



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

JAN 15 1998

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U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Attn: Document Control Desk

**MONTHLY OPERATING REPORT  
SALEM UNIT NO. 2  
DOCKET NO. 50-311**

Gentlemen:

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original Monthly Operating Report for December, 1997, is attached.

Sincerely,

A. C. Bakken III  
General Manager -  
Salem Operations

260063

RBK/tcp  
Enclosures

C Mr. H. J. Miller  
Regional Administrator USNRC, Region 1  
475 Allendale Road  
King of Prussia, PA 19046

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The power is in your hands.

SALEM GENERATING STATION DOCKET NO.: 50-311  
UNIT: Salem 2  
DATE: 1/15/98  
COMPLETED BY: R. Knieriem  
TELEPHONE: (609) 339-1782

**MONTHLY OPERATING SUMMARY**

**MONTH DECEMBER 1997**

Salem Unit 2 began the month of December operating at full power. Full power operation continued until December 21, when power was reduced to repair a broken stem on the 23 Heater Drain Pump Recirculation Valve. The unit returned to service on December 24, and completed the month at full power.

DOCKET NO.:

50-311

UNIT: Salem 2

DATE: 01/09/98

COMPLETED BY: F. Todd

TELEPHONE: (609) 339-1316

**OPERATING DATA REPORT**

**OPERATING STATUS**

1	Reporting Period DECEMBER 1997	Hours in Report Period	<u>744</u>
2	Currently Authorized Power Level (MWt)		<u>3411</u>
	Max Dependable Capacity (MWe-Net)		<u>1106</u>
	Design Electrical Rating (MWe-Net)		<u>1115</u>
3	Power level to which restricted (if any) (MWe Net)		<u>None</u>
4	Reason For Restriction (if any)		

	<u>This Month</u>	<u>Yr To Date</u>	<u>Cumulative</u>	
5	No. of hours reactor was critical	744	3123	81207
6	Reactor reserve shutdown hours	0.0	0.0	0.0
7	Hours generator on line	709	2834	78064
8	Unit reserve shutdown hours	0.0	0.0	0.0
9	Gross thermal energy generated (MWH)	2335319	8249240	196030245
10	Gross electrical energy generated (MWH)	778320	2700083	81348681
11	Net electrical energy generated (MWH)	745517	2472951	77175585
12	Unit Service Factor	95.3%	32.4%	49.2%
13	Unit Availability Factor	95.3%	32.4%	49.2%
14	Unit Capacity Factor (MDC)	90.6%	25.5%	43.9%
15	Unit Capacity Factor (DER)	89.9%	25.3%	43.6%
16	Unit Forced Outage Rate	7.7%	67.7%	33.4%
17	Shutdowns scheduled over next 6 months (type, date, duration):			
18	If shutdown at end of report period, estimated date of Startup:			

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**OPERATING DATA REPORT**  
**UNIT SHUTDOWNS AND POWER REDUCTIONS**

MONTH DECEMBER 1997

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTION/COMMENT
3504	12/21 - 12/24	F	0	A	4	Failed stem on the 23 Heater Drain Pump Recirculation Valve. Power was reduced and the turbine was removed from service. Reactor remained critical.

1) Reason

- A - Equipment Failure (Explain)
- B - Maintenance or Test
- C - Refueling
- D - Regulatory Restriction
- E - Operator Training/License Examination
- F - Administrative
- G - Operational Error (Explain)
- H - Other

(2) Method

- 1 - Manual
- 2 - Manual Trip
- 3 - Automatic Trip/Scram
- 4 - Other (Explain)

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**AVERAGE DAILY UNIT POWER LEVEL**

**MONTH DECEMBER 1997**

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1,103	17	1,105
2	1,103	18	1,106
3	1,104	19	1,106
4	1,105	20	1,105
5	1,106	21	905
6	1,107	22	128
7	1,102	23	194
8	1,102	24	521
9	1,103	25	955
10	1,107	26	1,084
11	1,106	27	1,099
12	1,104	28	1,104
13	1,100	29	1,105
14	1,100	30	1,101
15	1,104	31	1,102
16	1,105		

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**SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS**  
**FOR THE SALEM UNIT 2 GENERATING STATION**

**MONTH DECEMBER 1997**

The following items completed during **December 1997** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

**Design Changes Summary of Safety Evaluations**

**2EC-3340, Pkg. 1 Background Subtraction Performance Upgrade & Declassification Of Post-Accident Plant Vent Monitor 2R45A, B, C, and D.**

This design installed a small, keep-alive source in the 2R45A and 2R45D detectors to prevent "low fail" occurrences and nuisance alarms. The modification also changed the classification of the 2R45 monitors to be non-safety related Reg Guide 1.97 Accident monitors.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

**2EC-3395, Pkg. 1, Moisture Separator Reheater Drain And Vent System Modifications.** This design change involved piping replacement and rerouting for the MSR drain and vent system to reduce excessive vibration.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

**Temporary Modifications Summary of Safety Evaluations**

There were no changes in this category implemented during December, 1997.

**Procedures Summary of Safety Evaluations**

There were no changes in this category implemented during December, 1997.

**UFSAR Change Notices Summary of Safety Evaluations**

There were no changes in this category implemented during December, 1997.

**Deficiency Reports Summary of Safety Evaluations**

There were no changes in this category implemented during December, 1997.

**Other Summary of Safety Evaluation**

**Safety Evaluation - Removal Of Insulation Of 23SG Main Steam Sensing Lines.** This Safety Evaluation evaluated the temporary removal of insulation from the 23 Main Steam and process sensing lines to demonstrate the effects of condensate buildup in the sensing lines on steam flow indication.

This Safety Evaluation does not negatively impact any accident response. It does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this Technical Specification Bases change does not involve an Unreviewed Safety Question.