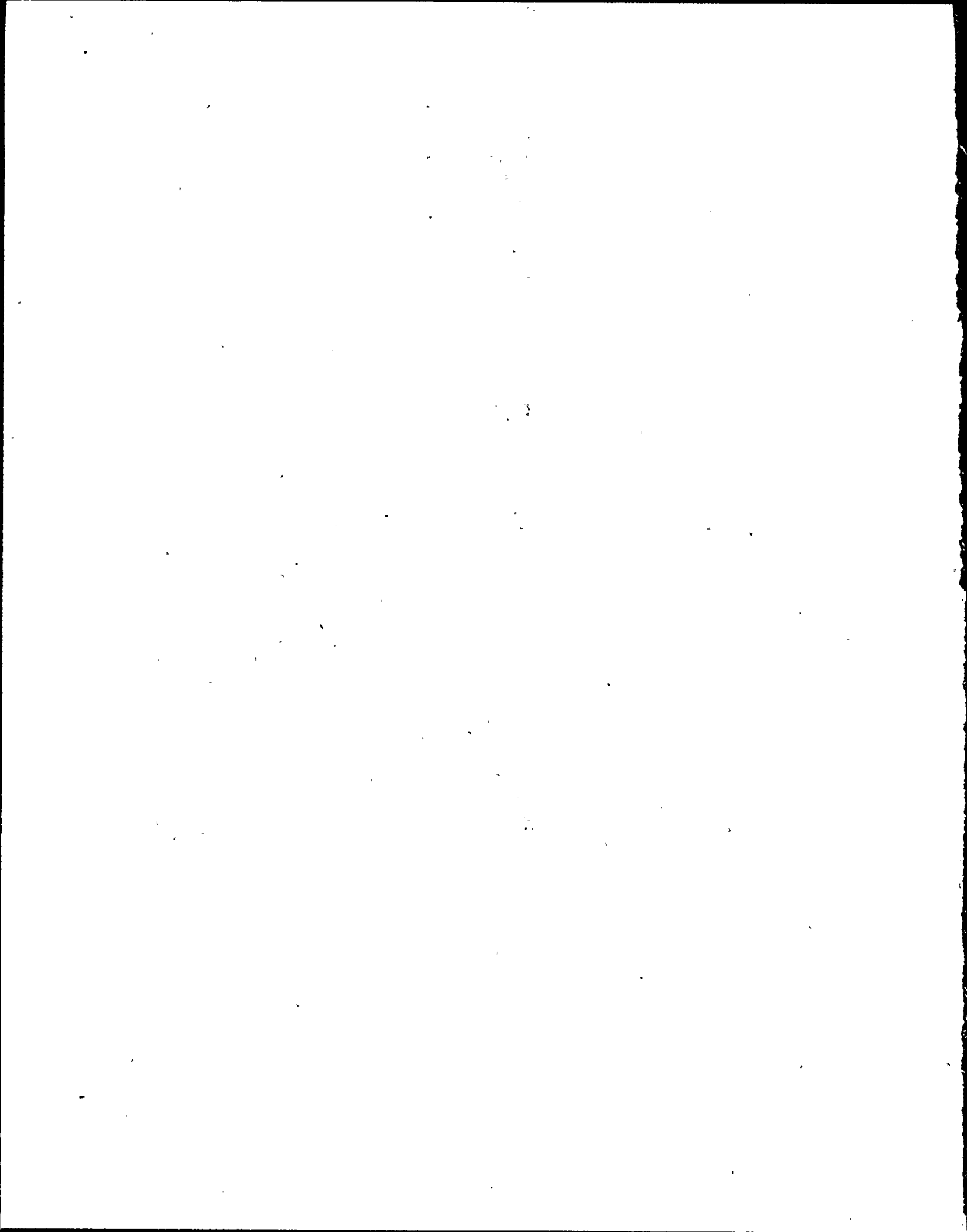
CP-63.5.0, Calibration and/or Maintenance of "63-5" Auto Stop Pressure Channel, PCN 79-1800.

CP-63.5.1, General Defeat Procedure "63-5" Auto Stop Pressure, PCN 79-1801.

CP-102.1, Defeat of Boric Acid Tank Level Channel "102", PCN 79-1811.

CP-102.2, Reinstatement of Boric Acid Tank Level Channel "102", PCN 79-1812.

CP-103.1, Boric Acid Tank Temperature Controller, PCN 79-1821.



PORC
ITEM #

PROCEDURE

CP-103.2, Boric Acid Tank Temperature Controller Alarms, PCN 79-1822.

CP-104.1, Boric Acid Tank Temperature Indicator TI-104, PCN 79-1823.

CP-105.1, Boric Acid Tank Temperature Indicator TI-105, PCN 79-1824.

CP-106.1, Defeat of Boric Acid Tank Level Channel "106", PCN 79-1825.

CP-106.2, Reinstate of Boric Acid Tank Level Channel "106", PCN 79-1826.

CP-107.1, Boric Acid Tank Temperature Controller, PCN 79-1833.

CP-107.2, Boric Acid Tank Temperature Controller Alarms, PCN 79-1832.

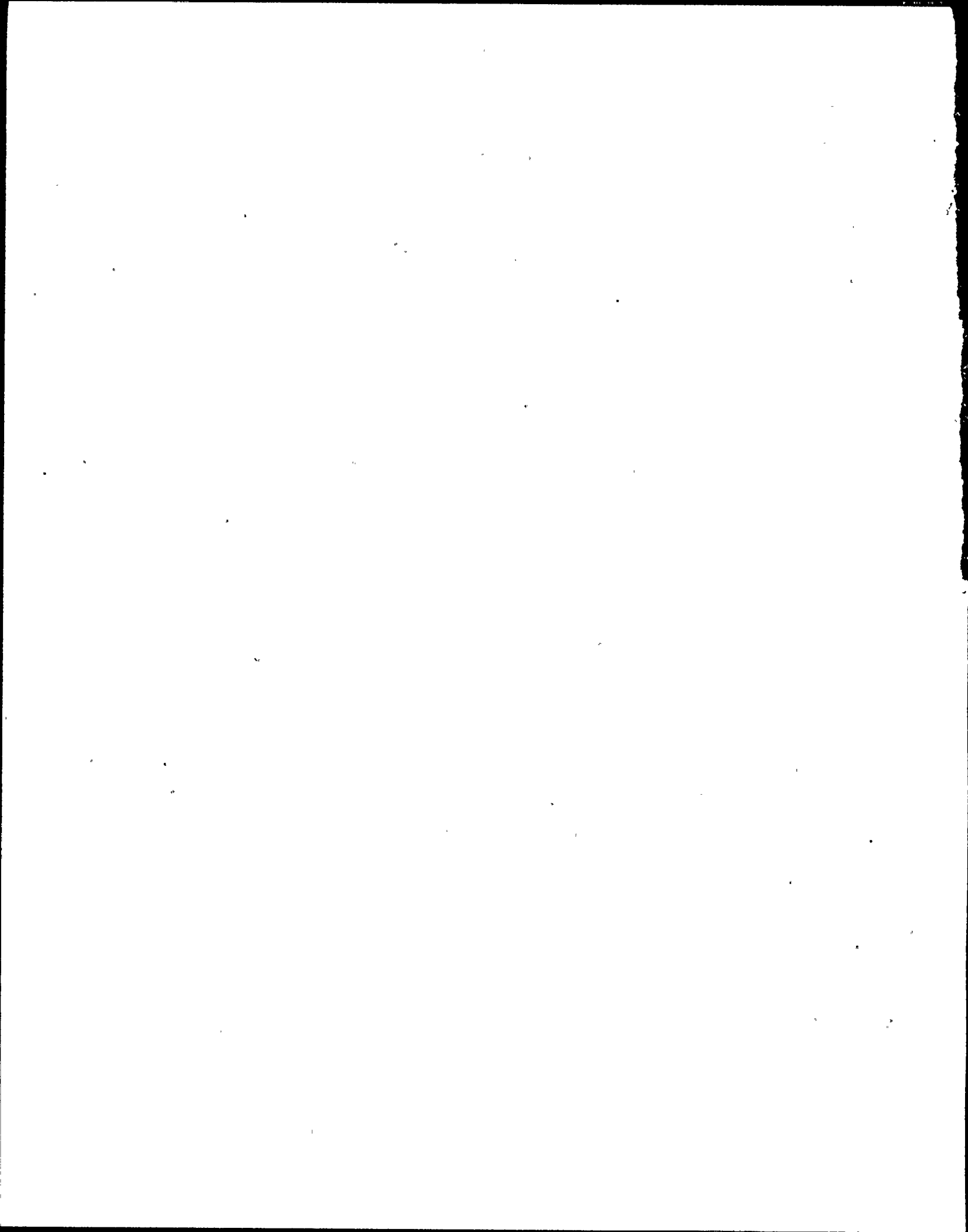
CP-110.1, General Defeat Procedure for Boric Acid Control Flow Channel 110, PCN 79-1830.

CP-111.1, General Defeat Procedure Reactor Makeup Water Flow Channel "111", PCN 79-1831.

79-1208 *CP-30.0, Calibration and/or Maintenance of Delta Flux Controllers Channel 3, PCN 79-1792.

79-1209 CP-40.1, Defeat of Delta Flux Signal Channel 4, PCN 79-1794.

CP-40.2, Reinstate of Delta Flux Signal Channel 4, PCN 79-1795.



PORC
ITEM #

PROCEDURE

- 79-1210 CP-102.9, Boric Acid Tank Level Nitrogen Bubbler Supply Pressure Switch PS-102, PCN 79-1835.
- CP-102.10, Boric Acid Tank Level Nitrogen Header Pressure Switch PAS-102, PCN 79-1834.
- CP-106.9, Boric Acid Tank Level Nitrogen Bubbler Supply Pressure Switch PS-106, PCN 79-1837.
- CP-106.10, Boric Acid Tank Level Nitrogen Header Pressure Switch PAS-106, PCN 79-1836.
- 79-1211 *E-10, Malfunction Rod Position Indicator, PCN 79-1760.
- 79-1212 E-26.1, Emergency Shutdown Resulting from a Reactor Trip From 8.5% Power or Less, PCN 79-1838.
- E-26.2, Emergency Shutdown Resulting from a Reactor Trip When Between 50% and 8.5% Power, PCN's 79-1796 and 79-1839.
- E-26.4, Turbine Trip Without Reactor Trip, PCN 79-1841.
- 79-1213 *E-26.3, Emergency Shutdown Resulting From a Reactor Trip When Operating at Greater Than 50% Power, PCN 79-1840.
- 79-1215 *GS-1.2, Security Event Report, PCN 79-1806.
- 79-1216 *GS-24.0, Security Surveillance Requirements and Procedures, PCN 79-1805.
- 79-1217 *HP-1.3, External Exposure Records, PCN 79-1828.
- 79-1218 *HP-12.1, Usage of Respirators, PCN 79-1809.

<u>FORC</u> <u>ITEM #</u>	<u>PROCEDURE</u>
<u>79-1222</u>	<u>*SC-3.15.15, Emergency Fire Equipment Locker Inventory and Inspection, PCN 79-1818.</u>
<u>79-1223</u>	<u>*T-27.1, 1A Emergency Diesel Generator Pre-Start Alignment, PCN 79-1829.</u>
<u>79-1224</u>	<u>*T-26.2, 1B Emergency Diesel Generator Pre-Start Alignment, PCN 79-1820.</u>
<u>79-1236</u>	<u>*P-3, Precautions, Limitations and Setpoints, P-3, Chemical and Volume Control System, PCN 79-1845.</u>
<u>79-1237</u>	<u>*S-16.16, Safety Injection Pump Isolation, PCN 79-1847.</u>
<u>79-1238</u>	<u>*AR/AA-21, Auxiliary Feedwater Defeat Switch, PCN 79-1808.</u>
<u>79-1239</u>	<u>*A-1.6, ALARA Committee Operating Procedure, PCN 79-1866.</u>
<u>79-1240</u>	<u>*A-1.6.1, Documentation of "As Low As Reasonably Achievable" (ALARA) Program, PCN 79-1869.</u>
<u>79-1241</u>	<u>*A-7.2, Inventory of First Aid Emergency Kit, Hospital Radiation Kit Hospital Radiation Equipment, PCN 79-1851.</u>
<u>79-1242</u>	<u>*A-52.4, Control of Limiting Conditions for Operating Equipment, PCN 79-1878.</u>
<u>79-1243</u>	<u>*A-52.5, Control of Limiting Conditions for System Specifications, PCN 79-1879.</u>
<u>79-1244</u>	<u>*A-1602, Preventive Action Report, PCN 79-1849.</u>

FORC
ITEM #

PROCEDURE

79-1245 CP-110.0, Calibration and/or Maintenance of Boric Acid Flow Channel "110", PCN 79-1867.

CP-110.8, Boric Acid Control Bistable YC-110, PCN 79-1862.

CP-111.0, Calibration and/or Maintenance of Reactor Makeup Water Flow Channel "111", PCN 79-1863.

CP-111.8, Reactor Makeup Water Flow Bistable YC-111, PCN 79-1861.

79-1246 CP-112.0, Calibration and/or Maintenance of Volume Control Tank Level Channel "112", PCN 79-1881.

CP-112.3, Volume Control Tank Level Transmitter LT-112, PCN 79-1883.

79-1247 CP-112.1, General Defeat Procedure Volume Control Tank Level Channel "112", PCN 79-1882.

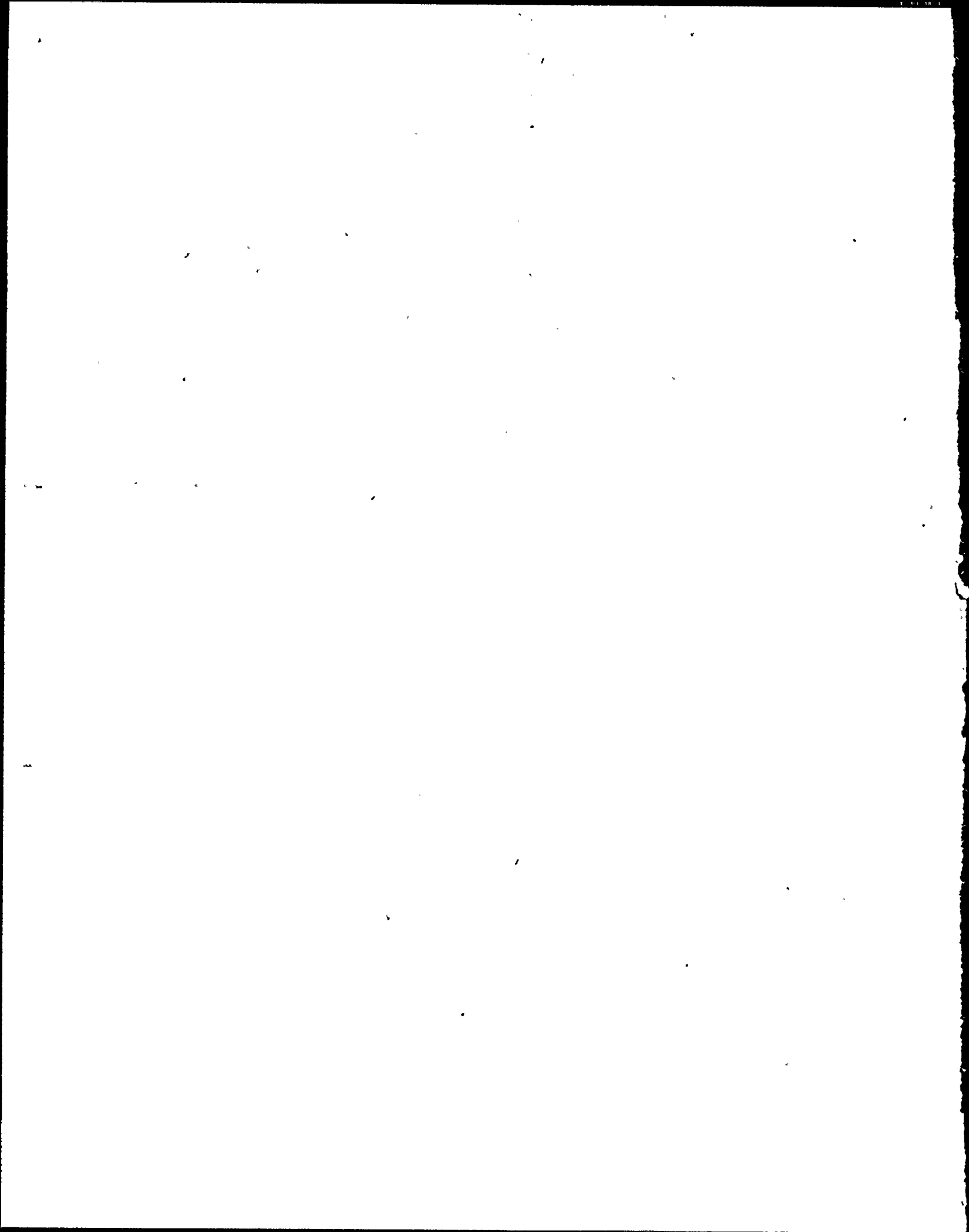
CP-139.1, General Defeat Procedure "139" Volume Control Tank Level Channel, PCN 79-1886.

79-1248 *CP-139.0, Calibration and/or Maintenance of Volume Control Tank Level Channel "139", PCN 79-1884.

79-1249 *CP-139.8, Calibration and/or Maintenance of Volume Control Tank Level Power Supply "LQ-139", PCN 79-1885.

79-1250 GS-3.0, Security Call List, PCN 79-1889.

GS-30.0, Threatening Phone Call or Bomb Threat Procedure, PCN 79-1888.



FORC
ITEM #

PROCEDURE

79-1253 *O-1.1C, Precritical Technical Specification Check Sheet, PCN 79-1877.

79-1254 PC-1.1, Primary Coolant Analysis Schedule and Limits, PCN 79-1864.
PC-1.2, Non-Primary Coolant Analysis Schedule and Limits, PCN 79-1865.

79-1255 *PT-13.1.13, Star Corporation Heat Detector System Test, PCN 79-1844.

79-1256 *PT-2.1, Safety Injection System Pumps, PCN's 79-1846 and 79-1848.

79-1257 *PT-2.8, Component Cooling Water Pump System, PCN 79-1856.

79-1258 *S-23.2.2, Containment Purge Procedure, PCN 79-1891.

79-1259 *S-30.3, Containment Spray System Valve Position Verification, PCN 79-1894.

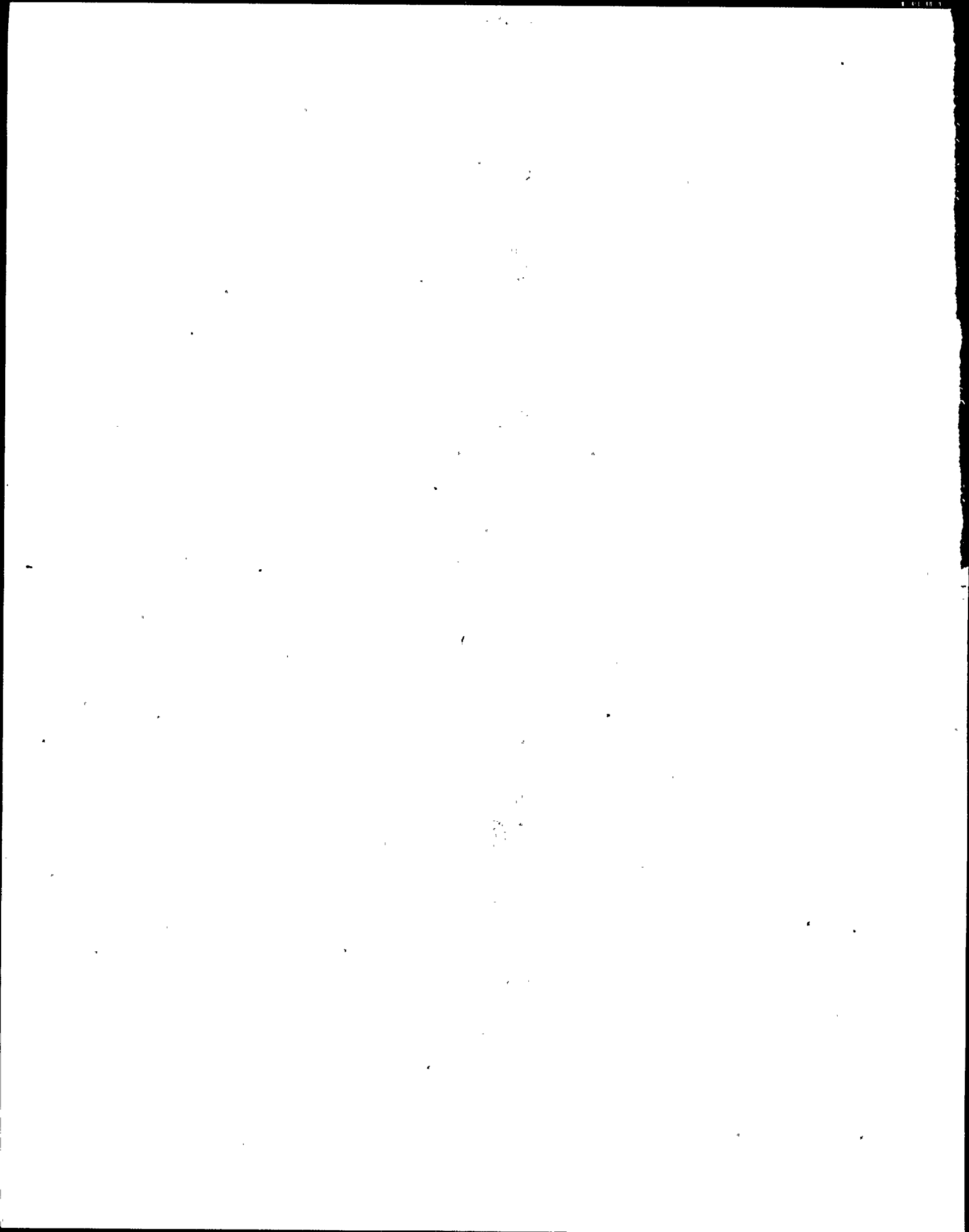
79-1260 *SC-1.8, Post Accident Environmental Sampling, PCN 79-1893.

79-1261 *SC-1.7C, Monitoring Site Radiation Level by TLD, PCN 79-1892.

79-1262 *SC-1.15, Inspection of Emergency Equipment, PCN 79-1819.

79-1263 *T-44.4, HVAC Lineup for Standby Auxiliary Feedwater System, PCN 79-1890.

79-1272 ES-7.1, Collecting Environmental and Post Accident TLD's, PCN 79-1852.



PORC
ITEM #

PROCEDURE

ES-7.2, Readout of Environmental and Post Accident TLD's, PCN 79-1853.

RD-13.2, Ingestion Doses from Liquid Releases, PCN 79-1854.

RD-13.3, Dose Commitment from Gaseous Releases, PCN 79-1855.

S-16.16A, 1A Safety Injection Pump Isolation Restoration, PCN 79-1857.

S-16.16B, 1B Safety Injection Pump Isolation Restoration, PCN 79-1858.

S-16.16C, 1C Safety Injection Pump Isolation Restoration, PCN 79-1859.

79-1273

EM-174, Emergency Maintenance Procedure - EM 174 - Check Valve 7226 Isolation for Inspection and Maintenance, PCN 79-1928.

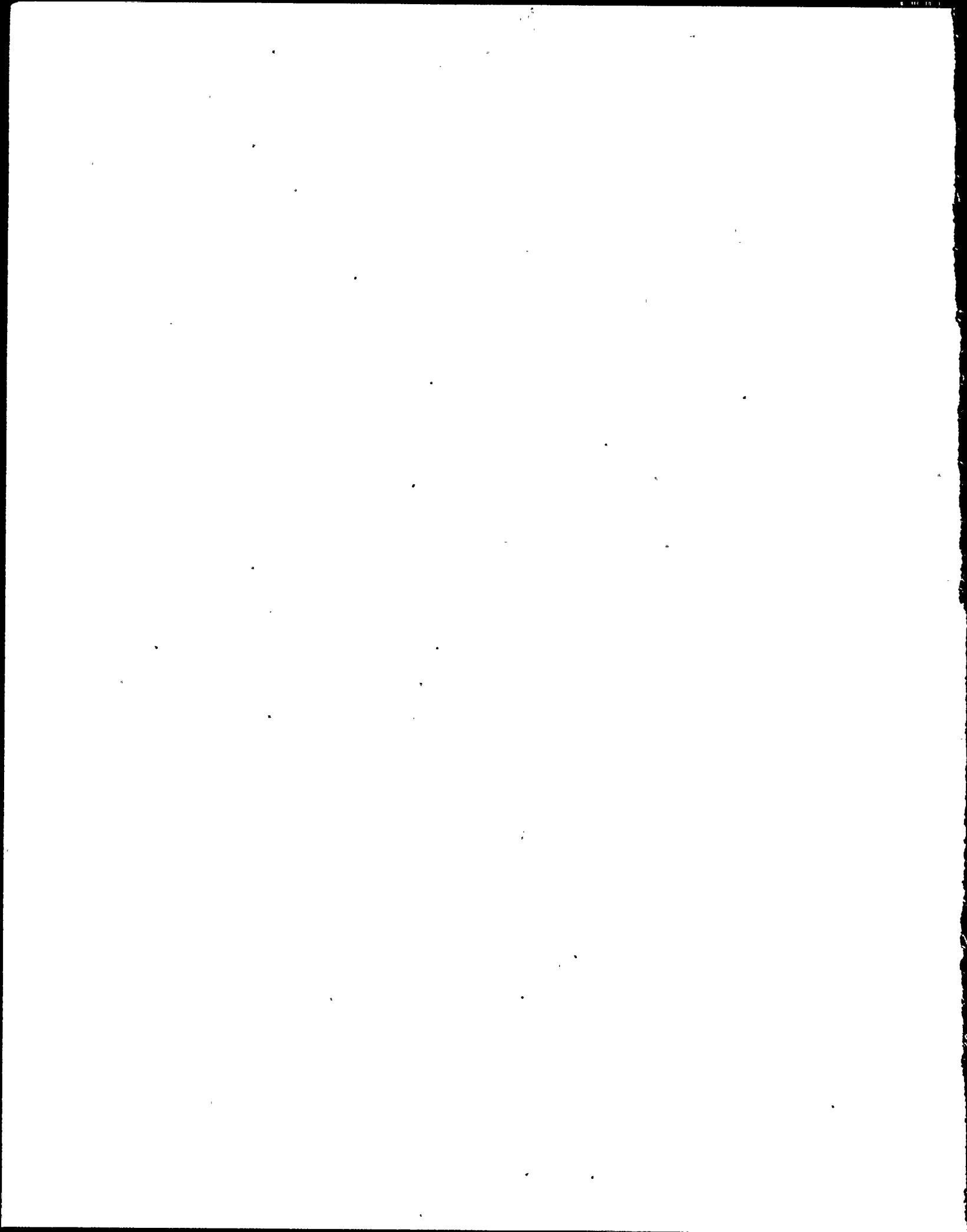
EM-179, Replacement of CVCS Letdown Vent Valve 2234, PCN 79-1916.

EM-188, Repair of Weld on "C" Charging Pump Discharge Line to Drain Valve 292E, PCN 79-1895.

EM-194, Repair of CVCS Hanger CH54, PCN 79-1896.

EM-197, Repair of Weld on "B" Charging Pump Discharge Line to Relief Valve 284, PCN 79-1897.

EM-198, Replacement of the Buchanan Terminal Blocks for Pressurizer Pressure and Level Transmitters Terminations With a Westinghouse Style No. 542247 Terminal Block, PCN 79-1898.



FORC
ITEM #

PROCEDURE

EM-202, Safety Injection Sequence Timing Relay Replacement, PCN 79-1899.

EM-205, Turbine Runback Relay in RPI System Replacement, PCN 79-1900.

EM-232, Repair of Feedwater Piping Inside Containment, PCN 79-1901.

EM-234, Repair of Feedwater - Steam Generator Nozzles, PCN 79-1902.

EM-237, Weld Repair - Connection at "A" Feedwater Line from MDAFWP, PCN 79-1903.

EM-238, Weld Repair - Connection at "A" Feedwater Line from TDAFWP, PCN 79-1904.

EM-239, Containment Spray Ring Header Pipe Clamp Replacement, PCN 79-1905.

79-1274 *PT-13.1.5, Flood Valve System Test System #3, 4, 5, 6, PCN 79-1913.

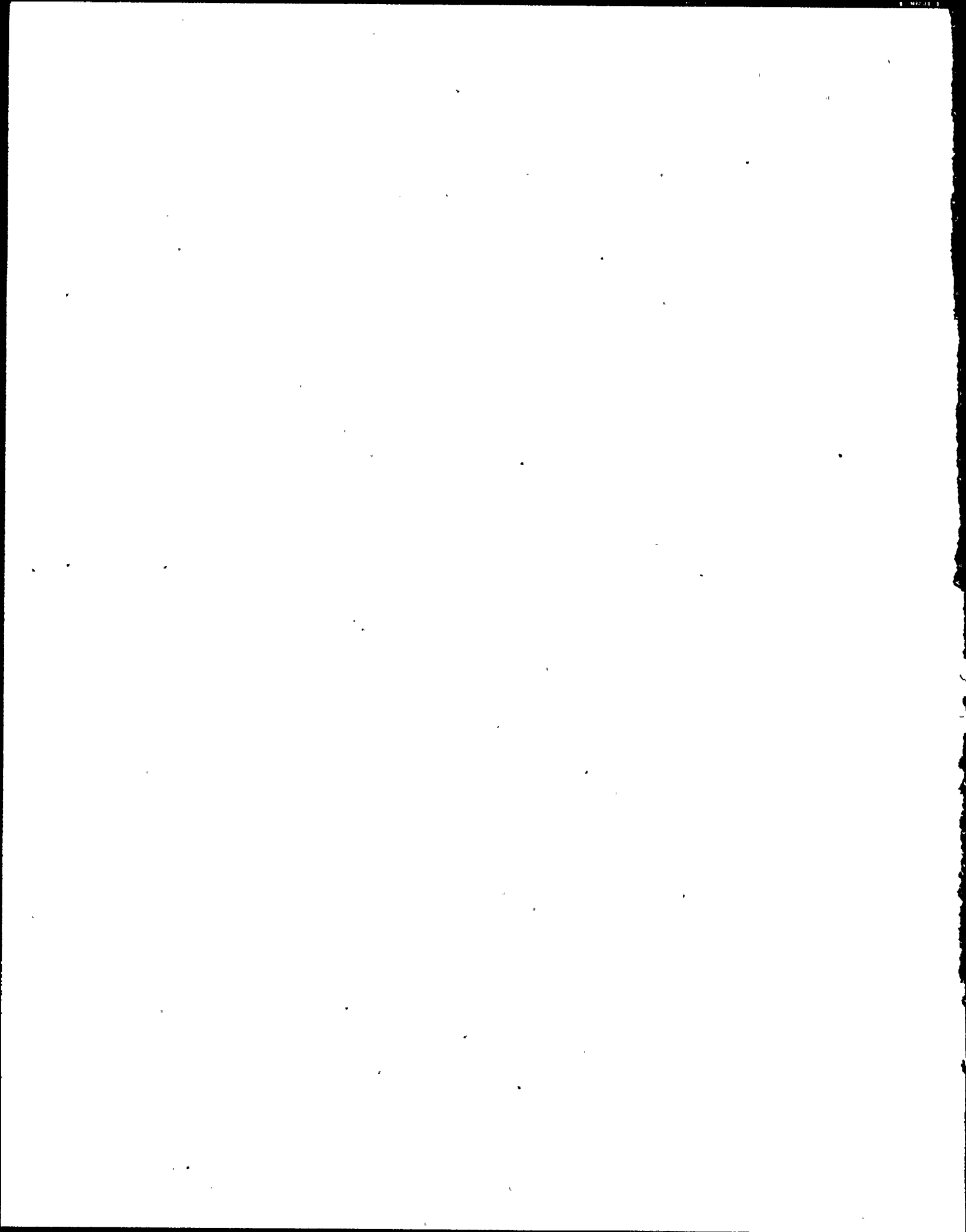
79-1275 *PT-12.2, Emergency Diesel Generator 1B, PCN 79-1934.

79-1277 *M-37.54, Isolation and Maintenance of Letdown Orifice Valves 200A, 200B and 202, PCN 79-1917.

79-1278 *M-48.4, Isolation of Bus 16, PCN 79-1921.

79-1281 *A-35, Prime Mover Holding and Markup Requirements, PCN 79-1924.

