



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

SEP 15 1998

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U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attn: Document Control Desk

**MONTHLY OPERATING REPORT
SALEM UNIT NO. 2
DOCKET NO. 50-311**

Gentlemen:

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original Monthly Operating report for August 1998 is attached.

Sincerely,

A. C. Bakken III
General Manager -
Salem Operations

/rbk
Enclosures

C Mr. H. J. Miller
Regional Administrator USNRC, Region 1
475 Allendale Road
King of Prussia, PA 19046

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The power is in your hands.

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DOCKET NO.: 50-311
 UNIT: Salem 2
 DATE: 9/15/98
 COMPLETED BY: R. Knieriem
 TELEPHONE: (609) 339-1782

Reporting Period: August 1998

OPERATING DATA REPORT

Design Electrical Rating (MWe-Net)
 Maximum Dependable Capacity (MWe-Net)

No. of hours reactor was critical
 No. of hours generator was on line (service hours)

Unit reserve shutdown hours
 Net Electrical Energy (MWH)

1115		
1106		
Month	Year-to-date	Cumulative
545	4736	85942
524	4623	82687
0.0	0.0	0.0
551651	4923208	82098793

UNIT SHUTDOWNS

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/COMMENT
4	8/1/98 - 8/10/98	F	220	B	1	Pressurizer Safety Valve Replacement

(1) Reason

- A - Equipment Failure (Explain)
- B - Maintenance or Test
- C - Refueling
- D - Regulatory Restriction
- E - Operator Training/License Examination
- F - Administrative
- G - Operational Error (Explain)
- H - Other

(2) Method

- 1 - Manual
- 2 - Manual Trip/Scram
- 3 - Automatic Trip/Scram
- 4 - Continuation
- 5 - Other (Explain)

Summary:

Salem Unit 2 began the month of August 1998 shutdown to replace three pressurizer safety valves and to conduct various technical specification surveillances. This shutdown was completed on August 10. Salem Unit 2 returned to full power on August 11, and operated at full power for the remainder of the month.

DOCKET NO.: 50-311
UNIT: Salem 2
DATE: 915/98
COMPLETED BY: R. B. Knieriem
TELEPHONE: (609) 339-1782

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE SALEM UNIT 2 GENERATING STATION

MONTH: August 98

The following items completed during **August 1998** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant; nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes - Summary of Safety Evaluations

2EC-0189, Pkg. 1, Replacement Of Pressurizer Safety Valves 2PR3, 2PR4, and 2PR5.

This modification replaced the existing seismic category I, safety-related Unit 2 Pressurizer Safety Valves (PSVs) 2PR3, 2PR4, and 2PR5 with new seismic category I safety-related valves meeting the requirements of ASME Section III, Class A, 1968 Edition through S68 Addendum. The replacement valves maintained the same internals and performance characteristics as the existing valves but differ with regard to the valve body and bolting arrangement. The installation of the new valves also incorporated new position indicators of a more reliable and modern design.

Review of this modification under 10CFR50.59 was required because the replacement of the PSVs constituted a change to the facility as described in the Safety Analysis Report (SAR). This review determined that the change did not increase the probability or the consequences of an accident previously evaluated in the SAR because the modification is bounded by existing accident evaluations. The review concluded that the change would not increase the

probability or the consequences of a malfunction of equipment important to safety and would not create the possibility of an accident or malfunction of a different type from any previously evaluated in the SAR, because the change did not modify the operation or the design basis function of any safety equipment. Because the change did not affect the existing analysis that forms the basis for the Technical Specifications, and will not violate Technical Specification and Updated Final Safety Analysis Report (UFSAR) requirements, the change will not reduce the margin of safety as defined in the basis for the Technical Specifications.

2EE-0327, Elimination Of Reactor Coolant System (RCS) Valve Stem Leakoff Lines

This modification eliminated the valve stem leak-off lines from valves 2PS1, 3, 24, 25, 28, 29; and 2PR6, and 2PR7 in the Salem Unit 2 Coolant Pressurizing and Pressure Relief systems in order to reduce identified RCS leakage to the Pressurizer Relief Tank (PRT).

Review of this modification under 10CFR50.59 was required because the elimination of the RCS system valve stem leakoff lines constituted a change to the facility as described in the Safety Analysis Report (SAR). This review determined that the change did not increase the probability or consequences of an accident or the malfunction of equipment important to safety previously evaluated in the SAR because the modification is bounded by existing accident evaluations. The review concluded that the modification would not create the possibility of an accident or malfunction of a different type from any previously evaluated in the SAR, because the change did not adversely impact the function of the valves that were modified. The modification eliminated the routing of any potential valve packing leakage to the PRT. Because the change did not affect the existing analysis that forms the basis for the Technical Specifications, and will not violate Technical Specification and Updated Final Safety Analysis Report (UFSAR) requirements, the change will not reduce the margin of safety as defined in the basis for the Technical Specifications.

Temporary Modifications - Summary of Safety Evaluations

There were no changes in this category implemented during August 1998.

Procedures - Summary of Safety Evaluations

There were no changes in this category implemented during August 1998.

UFSAR Change Notices - Summary of Safety Evaluations

There were no changes in this category implemented during August 1998.

Deficiency Reports - Summary of Safety Evaluations

There were no changes in this category implemented during August 1998.

Other - Summary of Safety Evaluations

There were no changes in this category implemented during August 1998.