

2008/02/14

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)  
DISTRIBUTION FOR INCOMING MATERIAL 50-335

REC: OREILLY J P  
NRC

ORG: SCHMIDT A D  
FL PWR & LIGHT

DOC DATE: 07/07/78  
DATE RCVD: 07/31/78

DOCTYPE: LETTER NOTARIZED: NO  
SUBJECT:

COPIES RECEIVED  
LTR 1 ENCL 1

FORWARDING LICENSEE EVENT REPT (RO 50-335/78-021) ON 06/08/78 CONCERNING  
DURING PWR ASCENSION TESTING FOLLOWING REFUELING OUTAGE, CEA #65 DROPPED FOUR  
TIMES DUE TO FAILURE OF ONE OR MORE OF ITS COIL PWR PROGRAMMER TIMING MODULE,  
INTEGRAL TIMER , OR 15 VO

PLANT NAME: ST LUCIE #1

REVIEWER INITIAL: XJM  
DISTRIBUTOR INITIAL: DL

\*\*\*\*\* DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS \*\*\*\*\*

INCIDENT REPORTS  
(DISTRIBUTION CODE A002)

FOR ACTION: BR CASE ORB#4 BC\*\*W/4 ENCL

INTERNAL:

REG FILE\*\*W/ENCL  
I & E\*\*W/2 ENCL

I & C SYSTEMS BR\*\*W/ENCL  
NOVAK/CHECK\*\*W/ENCL  
AD FOR ENG\*\*W/ENCL  
HANAUER\*\*W/ENCL  
AD FOR SYS & PROJ\*\*W/ENCL  
ENGINEERING BR\*\*W/ENCL  
KREGER/J. COLLINS\*\*W/ENCL  
K SEYFRIT/IE\*\*W/ENCL

NRC PDR\*\*W/ENCL  
MIPC\*\*W/3 ENCL  
EMERGENCY PLAN BR\*\*W/ENCL  
EEB\*\*W/ENCL  
PLANT SYSTEMS BR\*\*W/ENCL  
AD FOR PLANT SYSTEMS\*\*W/ENCL  
REACTOR SAFETY BR\*\*W/ENCL  
VOLLMER/BUNCH\*\*W/ENCL  
POWER SYS BR\*\*W/ENCL

EXTERNAL:

LPDR'S  
FT PIERCE, FL\*\*W/ENCL  
TIC, LIZ CARTER\*\*W/ENCL  
NSIC\*\*W/ENCL  
ACRS CAT B\*\*W/16 ENCL

DISTRIBUTION: LTR 45 ENCL 45  
SIZE: 1P+1P+1P

CONTROL NBR: 782140095

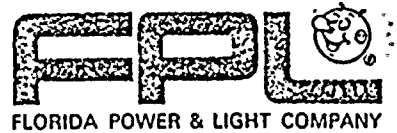
AO 4  
30

\*\*\*\*\* THE END \*\*\*\*\*



[The text in this section is extremely faint and illegible. It appears to be a multi-paragraph document, possibly a letter or a report, but the specific words and sentences cannot be discerned.]

REGULATORY DOCKET FILE COPY



July 7, 1978

PRN-LI-78-182

Mr. James P. O'Reilly, Director, Region II  
Office of Inspection and Enforcement  
U. S. Nuclear Regulatory Commission  
230 Peachtree Street, N. W., Suite 1217  
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

REPORTABLE OCCURRENCE 335-78-21  
ST. LUCIE UNIT 1  
DATE OF OCCURRENCE: JUNE 8, 1978

TECHNICAL SPECIFICATION 3.1.3.1.e  
CEA 68

1978 JUL 31 AM 10 35  
RECEIVED DISTRIBUTION SERVICES UNIT  
US NRC DISTRIBUTION SERVICES BRANCH

The attached Licensee Event Report is being submitted in accordance with Technical Specification 6.9 to provide notification of the subject occurrence.

Very truly yours,

*A. D. Schmidt*  
A. D. Schmidt  
Vice President  
Power Resources

MAS/cpc

Attachment

cc: Harold F. Reis, Esquire  
Director, Office of Inspection and Enforcement (30)  
Director, Office of Management Information and  
Program Control (3)

782140095

*1002  
5/11*



100

100

100

100

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | | ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

⑦ 0 ⑧ 1 | F | I | L | S | I | S | I | | ② | 0 | 0 | 1 | - | 1 | 0 | 1 | 0 | 1 | 0 | - | 1 | 0 | 1 | 0 | | ③ | 4 | 1 | 1 | 1 | 1 | 1 | | ④ | | | | | ⑤

CON'T  
⑦ ⑧ 0 ⑨ 1 | REPORT SOURCE | L | ⑤ | 0 | 1 | 5 | 1 | 0 | 1 | 0 | 1 | 3 | 1 | 3 | 5 | ⑦ | 0 | 1 | 6 | 1 | 0 | 8 | 7 | 1 | 8 | ③ | 0 | 1 | 7 | 1 | 0 | 1 | 7 | 1 | 7 | 1 | 8 | | ⑨

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

⑩ ⑭ | During power ascension testing following a refueling outage, CEA #68 |  
⑭ ⑮ | dropped four times within an eight and one-half hour time span. After |  
⑮ ⑯ | each of the first three drops, CEA #68 was restored to its normal position |  
⑯ ⑰ | within the time limit required by the applicable Technical Specification |  
⑰ ⑱ | 3.1.3.1.e action statement. After the fourth drop, CEA #68 was declared |  
⑱ ⑳ | inoperable. Several components were replaced, after which CEA #68 was |  
㉑ ㉒ | operated satisfactorily and declared operable. |