79-1282 *A-53.1, Equipment Inspection Period and Lubricant List, PCN 79-1936.



PORC
ITEM #

PROCEDURE

79-1283 A-301, Control of Station Modifications, PCN 79-1940.

A-304, Preparation, Review and Approval of Design Output Documents for Minor Modifications, PCN 79-1939.

79-1284 *A-1001, Inspection and Surveillance Activities, PCN 79-1935.

79-1285 *GS-5.0, Security Guard General Instructions, PCN 79-1925.

79-1286 *GS-8.0, Detex Tours By Security Guards, PCN 79-1949.

79-1287 *GS-9.0, Walkie Talkie Procedure for the Security Force, PCN 79-1950.

79-1288 *GS-18.1, Conduct of On-The-Job Training of Guards, PCN 79-1951.

79-1289 *GS-19.0, Issuance of Electronic Card Keys, PCN 79-1946.

79-1290 *GS-21.2, Unoccupied Vital Area Access Control, PCN 79-1907.

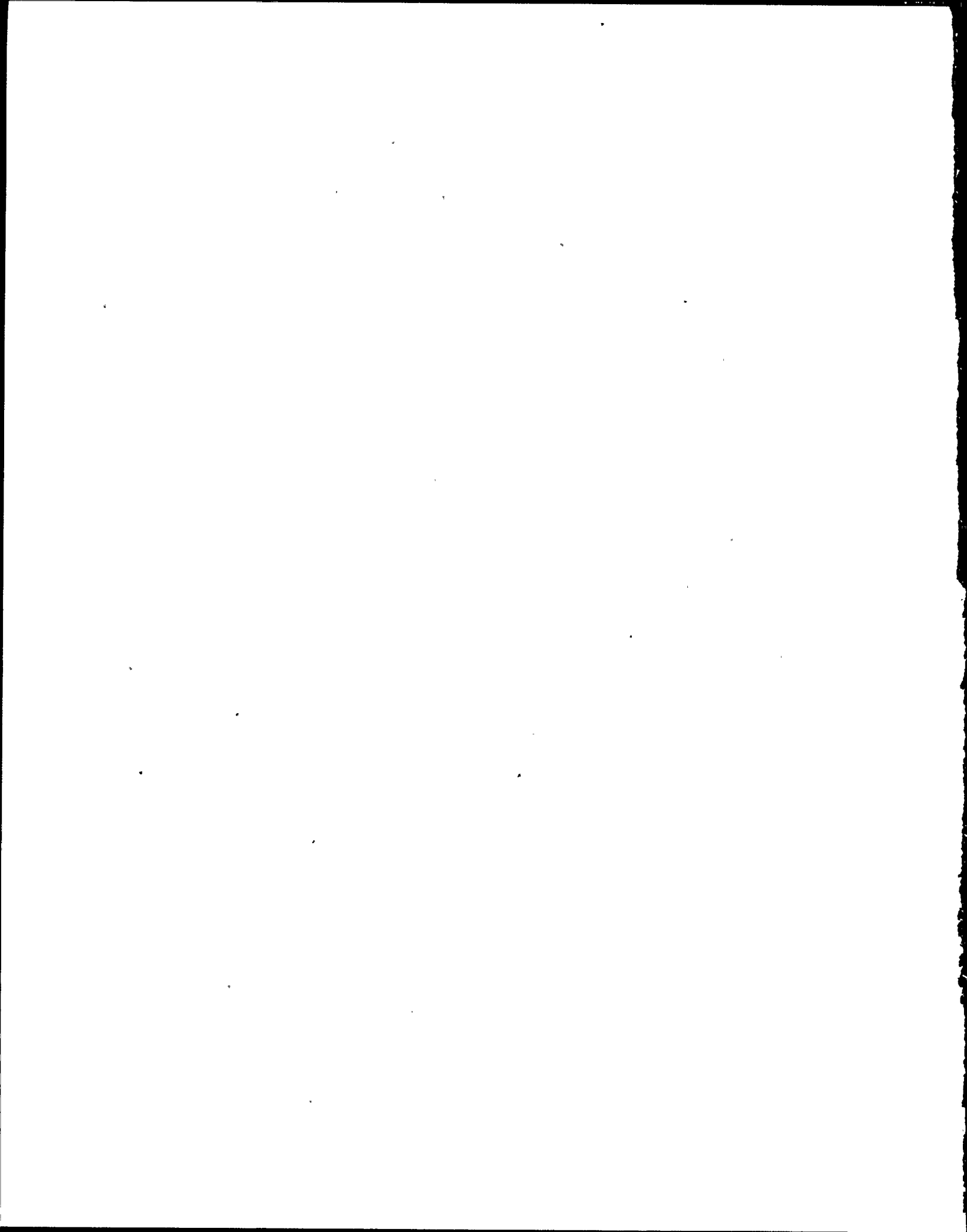
79-1291 *GS-26.0, Security Key and Lock Controls, PCN 79-1947.

79-1292 *GS-32.0, Civil Disturbance Procedure, PCN 79-1945.

79-1293 *GS-33.1, Use of Rifles and Shotguns, PCN 79-1942.

79-1294 *GS-34.0, Vital Areas, PCN 79-1944.

79-1295 *GS-35.0, Issuing and Destruction of Ginna Photo Identification Cards, PCN 79-1948.



PORC
ITEM #

PROCEDURE

79-1296 *GS-39.0, Protected/Vital Area Access, PCN 79-1943.

79-1297 *M-11.5K, Turbine Driven Auxiliary Feedwater Pump Major or Mechanical Inspection, PCN 79-1937.

79-1298 *M-48.2, Isolation of Bus 14, PCN 79-1922.

79-1299 O-1.1, Plant Heatup from Cold Shutdown to Hot Shutdown, PCN 79-1880.

O-6.2, Main Control Board System Status Verification, PCN 79-1906.

79-1300 *PC-1.3, Daily Chemical Analysis Results, PCN 79-1938.

79-1301 *PT-2.5.2, Air Operated Valves, Quarterly Surveillance (Valves 112B and 112C), PCN 79-1870.

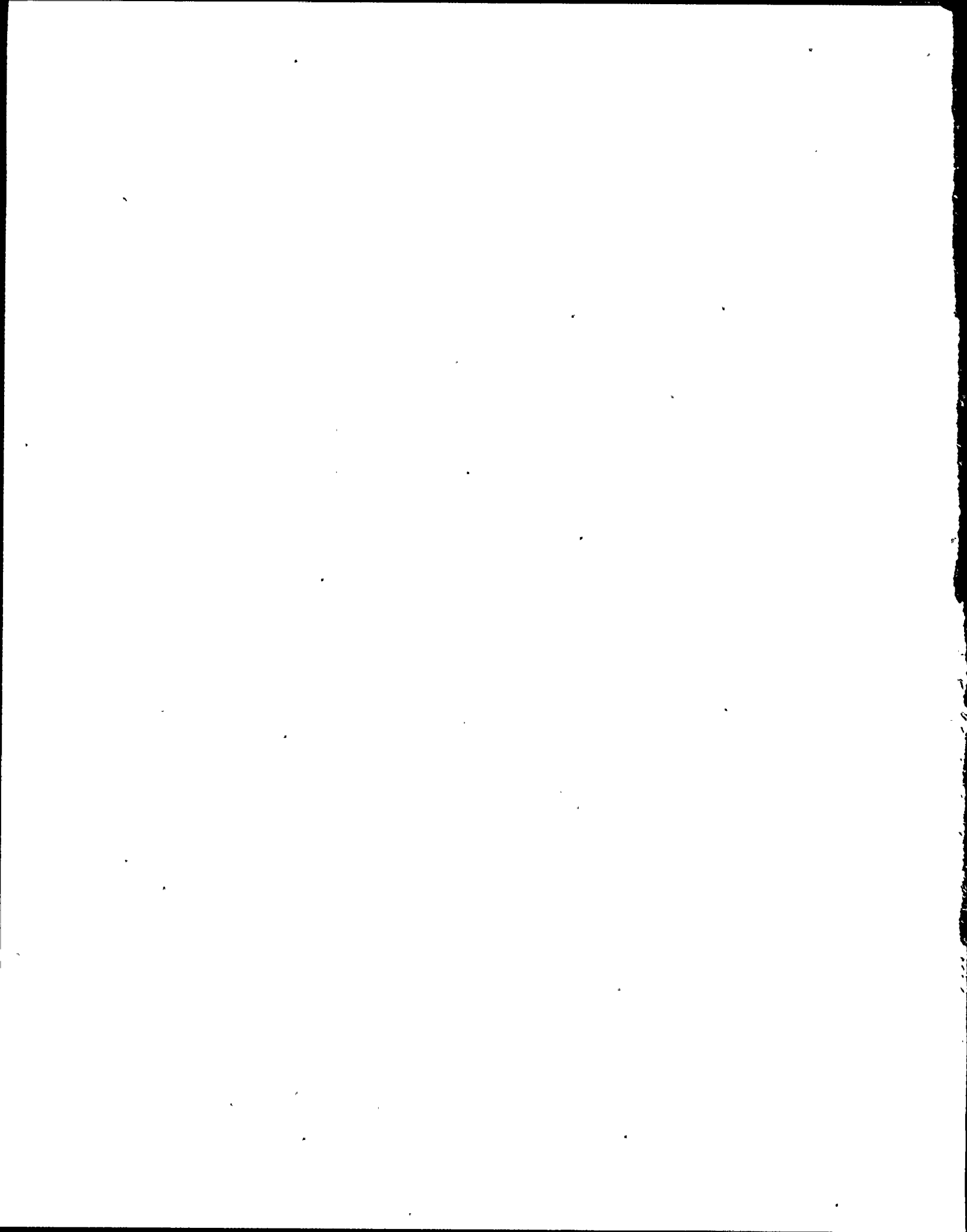
79-1302 PT-5.10, Process Instrumentation Reactor Protection Channel Trip Test (Channel 1), PCN 79-1952.

PT-5.20, Process Instrumentation Reactor Protection Channel Trip Test (Channel 2), PCN 79-1953.

PT-5.30, Process Instrumentation Reactor Protection Channel Trip Test (Channel 3), PCN 79-1954.

79-1303 *RD-4, Vent Radiogas Background and Factor Determination, PCN 79-1915.

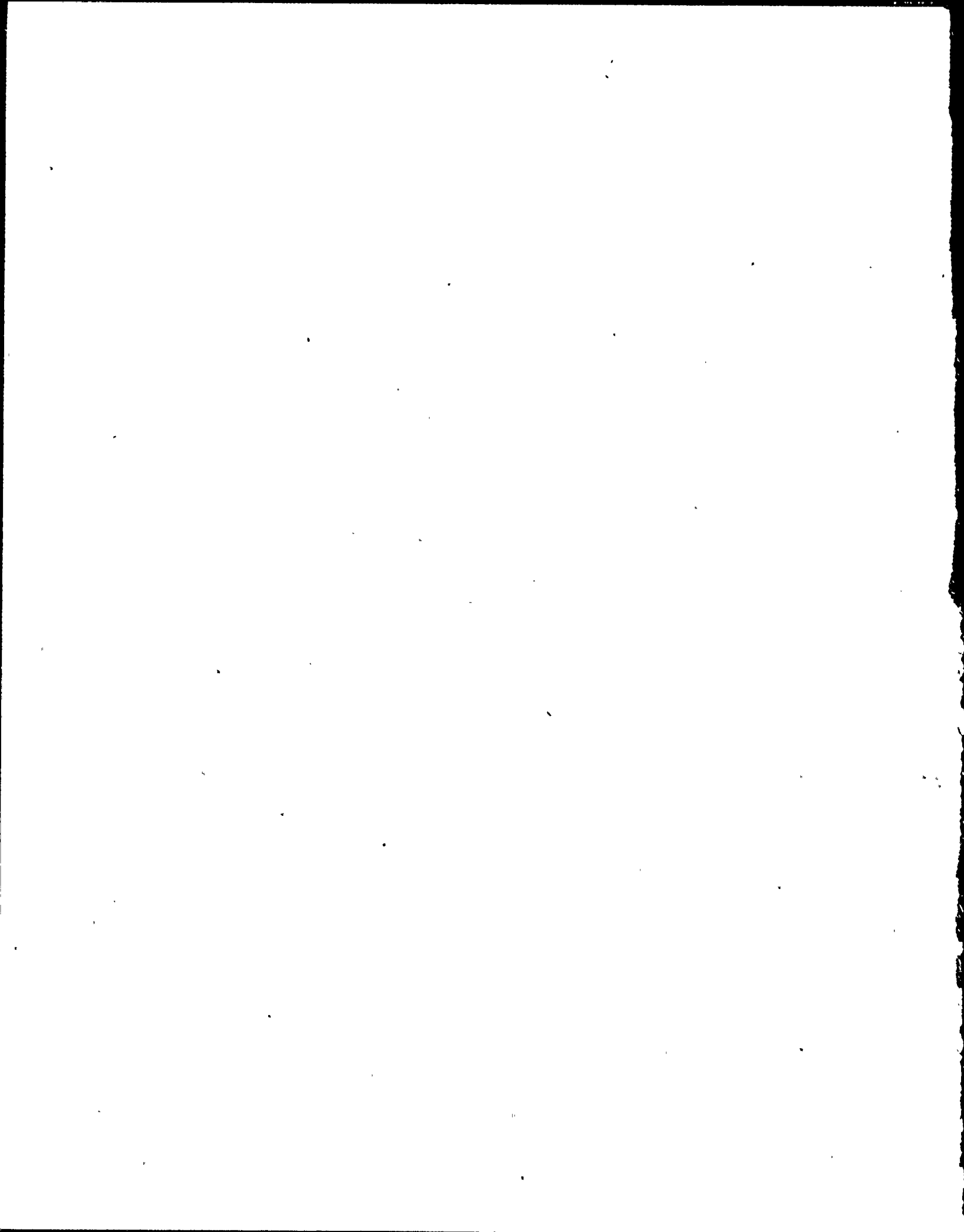
79-1305 S-11, Batching Tank, PCN 79-1933.



FORC
ITEM #

PROCEDURE

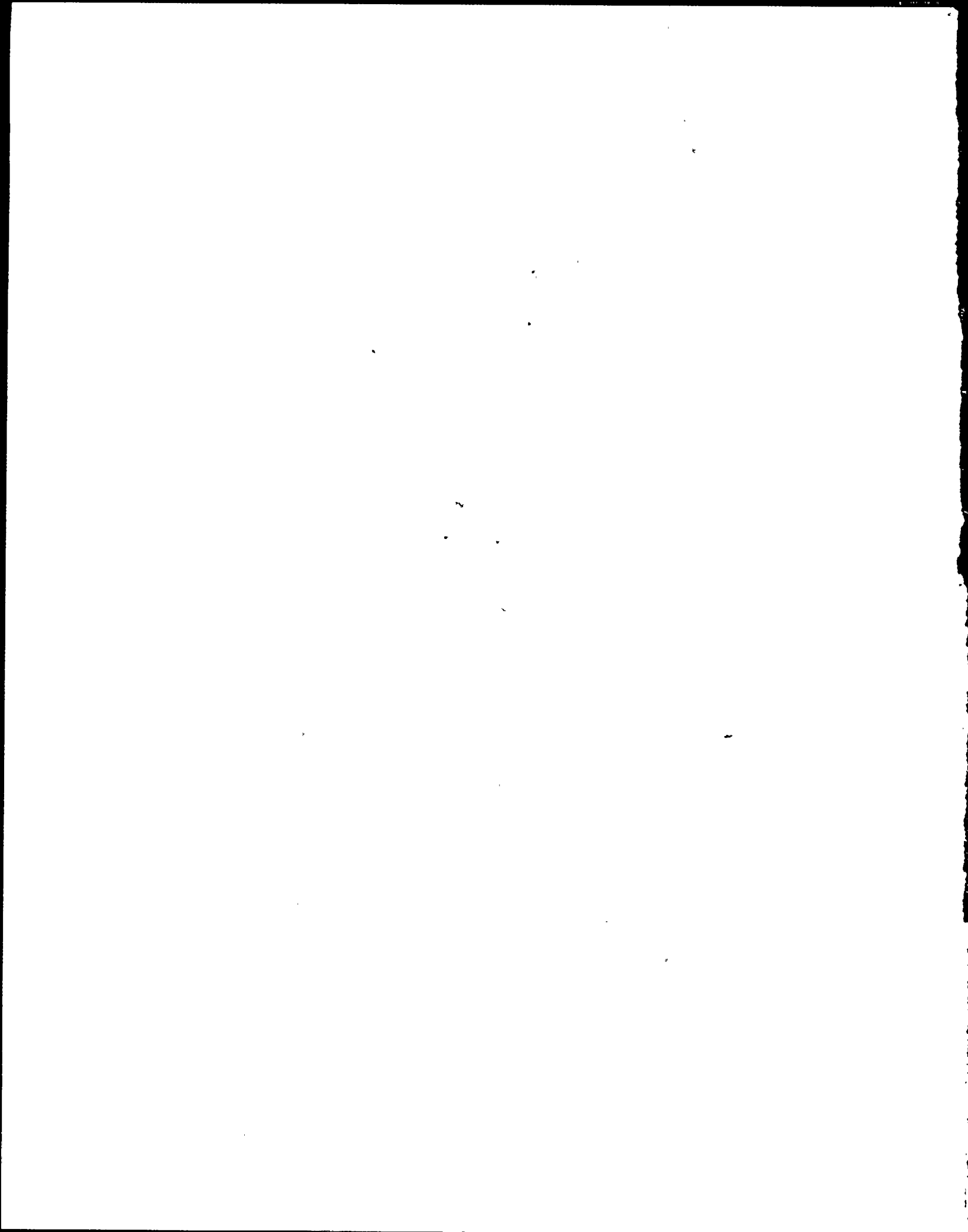
- S-11.1, Boric Acid Storage Tank Transfer to Batch Tank, PCN 79-1932.
- 79-1306 *S-30.5, Standby Auxiliary Feedwater Pump Valve Position Verification, PCN 79-1912.
- 79-1307 *SC-1.7B, Determination of Iodine or Particulate, PCN 79-1926.
- 79-1308 SC-1.7D, Emergency Radiation Monitoring Off-Site Survey Team #9D (Red), PCN 79-1873.
- SC-1.7E, Emergency Radiation Monitoring On-Site Survey Team #9E (Blue), PCN 79-1871.
- SC-1.7F, Emergency Radiation Monitoring Off-Site Survey Team #9F (Green), PCN 79-1874.
- SC-1.7G, Emergency Radiation Monitoring On-Site Survey Team #9G (Yellow), PCN 79-1872.
- SC-1.7H, Emergency Radiation Monitoring Off-Site Survey Team #9H, (Orange), PCN 79-1875.
- 79-1310 *T-44.2, Standby Auxiliary Feedwater (SAFW) System Alignment for Normal Operation, PCN 79-1910.
- 79-1319 E-1.1, Safety Injection System Actuation, PCN 79-2014.
- E-1.2, Loss of Coolant Accident, PCN 79-2013.
- E-1.3, Steam Line Break Accident, PCN 79-2012.



PORC
ITEM #

PROCEDURE

- E-1.4, Steam Generator Tube Rupture Accident, PCN 79-2011.
- E-1.5, Void Formation in the Reactor Coolant System, PCN 79-2015.
- 79-1322 *M-48.1, Isolation and Restoration of Bus #13, PCN 79-2017.
- *M-48.3, Isolation and Restoration of Bus #15, PCN 79-2018.
- 79-1323 *A-52.1, Shift Organization Relief and Turnover, PCN 79-2079.
- 79-1324 *AR-D-4, Pressurizer High Level, PCN 79-2063.
- 79-1325 *A-502, Plant Procedure Content and Format Requirements, PCN 79-1799.
- 79-1326 *P-1, Reactor Control and Protection System, PCN 79-2064.
- 79-1327 *A-1002, Qualification of Surveillance and Inspection Personnel, PCN 79-2016.
- 79-1328 *M-76.1, Installation and Test Sampling of Grouting, PCN 79-2009.
- 79-1329 *PT-2.4.1, Cold/Refueling Motor Operated Valve Surveillance (RHR System - 700 Valves), PCN 79-2006.
- 79-1330 *PT-2.9, Check Valve Exercising Quarterly Requirement, PCN 79-2010.
- 79-1331 *PT-5.20, Process Instrumentation Reactor Protection Channel Trip Test, (Channel 2), PCN 79-2008.



PORC
ITEM #

PROCEDURE

79-1332 *PT-6.1, Source Range Nuclear Instrumentation System, PCN 79-2005.

79-1333 SC-1.7E, Emergency Radiation Monitoring On-Site Survey Team #9E (Blue), PCN 79-1871.

SC-1.7G, Emergency Radiation Monitoring On-Site Survey Team #9G (Yellow), PCN 79-1872.

SC-1.7D, Emergency Radiation Monitoring Off-Site Survey Team #9D (Red), PCN 79-1873.

SC-1.7F, Emergency Radiation Monitoring Off-Site Survey Team #9F (Green), PCN 79-1874.

SC-1.7H, Emergency Radiation Monitoring Off-Site Survey Team #9H (Orange), PCN 79-1875.

79-1334 *SC-1.7B, Determination of Iodine or Particulate, PCN 79-1926.

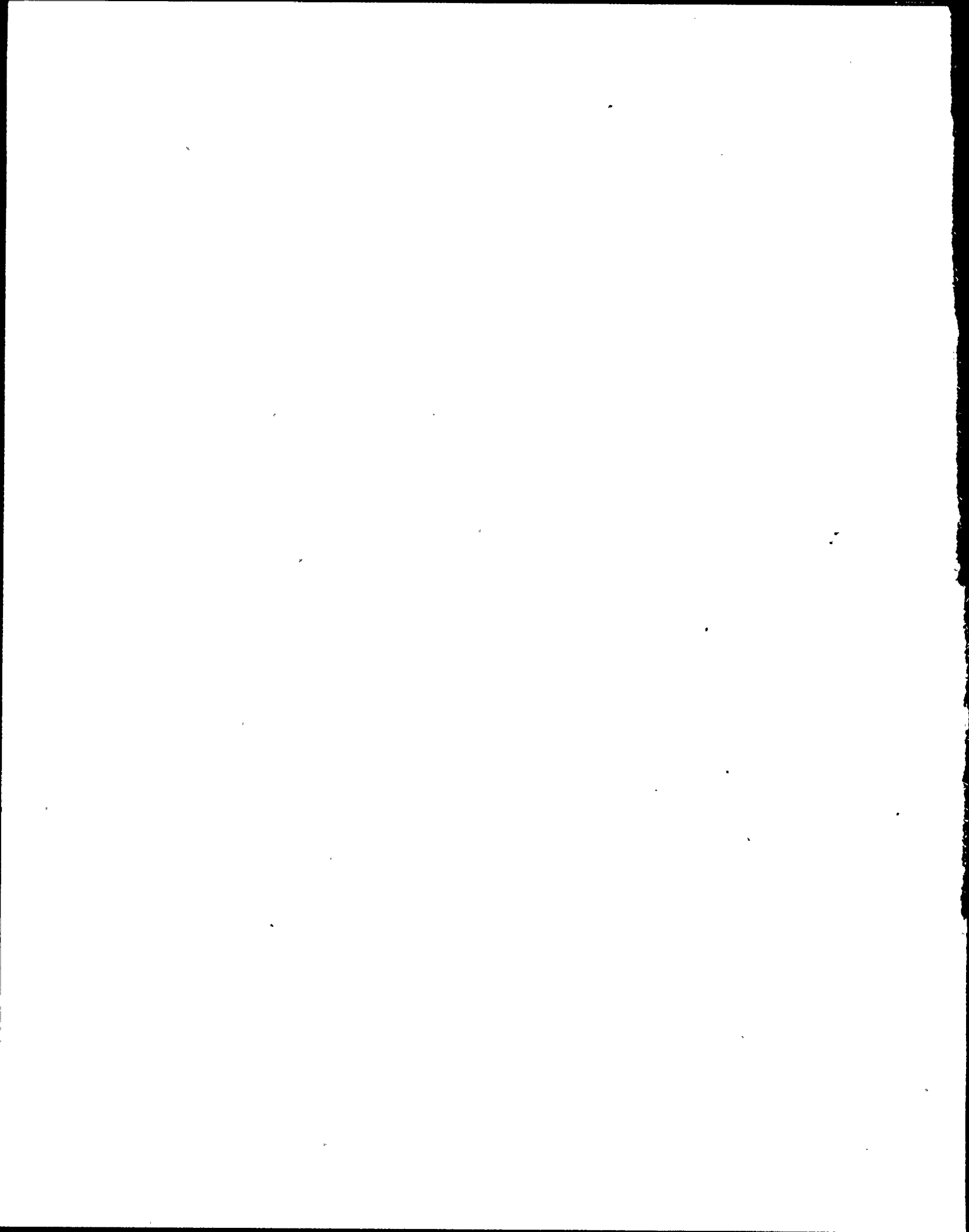
79-1335 SC-3.15.16, Fire Protection System Inspections, PCN 79-2023.

*S-16.16, Safety Injection Pump Isolation, PCN 79-1860.

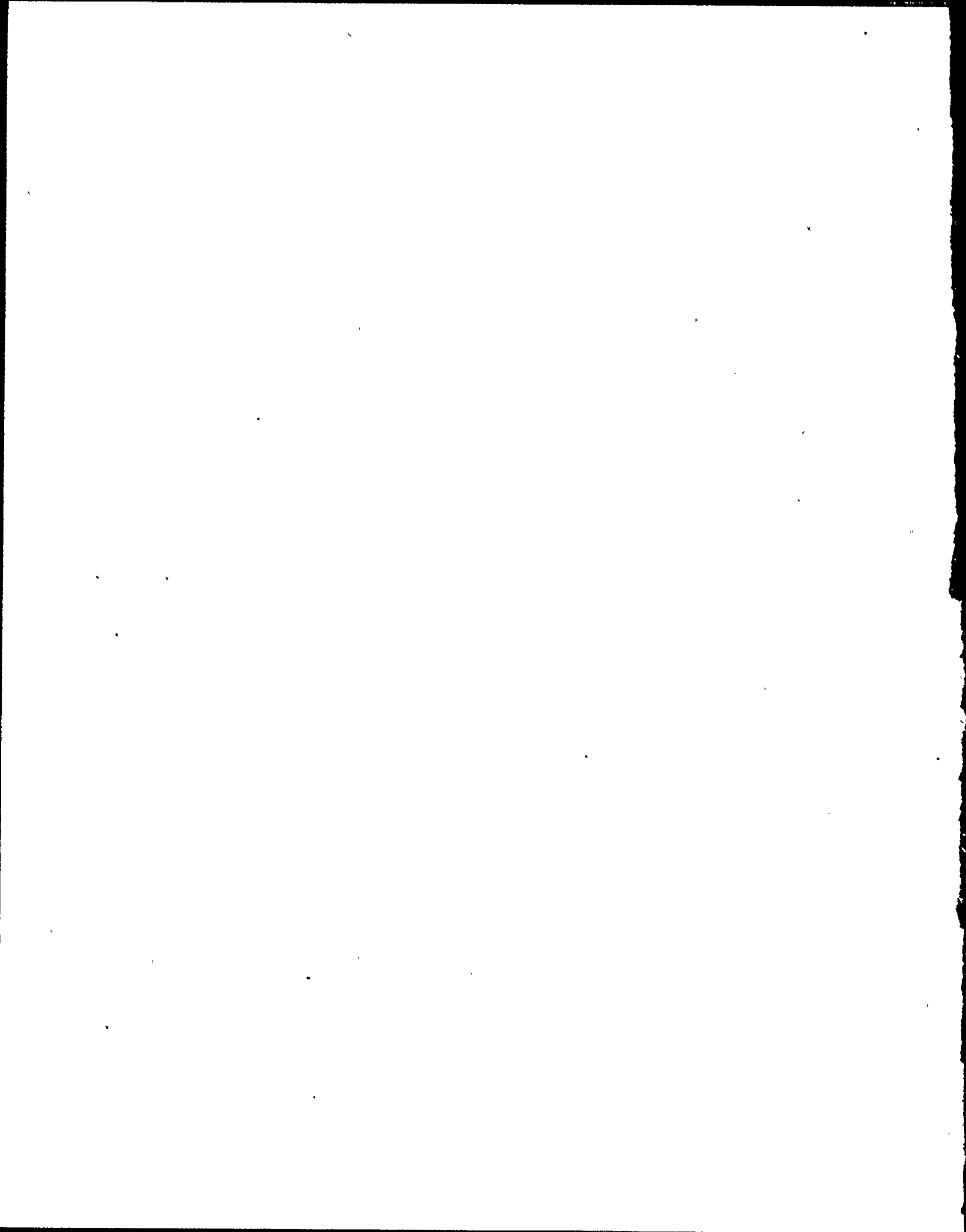
79-1337 PT-24.1, Spent Fuel Pit Charcoal Filter By-Pass Flow, PCN 79-2007.

PT-5.10, Process Instrumentation Reactor Protection Channel Trip Test, Channel 1, PCN 79-2004.

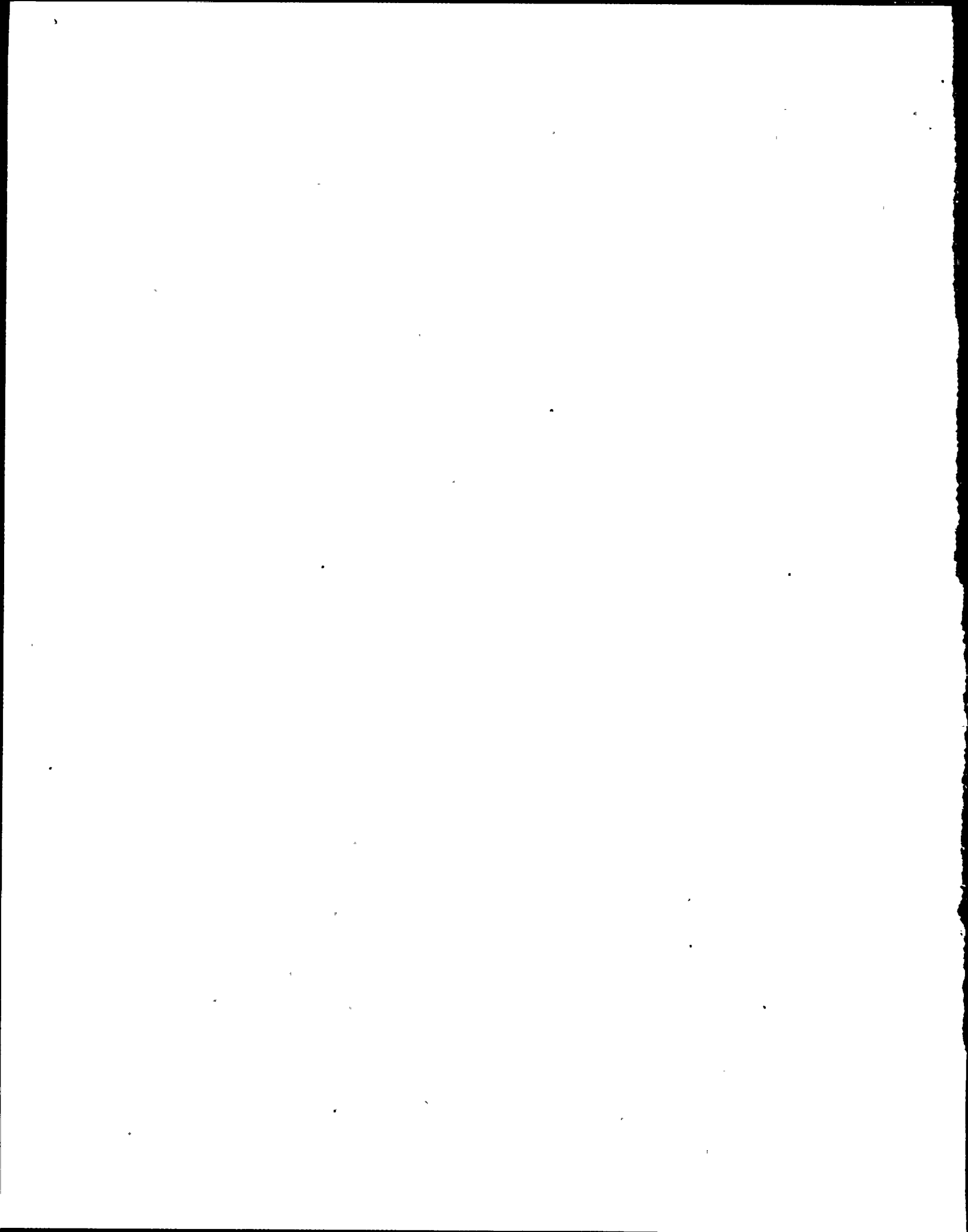
PT-2.8, Component Cooling Water Pump System, PCN 79-2075.



