PSEG

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

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June 15, 1998

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attn: Document Control Desk

MONTHLY OPERATING REPORT
SALEM UNIT NO. 1
DOCKET NO. 50-272

Gentlemen:

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original Monthly Operating report for May 1998 is attached.

Sincerely,

A. C. Bakken III
General Manager -
Salem Operations

/rbk
Enclosures

C Mr. H. J. Miller
Regional Administrator USNRC, Region 1
475 Allendale Road
King of Prussia, PA 19046

DOCKET NO.: 50-272
UNIT: Salem 1
DATE: 6/15/98
COMPLETED BY: R. Knieriem
TELEPHONE: (609) 339-1782

Reporting Period: **May 1998**

OPERATING DATA REPORT

Design Electrical Rating (MWe-Net)	**1115**		
Maximum Dependable Capacity (MWe-Net)	**1106**		
	Month	**Year-to-date**	**Cumulative**
No. of hours reactor was critical	**744**	**1303**	**105684**
No. of hours generator was on line (service hours)	**744**	**1062**	**101451**
Unit reserve shutdown hours	**0**	**0**	**0**
Net Electrical Energy (MWH)	**778176**	**960076**	**101096619**

UNIT SHUTDOWNS

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/COMMENT

(1) Reason

A - Equipment Failure (Explain)
B - Maintenance or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training/License Examination
F - Administrative
G - Operational Error (Explain)
H – Other

(2) Method

1 - Manual
2 - Manual Trip/Scram
3 - Automatic Trip/Scram
4 - Continuation
5 - Other (Explain)

Summary:

Salem Unit 1 began the month in power ascension, operating at 70% power. The unit achieved 100% power on May 2. Power was reduced to 90% on May 5, to perform Condensate Polisher maintenance. The unit returned to full power on May 8. Power was reduced on May 14, for performance of a 40% load rejection test and again on May

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15, for the performance of a Steam Generator Feed pump trip and turbine runback test. The unit returned to full power on May 16, and operated at full power for the remainder of the month.

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SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS FOR THE SALEM UNIT 1 GENERATING STATION

MONTH: MAY 1998

The following items completed during **May 1998** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant; nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes - Summary of Safety Evaluations

1EC-3394, Pkg. 1, Feedwater Flow Meter Nozzles Replacement

This design change involved the removal and replacement of the Feedwater Flow Meter Nozzles with ASME flow nozzles. The design change also added a Chordal Type, Leading Edge Ultrasonic Flow meter in each of the four feedwater supply lines.

This design change does not negatively impact any accident response nor does it increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

1EC-3561, Pkg. 1, Modification To Station Air Compressor Controls

This modification changed the Station Air Compressor 1SAE1 from the existing total closure control scheme to a total closure or constant pressure control scheme. This configuration was proposed by the compressor vendor to reduce cycling that normally occurs during low load operation.

This design change does not negatively impact any accident response nor does it increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

Temporary Modifications - Summary of Safety Evaluations

There were no changes in this category implemented during May 1998.

Procedures - Summary of Safety Evaluations

There were no changes in this category implemented during May 1998.

UFSAR Change Notices - Summary of Safety Evaluations

UFSAR Change Notice 98-017, Accumulator Sample Line Purging

This revision reflects changes to the procedure for purging of the Reactor Coolant system accumulator sample lines to a different location than currently described in the UFSAR. This proposed change will allow accumulator purges to other plant systems, such as the liquid waste system.

This design change does not negatively impact any accident response nor does it increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

UFSAR Change Notice S98-013, Nuclear Business Unit Organization Change – Succession Of Authority

This change revises the succession of authority and responsibility for Salem to reflect an Operations led organization.

This change does not negatively impact any accident response nor does it increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

Deficiency Reports - Summary of Safety Evaluations

There were no changes in this category implemented during May 1998.

Other - Summary of Safety Evaluations

Safety Evaluation – Auxiliary Feedwater Weld Repair Retest

This Safety Evaluation considered an alternate retest of a weld repair performed on Auxiliary Feedwater piping for which the proper code case (Code Case N-416-1) was not imposed. The error was identified post-installation. Proper use of the code case would have required a hydrostatic test or a root pass dye penetrant examination. This safety evaluation considered a radiographic test in lieu of the root pass dye penetrant examination.

This change does not negatively impact any accident response nor does it increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.