⑦ ⑧ ⑨ ⑩ SYSTEM CODE | ⑪ ⑫ CAUSE CODE | ⑬ ⑭ CAUSE SUBCODE | ⑮ ⑯ COMPONENT CODE | ⑰ ⑱ COMP. SUBCODE | ⑲ ⑳ VALVE SUBCODE  
⑦ ⑧ ⑨ ⑩ R | B | ⑪ ⑫ E | ⑬ ⑭ G | L | I | N | I | S | I | T | I | R | U | ⑮ ⑯ P | ⑰ ⑱ Z |  
⑰ ⑱ LEA/RO REPORT NUMBER | ㉑ ㉒ EVENT YEAR | ㉓ ㉔ SHUTDOWN METHODCD | ㉕ ㉖ SEQUENTIAL REPORT NO. | ㉗ ㉘ ATTACHMENT SUBMITTED | ㉙ ㉚ OCCURRENCE CODE | ㉛ ㉜ REPORT TYPE | ㉝ ㉞ REVISION NO. | ㉟ ㊱ COMPONENT MANUFACTURER  
⑰ ⑱ A | ㉑ ㉒ 7 1 8 | ㉓ ㉔ Z | ㉕ ㉖ 0 2 1 1 | ㉗ ㉘ Y | ㉙ ㉚ 0 1 3 | ㉛ ㉜ L | ㉝ ㉞ 0 | ㉟ ㊱ C 4 9 1 0 |  
⑰ ⑱ ACTION TAKEN | ㉑ ㉒ FUTURE ACTION | ㉓ ㉔ EFFECT ON PLANT | ㉕ ㉖ SHUTDOWN METHODCD | ㉗ ㉘ HOURS | ㉙ ㉚ ATTACHMENT SUBMITTED | ㉛ ㉜ NPRO-1 FORM SUB. | ㉝ ㉞ PRIME COMP. SUPPLIER | ㉟ ㊱ COMPONENT MANUFACTURER  
⑰ ⑱ A | ㉑ ㉒ Z | ㉓ ㉔ B | ㉕ ㉖ Z | ㉗ ㉘ 0 1 0 1 5 | ㉙ ㉚ Y | ㉛ ㉜ N | ㉝ ㉞ N | ㉟ ㊱ C 4 9 1 0 |

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ⑳

㉑ ㉒ | CEA #68 dropped due to failure of one or more of its coil power programmer |  
㉓ ㉔ | timing module, integral timer, or 15 volt power supplier. Since all three |  
㉕ ㉖ | were replaced, it is not known which was the cause of the CEA drops. CEA |  
㉗ ㉘ | #68 was restored to its normal position and has performed normally for four |  
㉙ ㉚ | weeks. |

⑦ ⑧ ⑨ ⑩ FACILITY STATUS | ⑪ ⑫ % POWER | ⑬ ⑭ OTHER STATUS | ⑮ ⑯ METHOD OF DISCOVERY | ⑰ ⑱ DISCOVERY DESCRIPTION  
⑦ ⑧ ⑨ ⑩ B | ⑪ ⑫ 0 8 1 0 | ⑬ ⑭ NA | ⑮ ⑯ A | ⑰ ⑱ Operator Observation

⑦ ⑧ ⑨ ⑩ ACTIVITY CONTENT RELEASED OF RELEASE | ⑪ ⑫ AMOUNT OF ACTIVITY | ⑬ ⑭ LOCATION OF RELEASE  
⑦ ⑧ ⑨ ⑩ Z | ⑪ ⑫ Z | ⑬ ⑭ NA | ⑮ ⑯ NA

⑦ ⑧ ⑨ ⑩ PERSONNEL EXPOSURES NUMBER | ⑪ ⑫ TYPE | ⑬ ⑭ DESCRIPTION  
⑦ ⑧ ⑨ ⑩ 0 | ⑪ ⑫ 0 | ⑬ ⑭ Z | ⑮ ⑯ NA

⑦ ⑧ ⑨ ⑩ PERSONNEL INJURIES NUMBER | ⑪ ⑫ DESCRIPTION  
⑦ ⑧ ⑨ ⑩ 0 | ⑪ ⑫ 0 | ⑬ ⑭ 0 | ⑮ ⑯ NA

⑦ ⑧ ⑨ ⑩ LOSS OF OR DAMAGE TO FACILITY TYPE | ⑪ ⑫ DESCRIPTION  
⑦ ⑧ ⑨ ⑩ Z | ⑪ ⑫ NA

⑦ ⑧ ⑨ ⑩ PUBLICITY ISSUED DESCRIPTION  
⑦ ⑧ ⑨ ⑩ N | ⑪ ⑫ NA

Additional Event Description

The coil power programmer timing module, the integral timer, and the 15 volt power supplier for CEA #68 were replaced, after which CEA #68 operated satisfactorily. CEA #68 was aligned to its normal position and power ascension testing continued.

As a result of the reactivity insertions of the dropped CEA's, azimuthal power tilt ( $T_q$ ) increased and twice exceeded the limit requiring action in accordance with Technical Specification 3.2.4. Also, the total planar radial peaking factor ( $F_{xy}$ ) increased and exceeded the limit requiring action in accordance with Technical Specification 3.2.2 several times. In all of the above cases, reactor power was reduced, and both  $T_q$  and  $F_{xy}$  were returned to normal within the time limits described in their respective Specifications.