<u>FORC</u> <u>ITEM #</u>	<u>PROCEDURE</u>
<u>79-1338</u>	<u>GS-45.0, Vital Area Security Surveillance, PCN 79-1908.</u>
	<u>GS-46.0, Electronic Keycard System Testing Procedures, PCN 79-1931.</u>
	<u>ISI-10, Inspection of Stagnant, Oxygenated, Borated Systems, PCN 79-1911.</u>
	<u>M-37.62, Check Valve 7226, Inspection and Maintenance, PCN 79-1927.</u>
	<u>O-6.13, Daily Surveillance Log, PCN 79-1817.</u>
<u>79-1346</u>	<u>*EM-249, Removal of the Temporary Shield Wall Around The Refueling Water Storage Tank, PCN 79-2104.</u>
<u>79-1347</u>	<u>*EM-250, RWST Supports - Sample Removal, PCN 79-2128.</u>
<u>79-1348</u>	<u>*O-6.14, Monthly Surveillance Schedule, PCN 79-2003.</u>
	<u>*SC-3.15.17, Technical Specification Firewatch Posting, PCN 79-1929.</u>
<u>79-1349</u>	<u>*A-1105, Calibration Surveillance Program for Instrumentation/Equipment of Safety Related Components, PCN 79-2105.</u>
<u>79-1350</u>	<u>*EM-250, RWST Supports - Sample Removal, PCN 79-2129.</u>
<u>79-1351</u>	<u>*HP-7.8, Alarming Dosimeter Accuracy Test, PCN 79-2076.</u>
<u>79-1352</u>	<u>*HP-11.2, Iodine In Air - Charcoal Cartridge Method, PCN 79-2082.</u>
<u>79-1353</u>	<u>*HP-12.1, Usage of Respirators, PCN 79-2080.</u>



<u>PORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-1354</u>	<u>*HP-12.7, Constant Flow Breathing Air System Set Up, PCN 79-2081.</u>
<u>79-1355</u>	<u>*PT-2.7, Service Water System, PCN 79-2130.</u>
<u>79-1356</u>	<u>*PT-13.1.12, Containment Post-Accident (Recirc.) and/or Auxiliary Filter RTD Testing, PCN 79-2131.</u>
<u>79-1357</u>	<u>*PT-36, Standby Auxiliary Feedwater Pumps Flow Test, PCN 79-2127.</u>
<u>79-1358</u>	<u>*RD-8, Liquid Radwaste Compositing and Analysis, PCN 79-2077.</u>
<u>79-1359</u>	<u>*S-4.4, Spent Resin Removal to Shipping Casks, PCN 79-2103.</u>
<u>79-1360</u>	<u>*S-5, Operating Instruction S-5, Nuclear Sample Room Sampling System, PCN 79-2078.</u>
<u>79-1361</u>	<u>*S-30.4, Auxiliary Feedwater System Valve Position Verification, PCN's 79-2143 and 79-2144.</u>
<u>79-1373</u>	<u>*PT-27.1, Tendon Inspection and Lift Off Verification, PCN 79-2157.</u>
<u>79-1374</u>	<u>*M-82.1, Degreasing of Containment Tendon Canisters, PCN 79-2158.</u>
<u>79-1376</u>	<u>*CP-428.9, Pressurizer Level Recorder LR-428, PCN 79-2176.</u>
<u>79-1377</u>	<u>*RD-4, Vent Radiogas Background and Factor Determination, PCN 79-2074.</u>
<u>79-1378</u>	<u>*O-6.13, Daily Surveillance Log, PCN 79-2183.</u>



PORC
ITEM #

PROCEDURE

79-1379 *O-6.2; Main Control Board System Status Verification, PCN 79-2184.

79-1380 HP-7.12; PIC-6A Operation and Calibration; PCN 79-1957.

HP-7.13; RO-1 (Rad Owl) Operation and Calibration; PCN 79-1956.

M-82.2; Containment Tendon Greasing; PCN 79-2162.

PR-10.1; ITE Circuit Shield Undervoltage Relay Acceptance Test, PCN 79-2092.

79-1383 *A-52.4; Control of Limiting Conditions for Operating Equipment, PCN 79-2155.

79-1384 *A-1601; Corrective Action Report; PCN 79-2156.

79-1385 CP-128.0; Calibration and/or Maintenance of Charging Line Flow Channel "128"; PCN 79-1982.

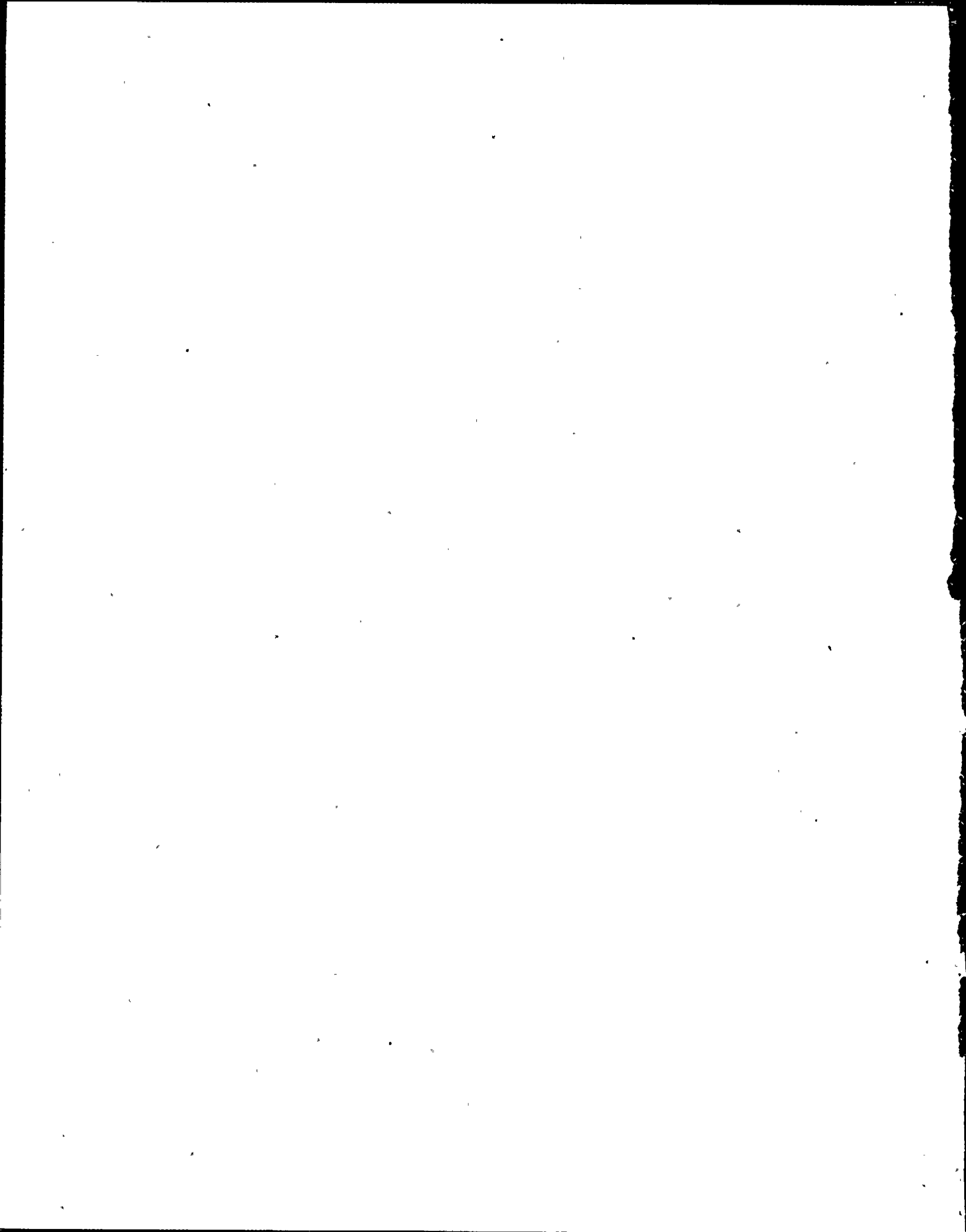
CP-420.4; Reactor Coolant Pressure Indicator PI-420 and Recorder PR-420; PCN 79-1972.

79-1386 CP-139.3; Volume Control Tank Level Transmitter LT-139; PCN 79-1984.

CP-172.3; Boric Acid Tank Level Transmitter LT-172; PCN 79-1964.

79-1387 *CP-128.3; Charging Line Flow Transmitter FT-128 and Local Indicator FI-128A; PCN 79-1983.

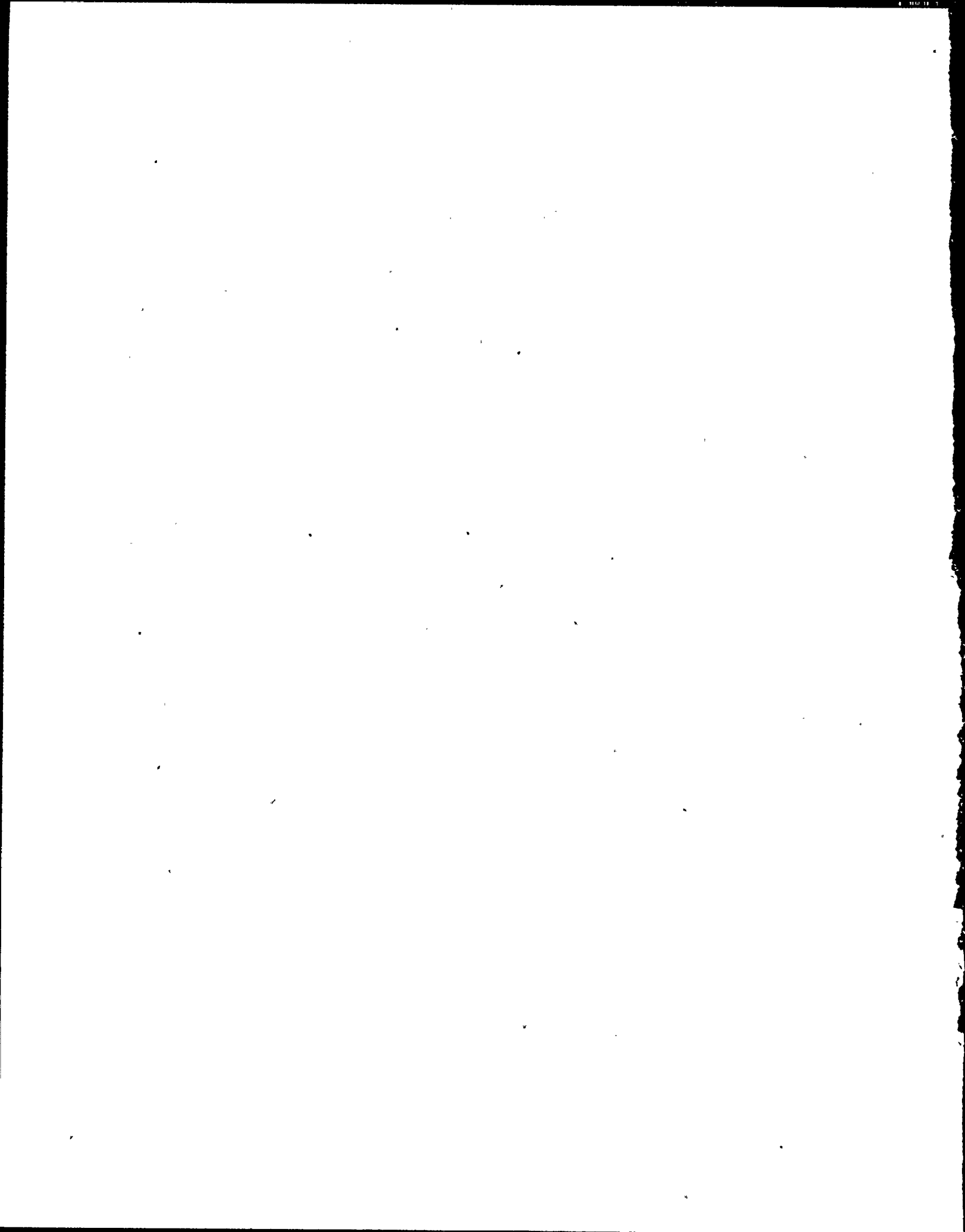
79-1388 *CP-171.3; Boric Acid Tank Level Transmitter LT-171; PCN 79-1968.



FORC
ITEM #

PROCEDURE

- 79-1389 *CP-171.7, Boric Acid Tank Level Indicator LT-171A, PCN 79-1960.
- 79-1390 CP-171.9, Boric Acid Tank Level Nitrogen Bubbler Supply Pressure Switch PS-171, PCN 79-1978.
- CP-172.9, Boric Acid Tank Level Nitrogen Bubbler Supply Pressure Switch PS-172, PCN 79-1977.
- 79-1391 CP-171.1, Defeat of Boric Acid Tank Level Channel "171", PCN 79-1967.
- CP-172.1, Defeat of Boric Acid Tank Level Channel "172", PCN 79-1963.
- CP-172.2, Reinstate of Boric Acid Tank Level Channel "172", PCN 79-1981.
- 79-1392 CP-171.5, Boric Acid Tank Level Bistable LC-171A/B, PCN 79-1969.
- CP-171.6, Boric Acid Tank Level Bistable LC-171C, PCN 79-1970.
- CP-172.5, Boric Acid Tank Level Bistable LC-172A/B, PCN 79-1965.
- CP-172.6, Boric Acid Tank Level Bistable LC-172C, PCN 79-1961.
- CP-250.0, Calibration and/or Maintenance of Ginna Station Meteorological Instrumentation, PCN 79-1980.
- 79-1393 *CP-250.6, Calibration and/or Maintenance of Wind Speed Voltage Signal Conditioner, PCN 79-1979.
- 79-1394 *CP-401.2, Reinstate T AVG Signal for Channel "1", PCN 79-1975.



PORC
ITEM #

PROCEDURE

79-1395 *CP-401.6, T AVG Bistable TC-401A, PCN 79-1976.

79-1396 *CP-401.9, T AVG Indicator TI-401, PCN 79-1974.

79-1397 CP-171.0, Calibration and/or Maintenance of Boric Acid Tank
Level Channel "171", PCN 79-1966.

CP-172.0, Calibration and/or Maintenance of Boric Acid Tank
Level Channel "172", PCN 79-1962.

CP-420.0, Calibration and/or Maintenance of Reactor Coolant
Pressure Channel 420, PCN 79-1971.

79-1398 *CP-428.4, Pressurizer Level Power Supply LQ-428, PCN
79-1985.

79-1399 *CP-428.7, Pressurizer Level Repeater LM-428B, PCN
79-1986.

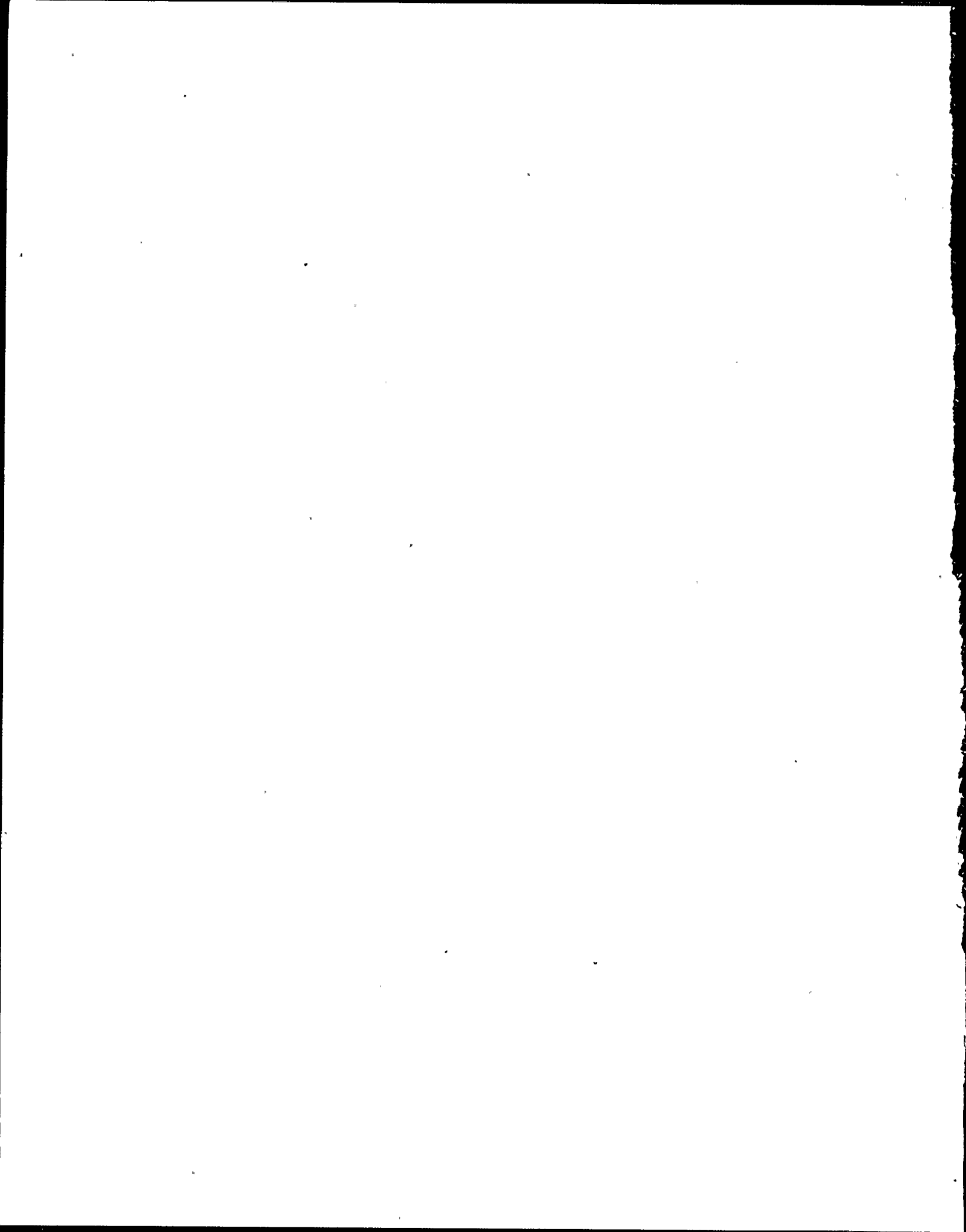
79-1400 *CP-428.9, Pressurizer Level Recorder LR-428, PCN
79-1987.

79-1401 O-1.1, Plant Heatup From Cold Shutdown to Hot
Shutdown, PCN 79-2185.

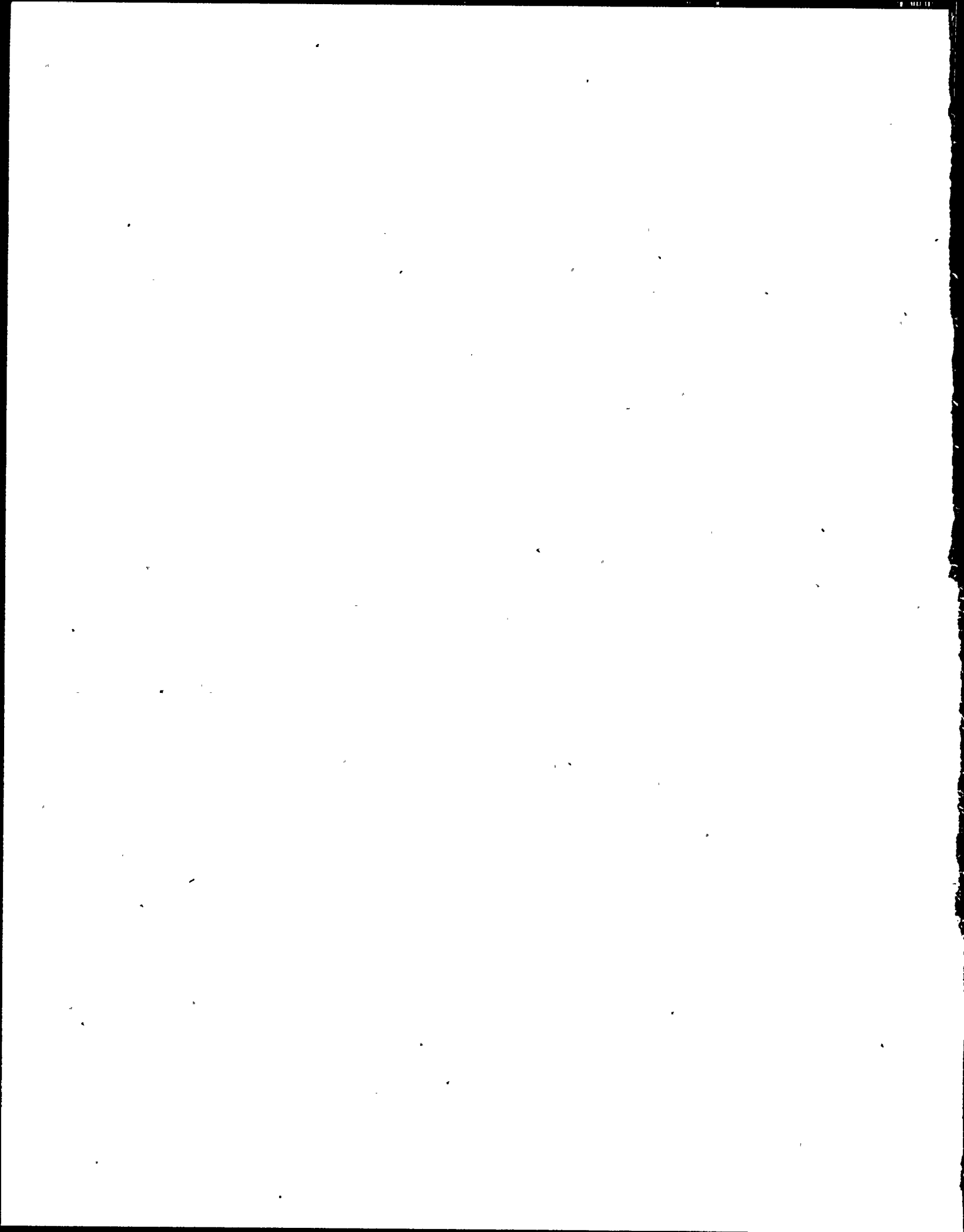
O-1.2, Plant From Hot Shutdown to Steady Load, PCN
79-2187.

O-2.1, Normal Shutdown to Hot Shutdown, PCN 79-2186.

79-1402 *PT-2.8, Component Cooling Water Pump System, PCN
79-2153.



<u>PORC</u> <u>ITEM #</u>	<u>PROCEDURE</u>
<u>79-1403</u>	<u>*S-12.2; Operator Action in the Event of Indication at Significant Increase in Leakage, PCN 79-2182.</u>
<u>79-1404</u>	<u>*SC-3.2.9; Immediate Action - Containment Vessel Fire, PCN 79-2154.</u>
<u>79-1416</u>	<u>*SM 77-1850.1; Fire Barrier Penetration Seals Installation, PCN 79-2247.</u>
<u>79-1417</u>	<u>*M-37.50; MOV-721 RHR to "B" Loop Maintenance and Inspection, PCN 79-2194.</u>
<u>79-1418</u>	<u>*S-4.2.5; Operating Instruction S-4.2.5, Release of Gas Decay Tank, PCN 79-2251.</u>
<u>79-1419</u>	<u>HP-7.10; Area Monitor Calibration Emergency Center, PCN 79-1959.</u>
	<u>HP-7.11; Teletector Operation and Calibration, PCN 79-1958.</u>
	<u>HP-7.14; RD-2A Operation and Calibration, PCN 79-1955.</u>
	<u>HP-7.15; XETEX 302A High Level Probe Operation and Calibration, PCN 79-2033.</u>
	<u>HP-7.16; XETEX 302B High Level Probe Operation and Calibration, PCN 79-2034.</u>
	<u>RD-13; Offsite Dose Calculation Manual, PCN 79-2179.</u>
<u>79-1421</u>	<u>*A-1, Radiation Control Manual, PCN's 79-1804 and 79-1914.</u>
<u>79-1422</u>	<u>CP-5.0; Calibration Alignment of T Avg. and Delta T at Zero Power Channel 1 Loop A, Unit 1, PCN 79-2060.</u>



PORC
ITEM #

PROCEDURE

CP-5.10, T Avg. and Delta T Alignment at 70% Power or Greater Loop A, Unit 1, PCN 79-2057.

CP-6.0, Calibration Alignment of T Avg. and Delta T at Zero Power Channel 2 Loop A, Unit 1, PCN 79-2061.

CP-6.10, T Avg. and Delta T Alignment at 70% Power or Greater Loop A, Unit 2, PCN 79-2056.

CP-7.0, Calibration Alignment of T Avg. and Delta T at Zero Power Channel 3 Loop B, Unit 1, PCN 79-2059.

CP-7.10, T Avg. and Delta T Alignment at 70% Power or Greater Loop B, Unit 1, PCN 79-2055.

CP-8.0, Calibration Alignment of T Avg. and Delta T at Zero Power Channel 4, PCN 79-2058.

CP-8.10, T Avg. and Delta T Alignment at 70% Power or Greater Loop B, Unit 2, PCN 79-2054.

CP-31.3, Calibration of Source Range N-31, PCN 79-2029.

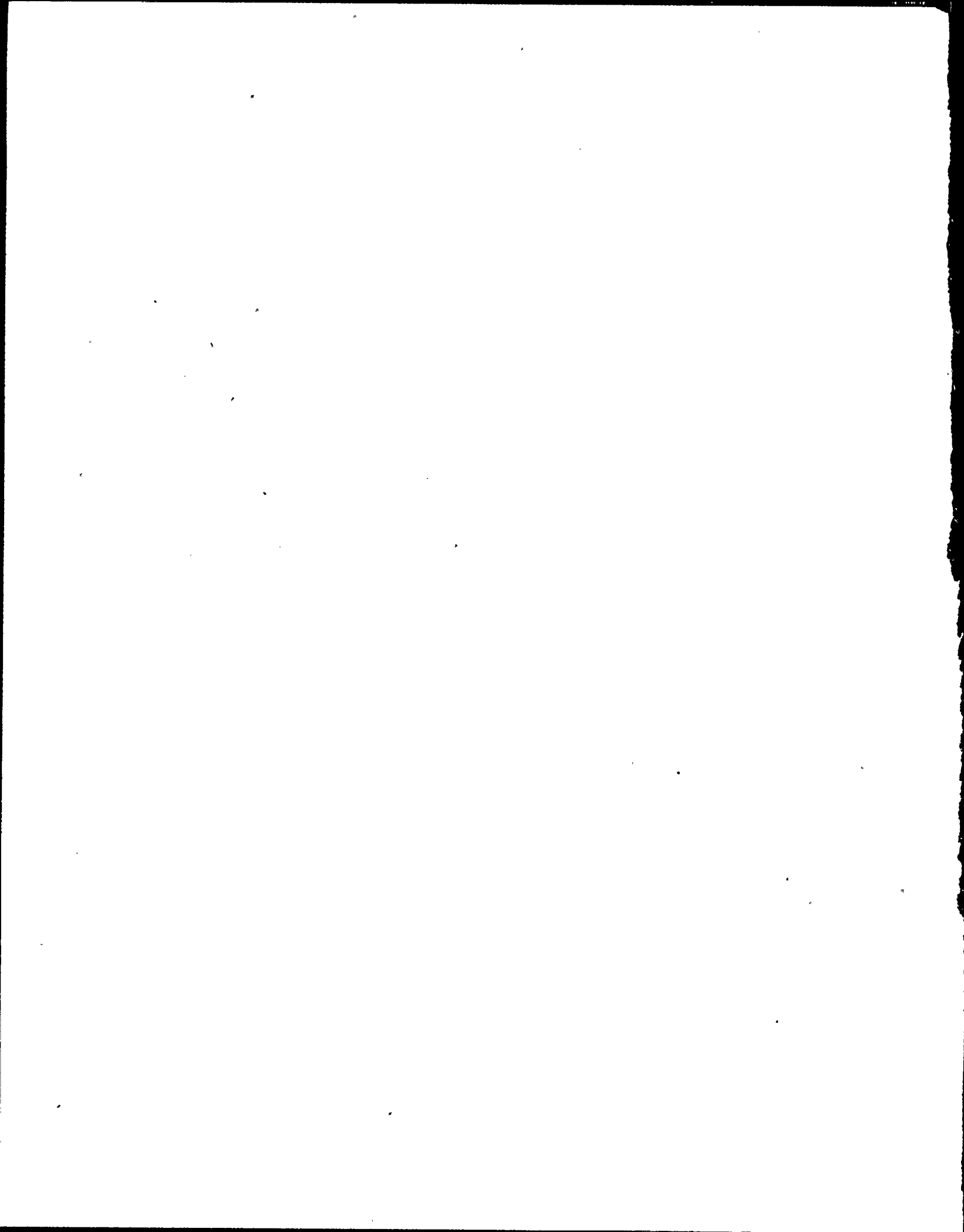
79-1423

CP-12.0, Rod Insertion Limit and Delta T Reactor Control Rack, PCN 79-2025.

CP-12.19, Rod Insertion Limit Medium Current Selector TW-405X, PCN 79-2026.

CP-12.22, Rod Insertion Limit Computer Unit TM-405J, PCN 79-2027.

CP-422.0, Calibration and/or Maintenance of Pressurizer Spray Line Temperature "A" Loop, PCN 79-1994.



FORC
ITEM #

PROCEDURE

CP-423.0, Calibration and/or Maintenance of Pressurizer Spray Line Temperature "B" Loop, PCN 79-1993.

CP-423.3, Calibration and/or Maintenance of Pressurizer Spray Line Temperature Indicator, PCN 79-1991.

CP-424.0, Calibration and/or Maintenance of Pressurizer Liquid Temperature, PCN 79-1989.

CP-425.0, Calibration and/or Maintenance of Pressurizer Vapor Temperature TE-425, PCN 79-1988.

CP-429.1, General Defeat Procedure Pressurizer Channel 429, PCN 79-2051.

CP-931.0, Calibration and/or Maintenance of Spray Additive Tank Level Channel 931, PCN 79-2038.

CP-931A, Calibration of RHR Refueling Water Return Line Flow Transmitters and Associated Indicators, PCN 79-2040.

CP-931B, Calibration of RHR Refueling Water Return Line Flow Transmitters and Associated Indicators, PCN 79-2039.

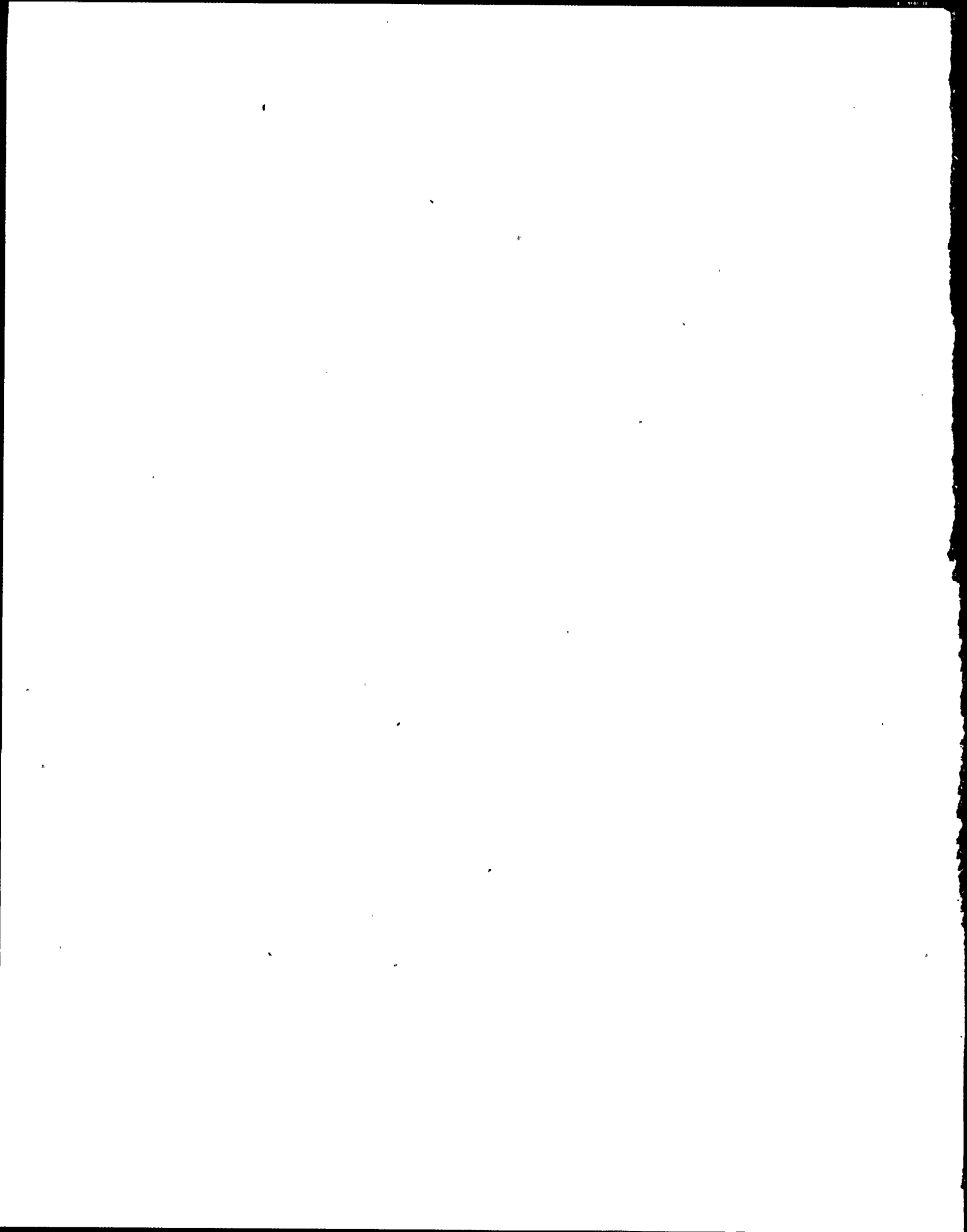
79-1424

CP-31.1, General Defeat Procedure for N-31, PCN 79-2036.

CP-32.1, General Defeat Procedure for N-32, PCN 79-2031.

CP-421.1, Calibration and/or Maintenance Pressurizer Surge Line Temperature R/I Converter TM-421, PCN 79-1999.

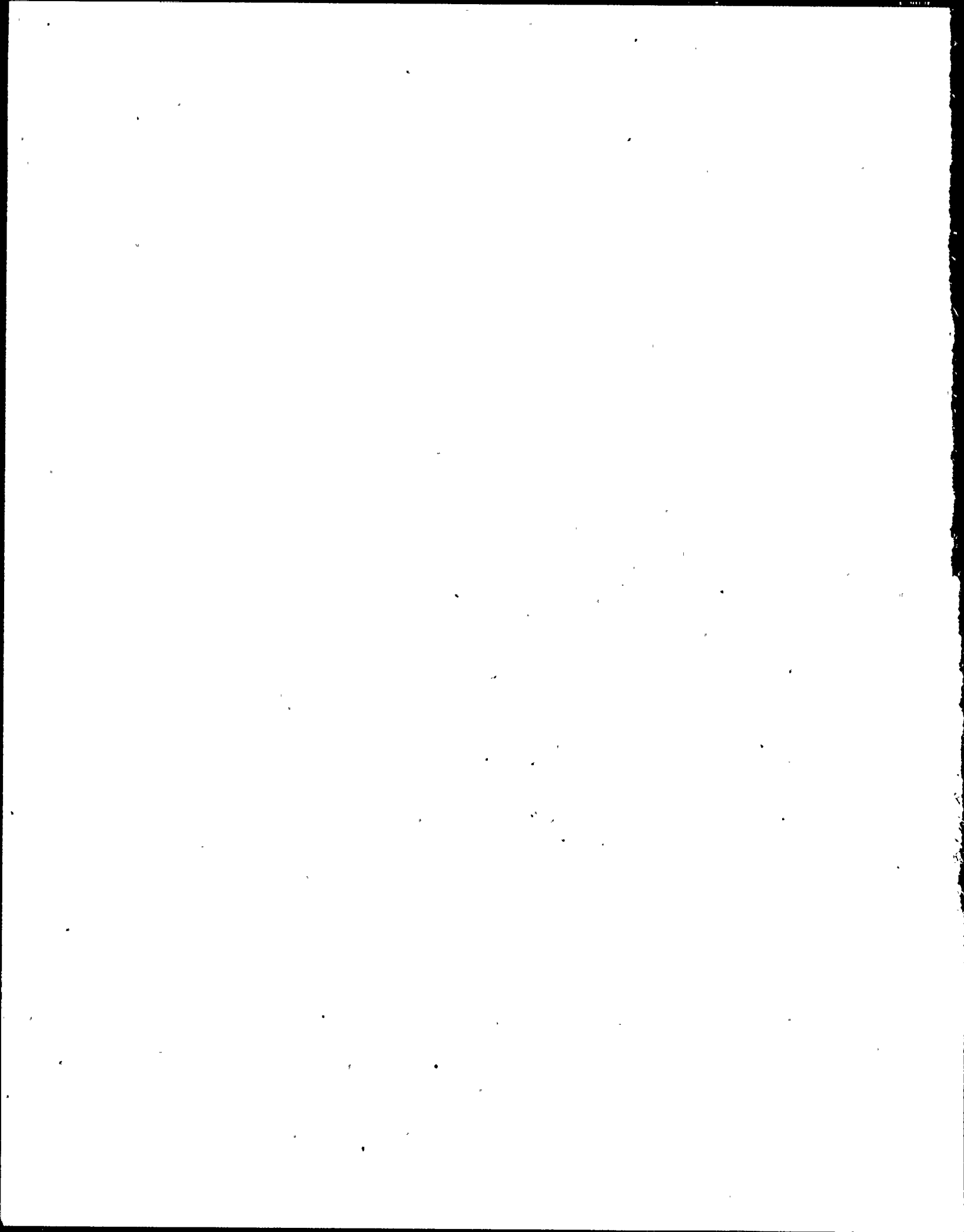
CP-421.2, Pressurizer Surge Line Temperature Bistable TC-421, PCN 79-2000.



PORC
ITEM #

PROCEDURE

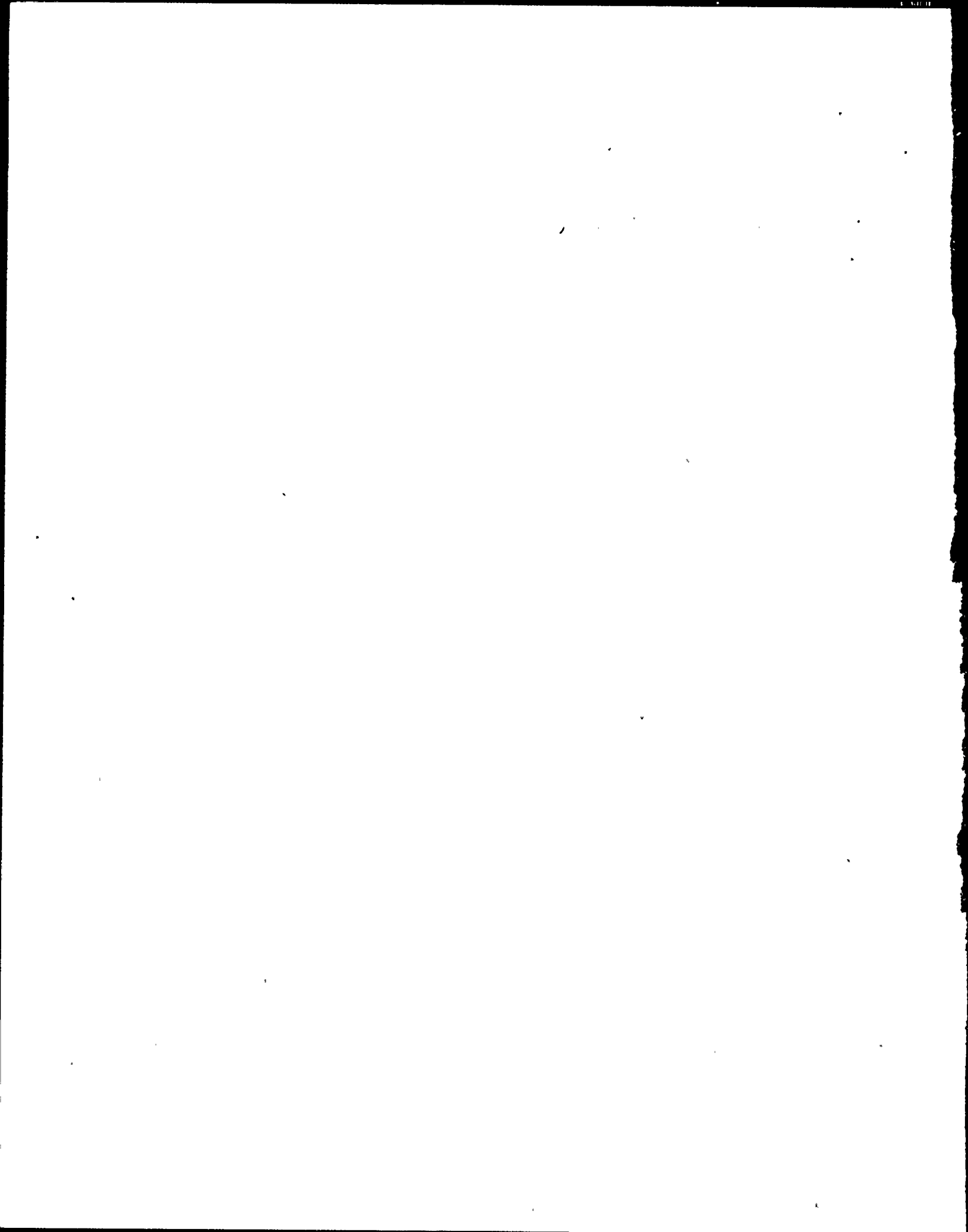
- CP-423.1; Calibration and/or Maintenance of Pressurizer Spray Line Temperature R/I Converter, PCN 79-2052.
- CP-630.1; RHR Loop Reactor Coolant Inlet Temperature R/I Converter TM-630, PCN 79-2043.
- 79-1425 CP-31.4; Calibration of Source Range N-31 Bistable Relay Drivers, PCN 79-2035.
- CP-32.3; Calibration of Source Range N-32, PCN 79-2028.
- CP-32.4; Calibration of Source Range N-32 Bistable Relay Drivers, PCN 79-2032.
- CP-424.2; Calibration and/or Maintenance of Pressurizer Liquid Temperature Bistable TC-424, PCN's 79-1990 and 79-2180.
- 79-1426 *CP-35.0; Intermediate Range N-35 Calibration and/or Maintenance, PCN 79-2030.
- 79-1427 CP-420.1; Reactor Coolant Pressure Power Supply PQ-420, PCN 79-1997.
- CP-428.0; Calibration and/or Maintenance of Pressurizer Level Channel "428", PCN 79-2002.
- CP-924.0; Calibration and/or Maintenance of Safety Injection Flow Loop "B", PCN 79-2042.
- CP-925.0; Calibration and/or Maintenance of Safety Injection Flow Loop "A", PCN 79-2041.
- 79-1428 *CP-420.2; Reactor Coolant Pressure Transmitter PT-420, PCN 79-1998.



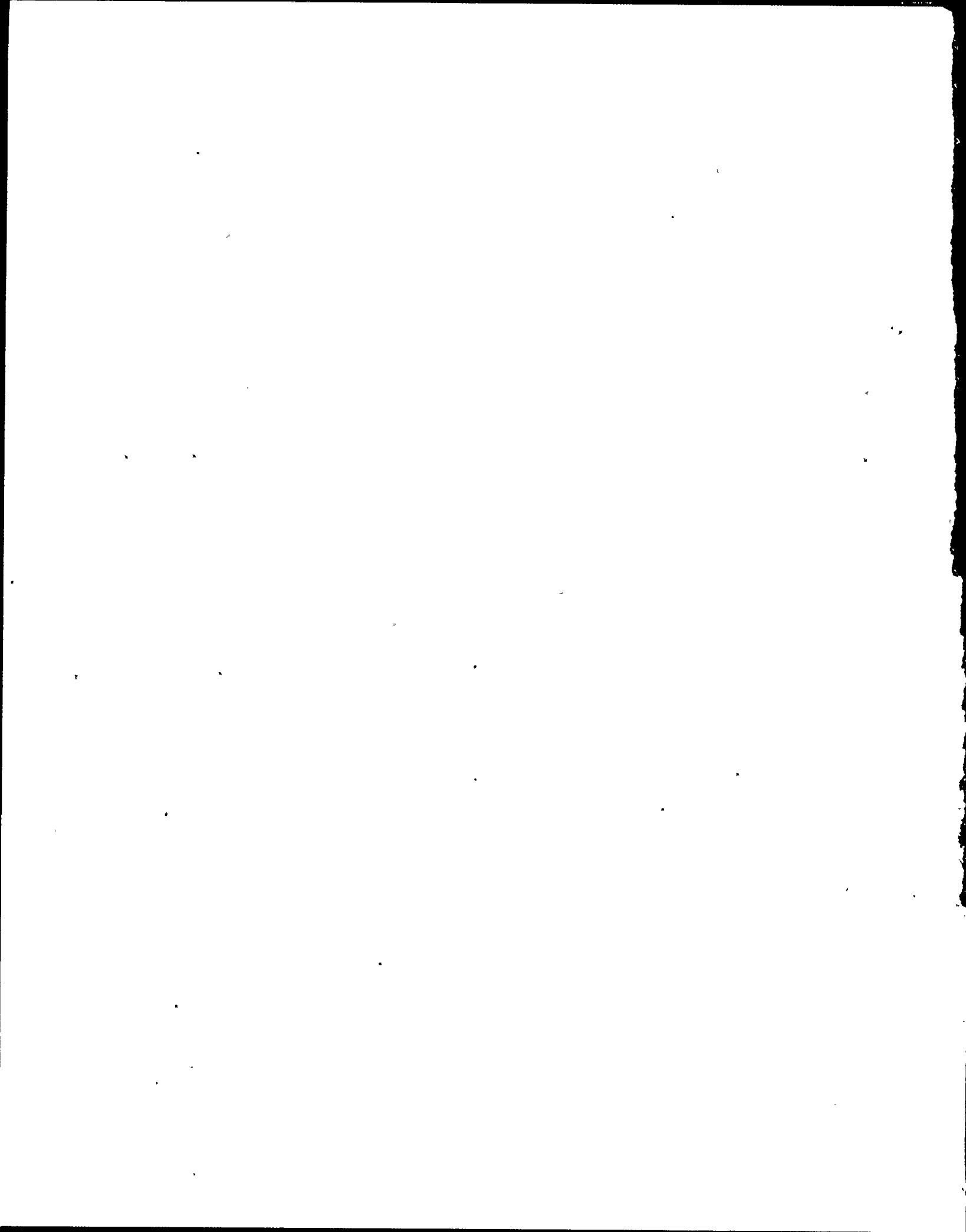
FORC
ITEM #

PROCEDURE

- 79-1429 *CP-421.3, Calibration and/or Maintenance of Pressurizer Surge Line Temperature Indicator TI-421, PCN 79-2001.
- 79-1430 CP-422.2, Calibration and/or Maintenance of Pressurizer Spray Line Temperature Bistable TC-422, PCN 79-1996.
- CP-422.3, Calibration and/or Maintenance of Pressurizer Spray Line Temperature Indicator TI-422, PCN 79-1995.
- CP-423.2, Calibration and/or Maintenance of Pressurizer Spray Line Temperature Bistable TC-423, PCN 79-1997.
- 79-1431 CP-426.0, Calibration and/or Maintenance of "426" Pressurizer Level Channel, PCN 79-2069.
- CP-427.0, Calibration and/or Maintenance of Pressurizer Level Channel "427", PCN 79-2065.
- CP-428.0, Calibration and/or Maintenance of Pressurizer Level Channel "428", PCN 79-2067.
- 79-1432 CP-426.0, Calibration and/or Maintenance "426" Pressurizer Level Channel, PCN 79-2171.
- CP-427.0, Calibration and/or Maintenance of Pressurizer Level Channel "427", PCN 79-2172.
- CP-428.0, Calibration and/or Maintenance of Pressurizer Level Channel "428", PCN 79-2177.
- 79-1433 *CP-429.2, General Reinstatement Procedure Pressurizer Channel 429, PCN 79-2050.



<u>FORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-1434</u>	<u>*CP-429.11, Pressurizer Pressure Repeater PM-429C, PCN 79-2053.</u>
<u>79-1435</u>	<u>CP-464.10, Steam Flow Indicator FI-464, PCN 79-2049.</u> <u>CP-466.9, Feedwater Flow Indicator FI-466, PCN 79-2048.</u>
<u>79-1436</u>	<u>*CP-467.10, Feedwater Flow Recorder FR-465, PCN 79-2047.</u>
<u>79-1437</u>	<u>CP-478.2, Reinstate of "B" Steam Generator Pressure Channel "478", PCN 79-2046.</u> <u>CP-482.2, General Reinstate of "A" Steam Generator Pressure Channel 482, PCN 79-2045.</u>
<u>79-1438</u>	<u>CP-482.0, Calibration and/or Maintenance of Steam Generator Pressure Channel "482", PCN 79-2037.</u> <u>CP-483.0, Calibration and/or Maintenance of Steam Generator Pressure Channel "483", PCN 79-2044.</u>
<u>79-1439</u>	<u>*CP-924.1, Calibration and/or Maintenance of Safety Injection Flow Loop "A" Transmitter FT-924, PCN 79-2024.</u>
<u>79-1440</u>	<u>E-4.1, Safeguard Buses Low Voltage Condition, PCN 79-2191.</u> <u>E-4.2, RG&E Low System Frequency Condition, PCN 79-2192.</u> <u>E-6.1, Loss of Component Cooling During Power Operation, PCN 79-2188.</u> <u>E-6.2, Loss of Component Cooling While Plant is Shutdown, PCN 79-2189.</u>



PORC
ITEM #

PROCEDURE

E-6.3, Loss of Component Cooling During Recirculation Phase of Accident, PCN 79-2190.

E-14, High Reactor Coolant Activity, PCN 79-2198.

E-17.1, Reactor Coolant Drain Tank Pump Operation for Core Cooling, PCN 79-2199.

E-38.1, Operator Actions to be Taken in the Event of a Loss of the Service Water Return Line Outside the Auxiliary Building, PCN 79-2200.

79-1441 *HP-4.3, Work Permit Use, PCN 79-2249.

79-1442 *ISI-1.2.1, Quality Group A Inservice Inspection, PCN 79-2195.

79-1443 *M-40.8, Functional Testing of Hydraulic Snubbers, PCN 79-2246.

79-1444 O-1.1C, Precritical Technical Specification Check Sheet, PCN 79-2231.

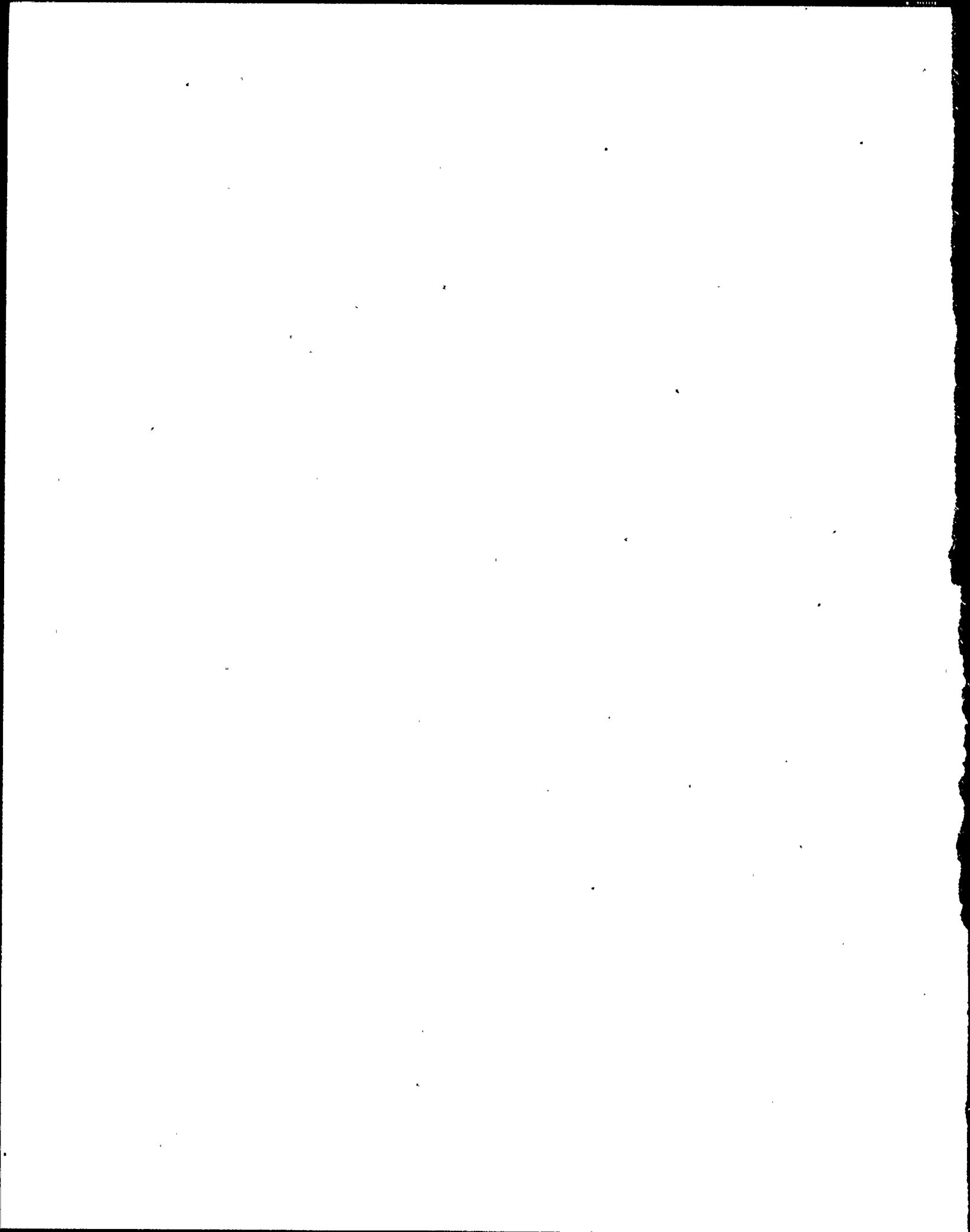
O-1.1D, Pre-heatup Plant Requirement Check List, PCN 79-2230.

79-1445 *O-6.7, Weekly Alarm Status Check, PCN 79-2252.

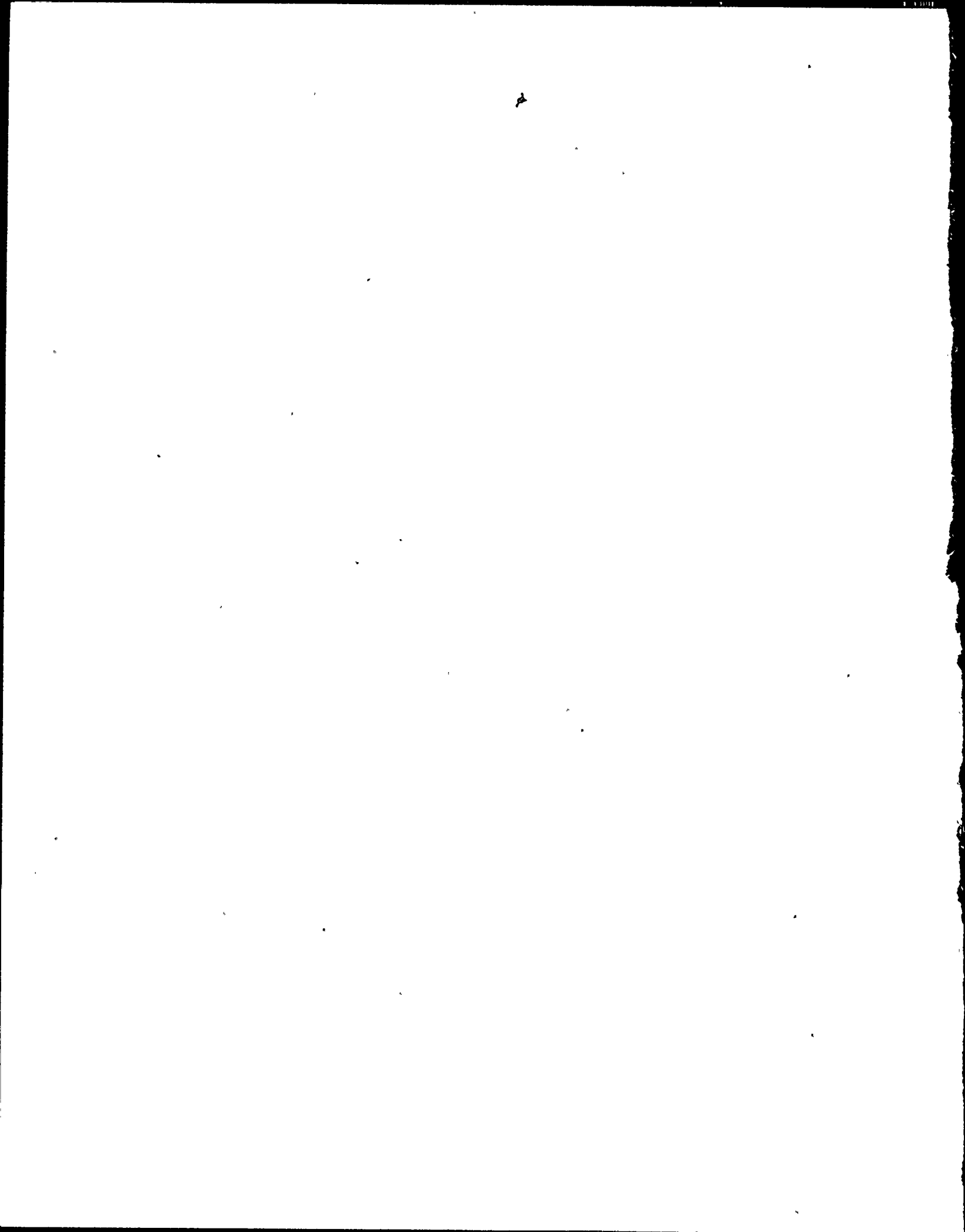
79-1446 *PT 2.1, Safety Injection System Pumps, PCN 79-2181.

79-1448 *S-3.6, Auxiliary Building Heat Tracing System Operation, PCN 79-2178.

79-1449 *S-12.4, RCS Leakage Surveillance Record Instructions, PCN 79-2196.



<u>PORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-1450</u>	<u>*T-41A, Alignment of Auxiliary Feedwater System Prior to Power Operation, PCN 79-2232.</u>
<u>79-1461</u>	<u>*EM-251, Rock Anchor Coupling Enclosure Broken Bolt Repair, PCN 79-2284.</u>
<u>79-1464</u>	<u>*A-102.10, Health Physics Training and Responsibility Limits, PCN 79-2248.</u>
<u>79-1465</u>	<u>*GS-26.0, Security Key and Lock Controls, PCN 79-2285.</u>
<u>79-1466</u>	<u>*ISI-2.1.6A, Quality Group B Inservice Inspection, PCN 79-2283.</u>
<u>79-1467</u>	<u>*S-3.4F, Boron Recycle Process Shutdown, PCN 79-2304.</u>
<u>79-1468</u>	<u>CP-426.5, Pressurizer Level Bistable LC-426 A/B, PCN 79-2070.</u>
	<u>CP-428.5, Pressurizer Level Bistable LC-428 A/E, PCN 79-2068.</u>
	<u>CP-427.5, Pressurizer Level Bistable LC-427 A/C, PCN 79-2066.</u>
<u>79-1469</u>	<u>CP-406.2, Reinstate Delta T Signal for Channel 2, PCN 79-2106.</u>
	<u>CP-407.2, Reinstate Delta T Signal for Channel 3, PCN 79-2107.</u>
	<u>CP-408.2, Reinstate Delta T Signal for Channel 4, PCN 79-2108.</u>
	<u>CP-134.1, Letdown Line Flow Transmitter FT-134, PCN 79-2109.</u>



FORC
ITEM #

PROCEDURE

CP-134.2, Letdown Line Flow Power Supply FQ-134, PCN 79-2110.

CP-134.3, Letdown Line Flow Square Root Converter FM-134, 79-2111.

CP-134.4, Letdown Line Flow Bistable FC-134, PCN 79-2112.

CP-134.5, Letdown Line Flow Indicator FI-134, PCN 79-2113.

CP-405.2, Reinstate Delta T Signal for Channel 1, PCN 79-2119.

CP-405.20.1, Defeat of Delta T Setpoint 2, Channel 1, PCN 79-2134.

CP-408.20.1, Defeat of Delta T Setpoint 2, Channel 4, PCN 79-2136.

CP-406.20.1, Defeat of Delta T Setpoint 2, Channel 2, PCN 79-2137.

CP-408.10.1, Defeat of Delta T Setpoint 1, Channel 4, PCN 79-2138.

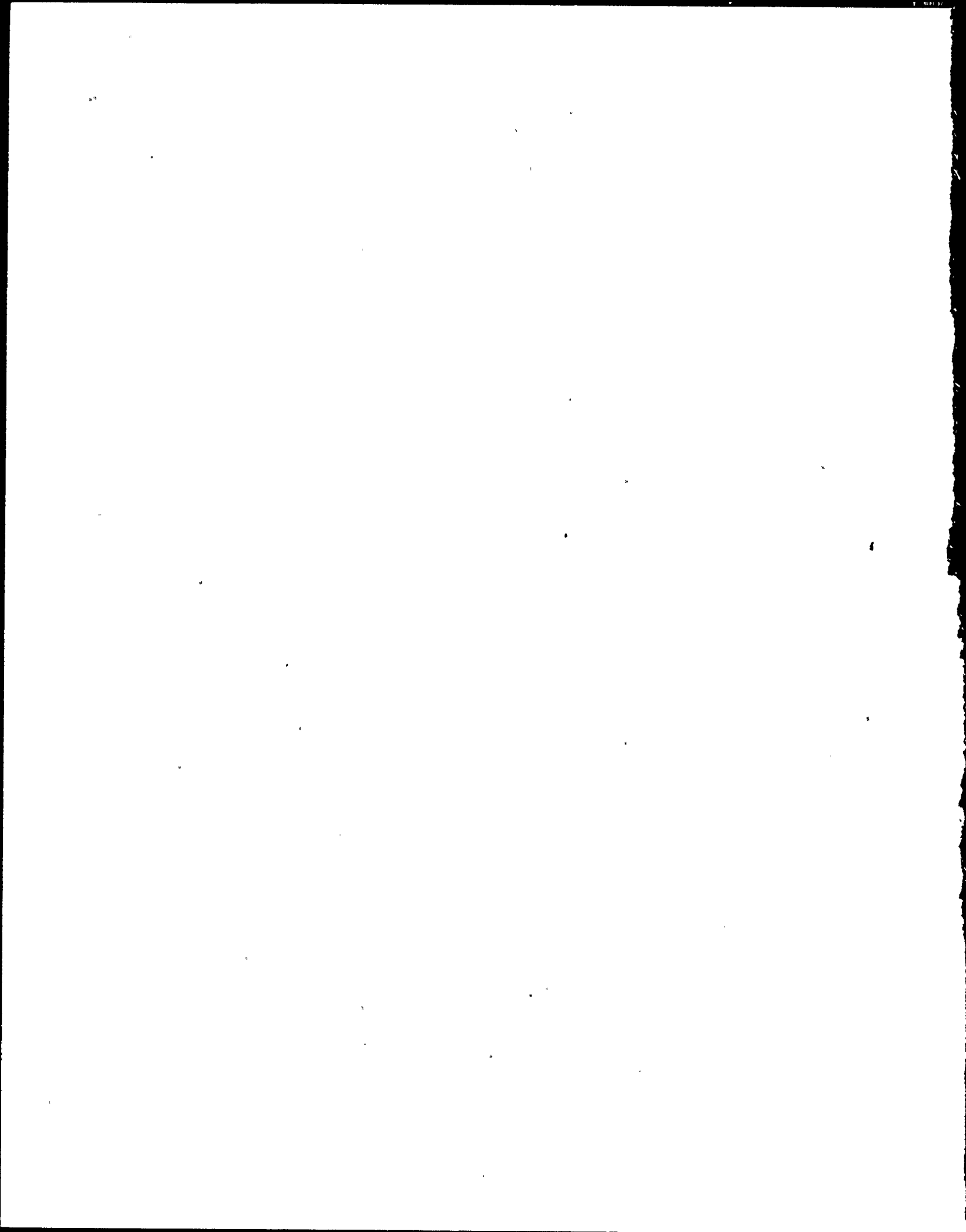
79-1470

CP-42.1, General Defeat Procedure For N-2, PCN 79-2095.

CP-43.1, General Defeat Procedure for N-43, PCN 79-2096.

CP-44.1, General Defeat Procedure for N-44, PCN 79-2097.

CP-41.1, General Defeat Procedure for N-41, PCN 79-2098.



FORC
ITEM #

PROCEDURE

CP-41.2, General Reinstatement Procedure for N-41, PCN 79-2099.

CP-42.2, General Reinstatement Procedure for N-42, PCN 79-2100.

CP-43.2, General Reinstatement Procedure for N-43, PCN 79-2101.

CP-44.2, General Reinstatement Procedure for N-44, PCN 79-2102.

CP-404.2, Reinstatement T AVG Signal for Channel "4", PCN 79-2123.

CP-403.2, Reinstatement T AVG Signal for Channel "3", PCN 79-2124.

CP-402.2, Reinstatement T AVG Signal for Channel "2", PCN 79-2125.

CP-401.5, T AVG (3 SEC) Lag Unit TM - 401 BB, PCN 79-2115.

79-1471 *CP-401.5, T AVG (3 SEC) Lag Unit TM - 401 BB, PCN 79-2115.

79-1472 *CP-209.4, Nuclear Measurements Corporation Air Monitor, PCN 79-2114.

79-1473 CP-406.10, Calibration and/or Maintenance of Delta TSP1 Channel 2, PCN 79-2140.

CP-405.10, Calibration and/or Maintenance of Delta TSP1 Channel 1, PCN 79-2139.

79-1474 *CP-37.0, Calibration and/or Maintenance of Comparator Drawer N-37, PCN 79-2093.

PORC
ITEM #

PROCEDURE

- 79-1475 CP-401.7, T AVG Repeater TM-401C, PCN 79-2116.
- CP-401.8, T AVG Repeater TM-401W, PCN 79-2117.
- CP-401.6, T AVG Bistable TC-401A, PCN 79-2126.
- CP-406.20.6, Delta T Setpoint 2 Repeater TM-402S,
PCN 79-2142.
- 79-1476 *CP-406.10.1, Defeat of Delta T Setpoint 1, Channel 2,
PCN 79-2135.
- 79-1477 CP-405.10.1, Defeat of Delta T Setpoint 1, Channel 1,
PCN 79-2133.
- CP-37.1, Low Voltage Power Supply NQ-401 Adjustments,
PCN 79-2094.
- CP-406.20.0, Calibration and/or Maintenance of Delta
TSP2, Channel 2, PCN 79-2141.
- 79-1480 *E-29.3, Standby Auxiliary Feedwater Pump Operation, PCN 79-
2229.
- 79-1490 *PT-36, Standby Auxiliary Feedwater System Flow Check, PCN 79-2419.
- 79-1491 *EM-252, Pipe Support Rework for System (Without Anchor
Bolt Rework), PCN 79-2402.
- 79-1492 *EM-154, Steam Generator Handhole Cover Gasket Leak Repair Using
Furmanite Process, PCN 79-2423.

<u>PORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-1493</u>	<u>*EM-253, Replace Reset Button for Containment Isolation and Key Operated Switch for Ventilation Isolation with a Key Operator Push Button, PCN 79-2424.</u>
<u>79-1494</u>	<u>*CP-216.3, Replacement of RMS Channel R-16 Detector, PCN 79-2430.</u>
<u>79-1495</u>	<u>*PC-1.1, Primary Coolant Analysis Schedule and Limits, PCN 79-2416.</u>
<u>79-1499</u>	<u>*CP-113.0, Immediate Boration Flow Channel 113, Calibration and/or Maintenance, PCN 79-2084.</u>
	<u>*CP-113.1, Immediate Boration Magnetic Flow Converter FM-113, PCN 79-2085.</u>
	<u>*CP-113.2, Immediate Boration Flow Integrator YM-113, PCN 79-2086.</u>
	<u>*CP-113.3, Immediate Boration Flow Counter YIC-113, PCN 79-2087.</u>
	<u>*CP-113.4, Immediate Boration Flow Indicator FI-113, PCN 79-2088.</u>
	<u>*SC-3.15.9, Exterior Hydrant Freeze Protection, PCN 79-2326.</u>
<u>79-1500</u>	<u>*A-25.1, Ginna Station Event Report, PCN 79-2398.</u>
<u>79-1501</u>	<u>*A-54.6, Health Physic's Note, PCN 79-2399.</u>
<u>79-1502</u>	<u>,*A-1702, Record Storage Facilities and Equipment, PCN 79-2382.</u>
<u>79-1503</u>	<u>*CP-10.1, Defeat of Delta Flux Signal Channel 1, PCN 79-2307.</u>
<u>79-1504</u>	<u>*CP-410.1, Reactor Coolant Temperature, Cold Leg, R/I TM-410, PCN 79-2145.</u>

<u>FORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-1505</u>	<u>*EM-246, Furmanite Repair Pressurizer Spray Valve 431A and B, PCN 79-2429.</u>
<u>79-1506</u>	<u>*HP-1.1, Issuing Personnel Dosimeters, PCN 79-2425.</u>
<u>79-1507</u>	<u>*HP-1.6, Neutron Exposure, PCN 79-2420.</u>
<u>79-1508</u>	<u>*HP-7.9, 1000B Gamma Calibrator Operation, PCN 79-2323.</u>
<u>79-1509</u>	<u>*ISI-1.1.9, Quality Group A Inservice Inspection, PCN 79-2282.</u>
<u>79-1510</u>	<u>*M-38.1, Diesel Fire Pump Engine Maintenance and Inspection, PCN 79-2308.</u>
<u>79-1511</u>	<u>*O-1.2, Plant from Hot Shutdown to Steady Load, PCN 79-2432.</u>
<u>79-1512</u>	<u>*O-2.1, Normal Shutdown to Hot Shutdown, PCN 79-2426.</u>
<u>79-1513</u>	<u>*PT-2.8, Component Cooling Water Pump System, PCN 79-2400.</u>
<u>79-1514</u>	<u>*PT-27.1, Tendon Inspection and Lift-off Verification, PCN 79-2418.</u>
<u>79-1515</u>	<u>*RD-7, Liquid Waste Release, PCN 79-2322.</u>
<u>79-1516</u>	<u>*RSSP-1.4, Valve Interlock Verification - Reactor Coolant System, PCN 79-2408.</u>
<u>79-1517</u>	<u>*SC-1.3B, Emergency Coordinator Check List, PCN 79-2324.</u>
<u>79-1518</u>	<u>*SC-1.4, General Radiation Emergency, PCN 79-2391.</u>

FORC
ITEM #

PROCEDURE

79-1519 *SC-1.13, Estimating Off-Site Doses, PCN 79-2390.

79-1520 *WC-1, A list of Sample Chemical Parameters and Sampling Schedule, PCN 79-2325.

79-1529 *PT-13.1.14, Fire Barrier Penetration Seals, PCN 79-2460.

79-1530 CP-4084.0, "C" Standby Auxiliary Feedwater Pump Flow, PCN 79-2267.

CP-4084.1, Transmitter FT-4084, PCN 79-2268.

CP-4084.2, Square Root Extractor FY-4084A, PCN 79-2269.

CP-4084.3, Repeater FY-4084B and Indicator FI-4084A, PCN 79-2270.

CP-4084.4, Repeater FY-4084C and Indicator FI-4084B, PCN 79-2271.

CP-4084.5, Bistable FB-4084A and B, PCN 79-2272.

CP-4084.6, Bistable FB-4084C, PCN 79-2273.

CP-4085.0, "D" Standby Auxiliary Feedwater Pump Flow, PCN 79-2277.

CP-4085.1, Transmitter FT-4085, PCN 79-2278.

CP-4085.2, Square Root Extractor FY-4085A, PCN 79-2279.

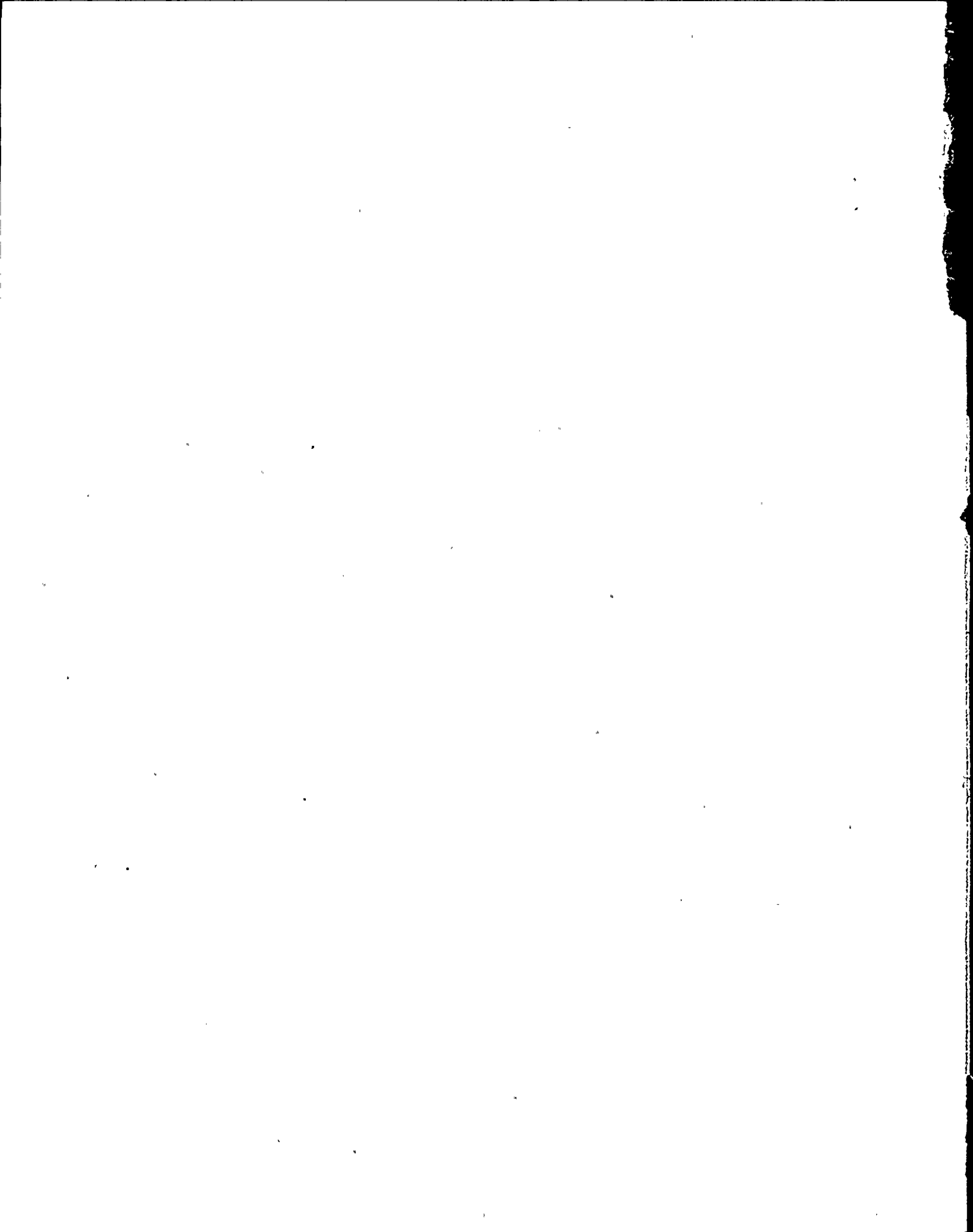
CP-4085.3, Repeater FY-4085B and Indicator FI-4085A, PCN 79-2280.

CP-4085.4, Repeater FY-4085C and Indicator FI-4085B, PCN 79-2274.

PORC
ITEM #

PROCEDURE

- CP-4085.5, Bistable FB-4085A and B, PCN 79-2276.
- CP-4085.6, Bistable FB-4085C, PCN 79-2275.
- 79-1532 *A-1102, Qualification of Test Personnel, PCN 79-2462.
- 79-1533 *A-1105, Calibration Surveillance Program for Instrumentation/Equipment of Safety Related Components, PCN 79-2436.
- 79-1534 CP-405.0, Calibration and/or Maintenance of Delta T Channel 1, PCN 79-2122.
- CP-406.0, Calibration and/or Maintenance of Delta T Channel 2, PCN 79-2120.
- CP-407.0, Calibration and/or Maintenance of Delta T Channel 3, PCN 79-2121.
- CP-408.0, Calibration and/or Maintenance of Delta T Channel 4, PCN 79-2118.
- 79-1535 *HP-2.2, Whole Body Counter Operation, PCN 79-2435.
- 79-1536 *HP-4.3, Work Permit Use, PCN 79-2417.
- 79-1537 *PT-31, Safeguard Pump Bearing Temperature Check, PCN 79-2437.
- 79-1538 *S-14.1, Radiation Monitoring and Related Systems Daily Plot Requirements, PCN 79-2433.
- 79-1546 *A-301, Control of Station Modifications, PCN 79-2480.



FORC
ITEM #

PROCEDURE

79-1547 *HP-4.1, Controlled Area Entry, PCN 79-2484.

79-1548 *PT-2.1, Safety Injection System Pumps, PCN's 79-2466 and 79-2467.

79-1550 CP-4086.0, "C" Standby Auxiliary Feedwater Pump Discharge Pressure Channel, PCN 79-2309.

CP-4086.1, Transmitter PT-4086, PCN 79-2310.

CP-4086.2, Power Supply PQ-4086, PCN 79-2311.

CP-4086.3, Bistable PB-4086, PCN 79-2312.

CP-4086.4, Indicators PI-4086A and PI-4086B, PCN 79-2313.

CP-4087.0, Standby Auxiliary Feedwater Pump Discharge Pressure Channel, PCN 79-2314.

CP-4087.1, Transmitter PT-4087, PCN 79-2315.

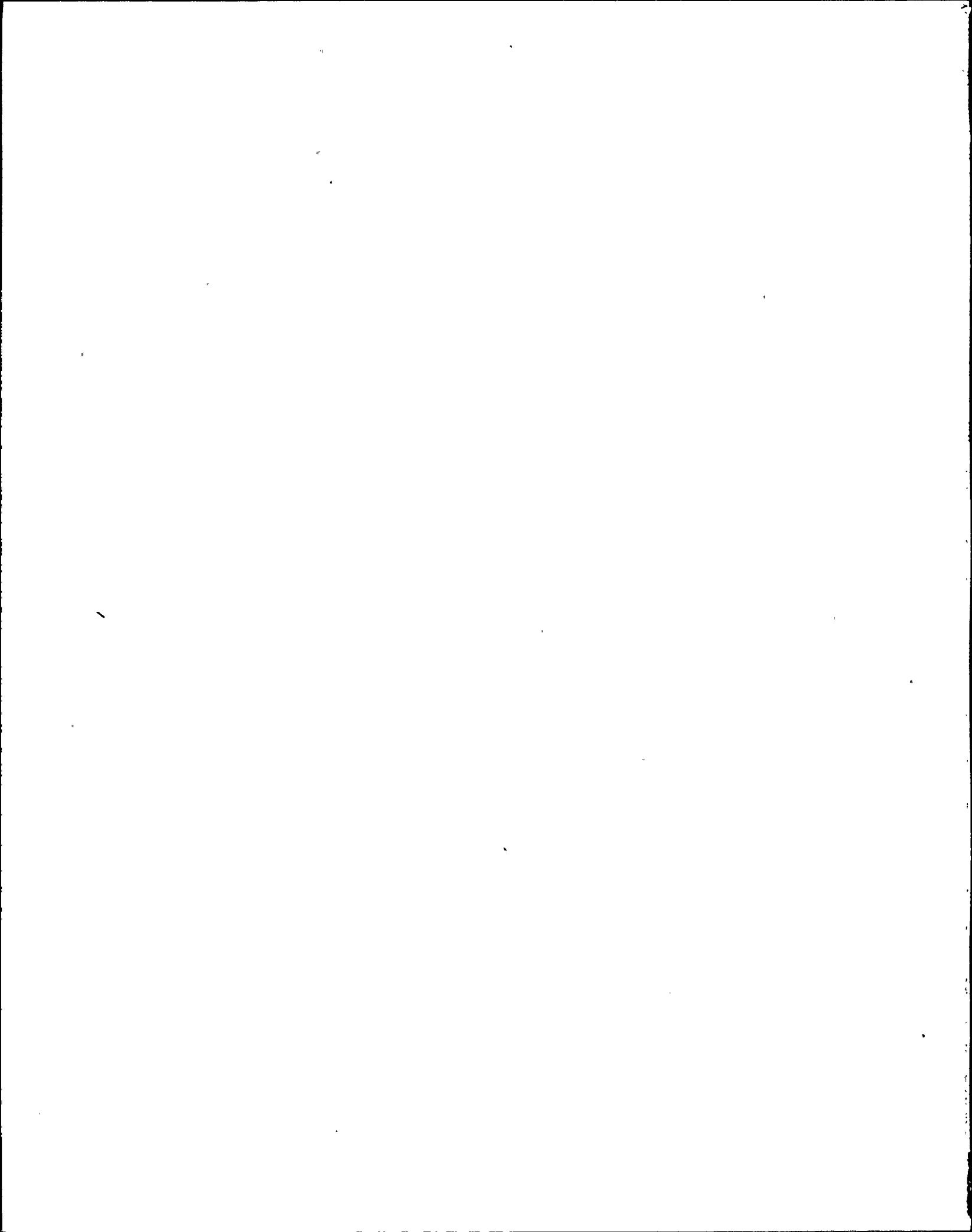
CP-4087.2, Power Supply PQ-4087, PCN 79-2316.

CP-4087.3, Bistable PB-4087, PCN 79-2317.

CP-4093.0, Standby Auxiliary Feedwater Pumps Condensate Tank Level, PCN 79-2374.

CP-4093.2, Power Supply LQ-4093, PCN 79-2376.

CP-4093.3, Bistable LB-4093A, PCN 79-2377.



PORC
ITEM #

PROCEDURE

CP-4093.4, Bistable LB-4093B, PCN 79-2378.

CP-4093.5, Indicator LI-4093, PCN 79-2379.

HP-7.17, GM E-530 Survey Meter Operation and Calibration, PCN 79-2383.

HP-7.18, Auto Digimaster 305A and B Operation and Calibration, PCN 79-2389.

HP-7.19, Jordon Rad Gun Operation and Calibration, PCN 79-2388.

HP-7.20, Digimaster Survey Meter Operation and Calibration, PCN 79-2387.

HP-7.21, Jordon Radector III Operation and Calibration, PCN 79-2386.

HP-7.22, High Range Survey Meter, CP-5-MU Operation and Calibration, PCN 79-2385.

HP-7.23, Xetex 501 A-2 Digital Area Monitor Operation and Calibration, PCN 79-2384.

HP-11.5.1, NMC Constant Air Monitor AM-3D Calibration, PCN 79-2465.

79-1551 *A-102.4, Indoctrination and Training of Quality Control Personnel, PCN 79-2475.

79-1552 *A-502, Plant Procedures and Format Requirements, PCN 79-2474.

79-1553 *A-1001, Performance of Tests, PCN 79-2470.

FORC
ITEM #

PROCEDURE

79-1554 CP-409.0, Reactor Coolant Temperature, Cold Leg "409B" Calibration and/or Maintenance, PCN 79-2151.

CP-410.0, Reactor Coolant Temperature, Cold Leg "410B" Calibration and/or Maintenance, PCN 79-2152.

79-1555 CP-424.3, Calibration and/or Maintenance of Pressurizer Liquid Temperature Indicator, PCN 79-2167.

CP-425.2, Calibration and/or Maintenance of Pressurizer Vapor Temperature Bistable TC-425, PCN 79-2169.

79-1556 *CP-427.2, Reinstate "427" Pressurizer Level, PCN 79-2173.

79-1557 *CP-428.2, Reinstate "428" Pressurizer Level, PCN 79-2174.

79-1558 CP-430.1, General Defeat Procedure 430 Pressurizer Pressure Channel, PCN 79-2227.

CP-430.2, General Reinstate Procedure 430 Pressurizer Pressure Channel, PCN 79-2228.

CP-431.1, General Defeat Procedure 431 Pressurizer Pressure Channel, PCN 79-2225.

CP-431.2, General Reinstate Procedure Pressurizer Pressure Channel 431, PCN 79-2226.

CP-449.1, General Defeat Procedure for Pressurizer Pressure Channel P-449, PCN 79-2222.

CP-449.2, General Reinstate Procedure for Pressurizer Pressure Channel P-449, PCN 79-2223.

PORC
ITEM #

PROCEDURE

- 79-1559 CP-450.0, Overpressure Protection Loop 450, PCN 79-2218.
- CP-452.0, Overpressure Protection Loop 452, PCN 79-2213.
- 79-1560 *CP-451.0, Overpressure Protection Loop 451, PCN 79-2219.
- 79-1561 CP-409.1, Reactor Coolant Temperature Cold Leg, R/I TM-409,
PCN 79-2148.
- CP-409.2, Reactor Coolant Temperature Cold Leg, Recorder TR-409,
PCN 79-2147.
- CP-409.3, Reactor Coolant Temperature Cold Leg, Bistable TC-409A/B,
PCN 79-2149.
- CP-410.2, Reactor Coolant Temperature Cold Leg, Recorder TR-410,
PCN 79-2146.
- CP-410.3, Reactor Coolant Temperature Cold Leg, Bistable TC-410A/B,
PCN 79-2150.
- CP-410.3, Reactor Coolant Temperature Cold Leg, Bistable TC-410A/B,
PCN 79-2150.
- CP-424.1, Calibration and/or Maintenance of Pressurizer Liquid
Temperature R/I Converter, PCN 79-2166.
- CP-425.1, Calibration and/or Maintenance of Pressurizer Vapor
Temperature R/I Converter, PCN 79-2168.
- CP-450.1, Overpressure Protection Loop 450 Transmitter,
PCN 79-2217.

FORC
ITEM #

PROCEDURE

CP-450.2, Overpressure Protection Loop 450 Power Supply and Bistable, PCN 79-2216.

CP-451.1, Overpressure Protection Loop 451 Transmitter, PCN 79-2220.

CP-451.2, Overpressure Protection Loop 451 Power Supply and Bistable, PCN 79-2221.

CP-452.1, Overpressure Protection Loop 452 Transmitter, PCN 79-2214.

CP-452.2, Overpressure Protection Loop 452 Power Supply and Bistable, PCN 79-2215.

CP-455.1, N₂ Accumulator Lo Loop 455 Transmitter, PCN 79-2209.

CP-455.2, N₂ Accumulator Pressure Low Bistable, Power Supply and Indicator Loop 455, PCN 79-2210.

CP-456.1, N₂ Accumulator Pressure Low Loop 456 Transmitter, PCN 79-2207.

CP-456.2, N₂ Accumulator Pressure Low Loop 456 Bistable, Power Supply and Indicator, PCN 79-2208.

79-1562

CP-455.0, N₂ Accumulator Pressure Low Loop 455, PCN 79-2212.

CP-456.0, N₂ Accumulator Pressure Low Loop 456, PCN 79-2211.

79-1563

CP-461.0, Calibration and/or Maintenance of Steam Generator Level Channel "461", PCN 79-2205.

FORC
ITEM #

PROCEDURE

CP-462.0, Calibration and/or Maintenance of Steam Generator Level Channel "462", PCN 79-2203.

CP-463.0, Calibration and/or Maintenance of Steam Generator Level Channel "463", PCN 79-2201.

79-1564 CP-464.7, Steam Flow Square Root Extractor FM-464B, PCN 79-2245.

CP-465.7, Steam Flow Square Root Extractor FM-465B, PCN 79-2244.

79-1565 CP-426.8, Pressurizer Level Indicator LI-426, PCN 79-2170.

CP-428.8, Pressurizer Level Indicator LI-428, PCN 79-2175.

CP-461.8, Steam Generator Level Indicator LI-461, PCN 79-2206.

CP-462.8, Steam Generator Level Indicator LI-462, PCN 79-2204.

CP-463.8, Steam Generator Level Indicator LI-463, PCN 79-2202.

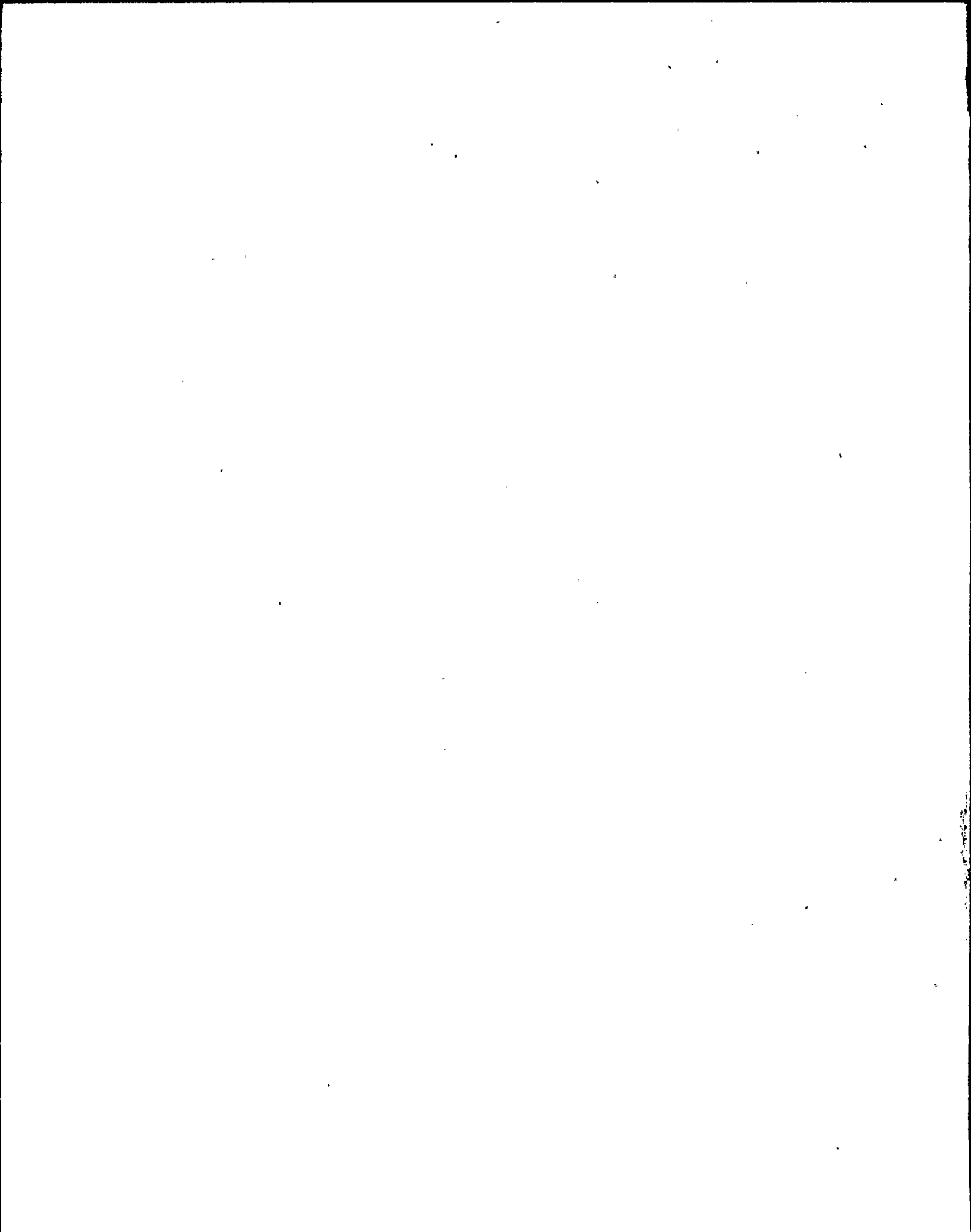
CP-465.10, Steam Flow Indicator FI-465, PCN 79-2243.

CP-468.10, Steam Generator Pressure Indicator PI-468, PCN 79-2239.

CP-469.5, Steam Generator Pressure Lead/Lag Unit PM-469A,
PCN 79-2238.

CP-469.8, Steam Generator Pressure Indicator PI-469, PCN 79-2237.

CP-471.8, Steam Generator Level Indicator LI-471, PCN 79-2236.



PORC
ITEM #PROCEDURE

- CP-472.8, Steam Generator Level Indicator LI-472, PCN 79-2235.
- 79-1566 *CP-465.11, Steam Flow Recorder FR-465, PCN 79-2242.
- 79-1567 CP-431.0, Calibration and/or Maintenance of Pressurizer Pressure Channel "431", PCN 79-2224.
- CP-467.0, Calibration and/or Maintenance of Feedwater Flow Channel "467", PCN 79-2240.
- 79-1568 *CP-472.0, Calibration and/or Maintenance of Steam Generator Level Channel "472", PCN 79-2234.
- 79-1569 *CP-473.3, Steam Generator Level Transmitter LT-473, PCN 79-2233.
- 79-1570 *CP-617.0, Component Cooling Water Pumps Discharge Pressure PIC-617, PCN 79-2481.
- 79-1571 CP-1012, Calibration and/or Maintenance of "A" Waste Condensate Tank Level "1012", PCN 79-2468.
- CP-1013, Calibration and/or Maintenance of "B" Waste Condensate Tank Level "1013", PCN 79-2469.
- 79-1572 *EM-154, Steam Generator Handhole Cover Gasket Leak Repair Using "Furmanite" Process, PCN 79-2483.
- 79-1573 *PT-13, Fire Pump Operation and System Alignment, PCN 79-2482.
- 79-1574 *S-4.1D, Drumming Waste Evaporator Bottoms, PCN 79-2472.
- 79-1575 *S-30.4, Auxiliary Feedwater System Valve Position Verification, PCN 79-2478.

PORC
ITEM #

PROCEDURE

79-1576 *SC-1.15, Inspection of Emergency Equipment, PCN 79-2477.

79-1577 *T-41A, Alignment of Auxiliary Feedwater System Prior to Power Operation, PCN 79-2479.

79-1586 *EM-255, Technical Support Center Intercom Systems Wiring Installation, PCN 79-2571.

79-1589 *M-38.10, Repair or Replacement of W-2 Switches, PCN 79-2473.

79-1590 *A-7.1, First Aid Supply Inventory for Plant Stretcher Boxes, PCN 79-2497.

79-1591 CP-474.7, Steam Flow Square Root Extractor FM-474B, PCN 79-2262.
CP-475.5, Steam Flow Bistable FC-475A, PCN 79-2260.

79-1592 CP-473.8, Steam Generator Level Indicator LI-473, PCN 79-2263.
CP-475.7, Steam Flow Square Root Extractor FM-475B, PCN 79-2259.
CP-475.11, Steam Flow Recorder FR-475, PCN 79-2254.
CP-474.10, Steam Flow Indicator FI-474, PCN 79-2261.
CP-477.9, Feedwater Flow Indicator FI-477, PCN 79-2257.

79-1593 *CP-476.9, "B" Feedwater Flow Indicator FI-476, PCN 79-2255.

79-1594 CP-477.5, Feedwater Flow Square Root Extractor FM-477A, PCN 79-2256.

FORC
ITEM #

PROCEDURE

CP-476.5; "B" Feedwater Flow Square Root Extractor FM-476A, PCN 79-2258.

79-1595 *HP-1.6; Neutron Exposure, PCN 79-2492.

79-1596 *HP-7.5; Pocket Dosimeter/Accuracy and Leak Test, PCN 79-2496.

79-1597 *ISI-10; Inspection of Stagnant, Oxygenated, Borated Systems, PCN 79-2476.

79-1598 *S-3.4D; Boron Recycle Process Startup, PCN 79-2488.

79-1599 *SC-1.15; Inspection of Emergency Equipment, PCN 79-2493.

79-1608 *A-1104; Ginna Station Technical Specification Surveillance Program, PCN 79-2499.

79-1609 *ES-1; Schedule for Environmental Samples and Parameters Analyzed, PCN 79-2423.

79-1610 *HP-4.3; Work Permit Use, PCN 79-2504.

79-1611 *S-5; Nuclear Sample Room Sampling System, PCN 79-2485.

79-1612 *SC-1.4; General Radiation Emergency, PCN 79-2502.

79-1613 *M-38.11; OT2 Switches Repair or Replacement for " ", PCN 79-2490.

79-1614 HP-7.1; Calibration of Gamma Survey Instruments - High Range, PCN 79-2510.

FORC
ITEM #

PROCEDURE

HP-7.2, Calibration of Gamma Survey Instruments - Low Range,
PCN 79-2509.

M-55.1, Repair or Replacement of Type W-2 Breaker Control
Switches, PCN 79-2512.

79-1616 *CP-473.0, Calibration and/or Maintenance of Steam Generator Level
Channel "473", PCN 79-2289.

79-1617 *CP-473.6, Steam Generator Level Bistable LC-473E, PCN 79-2290.

79-1618 *CP-474.5, Steam Flow Bistable FC-474A, PCN 79-2288.

79-1619 CP-478.5, Steam Generator Pressure Lead/Lag Unit PM-478A,
PCN 79-2303.

CP-478.10, Steam Generator Pressure Indicator PI-478, PCN 79-2302.

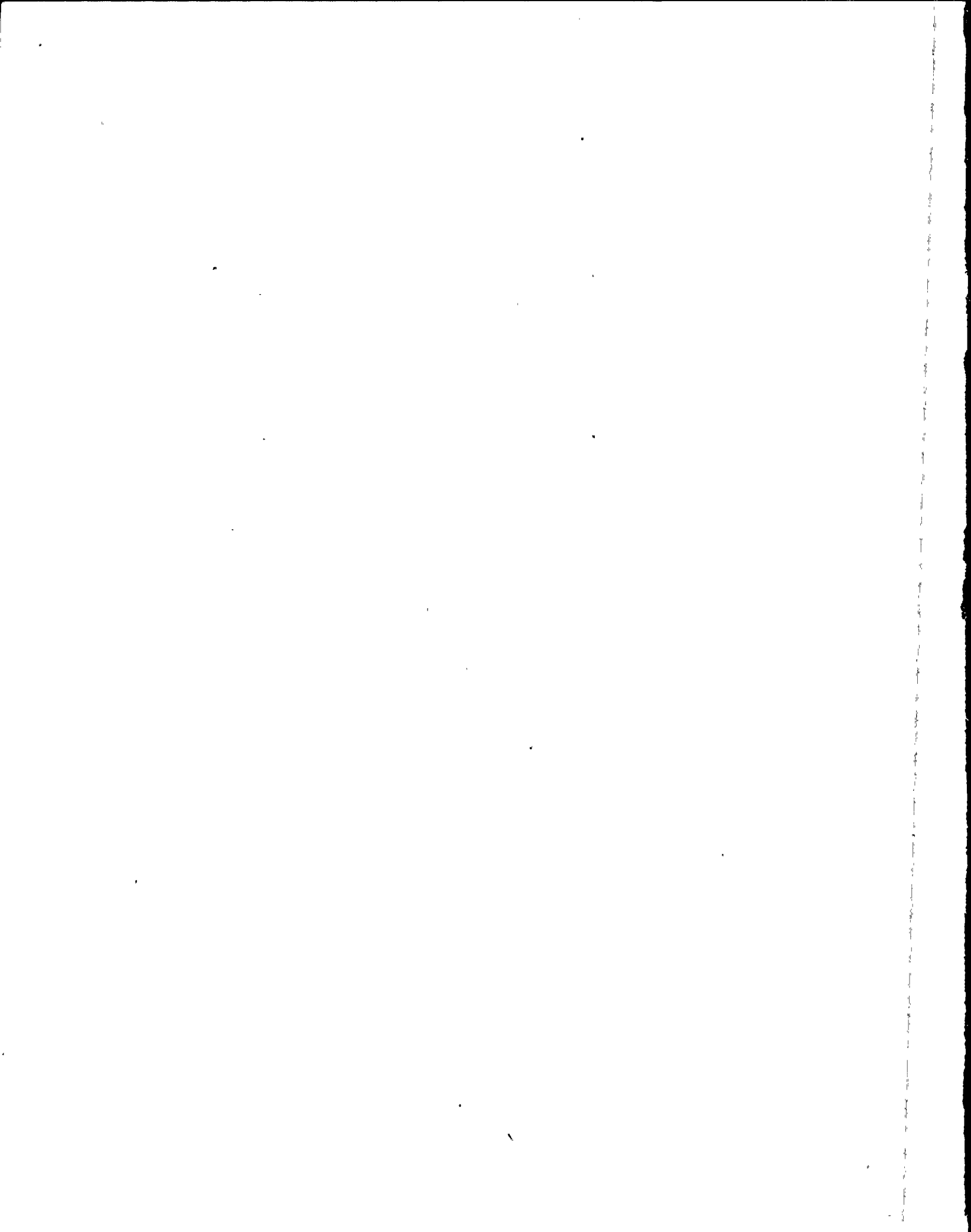
CP-479.5, Steam Generator Pressure Lead/Lag Unit PM-479A,
PCN 79-2301.

CP-479.8, Steam Generator Pressure Indicator PI-479, PCN 79-2300.

CP-482.5, Steam Generator Pressure Lead/Lag Unit PM-482A,
PCN 79-2298.

CP-482.9, Steam Generator Pressure Indicator PI-482A and
Remote PI-482A, PCN 79-2297.

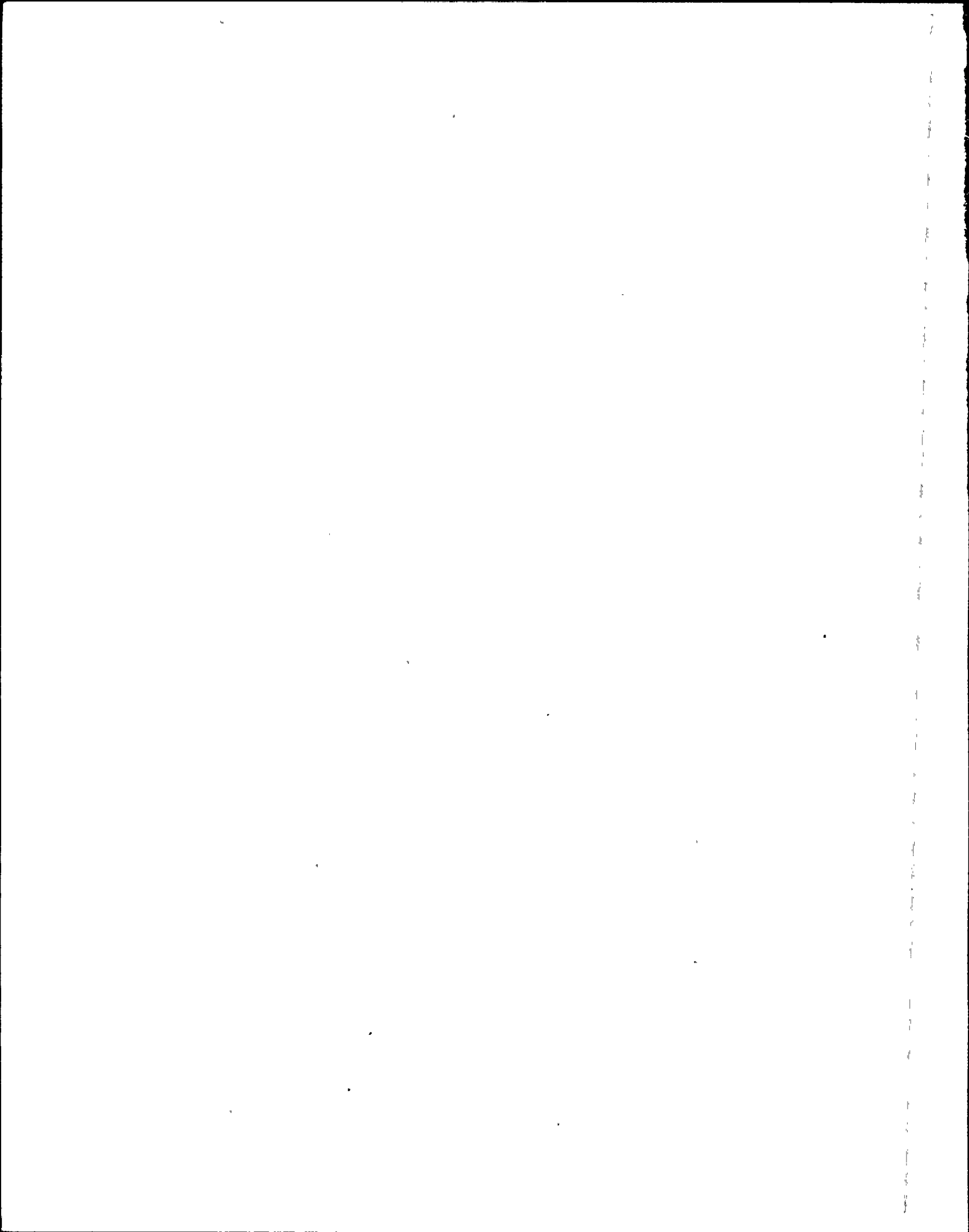
CP-483.5, Steam Generator Pressure Lead/Lag Unit PM-483A,
PCN 79-2296.



FORC
ITEM #

PROCEDURE

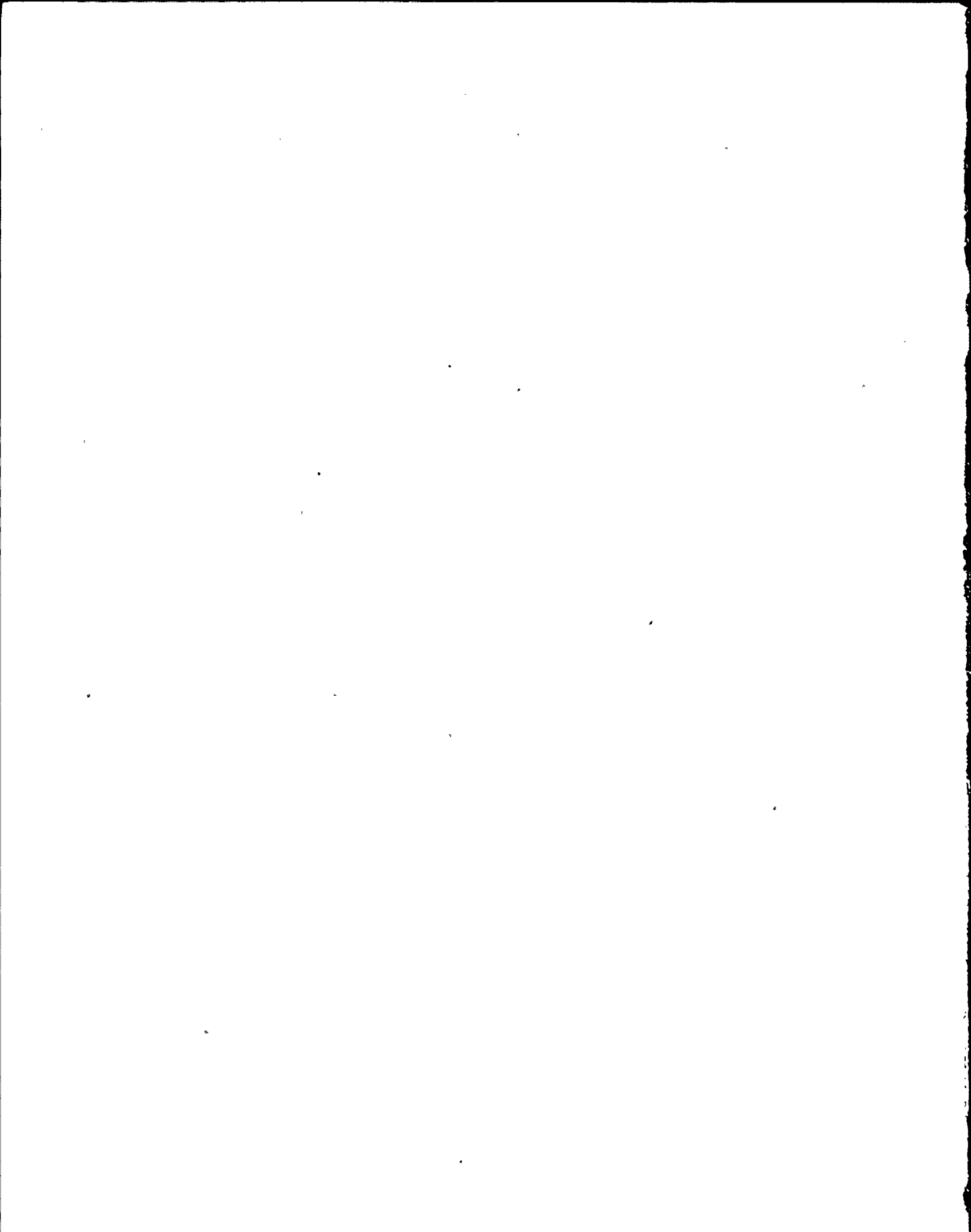
- CP-483.9, Steam Generator Pressure Indicator PI-483A and PI-483B, PCN 79-2295.
- CP-486.6, Turbine First Stage Pressure Impulse Unit PM-486B, PCN 79-2294.
- 79-1620 *CP-482.3, Steam Generator Pressure Transmitter PT-482, PCN 79-2299.
- 79-1621 *CP-2092.0, Calibration and/or Maintenance of "2092" Atmospheric Relief Valve IA (CV-56) Control, PCN 79-2518.
- 79-1622 CP-2092.2, General Reinstate "2092" Atmospheric Relief Valve IA (CV-56) Control, PCN 79-2515.
- CP-2093.1, General Defeat Procedure "2093" Atmospheric Relief Valve IB (CV-57) Control, PCN 79-2517.
- 79-1623 *CP-2093.0, Calibration and/or Maintenance of "2093" Atmospheric Relief Valve IB (CV-57) Control, PCN 79-2516.
- 79-1624 HP-7.3, Calibration of Alpha Survey Instruments PAC-4S, PCN 79-2508.
- HP-7.4, Calibration of Neutron Survey Instruments, PCN 79-2507.
- 79-1625 *HP-7.6, Tagging of Instruments Requiring Calibration or Maintenance PCN 79-2506.
- 79-1626 *HP-7.7, Calibration of Heat Survey Instruments, PCN 79-2505.
- 79-1627 *O-5.1, Load Reductions, PCN 79-2529.



PORC
ITEM #

PROCEDURE

- 79-1628 *SC-1.12A, Immediate Call List, PCN 79-2165.
- 79-1637 *EM-255, Technical Support Center Intercom Systems Wiring Installation, PCN 79-2571.
- 79-1638 *E-10, Malfunction Rod Position Indicator, PCN 79-2522.
- 79-1639 *E-12.1, Malfunctioning Rod Cluster Control Full Length Rods, PCN 79-2521.
- 79-1640 *M-42.2, Charcoal Filters Removal/Installation, PCN 79-2514.
- 79-1641 *M-37.56, Nuclear Sample System Manual Valve Replacement, PCN 79-2543.
- *M-43.11, Steam Generator Tube Removal - Steam Generator, PCN 79-2549.
- *M-43.23A, Steam Generator Recirculation to the "B" Condensate Storage Tank, PCN 79-2547.
- 79-1643 *EM-180, Manipulator Crane Gripper Maintenance, PCN 79-2551.
- *EM-184, Limitorque Valve Operator Removal and Replacement for MOV-3504, PCN 79-2552.
- *M-43.32, Steam Generator Tubes ID Measurement, PCN 79-2545.
- 79-1644 E-1.1, Immediate Action and Diagnostics for Spurious Actuation of SI, LOCA, Loss of Secondary Coolant, and Steam Generator Tube Rupture, PCN 79-2594.



PORC
ITEM #

PROCEDURE

E-1.2, Loss of Reactor Coolant, PCN 79-2593.

E-1.3, Loss of Secondary Coolant, PCN 79-2592.

E-1.4, Steam Generator Tube Rupture, PCN 79-2591.

79-1645

*O-8.1, Restoration of Pressurizer Heaters to Maintain Natural Circulation at HSD, PCN 79-2590.

79-1646

M-11.11, Crane Chempumps, Series G and GT Maintenance, PCN 79-2553.

M-11.13, Inspection and Maintenance of Gould Pumps, PCN 79-2560.

M-11.16, Durco-Dunrion Pump Maintenance, PCN 79-2566.

M-11.17, Weinman Pumps, PCN 79-2565.

M-11.18, Buffalo Forge Pumps Maintenance, PCN 79-2554.

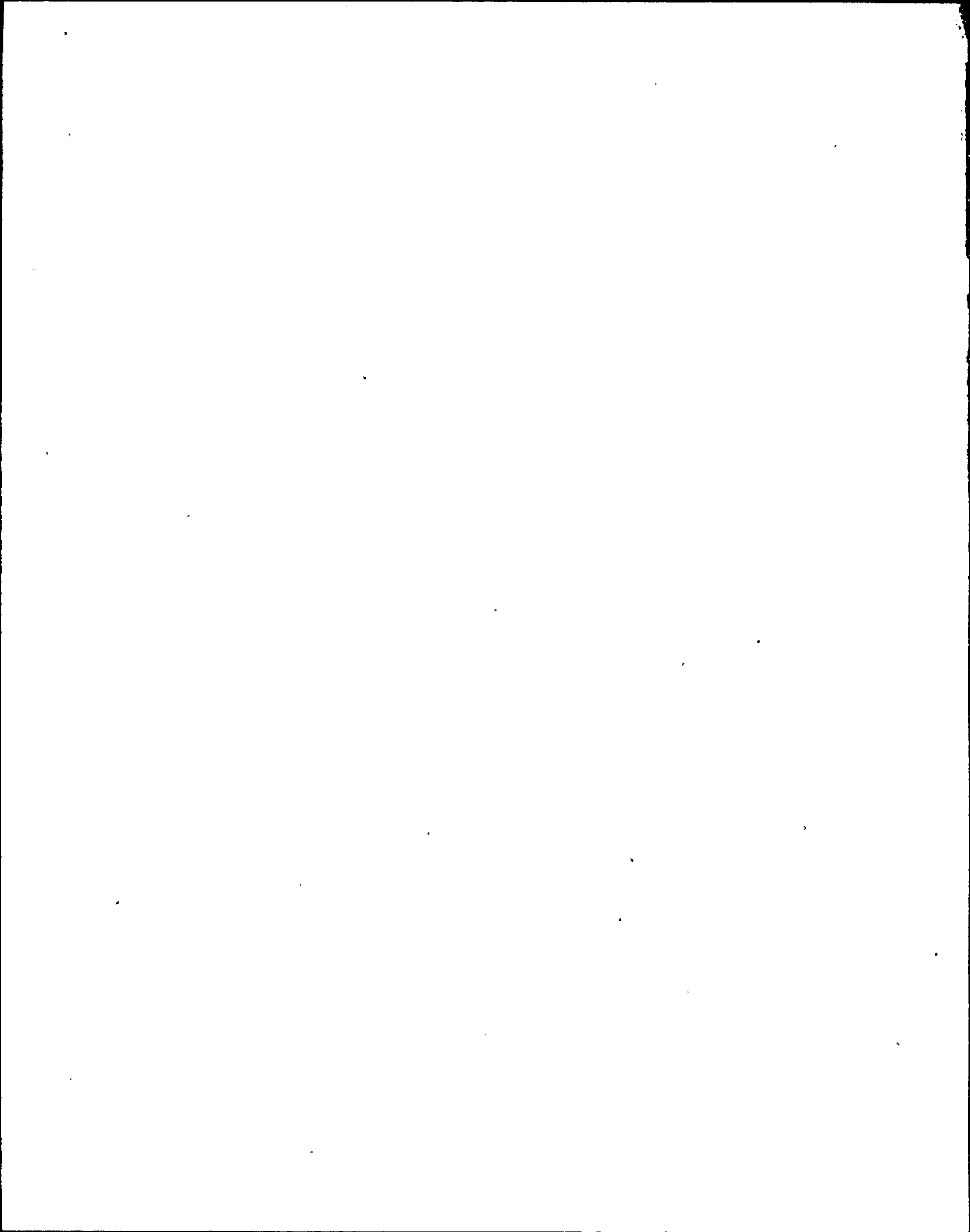
M-11.19, Crane-Deming Pumps Maintenance, PCN 79-2564.

M-11.20, Economy Pumps Vertical Type Maintenance, PCN 79-2463.

M-11.22, Lapp-Interpace Pulsa Feeder Pumps, PCN 79-2562.

M-11.31, Condensate Polishing Sundstrand Air Blower Maintenance, PCN 79-2561.

M-11.33, Condensate Booster Pump Inspection and Maintenance, PCN 79-2555.



PORC
ITEM #

PROCEDURE

M-37.12, Maintenance Isolation Procedure for V-392B Charging Line to Cold Leg "A", PCN 79-2542.

M-37.34, Main Steam Line Power Operated Relief Valves 3410 or 3411 Maintenance, PCN 79-2541.

M-43.23, Steam Generator Tubesheet Lancing, PCN 79-2548.

M-43.25, Installation of Steam Generator Handhole Cover, PCN 79-2546.

M-45.2, Non-Safeguard Motor for Inspection, PCN 79-2556.

M-61.2, Auxiliary Building Sump Tank Inspection and Cleaning, PCN 79-2540.

M-63.1, Fan Cooler Unit Maintenance, PCN 79-2557.

79-1647 *A-1602, Preventive Action Report, PCN 79-2544.

79-1649 *CP-945.7, Containment Pressure Indicator PI-945, PCN 79-2349.

*CP-946.7, Containment Pressure Indicator PI-946, PCN 79-2348.

*CP-947.7, Containment Pressure Indicator PI-947, PCN 79-2347.

*CP-948.7, Containment Pressure Indicator PI-948, PCN 79-2346.

*CP-949.7, Containment Pressure Indicator PI-949, PCN 79-2345.

*CP-950.7, Containment Pressure Indicator PI-950, PCN 79-2344.

FORC
ITEM #

PROCEDURE

79-1651

CP-624.1, Residual Heat Removal Heat Exchanger Outlet Valve HCV-624 Isolation, PCN 79-2366.

CP-625.1, Residual Heat Removal Heat Exchanger Valve HCV-625 Isolation, PCN 79-2365.

CP-626.1, Defeat of Residual Heat Removal Flow Channel "626", PCN 79-2364.

CP-934.2, Accumulator Level Bistable LC-934, PCN 79-2359.

CP-934.3, Accumulator Level Local Indicator LI-934, PCN 79-2358.

CP-945.1, Defeat of "A-1" Containment Pressure Channel "945", PCN 79-2339.

CP-945.2, Reinstate of "A-1" Containment Pressure Channel "945", PCN 79-2340.

CP-947.1, Defeat of "A-3" Containment Pressure Channel "947", PCN 79-2341.

CP-947.2, Reinstate of "A-3" Containment Pressure Channel "947", PCN 79-2342.

CP-949.1, Defeat of "B-1" Containment Pressure Channel "949", PCN 79-2337.

CP-949.2, Reinstate of "B-1" Containment Pressure Channel "949", PCN 79-2338.

CP-1090.1, #1 Fan Cooler Condensate Level Transmitter LT-1090, PCN 79-2336.

FORC
ITEM #

PROCEDURE

CP-1091.1, #2 Fan Cooler Condensate Level Transmitter LT-1091, PCN 79-2335.

CP-1092.1, #3 Fan Cooler Condensate Level Transmitter LT-1092, PCN 79-2334.

CP-1093.1, #4 Fan Cooler Condensate Level Transmitter LT-1093, PCN 79-2333.

CP-2006.0, Turbine Driven Auxiliary Feedwater Flow to Steam Generator 1A, Loop 2006, PCN 79-2338.

CP-2007.0, Turbine Driven Auxiliary Feedwater Flow to Steam Generator 1B, Loop 2007, PCN 79-2339.

79-1652 *CP-626.0, Calibration and/or Maintenance of Residual Heat Removal Flow Channel "626", PCN 79-2363.

79-1653 *CP-629.0, Residual Heat Removal Loop Pressure "629", PCN 79-2362.

79-1654 *CP-920.0, Calibration and/or Maintenance "920" Refueling Water Storage Tank Level Channel, PCN 79-2361.

79-1655 *CP-930.0, Calibration and/or Maintenance of Spray Additive Tank Inlet Flow Transmitter Loop, PCN 79-2360.

79-1656 CP-937.0, Calibration and/or Maintenance of Accumulator Pressure Channel "937", PCN 79-2357.

CP-939.0, Calibration and/or Maintenance of Accumulator Level Channel "939", PCN 79-2355.

FORC
ITEM #

PROCEDURE

CP-940.0, Calibration and/or Maintenance of Accumulator Pressure Channel "940", PCN 79-2354.

CP-941.0, Calibration and/or Maintenance of Accumulator Pressure Channel "941", PCN 79-2352.

79-1657 CP-938.0, Calibration and/or Maintenance of Accumulator Level Channel "938", PCN 79-2356.

CP-2030.0, 1B Motor Driven Auxiliary Feedwater Pump Discharge Pressure Loop 2030, PCN 79-2322.

79-1658 *CP-940.1, Accumulator Pressure Transmitter PT-940, PCN 79-2353.

79-1659 *CP-942.0, Check and/or Maintenance of Containment Sump #1 Level Switches and Indicating Lights, PCN 79-2351.

79-1660 *CP-943.0, Check and/or Maintenance of Containment Sump B #2 Level Switches and Indicating Lights, PCN 79-2350.

79-1661 *CP-1037, Calibration and/or Maintenance of #2 Gas Decay Tank Pressure Channel "1037", PCN 79-2343.

79-1663 CP-2092.1, General Defeat Procedure "2092" Atmospheric Relief Valve 1A (CV-56) Control, PCN 79-2331.

CP-2093.2, General Reinstatement "2093" Atmospheric Relief Valve 1B (CV-57) Control, PCN 79-2332.

79-1664 E-9, Leakage into the Component Cooling Loop, PCN 79-2523.

E-11, Continuous Rod Withdrawal, PCN 79-2520.

PORC
ITEM #

PROCEDURE

E-15.1, Malfunction of Pressurizer Power Relief or Safety Valves, PCN 79-2527.

E-15.2, Malfunction of Pressurizer Heaters or Spray Valves, PCN 79-2526.

E-17, Loss of Residual Heat Removal, PCN 79-2525.

E-19, Continuous Insertion of RCCA Control Bank, PCN 79-2524.

79-1665 *GS-39.0, Protection Vital/Area Access, PCN 79-2550.

79-1666 *HP-2.1, Whole Body Counting Guide, PCN 79-2559.

79-1667 *PT-17.2, Process Radiation Monitors R-11 - R-21 and R-10A and B, PCN 79-2511.

79-1668 PT-22.8, Mechanical Manifold "A" Leakrate Test, PCN 79-2538.

PT-22.9, Mechanical Manifold "B" Leakrate Test, PCN 79-2537.

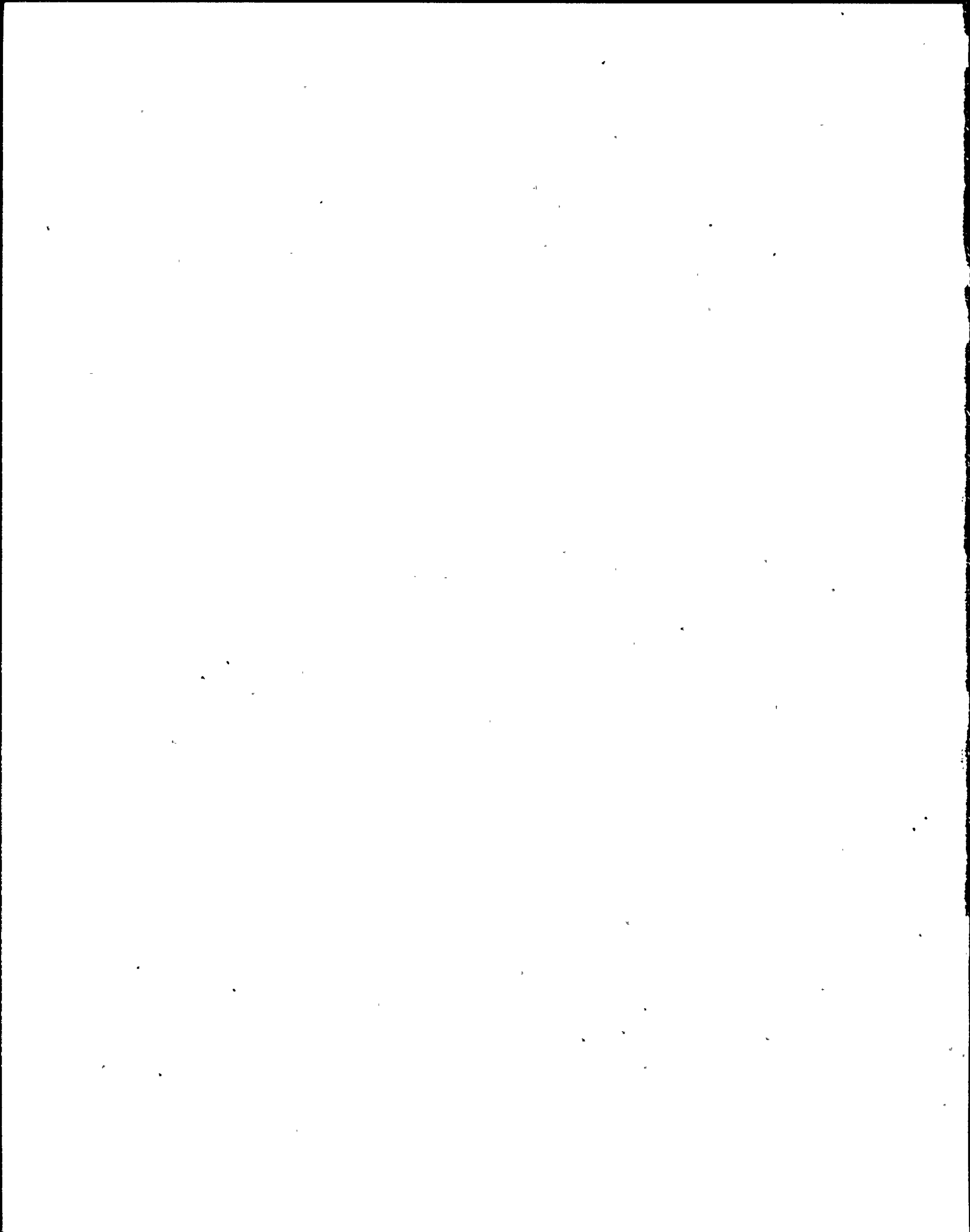
PT-22.10, Mechanical Manifold "C" Leakrate Test, PCN 79-2536.

PT-22.11, Mechanical Manifold "E" Leakrate Test, PCN 79-2535.

PT-22.12, Mechanical Manifold "F" Leakrate Test, PCN 79-2534.

PT-22.13, Mechanical Manifold "G" Leakrate Test, PCN 79-2533.

PT-22.14, Mechanical Manifold "H" Leakrate Test, PCN 79-2532.



PORC
ITEM #

PROCEDURE

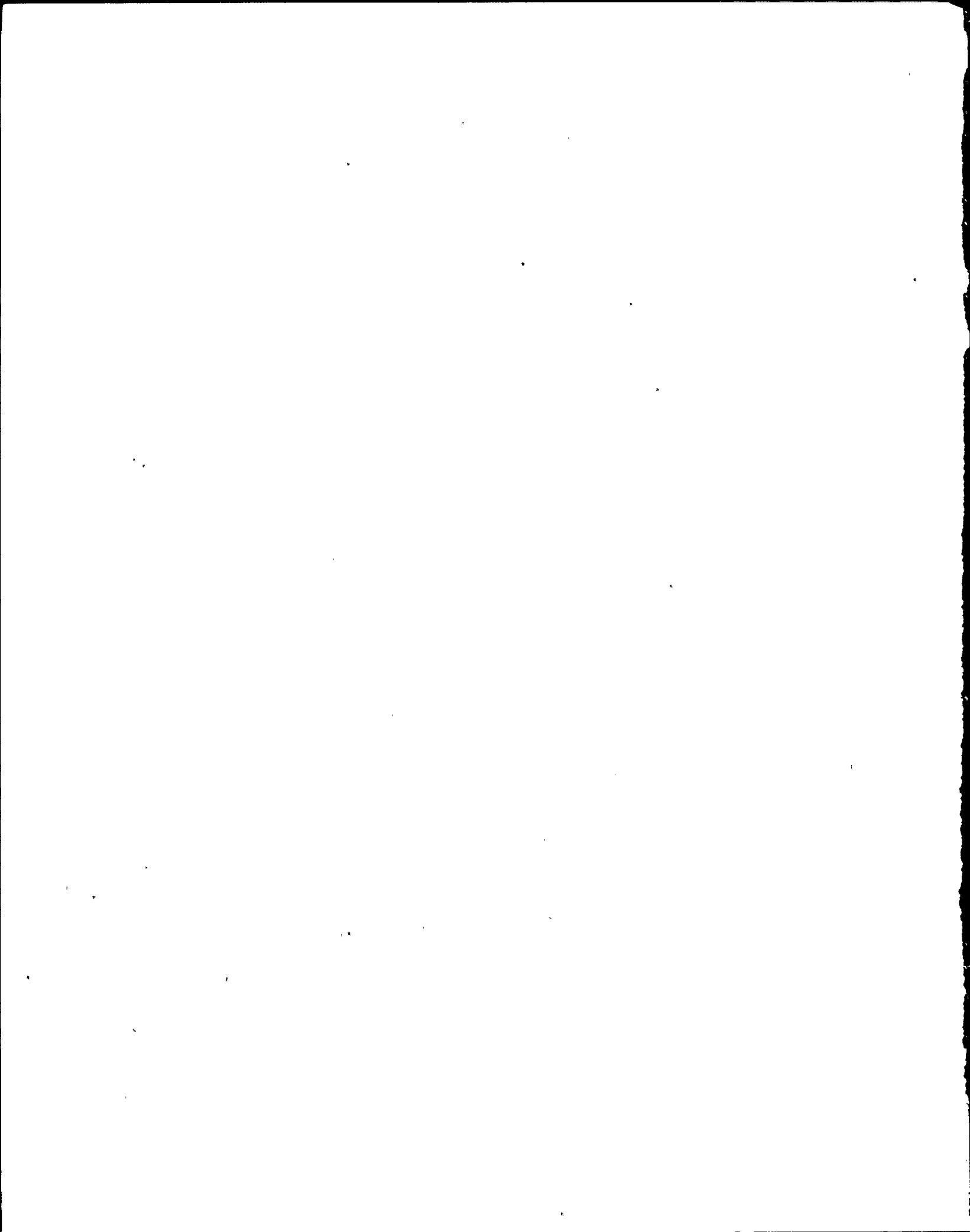
- PT-22.16, Mechanical Manifold "J" Leakrate Test, PCN 79-2531.
- PT-22.17, Mechanical Manifold "K" Leakrate Test, PCN 79-2530.
- 79-1669 *QCIP-21, Shipping Package LL-50-100 Inspection, PCN 79-2570.
- 79-1670 S-16A, Safety Injection System Alignment, PCN 79-2568.
- S-30.2, RHR System Valve Position Verification, PCN 79-2569.
- 79-1683 *SM 79-1837.2, Fire Doors Installation, PCN 79-2589.
- 79-1684 *M-57.2, Source Range and/or Intermediate Range NIS Detector Replacement, PCN 79-2598.
- 79-1685 *ISI-9.1, Inservice Inspection of Shell Type Anchor Bolts, PCN 79-2613.
- 79-1686 *RSSP-15.12, Hydro Test of Class "B" Piping (A and B Steam Generator Feedwater Piping, PCN 79-2614.
- 79-1687 *EWR-2603, Direct Indication of Valve Position Pressurizer Safety Valve Position Indication.
- 79-1688 *EWR-2603A, Direct Indication Valve Position Pressurizer Power Operated Relief Valve Indication.
- 79-1689 *M-43.16.2, Installation of Steam Generator Mechanical Tube Plugs, PCN 79-2615.
- 79-1690 *M-51.5, Westinghouse BFD Relay Armature Overtravel Measurement, PCN 79-2617.

<u>FORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-1693</u>	<u>*SM 79-2603.1, Pressurizer Safety Valve Direct Position Indicator, PCN 79-2629.</u>
<u>79-1694</u>	<u>*SM 79-2603A.1, Pressurizer PORV Direct Position Indicator to Computer, PCN 79-2631.</u>
<u>79-1695</u>	<u>*SM 79-2604.1, RCS Subcooling Margin Monitoring System, PCN 79-2630.</u>
<u>79-1697</u>	<u>*A-1.1, Locked Radiation Areas, PCN 79-2596.</u>
<u>79-1699</u>	<u>T-36.2, Service Water Redundant Return Line Operation, PCN 79-2623.</u>
	<u>T-44.1, Condensate Supply Tank for Standby Auxiliary Feedwater Pumps Filling, PCN 79-2623.</u>
<u>79-1700</u>	<u>M-73.4, Components to Stainless Steel Piping or Components, PCN 79-2575.</u>
	<u>SM 79-01.1, Feedwater Drain Lines Inside Containment, PCN 79-2638.</u>
	<u>SM 77-06.2, Low Pressure Purification System Modification - Hydrostatic Test, PCN 79-2640.</u>
	<u>SM 77-06.3, Low Pressure Purification System Modification, PCN 79-2639.</u>
<u>79-1701</u>	<u>SC-1.3D, Manning the Technical Support Center, PCN 79-2500.</u>
	<u>SC-1.3E, Manning the Operational Support Center, PCN 79-2501.</u>
<u>79-1702</u>	<u>M-38, Equalizing Charge Station Battery Systems, PCN 79-2605.</u>

PORC
ITEM #

PROCEDURE

- M-44.3, Isolation of 1C MCC and Restoring to Service, PCN 79-2609.
- M-44.4, Isolation of 1D MCC and Restoring to Service, PCN 79-2610.
- M-44.5, Isolation and Restoration of Motor Control Center 1E, PCN 79-2607.
- M-44.6, Isolation and Restoration of Motor Control Center 1F, PCN 79-2608.
- M-55.3, Maintenance of Control Rod Drive Mechanism Coil Stack, PCN 79-2606.
- 79-1703 *A-601, Plant Procedure Document Control, PCN 79-2637.
- 79-1705 E-4, Station Blackout Operation, PCN 79-2579.
- E-26.1, Reactor Trip-Turbine Not Latched, PCN 79-2580.
- E-26.2, Reactor Trip-Turbine Latched, PCN 79-2581.
- 79-1706 *EM-251, Rock Anchor Coupling Enclosure Broken Bolt Repair, PCN 79-2574.
- 79-1707 *HP-1.4, Noble Gas Exposure, PCN 79-2636.
- 79-1708 *HP-2.3, TLD Reader Calibration, PCN 79-2618.
- 79-1709 *ISI-9.1, Inservice Inspection of Shell Type Anchor Bolts, PCN 79-2620.



<u>FORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-1710</u>	<u>*M-18.1, Drum Preparation for Waste Evaporator Bottoms, PCN 79-2603.</u>
<u>79-1711</u>	<u>*M-57.2, Source Range and/or Intermediate Range NIS Detector Replacement, PCN 79-2604.</u>
<u>79-1712</u>	<u>*O-6.10, Plant Operation With Steam Generator Tube Leak Indication, PCN 79-2597.</u>
<u>79-1713</u>	<u>*O-8, Natural Circulation in the RCS, PCN 79-2578.</u>
<u>79-1714</u>	<u>*S-25.4, Processing Water from Either Steam Generator, PCN's 79-2599 and 79-2600.</u>
<u>79-1715</u>	<u>*SM 77-1836.2, Relocation of Halon System, PCN's 79-2611 and 79-2616.</u>
<u>79-1723</u>	<u>*M-43.31.1, Steam Generator Handhole Gasket Surface Weld Repair, PCN 79-2666.</u>
<u>79-1724</u>	<u>*EM-256, Repair of the Safe End for the Pressurizer Power Operated Relief Valve Line, PCN 79-2678.</u>
<u>79-1728</u>	<u>*EM-257, Furmanite Repair - Valve 427 CVCS Letdown Isolation, PCN 79-2732.</u>
<u>79-1729</u>	<u>*HP-4.3, Work Permit Use, PCN 79-2612.</u>
<u>79-1732</u>	<u>A-53.3, Valve Preventive Maintenance Program, PCN 79-2595.</u>
	<u>CP-430.11, Pressurizer Pressure Repeater PM-430A.1, PCN 79-2633.</u>
	<u>CP-4086.0, "C" Standby Auxiliary Feedwater Pump Discharge Pressure Channel, PCN 79-2309.</u>

FORC
ITEM #

PROCEDURE

CP-4086.1; "C" Standby Auxiliary Feedwater Pump Discharge Pressure Transmitter, PT-4086, PCN 79-2310.

CP-4086.2; "C" Standby Auxiliary Feedwater Pump Discharge Pressure Power Supply PQ-4086, PCN 79-2311.

CP-4086.3; "C" Standby Auxiliary Feedwater Pump Pressure Bistable PB-4086, PCN 79-2312.

CP-4086.4; "C" Standby Auxiliary Feedwater Pump Discharge Pressure Indicators PI-4086A and PI-4086B, PCN 79-2313.

CP-4087.0; "D" Standby Auxiliary Feedwater Pump Discharge Pressure Channel, PCN 79-2314.

CP-4087.1; "D" Standby Auxiliary Feedwater Pump Discharge Pressure Transmitter, PCN 79-2315.

CP-4087.2; "D" Standby Auxiliary Feedwater Pump Discharge Pressure Power Supply, PCN 79-2316.

CP-4087.3; "D" Standby Auxiliary Feedwater Pump Discharge Pressure Bistable PB-4087, PCN 79-2317.

CP-4087.4; "D" Standby Auxiliary Feedwater Pump Discharge Pressure Indicators PI-4087A and PI-4087B, PCN 79-2318.

CP-4093.1; Standby Condensate Level Transmitter LT-4093, PCN 79-2375.

M-83; Manipulator Crane Gripper and Inner Mast Maintenance, PCN 79-2567.

M-84; Cleaning of Feedwater Flow Venturis, PCN 79-2628.

FORC
ITEM #

PROCEDURE

79-1733

M-31, Stator Replacement for Part Length Control Rod Drive System, PCN 79-2717.

MHE-100-1, Maintenance and Inspection of 40/5 Ton Shaw Box Crane in the Auxiliary Building, PCN 79-2681.

MHE-100-2, Maintenance and Inspection of 100/20 Ton Whiting Crane in Containment Serial No. 9591, PCN 79-2682.

MHE-100-3, Maintenance and Inspection of 125/25 Ton Whiting Crane in the Turbine Building Serial No. 9464, PCN 79-2683.

MHE-200-1, Inspection and Maintenance of 5 Ton Robbins and Myers Hoist Serial No. 43924 TWL in the Auxiliary Building Drumming Area, PCN 79-2684.

MHE-300-1, Inspection and Maintenance of 10 Tons Robbins and Myers Hoist Serial No. 43635 TUI in Containment, PCN 79-2685.

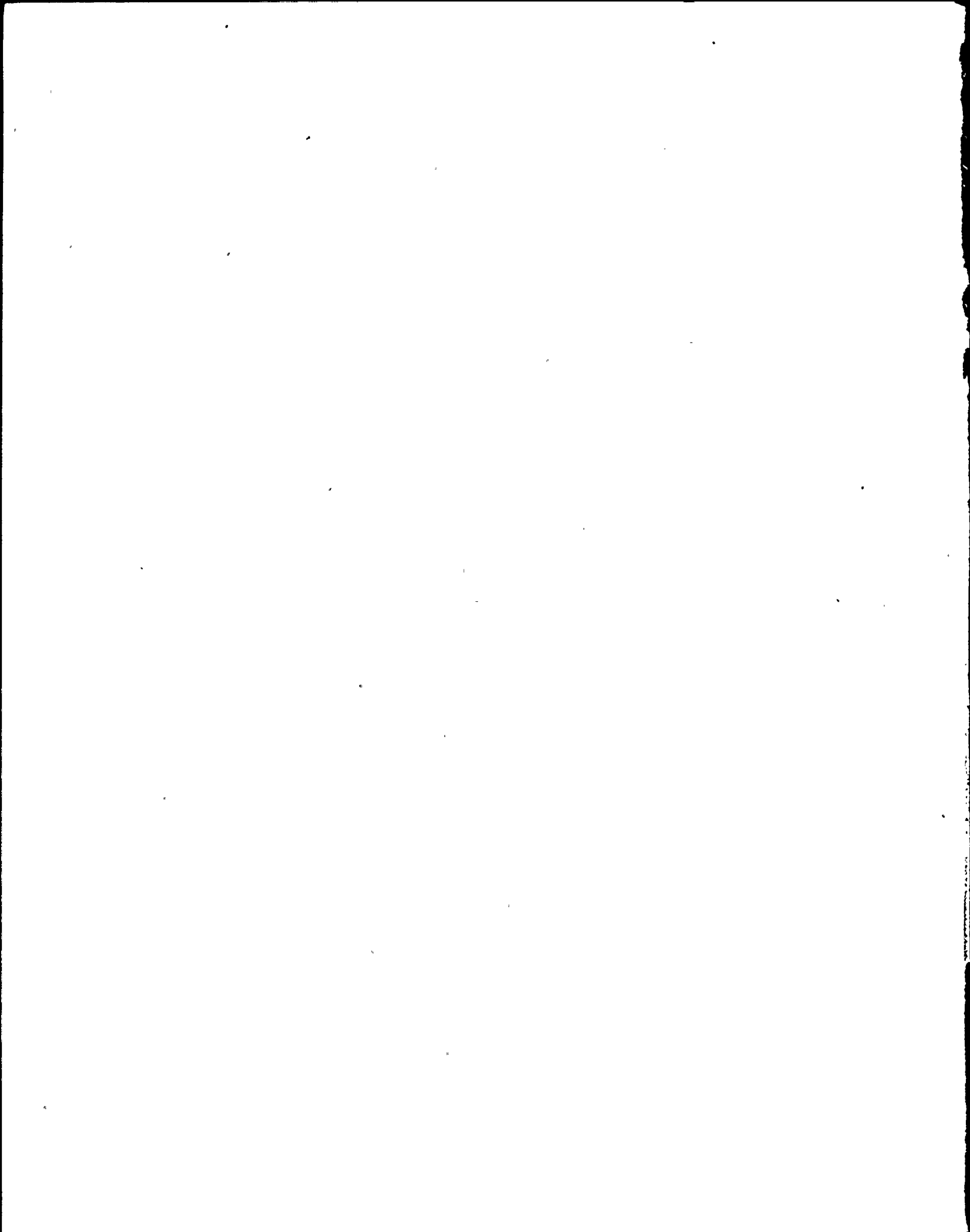
MHE-300-2, Inspection and Maintenance of 3 Ton P and H Jib Crane Serial No. T-42956 in Containment, PCN 79-2686.

MHE-400-1, Inspection and Maintenance of 18 Ton Grove Mobile Hydraulic Crane, PCN 79-2687.

MHE-500-1, Inspection and Maintenance of 1 Ton P and H Electric Operated Hoists in C.V., PCN 79-2688.

MHE-500-2, Inspection and Maintenance of 1 Ton P and H Electric Operated Hoists in the Auxiliary Building, PCN 79-2689.

MHE-600-1, Inspection and Maintenance of All Chishalm - Moore Electric Chain Hoists in the Stock Room, PCN 79-2690.



FORC
ITEM #

PROCEDURE

MHE-700-1; Inspection and Maintenance of Hand Operated Chain Hoists in the Range of 1/4 Ton to 1 Ton Capacity, PCN 79-2691.

MHE-700-2, Inspection and Maintenance of Hand Operated Chain Hoists in the Range of 1 1/2 Ton to 10 Ton Capacity, PCN 79-2692.

MHE-900-1; Inspection and Maintenance of Chishalm - Moore Lever Operated Chain Pullers (3/4, 1 1/2 and 6 Ton), PCN 79-2693.

MHE-900-2, Inspection and Maintenance of Ratcliff Lever Operated Chain Hoists, PCN 79-2694.

MHE-900-3; Inspection and Maintenance of Lug All Lever Operated Pullers, PCN 79-2695.

MHE-1000-1, Inspection of Slings, PCN 79-2696.

S-19; Failed Level Monitor System, PCN 79-2701.

S-28.2; Failed Fuel Detection Sample Line Isolation/Restoration, PCN 79-2664.

79-1734

*E-5; Control Room Inaccessibility, PCN 79-2655.

*E-28; Reactor Coolant Leak; PCN 79-2656.

*E-31; Automatic Turbine Load Runback, PCN 79-2674.

*E-32; Loss of Instrument Air; PCN 79-2675.

*M-55.2; Control Rod Drive Cables and Assemblies Maintenance and/or Inspection, PCN 79-2658.

PORC
ITEM #

PROCEDURE

*M-55.4; Permissible Rod Movement for Troubleshooting Rod Control Failure, PCN 79-2577.

T-5C; Condensate Pump Isolation, PCN 79-2626.

T-6.7, AVP Concentrated Acid Pumps - Isolating, Flushing and Returning to Normal, PCN 79-2624.

T-6.8, AVP Concentrated Caustic Pumps - Isolating, Flushing and Returning to Normal, PCN 79-2625.

79-1735 *A-7, Procedures for Handling Illness or Injuries at Ginna Station, PCN's 79-2159, 79-2373, and 79-2434.

79-1736 *A-54.4; Duty Engineer Responsibilities; PCN 79-2680.

79-1737 *CP-430.0, Calibration and/or Maintenance of Pressurizer Pressure Channel 430, PCN 79-2632.

79-1738 *E-16.2; High Iodine 131 in the Plant Vent; PCN 79-2665.

79-1739 *E-20; Immediate Boration; PCN 79-2657.

79-1740 E-23.1, Malfunction of #1 Reactor Coolant Pump Seal, PCN 79-2673.

E-23.2; Malfunction of #2 Reactor Coolant Pump Seal, PCN 79-2672.

E-23.3, Malfunction of #3 Reactor Coolant Pump Seal; PCN 79-2671.

E-23.4; Loss of Reactor Coolant Pump Seal Injection Supply, PCN 79-2670.

PORC
ITEM #

PROCEDURE

E-23.5, Reactor Coolant Pump Seal No. 1 Seal Leak-off Flow on or Off, PCN 79-2669.

E-25, Loss of Containment Integrity, PCN 79-2668.

E-30.1, Partial Loss of Condenser Vacuum, PCN 79-2663.

E-30.2, Complete Loss of Condenser Vacuum, PCN 79-2662.

E-36, Operator Actions to be Taken in the Event of Alarm Conditions on the Turbine Vibration Instruments, PCN 79-2661.

E-37.1, New UO₂ Fuel Assembly Dropping or Collision with Another Object, PCN 79-2676.

E-37.3, Spent Fuel Assembly or Rod Control Cluster Dropping or Collision with Another Object, PCN 79-2677.

79-1741 *GS-39.0, Protected Vital Area Access, PCN 79-2659.

79-1742 *HP-11.2, Iodine in Air - Charcoal Cartridge Method, PCN 79-2642.

79-1743 *HP-11.10, Air Sampling with Siersat Low Volume Air Samples, PCN 79-2643.

79-1744 *HP-12.6, Issuance, Proper Use and Return of Respirators, PCN 79-2649.

79-1745 *ISI-1.2.1, Quality Group A Inservice Inspection, PCN 79-2660.

79-1746 *O-1.1, Plant Heatup from Cold Shutdown to Hot Shutdown, PCN 79-2725.

PORC
ITEM #

PROCEDURE

79-1747 *O-2.3.2, Filling and Venting the RCS, PCN 79-2708.

79-1748 *PT-2.2, Residual Heat Removal System, PCN 79-2646.

79-1749 *PT-2.6, Cold/Refueling Shutdown Air Operated Valve Surveillance,
PCN 79-2697.

79-1750 *PT-7, Hydro Test of RCS, PCN's 79-2726 and 79-2728.

79-1751 *S-30.1, Safety Injection System Valve Position Verification,
PCN 79-2734.

79-1752 *S-30.6, Safeguard Valve Position Verification (Inside CV),
PCN 79-2727.

79-1753 *T-41C, Turbine Driven Auxiliary Feedwater Pump Restoration to
Service After Maintenance, PCN 79-2679.

79-1761 *A-201, Shift Technical Advisor Accountable to Technical Assistant
Operational Assessment Engineer, PCN 79-2647.

79-1762 *EM-235, Furmaniting MOV-700, RHR From "A" Loop, PCN 79-2722.

79-1763 *S-3.3A, Coolant Chemistry Addition Tank, PCN 79-2713.

79-1764 *A-52.1, Shift Organization and Responsibilities, PCN 79-2654.

79-1765 *A-52.1.1, Shift Relief Turnover, PCN 79-2651.

79-1766 *A-52.1.2, Shift Relief Turnover - Parameter Check, PCN 79-2652.

FORC
ITEM #

PROCEDURE

- 79-1767 *A-52.1.3, Shift Relief Turnover - Auxiliary Operators, PCN 79-2667.
- 79-1768 *E-1.5, Void Formation in the RCS, PCN 79-2582.
- 79-1769 *O-1.1, Plant Heatup From Cold Shutdown to Hot Shutdown, PCN 79-2758.
- 79-1770 *O-6.2, Main Control Board Verification, PCN 79-2650.
- 79-1771 *A-102.2, R. E. Ginna Administrative Controls Training Program, PCN 79-2746.
- 79-1772 *EM-257, Furmanite Repair - Valve 427 CVCS Letdown Isolation, PCN 79-2736.
- 79-1774 *S-9L, Batching Boric Acid and Transferring to Spent Fuel Pit, PCN 79-2700.
- 79-1775 *A-102.1, Facility Staff Training Program Ginna Station Personnel Training Program, PCN 79-2751.
- 79-1776 *A-403, Requisitioning Services from General Maintenance, PCN 79-2721.
- 79-1777 *A-604, Control of Construction Documents, PCN 79-2749.
- 79-1778 *A-1502, Non-Conformance Reports, PCN 79-2698.
- 79-1779 *E-1.6, CVCS Leak, PCN 79-2761.
- 79-1780 *E-7, Drop of a Full Length Rod Cluster Control Assembly, PCN 79-2558.

<u>PORC ITEM #</u>	<u>PROCEDURE</u>
<u>79-1781</u>	<u>*ISI-9.1, Inservice Inspection of Shell Type Anchor Bolts, PCN 79-2741.</u>
<u>79-1782</u>	<u>*M-56.1, Placement of Ceramic Fiber Penetration Seals, PCN 79-2754.</u>
<u>79-1783</u>	<u>*O-6.13, Daily Surveillance Log, PCN 79-2767.</u>
<u>79-1784</u>	<u>*PC-18.3, Determination of Chloride by Specific Ion Electrode, PCN 79-2744.</u>
<u>79-1785</u>	<u>*PT-7, RCS Hydro Test, PCN 79-2739.</u>
<u>79-1786</u>	<u>*PT-13, Fire Pump Operation and System Alignment, PCN 79-2710.</u>
<u>79-1787</u>	<u>*S-3.2D, Transferring Water From CVCS HUT's to RWST or SFP, PCN 79-2747.</u>
<u>79-1792</u>	<u>*T-27.3, Fuel Oil Receipt Emergency/Fire Diesel, PCN 79-2765.</u>
<u>79-1793</u>	<u>*T-32, Fire Service Water System, PCN 79-2714.</u>
<u>79-1794</u>	<u>*WC-9.0, Determination of Chromate, PCN 79-2705.</u>
<u>79-1802</u>	<u>*A-52.1.4, Health Physics and Chemistry Shift Changeover, PCN 79-2743.</u>
<u>79-1803</u>	<u>*PC-23.1, Emergency Sampling of Primary Coolant, PCN 79-2706.</u>
<u>79-1804</u>	<u>*PC-23.2, Containment Radiogas Sampling and Analysis During a Containment Isolation, PCN 79-2766.</u>
<u>79-1805</u>	<u>*PC-23.3, Estimate of Noble Gas Release Rate from the Plant Vent During Accident Conditions, PCN 79-2763.</u>

PORC
ITEM #

PROCEDURE

79-1806

*SC-1, Radiation Emergency Plan, PCN 79-2576.

79-1807

*SC-1.3A, Site Radiation Emergency (Shift Supervisor and Control Room). PCN's 79-2503 and 79-2421.

FORC
ITEM #

PROCEDURE

79-31 *SM 78-1890.2, Feedwater Line Drain Valves After the Containment Check Valves, PCN 78-2664. This procedure is to provide a procedure for performing modification. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-32 *SM 78-1594, A-1, Installation of Spent Fuel Pit Suction Strainer, PCN 79-29. This procedure is to describe the work necessary to install a new suction strainer for the upper section from the spent fuel pool. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE ending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-78 *SM 78-1845.1, Guide Stud Storage Racks, PCN 79-41. R. Yeckel presented this procedure change. It requested minor revisions be made to incorporate Revision 1 to Reference 1 and improve the procedure. The Committee recommended approval of this procedure change. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-90 SM 78-1594A.1, Installation of SFP Suction Strainer, PCN 79-83. This requested to N/A Steps 6.7, 6.7.1, 6.7.2, 6.7.3, 6.7.4, 6.9 and 6.9.1 as temporary strainer will not be required per field change to Drawing #10904-72.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

FORC
ITEM #

PROCEDURE

79-92

*SM 77-1499.1, Rod Insertion Limit Circuitry, PCN 78-2744. R. Yeckel presented this new procedure to provide instructions for modifying Rod Insertion Limit Circuitry as per EWR 1499 Design Criteria. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-93

*SM 78-1872.1, Manipulator Crane Gripper System Modification, PCN 78-2668. R. Yeckel presented this new procedure to describe the method to install a modified double acting gripper actuation system on the manipulator crane. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-130

SM 76-21.4, Battery Room Air Conditioning - Air Conditioner Discharge Duct South End of Battery Room, PCN 79-92.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 76-21.6, Battery Room Air Conditioning - Air Supply Duct to the Air Conditioner, PCN 79-94.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 76-21.5, Battery Room Air Conditioning - Service Water Piping Installation, PCN 79-95.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

PORC
ITEM #

PROCEDURE

SM 76-21.7, Battery Room Air Conditioning - Miscellaneous, PCN 79-93.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 74-28.1, Installation of Make-Up Lines to Condensate Storage Tanks, PCN 79-131.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 75-48.30, Rework of the Auxiliary Feedwater Crossover Piping Modification, PCN 79-132.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 77-1030.1, Neutralizing Tank Logic Modification, PCN 79-133.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 77-2110.1, Chem. Lab DI Storage Tank Cable Routing, PCN 79-134.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 77-1449.1, Diesel Fire Pump Heater, PCN 79-135.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

FORC
ITEM #

PROCEDURE

79-159

*SM 78-1594A.2, Standby Spent Fuel Pit Pump Cable Routing, PCN 79-180. R. Yeckel presented this new procedure to describe the steps necessary to install conduit and cable for the standby spent fuel pit pump. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-160

*SM 78-1594A.3, Installation of Spent Fuel Pit Cooling Backup System, PCN 79-181. R. Yeckel presented this new procedure to describe the steps necessary to isolate the system and to install butterfly valves and other mechanical components to existing plant systems. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-164

*SM 75-14.1, Batch Tank Recirculation Heat Trace Installation, PCN 79-211. J. Brown presented this temporary procedure change for review only. It requested minor revisions be made to the procedure as Field Change Request is no longer required. The Committee reviewed this procedure change. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-172

*SM 77-1682.5, Penetration Testing Modification - Penetration 143, PCN 79-202. J. Brown presented this procedure change. It requested several minor revisions be made to the procedure to incorporate Revision 1 to Bell/Schneider procedure and improve procedure. The Committee recommended approval of this procedure change. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

FORC
ITEM #

PROCEDURE

79-184 *SM-2714.1, "A" Steam Generator J-Tube Modification, PCN 78-2693.
J. Brown presented this new procedure to describe the steps necessary to install J-Tubes on top of the feedwater distribution ring header and plug the existing holes on the bottom of this ring header. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-185 *SM-2714.2, "B" Steam Generator J-Tube Modification, PCN 78-2692.
J. Brown presented this new procedure to describe the steps necessary to install J-Tubes on top of the feedwater distribution ring header and plug the existing holes on the bottom of this ring header. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-190 *SM 78-1657.1, Installation of 42" Duct for Control Room Ventilation, PCN 79-218. J. Brown presented the following procedure change. It requested that steps 6.2.1, 6.3, 6.4, 6.4.1, and 6.5 be performed in any sequence to assure job completion in a timely manner because it is not realistic to do each part of the installation in strict chronological sequential order. The Committee recommended approval of the procedure change. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-204 *SM 77-1623.1, Charging Pump Monorail, PCN 79-308. J. Brown presented this temporary procedure change for review only. It requested Steps 6.5.1, 6.5.2, and 6.5.3 be N/A'd as the monorail was not installed. The Committee reviewed this temporary procedure change. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

FORC
ITEM #

PROCEDURE

79-214

*SM 79-1880.1, Part Length Control Rod Drive Shaft Retention for Operation Without Part Length Rods, PCN 79-306.

J. Larizza presented this new procedure together with the Westinghouse Safety Evaluation, the RG&E Design Review of the part length rod removal and system modification, SM 79-1880.1 for review. Item II of the safety evaluation performed by Westinghouse addresses thimble plugs assembly mechanical devices. The plug assemblies to be installed at Ginna are of the flow mixer type and contain sixteen rather than twenty short rods. The Committee recommended approval of this new procedure and safety evaluation. This item IS NOT COMPLETE pending completion of the modification.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-215

*SM 78-2133.1, Valves 866A and 866B Modification for Testing, PCN 79-309. J. Brown presented this new procedure to describe the steps necessary to remove and reinstall Valves 866A and 866B and modify piping to install new valves for testing. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-216

*SM 79-1832A.1, Circuit Separation Fuse Installation, PCN 79-333. J. Brown presented this new procedure to describe the steps necessary to install fuse disconnect blocks and wiring in Buses 14-16-17 & 18. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

PORC
ITEM #

PROCEDURE

79-247

*SM 78-1845.1, Guide Stud Storage Racks, PCN 79-276. J. Brown presented this procedure change. It requested the following changes:

Revise Step 2.1 to read:

"2.1 Bell/Schneider Work Procedure EWR 1845, Revision 3."

Replace WP EWR 1845 Revision 1 with attached Revision 3. This is to update the procedure. The Committee recommended approval of this procedure change. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-250

*SM 79-2428, RCP Lube Oil Level Indication. J. St. Martin presented Revision 0 of the Safety Analysis and Revision 0 of the Design Criteria for this modification. The Committee performed a Safety Evaluation and determined there were no unreviewed safety questions or Technical Specification changes required. This item IS NOT COMPLETE pending completion of this modification.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-260

*SM 77-1431.1, AFW Flow Control Valve - Mechanical Installation, PCN 79-337. J. St. Martin presented this new procedure to describe the steps necessary to install the piping, valves and miscellaneous supports for the AFW Flow Control Valve Modification. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

FORC
ITEM #

PROCEDURE

79-288

*SM 77-1682.10, Hydrostatic Testing of Penetration 143 Piping Modification, PCN 79-297. J. Brown presented this procedure change. It requested minor revisions be made to make the procedure consistent and complete with respect to SM-1682.5. The Committee recommended approval of this procedure change. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-290

*SM 79-1832A.2, Circuit Separation Installation, PCN 79-351. J. Brown presented this new procedure to describe the steps necessary to install fuse disconnect blocks and wiring in Buses 13 and 15. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-308

*SM 77-1891.1, Electric Heat Tracing for Valve 825 A & B, PCN 79-443. J. Brown presented this procedure change. It requested several revisions be made to the procedure to conform with drawing change and to facilitate fabrication prior to installation activities. The Committee recommended approval of this procedure change. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

PORC
ITEM #

PROCEDURE

79-309

SM 1594A.4, Qualification and Leak Testing of Standby SFP Cooling Components, PCN 79-269. This change requested the following change to Step 6.4.3:

"Step 6.4.3 Service Water Inlet and Outlet Temperatures not taken during test."

This is because temperatures are not necessary for Engineering data.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 78-1872.1, Manipulator Crane Gripper System Modification, PCN 79-407. This change requested minor revisions be made to the procedure to facilitate the change as required without changing the intent.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 79-2417.1, "A" Steam Generator J-Tube Modification, PCN 79-415. This change requested a change to Step 6.2.4 - procedure should be 'M-43.24.1' not 'M-43.23.1'. This is to correct the incorrectly referenced procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 79-2417.1, "A" Steam Generator J-Tube Modification, PCN 79-437. This change requested minor revisions to Steps 3.4 and 3.20.1 as ventilation requirements and air sampling requirements were modified.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

PORC
ITEM #

PROCEDURE

SM 79-2417.2, "B" Steam Generator J-Tube Modification, PCN 79-414. This change requested a change to Step 6.2.4 - procedure should be 'M-43.24.1' not 'M-43-23.1'. This is to correct the incorrectly referenced procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 79-2417.2, "B" Steam Generator J-Tube Modification, PCN 79-438. This change requested minor revisions to Steps 3.4 and 3.20.1 as ventilation requirement and air sampling requirements were modified.

The Committee reviewed these temporary procedure changes. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-337

SM 78-1464.1, Replacement of Volume Control Tank Level Transmitters, PCN 79-493. This requested minor changes be made to the procedure to ensure proper hookup of the system.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 78-1464.2, Replacement of Accumulator Level Transmitters, LT-935, LT-934, LT-938 and LT-939, PCN 79-494. This requested minor changes be made to the procedure to ensure proper hook-up of the system.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

PORC
ITEM #

PROCEDURE

SM 77-1623.1, Charging Pump Monorails, PCN 79-470.
This requested to mark "Step 6.7 N/A" as Brady letters will be applied after painting is completed after shutdown for "C" Charging Pump.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 78-1890.2, Feedwater Line Drain Valves After the Containment Check Valves, PCN 79-503. This requested the delineation of Step 4.4 to allow contractor to drill as they see fit: pilot hole, then larger hole, using electric drill.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 78-2133.1, Valves 866A and 866B Modification for Testing, PCN 79-509. This requested the following steps be added to Step 6.1.

- | | | | |
|---------|---------------------------|---------------|-------|
| 6.1.4 | Ensure V-868A open. | V-868A Open | _____ |
| 6.1.5 | Ensure V-868B open. | V-868B Open | _____ |
| 6.1.6 | Open Spray Header Drain. | V-2825 Open | _____ |
| 6.1.7 | Open Spray Header Drain. | V-2826 Open | _____ |
| 6.6.2.1 | Close Spray Header Drain. | V-2825 Closed | _____ |
| 6.6.2.2 | Close Spray Header Drain. | V-2826 Closed | _____ |

These were omitted from the procedure when last issued.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

FORC
ITEM #

PROCEDURE

SM 79-2417.2, "B" Steam Generator J-Tube Modification, PCN 79-501. This requested the delineation of Steps 6.16.1 through 6.16.1.3; 6.16.1.4 and in Step 6.16.13.1 delete words "after grinding". Add new Step 6.16.1 to read: "Placement of feedwater ring plugs at the feedwater ring to tee connections to be done as per Westinghouse field change Number 3." This is because supports in the area are located differently than shown on drawings.

The Committee reviewed these temporary changes. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-353

*SM 77-1431.2, Hydrostatic Testing of Auxiliary Feedwater Modification, PCN 79-597. J. St. Martin presented this new procedure to perform required tests of the modified piping for the 1A and 1B Auxiliary Feedwater Pumps. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE pending completion of this modification.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-388

*SM 77-1431.4, Installation of Electrical Equipment for Auxiliary Feedwater Flow Control Bypass Valves, PCN 79-631. J. St. Martin presented this new procedure to describe the steps needed to install miscellaneous electric equipment for the Auxiliary Feedwater Flow Control Modification. Mechanical installation is in accordance with SM 77-1431.1. Electrical terminations and checkout are in accordance with SM 77-1431.3. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

PORC
ITEM #

PROCEDURE

79-396

*SM 78-1838.1, Containment Wall-Mounted Jib Crane, PCN 79-671. J. St. Martin presented this new procedure to provide guidelines for installation of the EWR-1838 Wall-Mounted Jib Crane in Containment. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-728

*SM 75-9.24, RWST Jet Shielding Installation Rework, PCN 79-1020. J. Brown presented this procedure for deletion because work has been completed and procedure is no longer needed. The Committee recommended approval of this deletion. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-763

*SM 75-6.23, Service Water Redundant Return Line Support Assemblies Installation, PCN 79-1093. This procedure is being deleted as the work has been completed and the procedure is no longer needed.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

*SM 75-50.31, Jet Shielding Rework, Main Steam and Feedwater, PCN 79-1118. This procedure is being deleted as the work has been completed and the procedure is no longer needed.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

*SM 76-01.1, Safeguard Buses DC Control Voltage Monitor, PCN 79-1117 This procedure is being deleted as the work has been completed and the procedure is no longer needed.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

PORC
ITEM #

PROCEDURE

*SM 76-21.8, Reroute of Service Water Supply to Battery Room Air Conditioner, PCN 79-1116. This procedure is being deleted as the work has been completed and the procedure is no longer needed.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

*SM 77-1474.1, Reactor Coolant Pump Seal Bypass Modification, PCN 79-1115. This procedure is being deleted as the work has been completed and the procedure is no longer needed.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

*SM 77-1499.1, Rod Insertion Limit Circuitry, PCN 79-1114. This procedure is being deleted as the work has been completed and the procedure is no longer needed.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

*SM 77-1623.1, Charging Pump Monorails, PCN 79-1113. This procedure is being deleted as the work has been completed and the procedure is no longer needed.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

*SM 77-1623.2, Wedge Washer Installation - Charging Pump Monorail, PCN 79-1112. The procedure is being deleted as the work has been completed and the procedure is no longer needed.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

PORC
ITEM #

PROCEDURE

*SM 77-1891.1; Electric Heat Tracing for Valves 825A and 825B, PCN 79-1110. This procedure is being deleted as the work has been completed and the procedure is no longer needed.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

*SM 78-1464.1; Replacement of Volume Control Tank Level Transmitters LT-139 and LT-112, PCN 79-1108. This procedure is being deleted as the work has been completed and the procedure is no longer needed.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

*SM 78-1464.2; Replacement of Accumulator Level Transmitters LT-935, LT-934, LT-938, and LT-939, PCN 79-1107. This procedure is being deleted as the work has been completed and the procedure is no longer needed.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

*SM 78-1890.2; Feedwater Line Drain Valves after the Containment Check Valves, PCN 79-1101. This procedure is being deleted as the work has been completed and the procedure is no longer needed.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

*SM 78-2133.1; Valves 866A and 866B Modification for Testing, PCN 79-1100. This procedure is being deleted as the work has been completed and the procedure is no longer needed.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

PORC
ITEM #

PROCEDURE

*SM 78-2273.1, Armor Plate on the East Wall of the Control Room.
 This procedure is being deleted as the work has been completed and the procedure is no longer needed.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-839

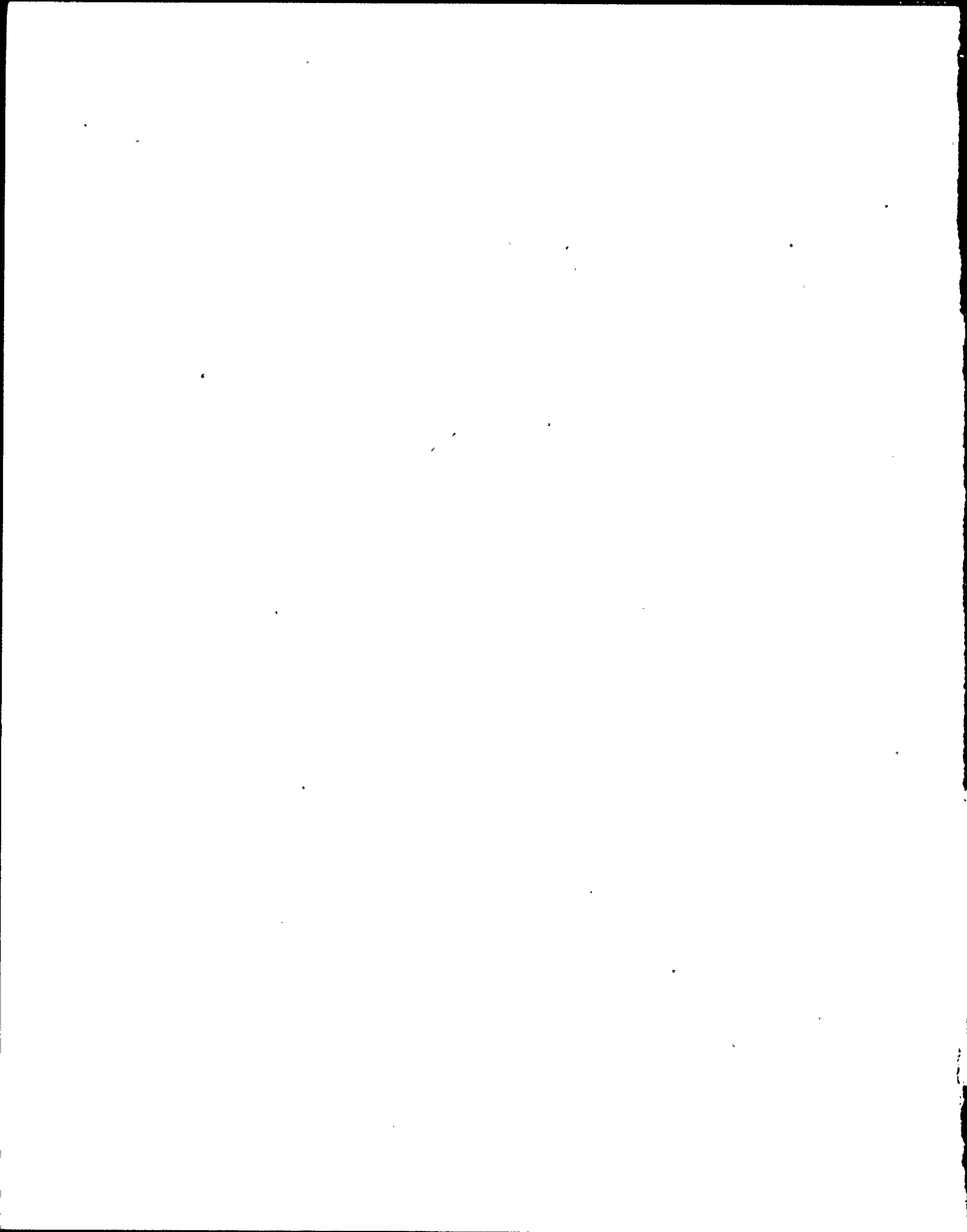
*SM 77-1431.3, Auxiliary Feedwater Flow Control Bypass Modification Termination and Checkout, PCN 79-1210. J. Brown presented this procedure change. It requested minor changes be made to agree with field change request. The Committee recommended that the PCN Reason should read "to agree with revised drawing" not "to agree with field change request". This is due to the fact that Engineering is taking care of the drawing revision. The Committee reviewed and recommended approval of this procedure change with modifications. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-882

*SM 77-1431.3, Auxiliary Feedwater Flow Control Bypass Modification Termination and Checkout, PCN 79-1260. J. Brown presented this procedure change. It requested that minor changes be made to the procedure to agree with Buffalo Electric drawings #12676-2R D, Revision IX and 12676-3R D, Revision VIII. The Committee reviewed and recommended approval of this procedure change. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.



PORC
ITEM #

PROCEDURE

79-893

*SM 79-2449.1, Safety Injection Logic Modification, PCN 79-1278. C. Rioch presented this new procedure to remove the pressurizer low level coincident with pressurizer low pressure signal for automatic safety injection initiation and install a modified design which will initiate safety injection when any two out of three pressurizer low pressure signals are received. The Committee determined that this procedure as written meets the test requirements of the design criteria. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE pending completion of this modification.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-924

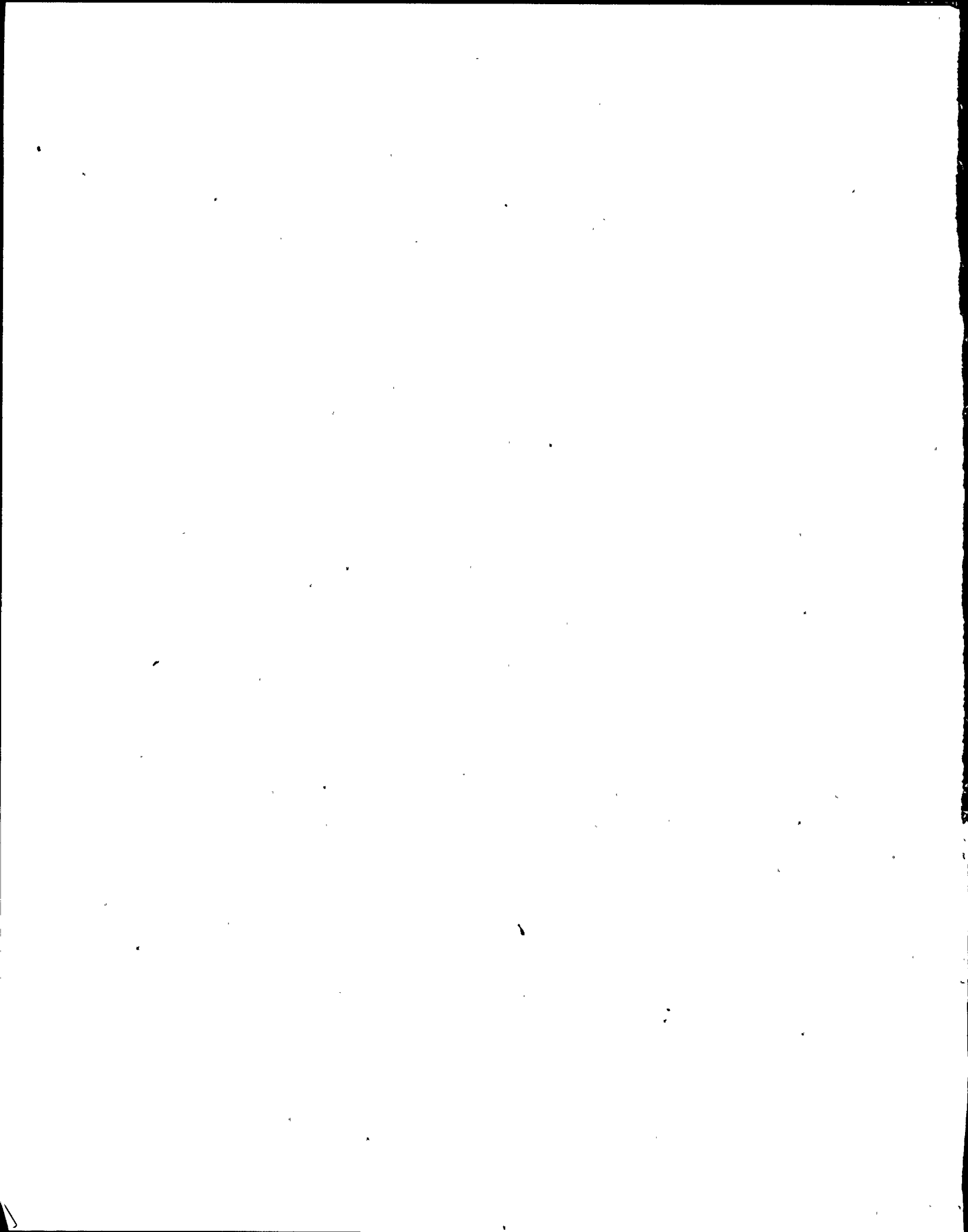
*SM 77-1431.3, Auxiliary Feedwater Flow Control Bypass Modification Termination and Checkout, PCN 79-1279. J. Brown presented this procedure change. It requested that minor changes be made to conform the procedure with the wiring diagrams. The Committee reviewed and recommended approval of this procedure change. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79 953

*SM 77-1431.3, Auxiliary Feedwater Flow Control Bypass Modification Termination and Checkout, PCN 79-1342. J. Brown presented this procedure change. It requested that minor changes be made to correct wording and to correct typographical errors. The Committee reviewed and recommended approval of this procedure change. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.



PORC
ITEM #

PROCEDURE

79-957

*SM 79-01.1, Feedwater Drain Lines Inside Containment, PCN 79-1359. J. Brown presented this new procedure. It is to describe the steps necessary to install a drain line with end cap on each feedwater li at the low point inside of containment. The Committee reviewed and recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-998

*SM 77-1660.50, RCS Overpressurization Protection System Annunciation PCN 79-1459. J. Brown presented this new procedure to the Committee for review. The purpose of this new procedure is to provide instruction for the installation of the alarm circuit to monitor the alignment of the PORV's isolation valves (MOV-515 and MOV-516). The Committee reviewed and recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed modification.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-1029

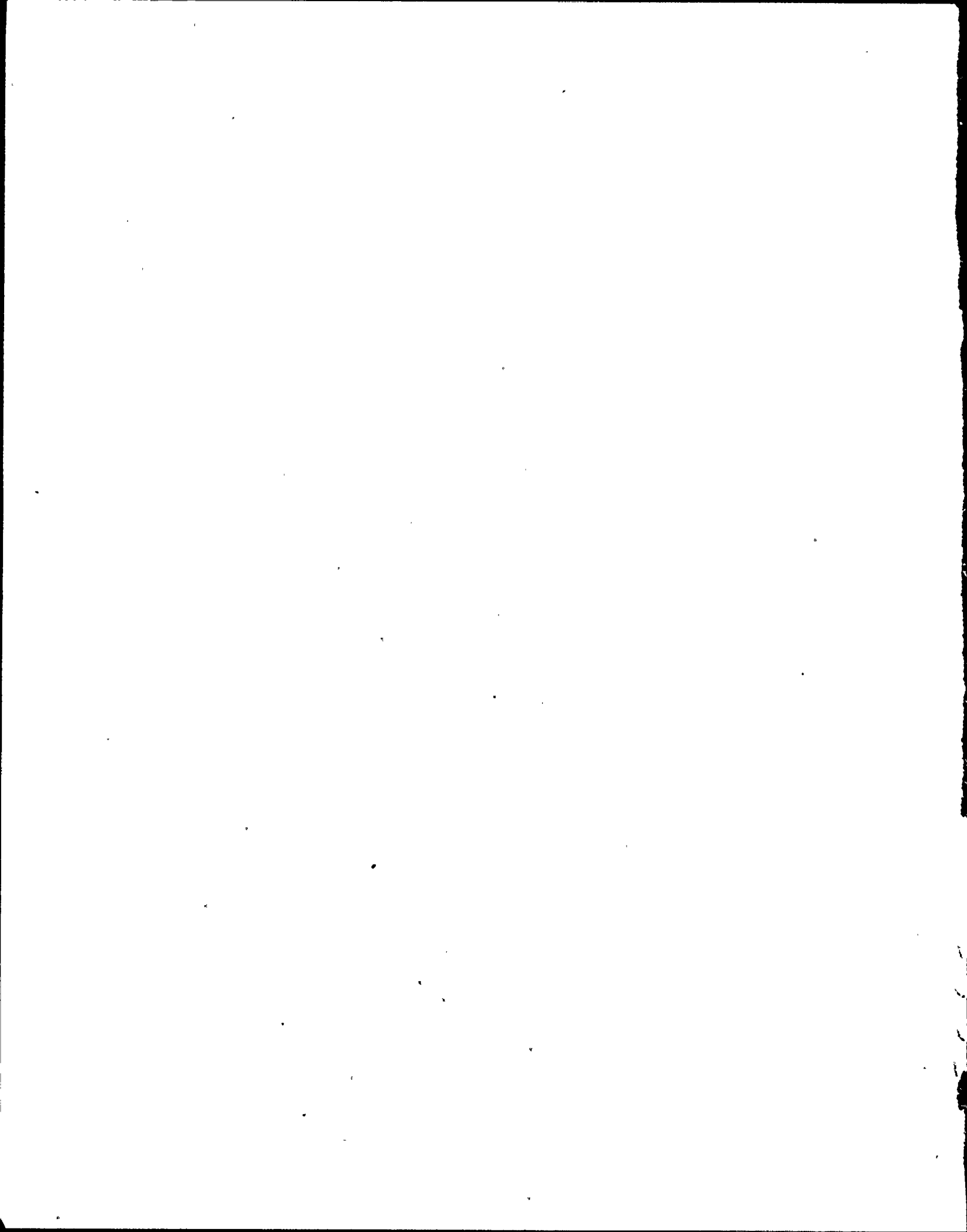
*SM 79-1662.1, Installation of Snubber Platforms, PCN 79-1510. J. Brown presented this new procedure to the Committee for review. The purpose of this procedure is to provide the guidelines necessary for installing Snubber Maintenance Platforms. The Committee reviewed and recommended approval of this new procedure. This item IS NOT COMPLETE pending completion of the modification.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-1052

*SM 79-2449.2, Bistable PC-449B, Control Channel Change (Field Change Request #2449) PCN 79-1603. J. St. Martin presented this new procedure to provide the steps necessary for interchanging the PC-449B bistable to actuate on Channel 431 which will interlock PCV-431C independently from 449, the normal controlling channel. The Committee recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.



FORC
ITEM #

PROCEDURE

79-1093

*SM 77-1660.50, RCS Overpressurization Protection System Annunciation PCN 79-1624. L. Depew presented this procedure change. It requested that Step 6.14 be N/A'd for O-7 as after discussion with the Operations Section it shows no need to revise O-7. The Committee reviewed and recommended approval of this procedure change. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-1141

*SM 79-1837.1, Computer Room Ceiling, PCN 79-1712. J. St. Martin presented this new procedure. It is to provide guidelines for installing a fire-rated ceiling in the computer room. The Committee reviewed and recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-1145

*SM 78-1875.2, "A" Feedwater Venturi Monorail Installation of Hoist, PCN 79-1711. L. Depew presented this new procedure. It is to provide the guidelines for installing and retraining the hoist for the "A" Feedwater Venturi Monorail. The Committee reviewed and recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-1148

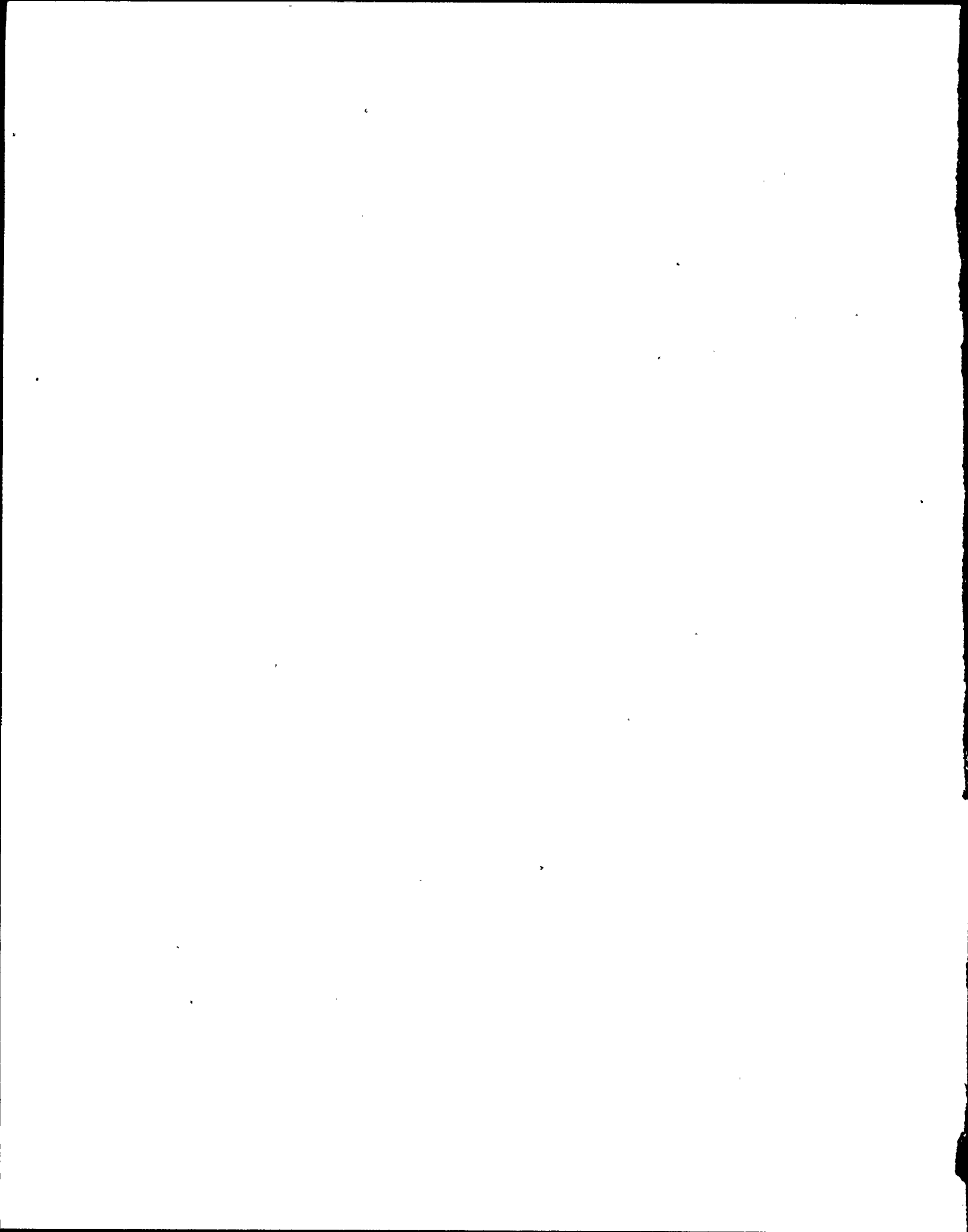
*L. Depew presented the following procedures for deletion as the procedure has been completed:

SM 75-5.38, Standby Auxiliary Feedwater System Operational Testing, PCN 79-1730.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 75-5.39, Standby Auxiliary Feedwater System Rework, PCN 79-1729.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.



FORC
ITEM #

PROCEDURE

SM 75-5.41, Standby Auxiliary Feedwater Cable and Conduit Installation Rework, PCN 79-1728.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 75-5.42, Electrical Installation and Cable Termination Rework for Air Handling Units, PCN 79-1727.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 75-5.43, Rework of Standby Auxiliary Feedwater System, PCN 79-1726.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 75-5.44, Rework of SARWP Seal Piping, PCN 79-1725.

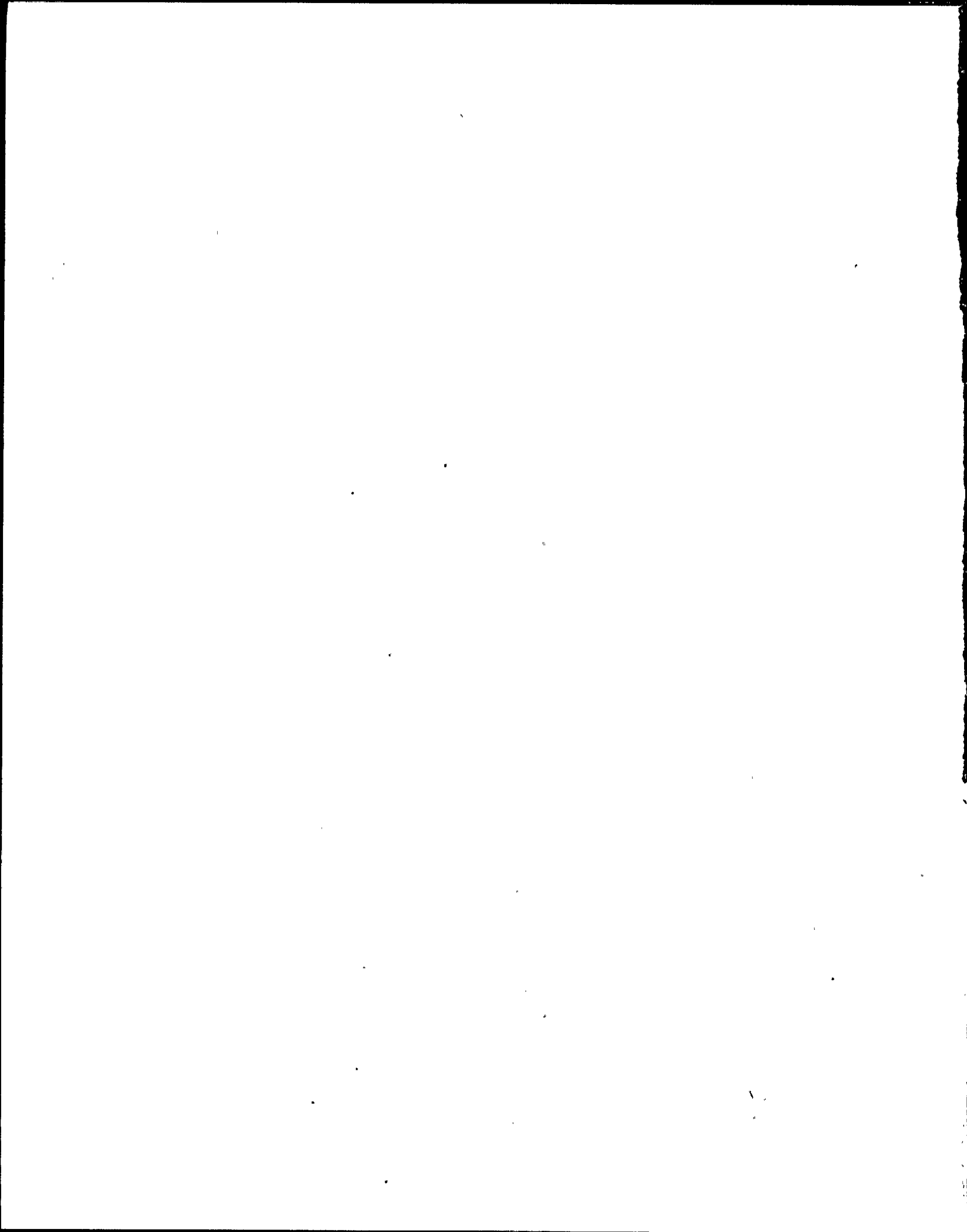
A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 75-10.28, Service Water Piping to Air Handling Units for Auxiliary Building Addition Rework, PCN 79-1724.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 75-34.23, Pipe Restraints and Jet Shields Rework for Steam Generator Blowdown Lines, PCN 79-1723.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.



PORC
ITEM #

PROCEDURE

SM 77-1660.50, RCS Overpressurization Protection System
Annunciation PCN 79-1722.

The Committee reviewed and recommended approval of these procedures for deletion. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-1416

*SM 77-1850.1, Fire Barrier Penetration Seals Installation, PCN 79-2247. C. Rioch presented this new procedure to provide the directions necessary to install and properly document fire barrier penetration seals and to upgrade existing fire barriers to meet present Technical Specification requirements. The Committee reviewed and recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-1683

*SM 79-1837.2, Fire Doors Installation, PCN 79-2589. J. St. Martin presented this new procedure to the Committee for review. The purpose of this procedure is to provide the guidelines for removing existing plant doors and replacing them with fire-rated frames, doors and associated hardware. The Committee reviewed and recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed modification.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-1693

*SM 79-2603.1, Pressurizer Safety Valve Direct Position Indicator, PCN 79-2629. J. St. Martin presented this new procedure. The purpose of this procedure is to provide the guidelines necessary to install LVDT and associated equipment for monitoring and alarming pressurizer safety valve position. The Committee reviewed and recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

PORC
ITEM #

PROCEDURE79-1694

*SM 79-2603A.1, Pressurizer PORV Direct Position Indicator to Computer, PCN 79-2631. J. St. Martin presented this new procedure. The purpose of this new procedure is to provide the instructions necessary for installation of the equipment for detecting PORV valve position and monitoring and alarming via the plant computer. The Committee reviewed and recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-1695

*SM 79-2604.1, RCS Subcooling Margin Monitoring System, PCN 79-2630. J. St. Martin presented this new procedure. The purpose of this new procedure is to provide the guidelines necessary to install the RCS Subcooling Margin Monitoring System. The Committee reviewed and recommended approval of this new procedure. This item IS NOT COMPLETE pending closeout review of the completed procedure.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-1700

SM 79-01.1, Feedwater Drain Lines Inside Containment, PCN 79-2638. As the modification is complete and closed out.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

SM 77-06.2, Low Pressure Purification System Modification - Hydrostatic Test, PCN 79-2640. As the procedure is complete.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

PORC
ITEM #

PROCEDURE

SM 77-06.3, Low Pressure Purification System Modification, PCN 79-2639. As the procedure is complete.

The Committee reviewed and recommended approval of these procedures for deletion. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

79-1715

*SM 77-1836.2, Relocation of Halon System, PCN's 79-2611 and 79-2616
L. Depew presented these procedure changes. They requested that minor changes be made as relocation of the Halon System will be administered by RG&E instead of Bell/Schnieder as originally planned and to update the procedure. The Committee reviewed and recommended approval of these procedure changes. This item IS COMPLETE.

A Safety Evaluation was performed and it was determined that the possibility of an accident is not increased. A change in the Plant Operating Technical Specifications is not necessary.

